

# Summary of Stress Analysis Results for the US-APWR Reactor Coolant Loop Branch Piping

**Non-Proprietary Version**

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**Revision History**

| Revision | Page  | Description  |
|----------|---|--|
| 0        | All   | Original Issue   |
| 1        | <p>i,ii,iii,iv,v</p> <p>ii, iii</p> <p>iv</p> <p>v</p> <p>vi</p> <p>1-1</p> <p>1-6 to 1-11</p> <p>2-1</p> | <p>Revised analysis condition and result based on following items.</p> <p>a) RHR Return Line (RH05,RH06) is separated from Accumulator Line (SI01,SI02).</p> <p>b) Design transient of Pressurizer Surge Line (RC01), Pressurizer Spray Line (RC02) and Pressurizer Safety Valve Line (RC04) is changed.</p> <p>c) Thermal analysis result is changed responding to detail calculation.</p> <p>Changed description for correction and clarification.</p> <p>Changed page number.</p> <p>Added Appendix 1-7, 1-8 and changed number of following Appendix.</p> <p>Changed description for clarification.</p> <p>Added Table 8.1-9 and changed following Table number.</p> <p>Added Figure 1.0-7 ,1.0-8 and changed following Figure number.</p> <p>Add Acronyms and changed description for clarification.</p> <p>Added description of RHR Return Line (RH05,RH06) and changed description for clarification.</p> <p>Added Figure 1.0-7, 1.0-8 and changed following Figure number.</p> <p>Charged Figure 1.0-9, 1.0-10 reflecting RHR Return Line separation.</p> <p>Added results of RHR Return Line (RH05,RH06).</p> <p>Changed results reflecting re-calculation.</p> |

**Revision History (Contd.)**

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|--------|--|--|
| 1      | 3-1  | Changed description for clarification.<br>Changed description of LBB evaluation.                                 |
|        | 4-1  | Changed description for clarification.   |
|        | 5-1  | Changed description for correction and clarification.<br>Added description about assumption of welding location. |
|        | 6-1,6-2  | Changed description for clarification.   |
|        | 7-1  | Revised revision number of document.   |
|        | 8-1  | Corrected design temperature and pressure.   |
|        | 8-2,8-3  | Changed description for clarification.   |
|        | 8-4  | Added description of Table 8.1-9 and changed following Table number.   |
|        | 8-5  | Added column of RHR Return Line (RH05,RH06).<br>Corrected mark of application of Accidental Load.                |
|        | 8-6,8-9,8-12,8-15  | Changed Remark for clarification.  |
|        | 8-6  | Corrected transient occurrence of Load regulation.   |
|        | 8-24 to 8-42   | Added Table 8.1-9 and changed following Table number.  |
|        | 9-5  | Changed description for clarification.<br>Added description about damping which apply to Pressurizer model.      |
|        | 9-14   | Changed description for clarification.   |
|        | 10-1   | Changed version of program.  |
|        | 11-1   | Changed description of LBB evaluation.   |
|        | 11-2,11-3  | Changed results reflecting re-calculation.   |
|        | 12-1   | Revised revision number and corrected document title.  |
|        | A1-1-2   | Changed description for clarification.   |
|        | A1-1-3   | Corrected misdescription.  |
| A1-1-4 | Changed piping isometrics reflecting analysis model modification.  |  |
| A1-1-6 | Corrected support stiffness.<br>Added point 9020 for clarification.<br>Corrected note about support point. |  |

**Revision History (Contd.)**

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|---|---|---|
| 1 | <p>A1-1-7</p> <p>A1-1-9 to<br/>A1-1-17</p> <p>A1-1-18 to<br/>A1-1-24</p> <p>A1-1-25</p> <p>A1-1-40</p> <p>A1-1-41,A1-1-42</p> <p>A1-1-43 to<br/>A-1-1-45</p> <p>A1-1-46 to<br/>A1-1-55</p> <p>A1-1-56</p> <p>A1-2-2</p> <p>A1-2-5 to A1-2-9</p> <p>A1-2-14 to<br/>A1-2-16</p> <p>A1-2-17</p> <p>A1-2-18</p> <p>A1-2-20 to<br/>A1-2-27</p> | <p>Changed figure of anchor coordinate for clarification.</p> <p>Corrected thermal displacement.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Corrected transient temperature and Pressure.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Added Earthquake Loads for clarification.</p> <p>Corrected Level C,D condition.</p> <p>Changed analysis model.</p> <p>Changed result of eigenvalue analysis reflecting re-calculation.</p> <p>Changed mode shape reflecting re-calculation.</p> <p>Changed result of thermal analysis based on design transient modification.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected transient temperature and Pressure.</p> <p>Corrected transient occurrence.</p> <p>Added Earthquake Loads for clarification.</p> <p>Changed stress analysis results reflecting re-calculation.</p> <p>Changed description for clarification.</p> <p>Changed piping isometrics reflecting analysis model modification.</p> <p>Changed support stiffness reflecting support modification.</p> <p>Corrected support stiffness.</p> <p>Added point 9030 for clarification.</p> <p>Corrected note about support point.</p> <p>Changed figure of anchor coordinate for clarification.</p> <p>Corrected thermal displacement.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected transient temperature.</p> |
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**Revision History (Contd.)**

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| 1 | A1-2-28 to<br>A1-2-45   | Corrected transient temperature.<br>Changed Load No. from CA No. for clarification.<br>Added Earthquake Loads for clarification.<br>Changed piping section applied transient.  |
|   | A1-2-46                 | Corrected Level C,D condition.<br>Changed piping section applied transient.  |
|   | A1-2-63                 | Corrected seismic displacement.<br>Change of equipment Nozzle or Building.   |
|   | A1-2-65                 | Changed analysis model.  |
|   | A1-2-66 to<br>A1-2-71   | Changed result of eigenvalue analysis reflecting re-calculation.   |
|   | A1-2-72 to<br>A1-2-74   | Changed mode shape reflecting re-calculation.  |
|   | A1-2-75 to<br>A1-2-152  | Changed result of thermal analysis based on design transient modification.<br>Changed Load No. from CA No. for clarification.<br>Added Earthquake Loads for clarification.<br>Corrected transient temperature.<br>Corrected transient occurrence.<br>Changed piping section applied transient. |
|   | A1-2-153 to<br>A1-2-154 | Changed stress analysis results reflecting re-calculation.   |
|   | A1-3-2                  | Changed description for clarification.   |
|   | A1-3-3 to A1-3-5        | Changed description for clarification.<br>Changed piping section applied transient.  |
|   | A1-3-6 to A1-3-8        | Changed piping isometrics reflecting analysis model modification.  |
|   | A1-3-14 to<br>A1-3-16   | Changed support stiffness reflecting support modification.   |
|   | A1-3-17,A1-3-18         | Corrected support stiffness.<br>Changed point 9010 for clarification.<br>Corrected note about support point.   |
|   | A1-3-19                 | Changed figure of anchor coordinate for clarification.   |

**Revision History (Contd.)**

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| 1 | <p>A1-3-21</p> <p>A1-3-22 to<br/>A1-3-42</p> <p>A1-3-43</p> <p>A1-3-55</p> <p>A1-3-58</p> <p>A1-3-59,A1-3-60</p> <p>A1-3-61 to<br/>A1-3-65</p> <p>A1-3-66 to<br/>A1-3-68</p> <p>A1-3-69 to<br/>A1-3-97</p> <p>A1-3-99</p> <p>A1-4-2</p> <p>A1-4-3</p> <p>A1-4-4</p> <p>A1-4-6</p> <p>A1-4-8</p> | <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected transient temperature and pressure.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Added Earthquake Loads for clarification.</p> <p>Corrected Level C,D condition.</p> <p>Corrected seismic displacement.</p> <p>Changed analysis model.</p> <p>Changed description of Water hammer analysis model.</p> <p>Changed result of eigenvalue analysis reflecting re-calculation.</p> <p>Changed mode shape reflecting re-calculation.</p> <p>Changed result of thermal analysis reflecting detail calculation.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Corrected transient temperature and pressure.</p> <p>Corrected transient occurrence.</p> <p>Added Earthquake Loads for clarification.</p> <p>Changed stress analysis results reflecting re-calculation.</p> <p>Changed description for clarification.</p> <p>Changed description for clarification.</p> <p>Changed piping section applied transient.</p> <p>Changed piping isometrics reflecting analysis model modification.</p> <p>Corrected support stiffness.</p> <p>Corrected note about support point.</p> <p>Changed figure of anchor coordinate for clarification.</p> <p>Changed Load No. from CA No. for clarification.</p> |
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**Revision History (Contd.)**

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| 1 | <p>A1-4-9 to<br/>A1-4-14</p> <p>A1-4-15</p> <p>A1-4-23,A1-4-24</p> <p>A1-4-25</p> <p>A1-4-26</p> <p>A1-4-27 to<br/>A1-4-29</p> <p>A1-4-30 to<br/>A1-4-41</p> <p>A1-4-42</p> <p>A1-5-2</p> <p>A1-5-6, A1-5-7</p> <p>A1-5-11</p> <p>A1-5-13</p> <p>A1-5-15 to<br/>A1-5-19</p> | <p>Corrected transient temperature.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Added Earthquake Loads for clarification.</p> <p>Changed piping section applied transient.</p> <p>Corrected Level C,D condition.</p> <p>Changed piping section applied transient.</p> <p>Changed analysis model.</p> <p>Changed description of Water hammer analysis model.</p> <p>Changed result of eigenvalue analysis reflecting re-calculation.</p> <p>Changed mode shape reflecting re-calculation.</p> <p>Changed result of thermal analysis based on design transient modification.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Corrected transient temperature.</p> <p>Corrected transient occurrence.</p> <p>Added Earthquake Loads for clarification.</p> <p>Changed piping section applied transient.</p> <p>Changed stress analysis results reflecting re-calculation.</p> <p>Changed description for clarification.</p> <p>Changed piping isometrics reflecting analysis model modification.</p> <p>Corrected support stiffness.</p> <p>Corrected note about support point.</p> <p>Changed figure of anchor coordinate for clarification.</p> <p>Corrected thermal displacement.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Corrected transient temperature.</p> |
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**Revision History (Contd.)**

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| 1 | <p>A1-5-20 to<br/>A1-5-52</p> <p>A1-5-53</p> <p>A1-5-61</p> <p>A1-5-64</p> <p>A1-5-65,A1-5-66</p> <p>A1-5-67 to<br/>A1-5-69</p> <p>A1-5-70 to<br/>A1-5-110</p> <p>A1-5-112</p> <p>A1-6-2</p> <p>A1-6-5,A1-6-6</p> <p>A1-6-8</p> <p>A1-6-9</p> <p>A1-6-10</p> <p>A1-6-12 to<br/>A1-6-16</p> <p>A1-6-17 to<br/>A1-6-37</p> <p>A1-6-38</p> <p>A1-6-46</p> <p>A1-6-48</p> | <p>Corrected transient temperature and Pressure.<br/>Changed Load No. from CA No. for clarification.<br/>Corrected Transient title for clarification.<br/>Added Earthquake Loads for clarification.</p> <p>Corrected Level C,D condition.</p> <p>Change of equipment Nozzle or Building.</p> <p>Changed analysis model.</p> <p>Changed result of eigenvalue analysis reflecting re-calculation.</p> <p>Changed mode shape reflecting re-calculation.</p> <p>Changed result of thermal analysis reflecting detail calculation.<br/>Changed Load No. from CA No. for clarification.<br/>Corrected Transient title for clarification.<br/>Corrected transient temperature and Pressure.<br/>Corrected transient occurrence.<br/>Added Earthquake Loads for clarification.</p> <p>Changed stress analysis results reflecting re-calculation.</p> <p>Changed description for clarification.</p> <p>Changed piping isometrics reflecting analysis model modification.</p> <p>Changed support stiffness reflecting support modification.</p> <p>Corrected support stiffness.<br/>Corrected note about support point.</p> <p>Changed figure of anchor coordinate for clarification.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Changed Load No. from CA No. for clarification.<br/>Added Earthquake Loads for clarification.</p> <p>Corrected Level C,D condition.</p> <p>Change of equipment Nozzle or Building.</p> <p>Changed analysis model.</p> |
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**Revision History (Contd.)**

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| 1 | <p>A1-6-49,A1-6-50</p> <p>A1-6-51 to<br/>A1-6-53</p> <p>A1-6-54 to<br/>A1-6-82</p> <p>A1-6-84</p> <p>A1-7-1</p> <p>A1-8-1</p> <p>Whole of<br/>Appendix 1-9</p> <p>A1-9-2</p> <p>A1-9-3</p> <p>A1-9-4</p> <p>A1-9-6</p> <p>A1-9-7</p> <p>A1-9-9 to<br/>A1-9-12</p> <p>A1-9-13 to<br/>A1-9-24</p> <p>A1-9-25</p> | <p>Changed result of eigenvalue analysis reflecting re-calculation.</p> <p>Changed mode shape reflecting re-calculation.</p> <p>Changed result of thermal analysis reflecting detail calculation.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Added Earthquake Loads for clarification.</p> <p>Changed stress analysis results reflecting re-calculation.</p> <p>Added Appendix 1-7 along with RHR Return A Line (RH05) separation.</p> <p>Added Appendix 1-8 along with RHR Return B Line (RH06) separation.</p> <p>Changed number of Appendix and associated page number, Table number and Figure number.</p> <p>Changed description for clarification.</p> <p>Changed piping section applied transient.</p> <p>Changed piping isometrics reflecting analysis model modification.</p> <p>Changed support stiffness reflecting support modification.</p> <p>Corrected support stiffness.</p> <p>Corrected note about support point.</p> <p>Changed figure of anchor coordinate for clarification.</p> <p>Corrected thermal displacement.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Corrected transient temperature.</p> <p>Corrected transient temperature and Pressure.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Added Earthquake Loads for clarification.</p> <p>Changed piping section applied transient.</p> <p>Corrected Level C,D condition.</p> <p>Changed piping section applied transient.</p> |
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**Revision History (Contd.)**

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| 1 | <p>A1-9-29 to<br/>A1-9-31</p> <p>A1-9-32</p> <p>A1-9-34</p> <p>A1-9-35</p> <p>A1-9-36 to<br/>A1-9-38</p> <p>A1-9-39 to<br/>A1-9-48</p><br><p>A1-9-49</p> <p>A1-9-51</p> <p>Whole of<br/>Appendix A1-10</p> <p>A1-10-2</p> <p>A1-10-3</p> <p>A1-10-4</p><br><p>A1-10-6</p><br><p>A1-10-7</p> <p>A1-10-9 to<br/>A1-10-12</p> | <p>Changed Floor response curve.</p> <p>Change of equipment Nozzle or Building.</p> <p>Changed analysis model.</p> <p>Changed result of eigenvalue analysis reflecting re-calculation.</p> <p>Changed mode shape reflecting re-calculation.</p> <p>Changed result of thermal analysis reflecting detail calculation.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Corrected transient temperature and Pressure.</p> <p>Corrected transient occurrence.</p> <p>Added Earthquake Loads for clarification.</p> <p>Changed piping section applied transient.</p> <p>Changed stress analysis results reflecting re-calculation.</p> <p>Added LBB evaluation results.</p> <p>Changed number of Appendix and associated page number, Table number and Figure number.</p> <p>Changed description for clarification.</p> <p>Changed piping section applied transient.</p> <p>Changed piping isometrics reflecting analysis model modification.</p> <p>Changed support stiffness reflecting support modification.</p> <p>Corrected support stiffness.</p> <p>Corrected note about support point.</p> <p>Changed figure of anchor coordinate for clarification.</p> <p>Corrected thermal displacement.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Corrected transient temperature.</p> |
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**Revision History (Contd.)**

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| 1 | <p>A1-10-13 to<br/>A1-10-24</p> <p>A1-10-25</p> <p>A1-10-29 to<br/>A1-10-31</p> <p>A1-10-32</p> <p>A1-10-34</p> <p>A1-10-35</p> <p>A1-10-36 to<br/>A1-10-38</p> <p>A1-10-39 to<br/>A1-10-48</p> <p>A1-10-49</p> <p>A1-10-51</p> <p>Whole of<br/>Appendix A1-11</p> <p>A1-11-2</p> <p>A1-11-6</p> <p>A1-11-7</p> | <p>Corrected transient temperature and Pressure.<br/>Changed Load No. from CA No. for clarification.<br/>Corrected Transient title for clarification.<br/>Added Earthquake Loads for clarification.<br/>Changed piping section applied transient.</p> <p>Corrected Level C,D condition.<br/>Changed piping section applied transient.</p> <p>Changed Floor response curve.</p> <p>Change of equipment Nozzle or Building.</p> <p>Changed analysis model.</p> <p>Changed result of eigenvalue analysis reflecting re-calculation.</p> <p>Changed mode shape reflecting re-calculation.</p> <p>Changed result of thermal analysis reflecting detail calculation.<br/>Changed Load No. from CA No. for clarification.<br/>Corrected Transient title for clarification.<br/>Corrected transient temperature and Pressure.<br/>Corrected transient occurrence.<br/>Added Earthquake Loads for clarification.<br/>Changed piping section applied transient.</p> <p>Changed stress analysis results reflecting re-calculation.</p> <p>Added LBB evaluation results.</p> <p>Changed number of Appendix and associated page number,<br/>Table number and Figure number.</p> <p>Changed description for clarification.</p> <p>Corrected support stiffness.<br/>Changed point 9010 for clarification.</p> <p>Changed figure of anchor coordinate for clarification.</p> |
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**Revision History (Contd.)**

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|---|---|---|
| 1 | <p>A1-11-9 to<br/>A1-11-12</p> <p>A1-11-13 to<br/>A1-11-21</p> <p>A1-11-22</p> <p>A1-11-29</p> <p>A1-11-32</p> <p>A1-11-33 to<br/>A1-11-35</p> <p>A1-11-36 to<br/>A1-11-44</p> <p>A1-11-46</p> <p>Whole of<br/>Appendix A1-12</p> <p>A1-12-2</p> <p>A1-12-6</p> <p>A1-12-7</p> <p>A1-12-9 to<br/>A1-12-12</p> | <p>Corrected thermal displacement.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Corrected transient temperature and Pressure.</p> <p>Corrected transient temperature and Pressure.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Added Earthquake Loads for clarification.</p> <p>Corrected Level C,D condition.</p> <p>Change of equipment Nozzle or Building.</p> <p>Changed result of eigenvalue analysis reflecting re-calculation.</p> <p>Changed mode shape reflecting re-calculation.</p> <p>Changed result of thermal analysis reflecting detail calculation.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Corrected transient temperature and Pressure.</p> <p>Corrected transient occurrence.</p> <p>Added Earthquake Loads for clarification.</p> <p>Change stress analysis results reflecting re-calculation.</p> <p>Changed number of Appendix and associated page number, Table number and Figure number.</p> <p>Changed description for clarification.</p> <p>Corrected support stiffness.</p> <p>Changed point 9010 for clarification.</p> <p>Changed figure of anchor coordinate for clarification.</p> <p>Corrected thermal displacement.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected Transient title for clarification.</p> <p>Corrected transient temperature and Pressure.</p> |
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**Revision History (Contd.)**

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|---|---|---|
| 1 | <p>A1-12-13 to<br/>A1-12-21</p> <p>A1-12-22</p> <p>A1-12-29</p> <p>A1-12-32</p> <p>A1-12-33 to<br/>A1-12-35</p> <p>A1-12-36 to<br/>A1-12-44</p> <p>A1-12-46</p> <p>Whole of<br/>Appendix A1-13</p> <p>A1-13-2</p> <p>A1-13-4</p> <p>A1-13-5</p> <p>A1-13-7</p> <p>A1-13-8</p> <p>A1-13-10 to<br/>A1-13-14</p> <p>A1-13-15 to<br/>A1-13-30</p> | <p>Corrected transient temperature and Pressure.<br/>Changed Load No. from CA No. for clarification.<br/>Corrected Transient title for clarification.<br/>Added Earthquake Loads for clarification.</p> <p>Corrected Level C,D condition.</p> <p>Change of equipment Nozzle or Building.</p> <p>Changed result of eigenvalue analysis reflecting re-calculation.</p> <p>Changed mode shape reflecting re-calculation.</p> <p>Changed result of thermal analysis reflecting detail calculation.<br/>Changed Load No. from CA No. for clarification.<br/>Corrected Transient title for clarification.<br/>Corrected transient temperature and Pressure.<br/>Corrected transient occurrence.<br/>Added Earthquake Loads for clarification.</p> <p>Changed stress analysis results reflecting re-calculation.</p> <p>Changed number of Appendix and associated page number,<br/>Table number and Figure number.</p> <p>Changed description for clarification.</p> <p>Changed piping isometrics reflecting analysis model<br/>modification.</p> <p>Changed Concentrated mass of support reflecting support<br/>modification.</p> <p>Corrected support stiffness.<br/>Corrected note about support point.</p> <p>Changed figure of anchor coordinate for clarification.</p> <p>Corrected thermal displacement.<br/>Changed Load No. from CA No. for clarification.<br/>Corrected transient temperature.</p> <p>Corrected transient temperature.<br/>Changed Load No. from CA No. for clarification.<br/>Added Earthquake Loads for clarification.</p> |
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**Revision History (Contd.)**

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| 1 | A1-13-31                   | Corrected Level C,D condition.  |
|   | A1-13-39                   | Corrected seismic displacement.<br>Change of equipment Nozzle or Building.  |
|   | A1-13-40                   | Corrected DBPB displacement.  |
|   | A1-13-41                   | Changed analysis model.   |
|   | A1-13-42                   | Changed result of eigenvalue analysis reflecting re-calculation.  |
|   | A1-13-43 to<br>A1-13-45    | Changed mode shape reflecting re-calculation.   |
|   | A1-13-46 to<br>A1-13-62    | Changed result of thermal analysis reflecting detail calculation.<br>Changed Load No. from CA No. for clarification.<br>Corrected transient temperature.<br>Added Earthquake Loads for clarification. |
|   | A1-13-64                   | Changed stress analysis results reflecting re-calculation.  |
|   | Whole of<br>Appendix A1-14 | Changed number of Appendix and associated page number,<br>Table number and Figure number.   |
|   | A1-14-2                    | Changed description for clarification.  |
|   | A1-14-3                    | Corrected misdescription.<br>Changed piping section applied transient.  |
|   | A1-14-4                    | Corrected piping specification due to omission.   |
|   | A1-14-5 to<br>A1-14-7      | Changed piping isometrics reflecting analysis model<br>modification.  |
|   | A1-14-8                    | Changed Concentrated mass of support reflecting support<br>modification.  |
|   | A1-14-10                   | Changed support stiffness reflecting support modification.<br>Corrected support stiffness.  |
|   | A1-14-11                   | Changed support stiffness reflecting support modification.<br>Corrected support stiffness.<br>Corrected note about support point.   |
|   | A1-14-12                   | Changed figure of anchor coordinate for clarification.  |
|   | A1-14-14 to<br>A1-14-17    | Corrected thermal displacement.<br>Changed Load No. from CA No. for clarification.<br>Corrected transient temperature.  |

**Revision History (Contd.)**

|   |                            |   |
|---|----------------------------|---|
| 1 | A1-14-18 to<br>A1-14-44    | Corrected transient temperature.<br>Changed Load No. from CA No. for clarification.<br>Added Earthquake Loads for clarification.  |
|   | A1-14-45                   | Corrected Level C,D condition.  |
|   | A1-14-52                   | Corrected seismic displacement.   |
|   | A1-14-53                   | Change of equipment Nozzle or Building.   |
|   | A1-14-55                   | Changed analysis model.   |
|   | A1-14-56                   | Changed result of eigenvalue analysis reflecting re-calculation.  |
|   | A1-14-57 to<br>A1-14-59    | Changed mode shape reflecting re-calculation.   |
|   | A1-14-60 to<br>A1-14-88    | Changed result of thermal analysis reflecting detail calculation.<br>Changed Load No. from CA No. for clarification.<br>Corrected transient temperature.<br>Added Earthquake Loads for clarification. |
|   | A1-14-90                   | Changed stress analysis results reflecting re-calculation.  |
|   | Whole of<br>Appendix A1-15 | Changed number of Appendix and associated page number,<br>Table number and Figure number.   |
|   | A1-15-2                    | Changed description for clarification.  |
|   | A1-15-3                    | Changed piping section applied transient.   |
|   | A1-15-6                    | Changed point 9010 for clarification.<br>Corrected support stiffness.   |
|   | A1-15-7                    | Changed figure of anchor coordinate for clarification.  |
|   | A1-15-9 to<br>A1-15-13     | Changed Load No. from CA No. for clarification.   |
|   | A1-15-14 to<br>A1-15-31    | Corrected transient temperature.<br>Changed Load No. from CA No. for clarification.<br>Added Earthquake Loads for clarification.<br>Changed piping section applied transient.                         |
|   | A1-15-32                   | Corrected Level C,D condition.  |
|   | A1-15-39                   | Change of equipment Nozzle or Building.   |
|   | A1-15-42                   | Changed result of eigenvalue analysis reflecting re-calculation.  |

**Revision History (Contd.)**

|   |                            |  |
|---|----------------------------|--|
| 1 | A1-15-43 to<br>A1-15-45    | Changed mode shape reflecting re-calculation.  |
|   | A1-15-46 to<br>A1-15-63    | Changed result of thermal analysis reflecting detail calculation.<br>Changed Load No. from CA No. for clarification.<br>Corrected transient temperature.<br>Added Earthquake Loads for clarification.<br>Changed piping section applied transient. |
|   | A1-15-64                   | Changed stress analysis results reflecting re-calculation.   |
|   | Whole of<br>Appendix A1-16 | Changed number of Appendix and associated page number,<br>Table number and Figure number.  |
|   | A1-16-2                    | Changed description for clarification.   |
|   | A1-16-6                    | Changed point 9010 for clarification.<br>Corrected support stiffness.  |
|   | A1-16-7                    | Changed figure of anchor coordinate for clarification.   |
|   | A1-16-9 to<br>A1-16-13     | Changed Load No. from CA No. for clarification.  |
|   | A1-16-14 to<br>A1-16-31    | Corrected transient temperature.<br>Changed Load No. from CA No. for clarification.<br>Added Earthquake Loads for clarification.   |
|   | A1-16-32                   | Corrected Level C,D condition.   |
|   | A1-16-39                   | Change of equipment Nozzle or Building.  |
|   | A1-16-42                   | Changed result of eigenvalue analysis reflecting re-calculation.   |
|   | A1-16-43 to<br>A1-16-45    | Changed mode shape reflecting re-calculation.  |
|   | A1-16-46 to<br>A1-16-63    | Changed result of thermal analysis reflecting detail calculation.<br>Changed Load No. from CA No. for clarification.<br>Corrected transient temperature.<br>Added Earthquake Loads for clarification.  |
|   | A1-16-64                   | Changed stress analysis results reflecting re-calculation.   |
|   | Whole of<br>Appendix A1-17 | Changed number of Appendix and associated page number,<br>Table number and Figure number.  |
|   | A1-17-2                    | Changed description for clarification.   |



**Revision History (Contd.)**

|   |                            |   |
|---|----------------------------|---|
| 1 | A1-17-6                    | Changed point 9010 for clarification.<br>Corrected support stiffness.   |
|   | A1-17-7                    | Changed figure of anchor coordinate for clarification.  |
|   | A1-17-9 to<br>A1-17-13     | Changed Load No. from CA No. for clarification.   |
|   | A1-17-14 to<br>A1-17-31    | Corrected transient temperature.<br>Changed Load No. from CA No. for clarification.<br>Added Earthquake Loads for clarification.  |
|   | A1-17-32                   | Corrected Level C,D condition.  |
|   | A1-17-39                   | Change of equipment Nozzle or Building.   |
|   | A1-17-42                   | Changed result of eigenvalue analysis reflecting re-calculation.  |
|   | A1-17-43 to<br>A1-17-45    | Changed mode shape reflecting re-calculation.   |
|   | A1-17-46 to<br>A1-17-63    | Changed result of thermal analysis reflecting detail calculation.<br>Changed Load No. from CA No. for clarification.<br>Corrected transient temperature.<br>Added Earthquake Loads for clarification.             |
|   | A1-17-64                   | Changed stress analysis results reflecting re-calculation.  |
|   | Whole of<br>Appendix A1-18 | Changed number of Appendix and associated page number,<br>Table number and Figure number.   |
|   | A1-18-2                    | Changed description for clarification.  |
|   | A1-18-3                    | Changed piping section applied transient.   |
|   | A1-18-6                    | Change point 9010 for clarification.<br>Corrected support stiffness.  |
|   | A1-18-7                    | Changed figure of anchor coordinate for clarification.  |
|   | A1-18-9 to<br>A1-18-13     | Changed Load No. from CA No. for clarification.   |
|   | A1-18-14 to<br>A1-18-31    | Corrected transient temperature.<br>Changed Load No. from CA No. for clarification.<br>Corrected transient temperature.<br>Added Earthquake Loads for clarification.<br>Changed piping section applied transient. |

**Revision History (Contd.)**

|   |   |   |
|---|---|---|
| 1 | <p>A1-18-32</p> <p>A1-18-39</p> <p>A1-18-42</p> <p>A1-18-43 to<br/>A1-18-45</p> <p>A1-18-46 to<br/>A1-18-63</p> <p>A1-18-64</p> <p>A2-3</p> <p>A2-6</p> <p>A2-8</p> <p>A2-10</p> <p>A2-11</p> | <p>Corrected Level C,D condition.</p> <p>Changed piping section applied transient.</p> <p>Change of equipment Nozzle or Building.</p> <p>Changed result of eigenvalue analysis reflecting re-calculation.</p> <p>Changed mode shape reflecting re-calculation.</p> <p>Changed result of thermal analysis reflecting detail calculation.</p> <p>Changed Load No. from CA No. for clarification.</p> <p>Corrected transient temperature.</p> <p>Added Earthquake Loads for clarification.</p> <p>Changed piping section applied transient.</p> <p>Changed stress analysis results reflecting re-calculation.</p> <p>Changed description for clarification.</p> <p>Changed description for clarification.</p> <p>Added description of LBB evaluation result of Accumulator Line.</p> <p>Added the description of note.</p> <p>Added the BAC for Accumulator Line.</p> <p>Added tabulated BAC points of Accumulator Line.</p> |
| 2 | 8-3   | Corrected description of design basis pipe break loads.   |

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**Abstract**

This report provides a summary of the stress analyses results of Reactor Coolant Loop (RCL) Branch Piping in accordance with MHI's commitment letter (Reference 11) concerning the content of the Technical Report.

From the results summarized in this report and a review of the component design drawings, it is concluded that the US-APWR RCL Branch Piping satisfies all of the requirements of the Design Specification (Reference 1) for structural integrity, operability, and safety.

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## List of Acronyms

The following list defines the acronyms used in this document.

|      |  |
|------|--|
| ABS  | Absolute Sum                             |
| ACC  | Accumulator                              |
| APWR | Advanced Pressurized-Water Reactor       |
| ASME | American Society of Mechanical Engineers |
| BAC  | Bounding Analysis Curve                  |
| CVCS | Chemical and Volume Control System       |
| DBPB | Design Basis Pipe Break                  |
| DVI  | Direct Vessel Injection                  |
| FRS  | Floor Response Spectrum                  |
| FW   | Feedwater                                |
| IC   | Inner Concrete                           |
| ISM  | Independent Support Motion               |
| LBB  | Leak-Before-Break                        |
| LOCA | Loss-of-Coolant Accident                 |
| LOF  | Left-out-Force                           |
| MCP  | Main Coolant Pipe                        |
| MS   | Main Steam                               |
| NPS  | Nominal Pipe Size                        |
| N/A  | Not Applicable                           |
| OBE  | Operating Basis Earthquake               |
| PZR  | Pressurizer                              |
| RCL  | Reactor Coolant Loop                     |
| RCP  | Reactor Coolant Pump                     |
| RCS  | Reactor Coolant System                   |
| RHRS | Residual Heat Removal System             |
| RV   | Reactor Vessel                           |
| SAM  | Seismic Anchor Motion                    |
| SI   | Safety Injection                         |
| SG   | Steam Generator                          |
| SRSS | Square Root Sum of the Squares           |
| SSE  | Safe-Shutdown Earthquake                 |
| UF   | Usage Factor                             |

## **1.0 INTRODUCTION**

This Stress Analysis Technical Report is a non-certified version of the ASME Design Report for the US-APWR RCL Branch Piping that has been prepared to support the US-APWR DCD Review. The content of this report follows the ASME guidelines for Design Reports (Section III Division 1 Appendix C) (Reference 5).

Design loads (pressure, deadweight and seismic inertia loads including loads associated with thermal expansion anchor motion and Seismic Anchor Motion (SAM)) used for pipe stress analysis were computed based on the conditions specified in the Design Specification (Reference 1). The thermal stress specified in NRC Bulletin 88-08 was not considered since the valve system configurations and valve disk position adjustments were made to assure that a thermal oscillation would not be generated as described in DCD 3.12.5.9. As for the pressurizer surge line thermal stratification described in NRC Bulletin 88-11, structural analysis was carried out by setting the thermal stratification profile based on the thermal flow analysis results.

This Technical Report meets the requirements of the ASME Code Section III Division 1 NCA-3551.1 (Reference 5) by providing a summary of results and conclusions based upon detailed analyses that demonstrate the validity of the RCL Branch Piping component to meet the requirements of the Design Specification (Reference 1).

For the design of Class 1 piping (i.e. NB-3600 and NC-3600), the 1992 Edition including the 1992 Addenda of ASME Section III was used as required by 10CFR50.55a (b) (1) (iii).

The scope of this Stress Analysis Report includes the piping systems and components (PSC) of the following RCL Branch Piping whose boundaries are identified in Figures 1.0-1 through 1.0-18. The selection of the RCL Branch Piping is consistent with MHI's intent to complete the Stress Analyses of "Risk Significant" PSCs prior to construction, and to enable NRC's audit during the DCD review phase.

- RC01 Pressurizer Surge Line
- RC02 Pressurizer Spray Line
- RC03 Pressurizer Safety Depressurization Valve Line
- RC04 Pressurizer Safety Valve Line
- RH01 and RH02 RHRS Suction Loop A and B Lines
- RH05 and RH06 RHRS Return Loop A and B Lines
- SI01 and SI02 Accumulator Loop A and B Lines
- SI05 and SI06 DVI A and B Lines
- CS01 CVCS Charging Line
- CS02 CVCS Let Down Line
- CS04 through CS07 CVCS Seal Injection A, B, C and D Lines

For each of the above Branch Piping, the Scope of the Report provides the following:

- A Summary of the Specification
- The Loads and Load Combinations

- The structural model of the piping including supports, flanges, valves, equipment and penetrations.
- The results of the piping analysis in accordance with the piping Design Specification (Reference 1)
- A review of the calculated stresses including effects of stress intensification, demonstration of ASME III acceptability, and LBB applicability checks for LBB applied piping

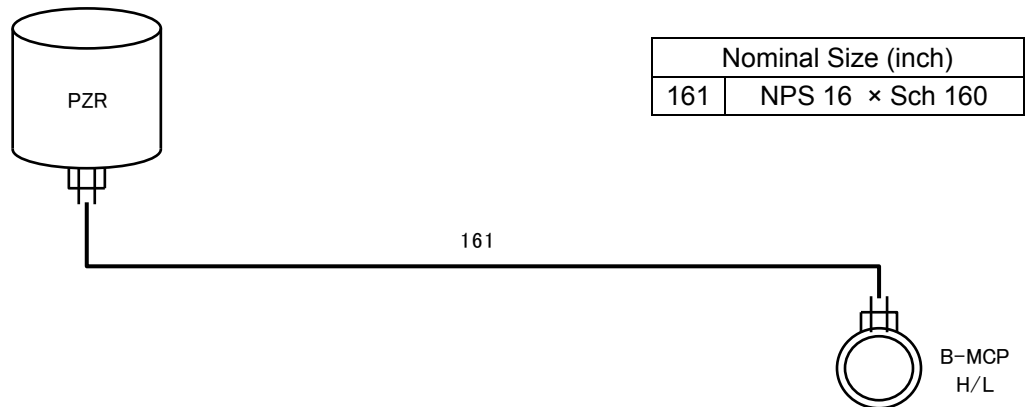


Figure 1.0-1 RC01 : Pressurizer surge line

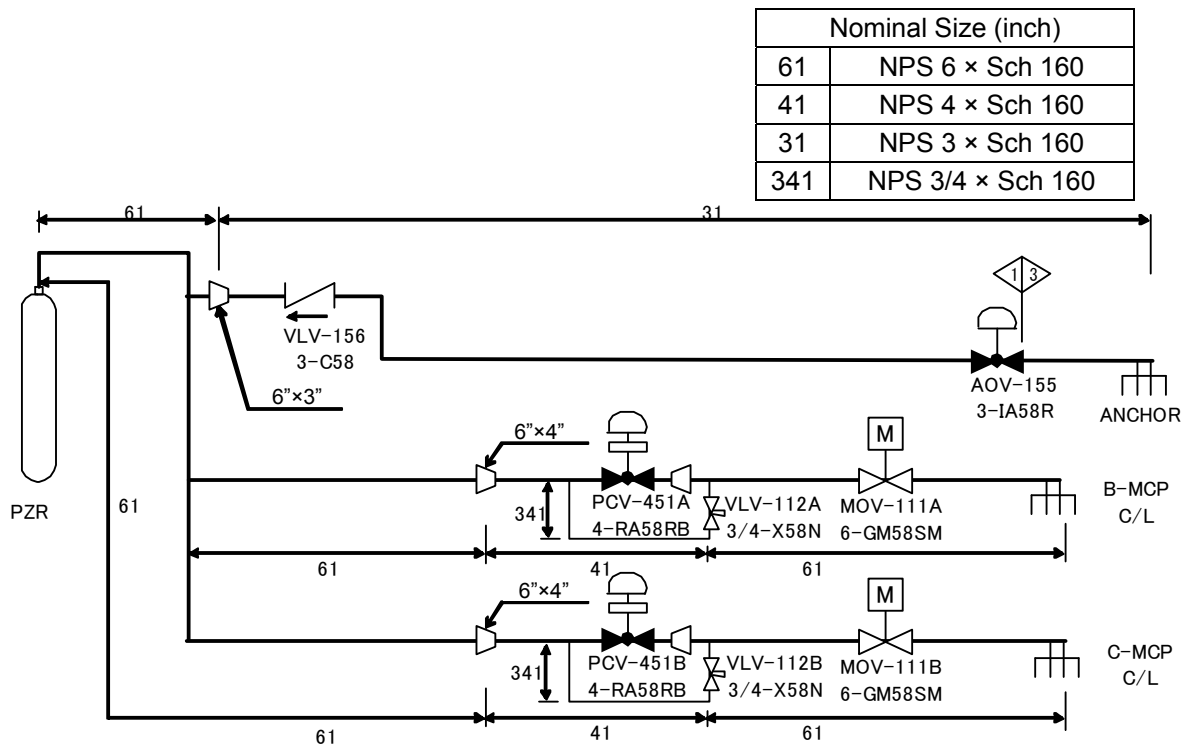


Figure 1.0-2 RC02 : Pressurizer spray line

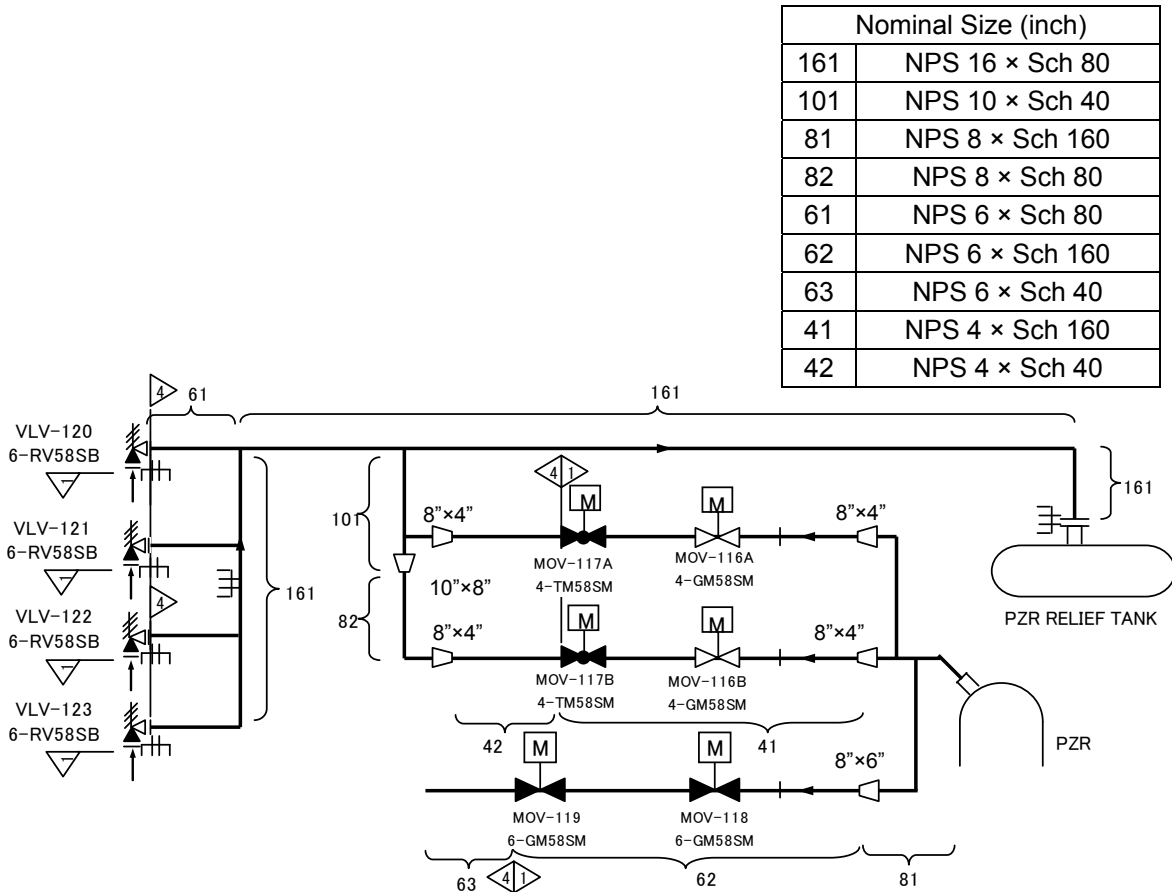


Figure 1.0-3 RC03: Pressurizer safety depressurization valve line

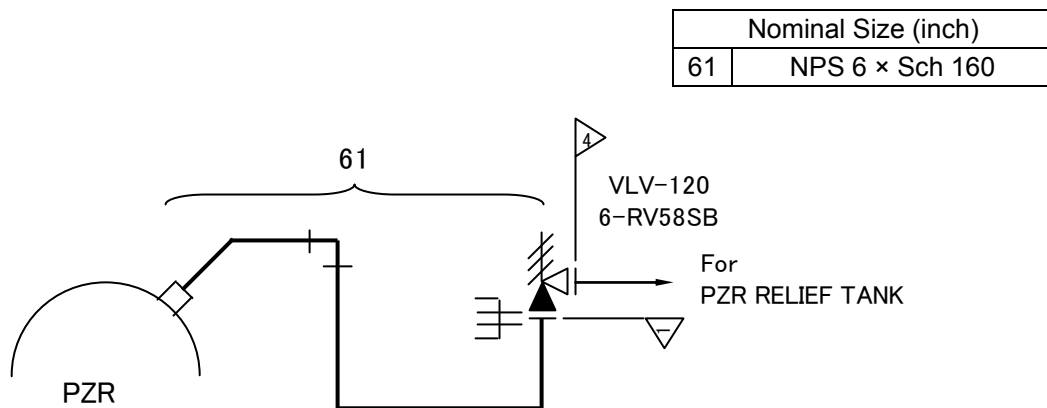


Figure 1.0-4 RC04: Pressurizer safety valve line

| Nominal Size (inch) |                  |
|---------------------|------------------|
| 101                 | NPS 10 × Sch 160 |
| 102                 | NPS 10 × Sch 80  |
| 81                  | NPS 8 × Sch 160  |
| 61                  | NPS 6 × Sch 80   |
| 41                  | NPS 4 × Sch 160  |
| 21                  | NPS 2 × Sch 160  |

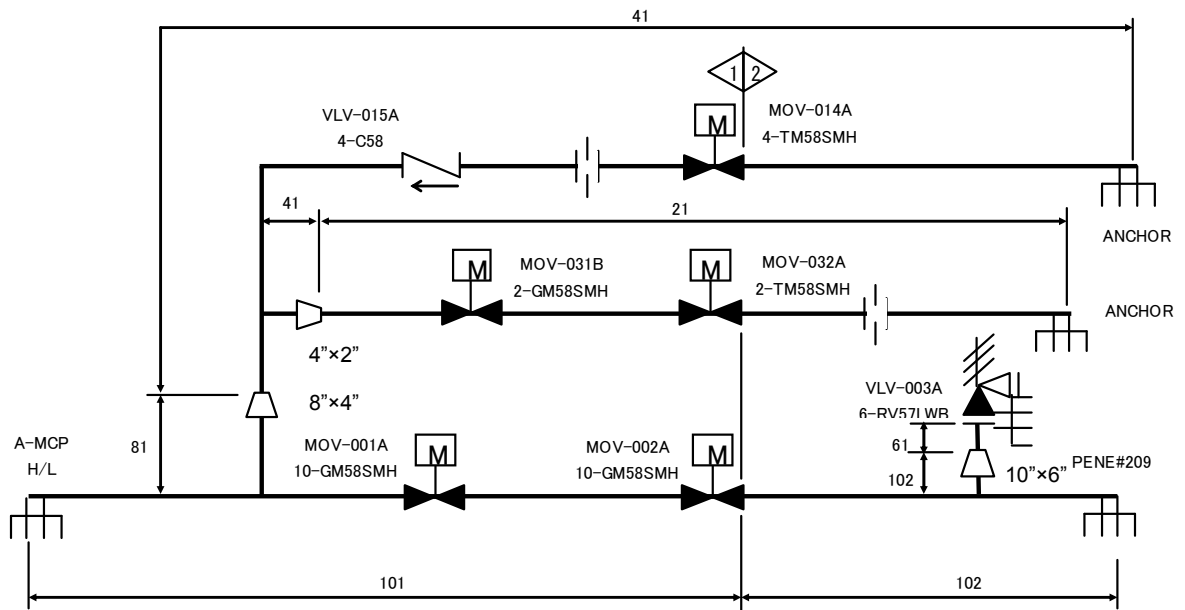


Figure 1.0-5 RH01: RHR suction loop A line

| Nominal Size (inch) |                  |
|---------------------|------------------|
| 101                 | NPS 10 × Sch 160 |
| 102                 | NPS 10 × Sch 80  |
| 81                  | NPS 8 × Sch 160  |
| 61                  | NPS 6 × Sch 80   |
| 41                  | NPS 4 × Sch 160  |

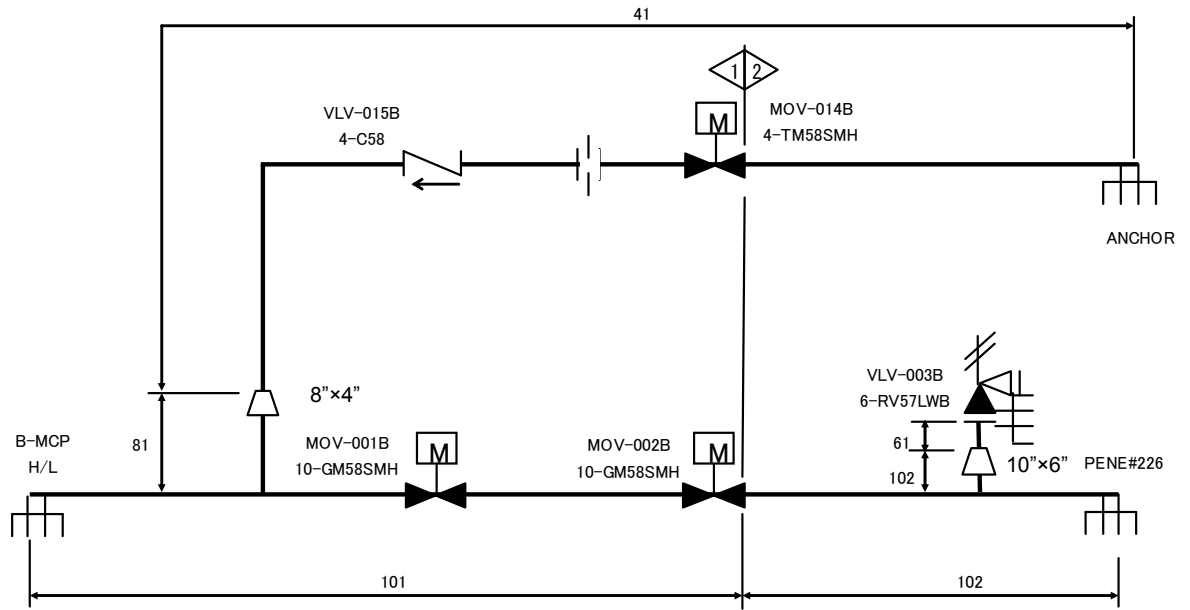


Figure 1.0-6 RH02: RHR suction loop B line

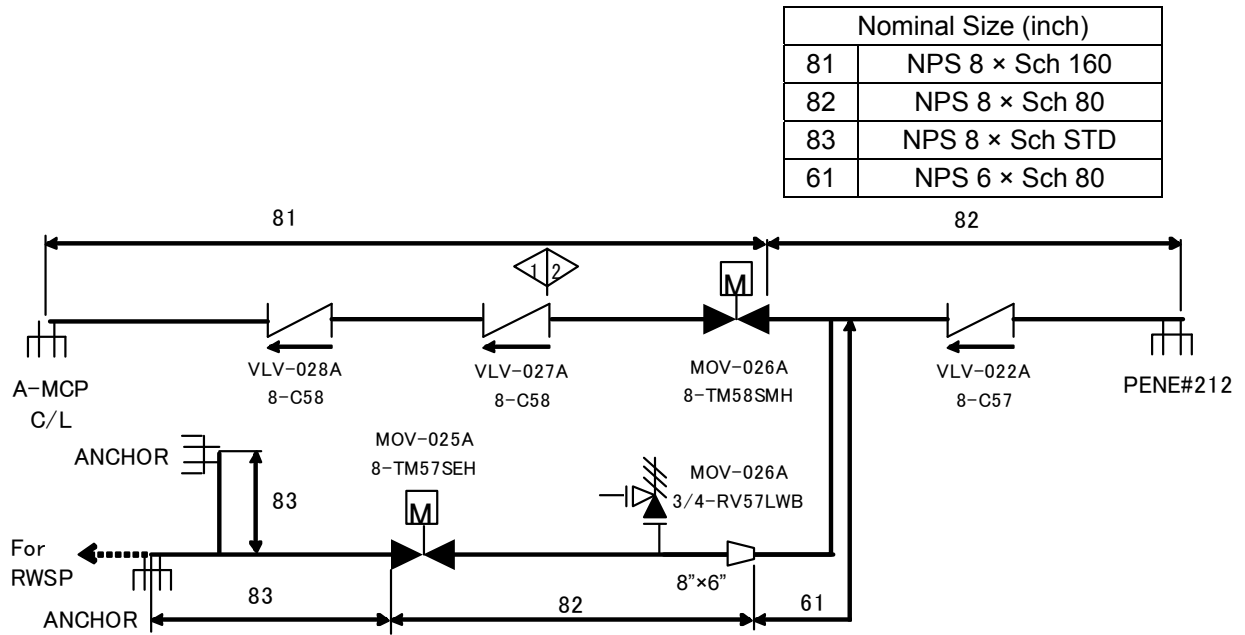


Figure 1.0-7 RH05: RHR return loop A line

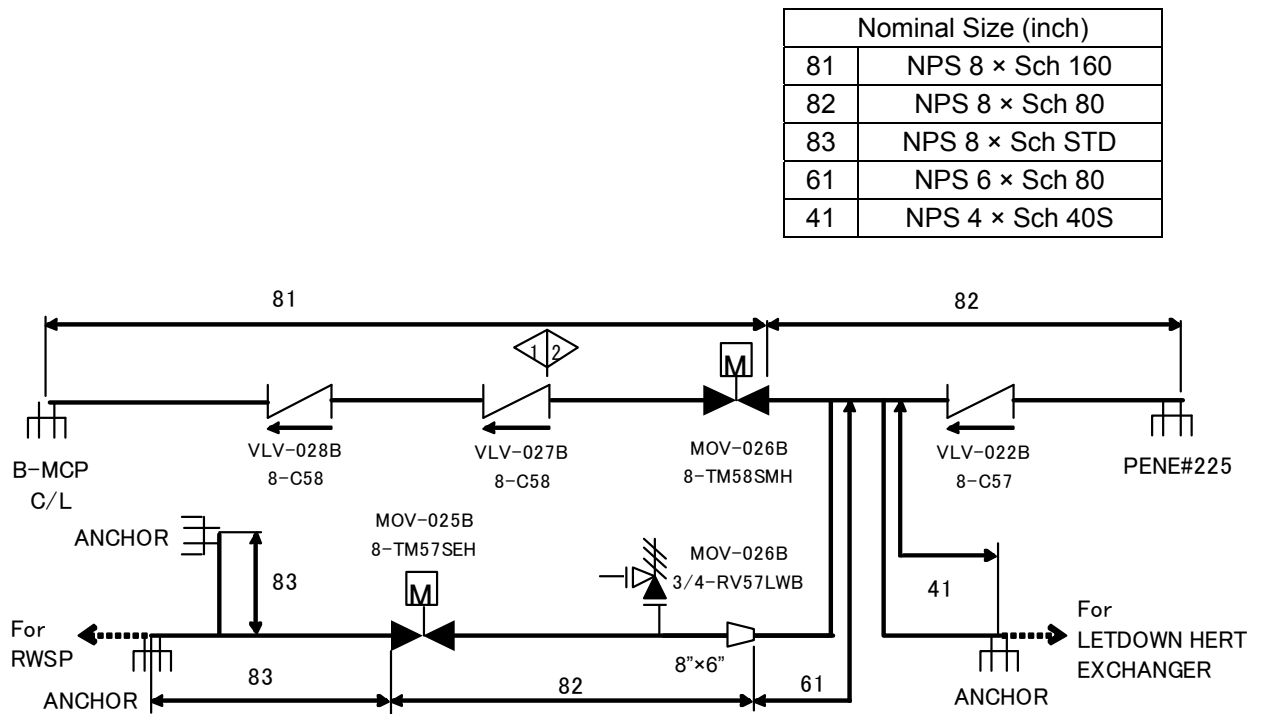


Figure 1.0-8 RH06: RHR return loop B line



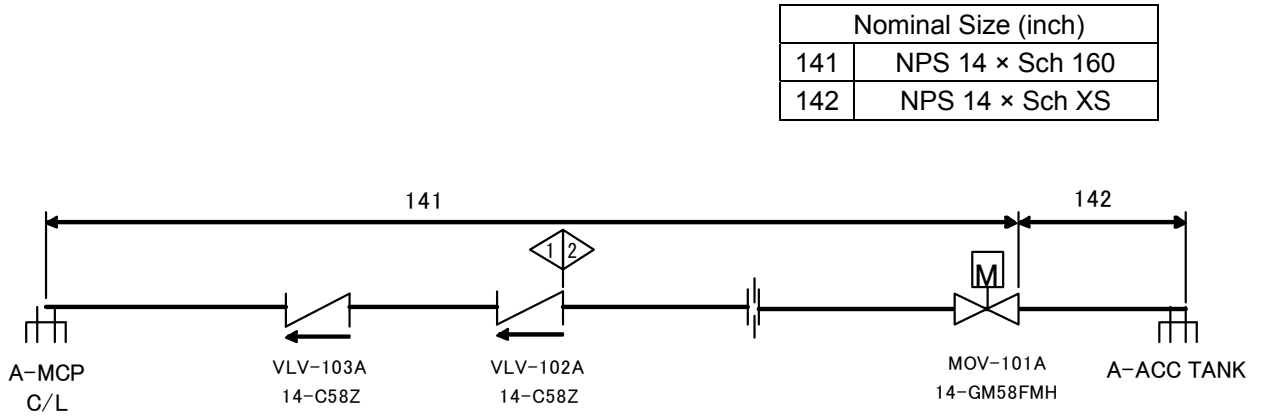


Figure 1.0-9 SI01: Accumulator loop A line

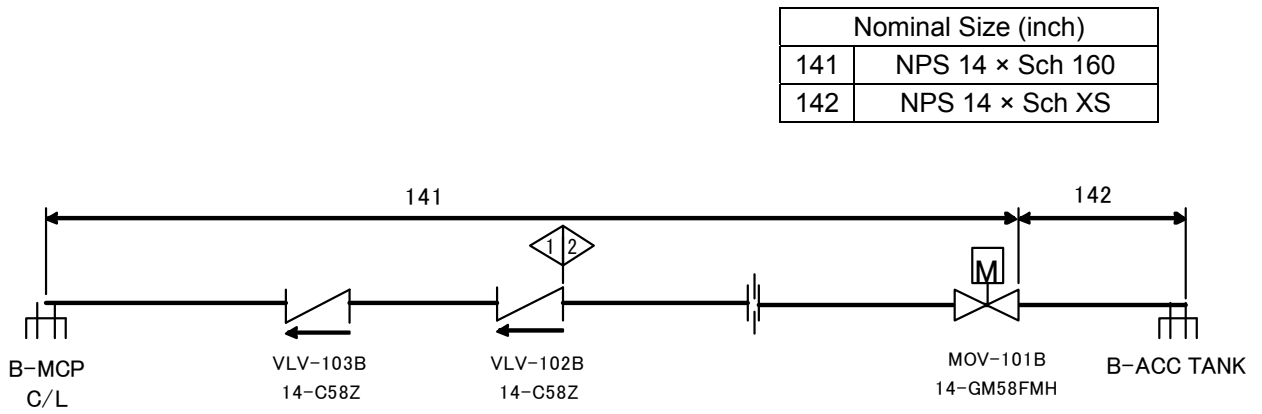


Figure 1.0-10 SI02: Accumulator loop B line

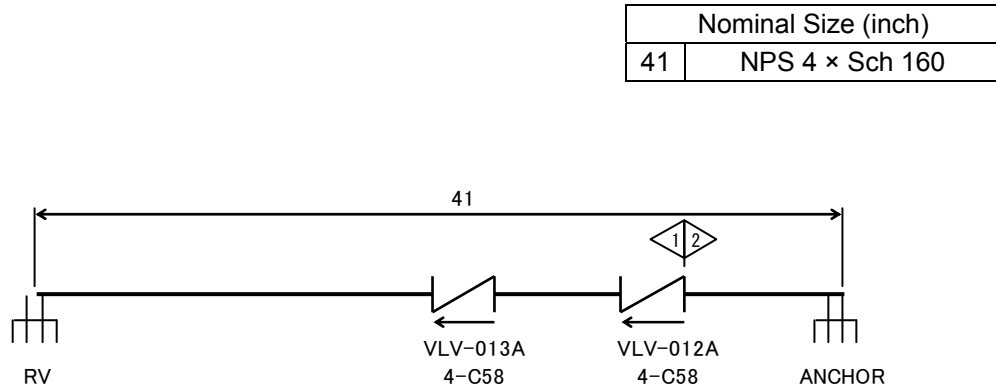


Figure 1.0-11 SI05: DVI A line

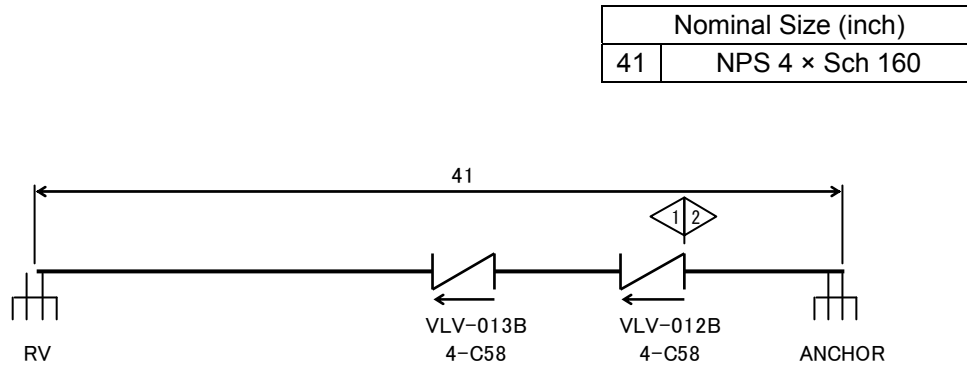


Figure 1.0-12 SI06: DVI B line

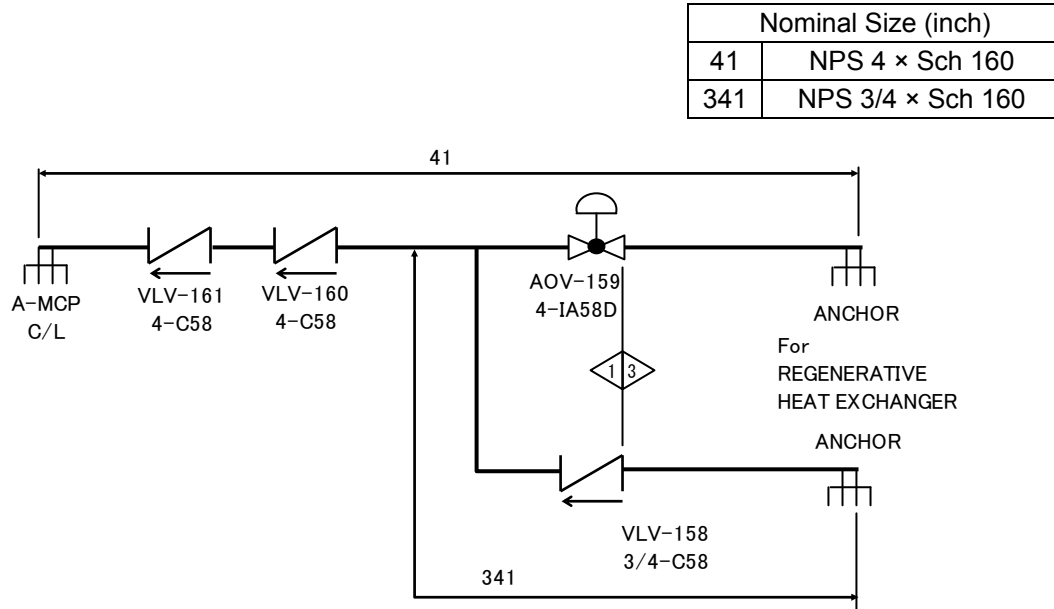


Figure 1.0-13 CS01: CVCS charging line

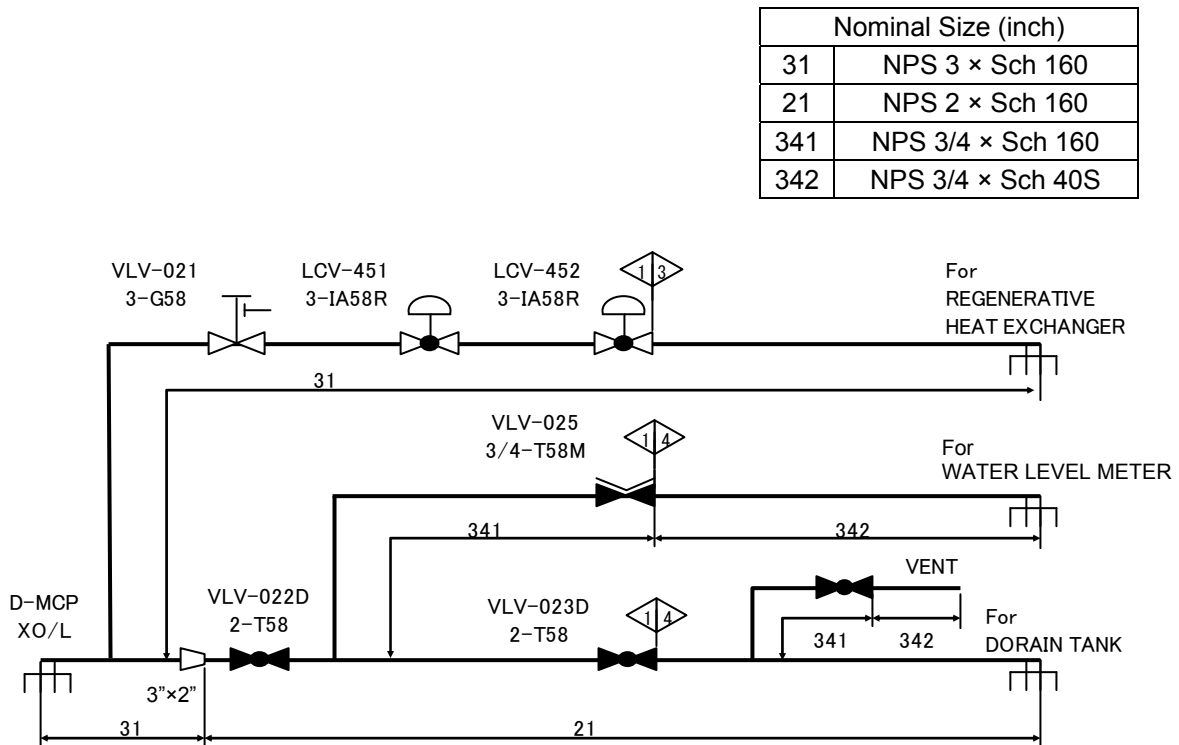


Figure 1.0-14 CS02: CVCS let down line

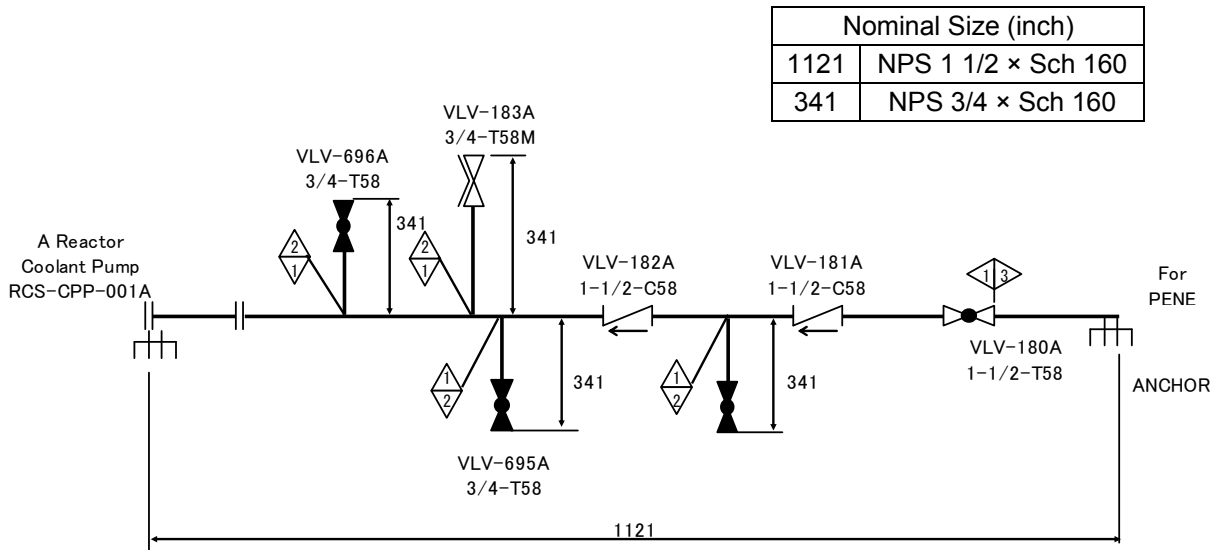


Figure 1.0-15 CS04: CVCS seal injection line (A-RCP)

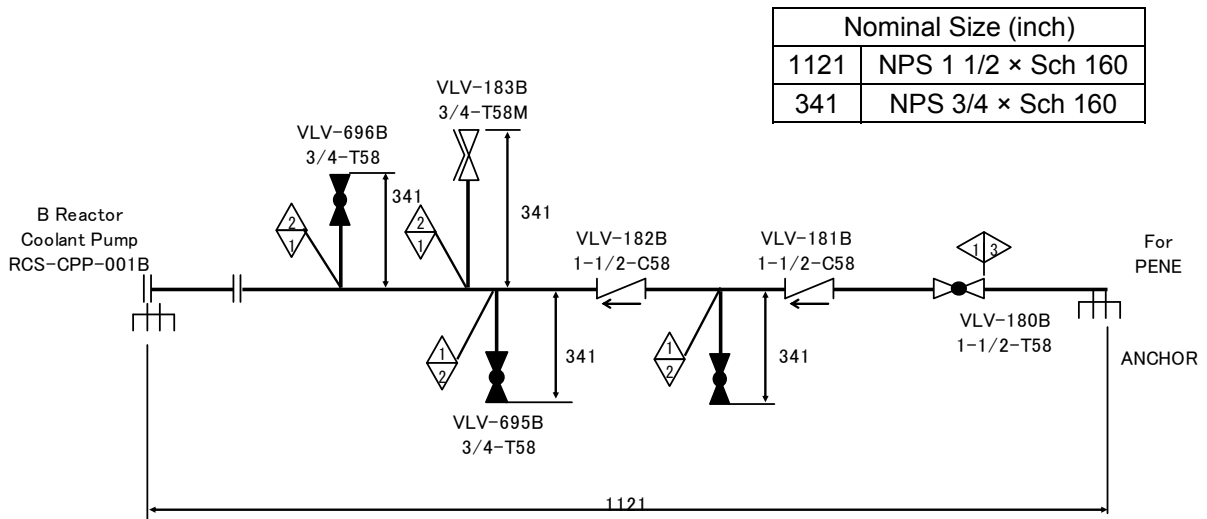


Figure 1.0-16 CS05: CVCS seal injection line (B-RCP)

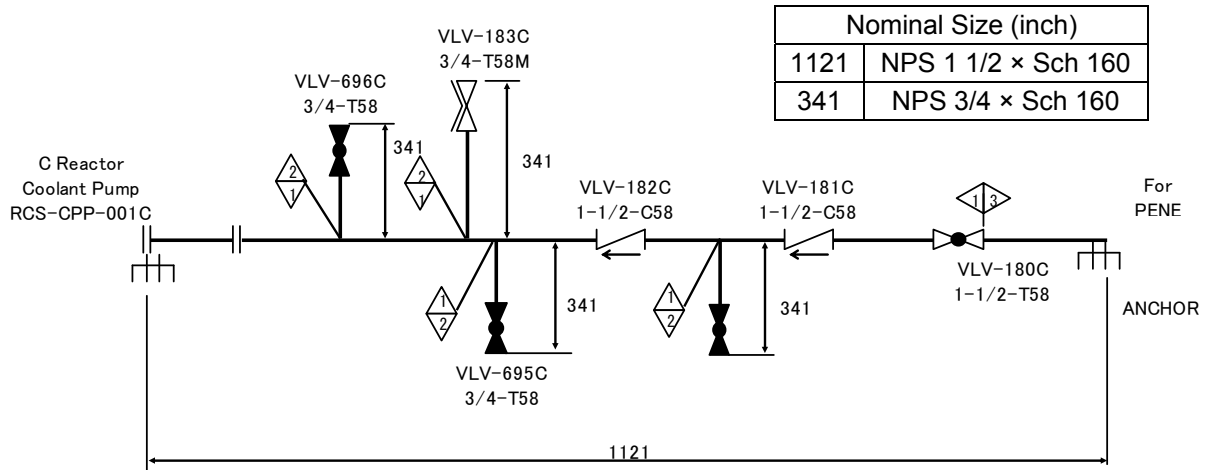


Figure 1.0-17 CS06: CVCS seal injection line (C-RCP)

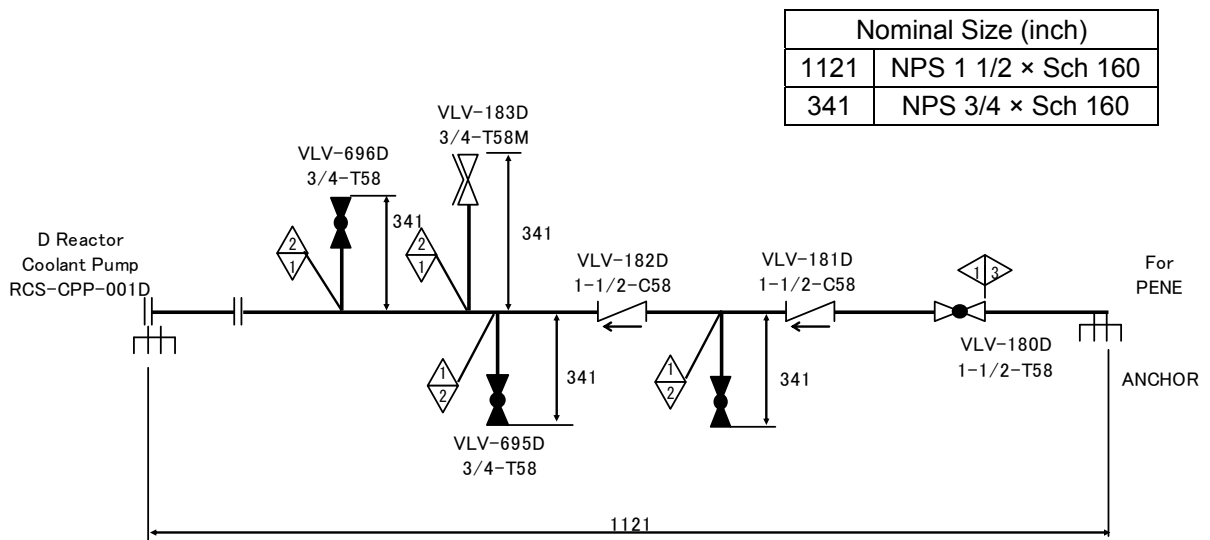


Figure 1.0-18 CS07: CVCS seal injection line (D-RCP)

**2.0 SUMMARY OF RESULTS**

The structural analysis results for each RCL Branch Piping are summarized in Section 11 and Appendix-1. The most limiting results in each evaluation are listed in Table 2.0-1 below.

**Table 2.0-1 Summary of Most Limiting Results**

| <b>Appendix</b> | <b>Evaluated Part</b>                                  | <b>Max Stress /<br/>Allowable Ratio</b> | <b>Highest<br/>Fatigue Usage<br/>Factor</b> | <b>LBB<br/>Evaluation</b> |
|-----------------|--|---|---|---------------------------|
| App. 1-1        | RC01 Pressurizer Surge Line                            |   |   |                           |
| App. 1-2        | RC02 Pressurizer Spray Line                            |   |   |                           |
| App. 1-3        | RC03 Pressurizer Safety<br>Depressurization Valve Line |   |   |                           |
| App. 1-4        | RC04 Pressurizer Safety Valve Line                     |   |   |                           |
| App. 1-5        | RH01 RHR Suction Loop A Line                           |   |   |                           |
| App. 1-6        | RH02 RHR Suction Loop B Line                           |   |   |                           |
| App. 1-7        | RH05 RHR Return Loop A Line                            |   |   |                           |
| App. 1-8        | RH06 RHR Return Loop B Line                            |   |   |                           |
| App. 1-9        | SI01 Accumulator Loop A Line                           |   |   |                           |
| App. 1-10       | SI02 Accumulator Loop B Line                           |   |   |                           |
| App. 1-11       | SI05 DVI A Line  |   |   |                           |
| App. 1-12       | SI06 DVI B Line  |   |   |                           |
| App. 1-13       | CS01 CVCS Charging Line                                |   |   |                           |
| App. 1-14       | CS02 CVCS Let Down Line                                |   |   |                           |
| App. 1-15       | CS04 CVCS Seal Injection A Line                        |   |   |                           |
| App. 1-16       | CS05 CVCS Seal Injection B Line                        |   |   |                           |
| App. 1-17       | CS06 CVCS Seal Injection C Line                        |   |   |                           |
| App. 1-18       | CS07 CVCS Seal Injection D Line                        |   |   |                           |

### **3.0 CONCLUSIONS**

The US-APWR RCL Branch Piping was designed to the requirements of the ASME Boiler and Pressure Vessel Code, 1992 Edition including the 1992 Addenda for the Design, Service Loadings, Operating Conditions, and Test Conditions as specified in the Design Specification (Reference 1).

Based on the results summarized in this report, it is concluded that the US-APWR RCL Branch Piping satisfy all of the requirements of the Design Specification (Reference 1) for structural integrity, operability, and safety, and it is confirmed that pressurizer surge line and accumulator loop A and B lines satisfies the LBB criteria using Bounding Analysis Curves (BACs) as described in Appendix 2.

4.0 NOMENCLATURE

Table 4.0-1 Symbol and Definition

| Symbol        | Unit | Definition  |
|---------------|------|---|
| $S_m$         | psi  | Design Stress Intensity   |
| $S_y$         | psi  | Yield Stress  |
| $S_c$         | psi  | Allowable Stress at minimum (cold) temperature  |
| $S_h$         | psi  | Allowable Stress at maximum (hot) temperature   |
| $S_A$         | psi  | Allowable Stress Range for Expansion Stress   |
| $DL$          | -    | Dead Load (The dead weight consists of the weight of the piping, insulation, and other loads permanently imposed upon the piping)                   |
| $P$           | -    | Design Pressure   |
| $P_R$         | -    | Range of Service Pressure   |
| $P_M$         | -    | Maximum Service Pressure  |
| $TH_{MTL}$    | -    | ASME Service Level A (Normal) and Service Level B (Upset) Miscellaneous Thermal Loads with Thermal Stratification and Thermal Cycling Effects       |
| $TH_{DISCON}$ | -    | Thermal Discontinuity Loads   |
| $TH_{GRAD}$   | -    | Thermal Radial Gradient Loads   |
| $L_{DM}$      | -    | Design Mechanical Loads   |
| $L_{DFN}$     | -    | ASME Service Level A (Normal) Dynamic Fluid Loads associated with hydraulic transients such as relief/safety valve opening or water/steam hammer    |
| $L_{DFU}$     | -    | ASME Service Level B (Upset) Dynamic Fluid Loads associated with hydraulic transients such as relief/safety valve opening or water/steam hammer     |
| $L_{DFE}$     | -    | ASME Service Level C (Emergency) Dynamic Fluid Loads associated with hydraulic transients such as relief/safety valve opening or water/steam hammer |
| $L_{DFD}$     | -    | ASME Service Level D (Faulted) Dynamic Fluid Loads associated with hydraulic transients such as relief/safety valve opening or water/steam hammer   |
| $1/3 SSE$     | -    | Design Condition & Level B Service Loading Earthquake (i.e. OBE)  |
| $SSEI$        | -    | Safe-Shutdown Earthquake Inertia Loads  |
| $SSEA$        | -    | Safe-Shutdown Earthquake Anchor Loads   |
| $BS$          | -    | Building Settlement   |
| $DBPB$        | -    | Design Basis Pipe Breaks, including LOCA and non-LOCA   |
| $LOCA$        | -    | Loss-of-Coolant Accident  |



## **5.0 ASSUMPTIONS AND OPEN ITEMS**

### **5.1 ASSUMPTIONS**

The basic modeling assumptions derived from the detailed analyses are as follows:

1. Because the valve weight and rigidity have not been set by the valve specifications or the procurer, data was used for a similar valve of the earlier PWR plant.
2. Because the rigidity of supports has not been set by the procurer, the value was set on the basis of a trial design that was consistent with the earlier PWR plant.
3. Because the locations of girth butt welds along straight pipes are to be determined in the detail design phase, local stress indices of girth butt welds are considered in both ends of each bend pipe in addition to all ends of pipe fittings. If the straight pipe length between a bend pipe and other bend pipe or pipe fitting is short enough to manufacture the straight pipe and the bend pipe without welding, the local stress indices at the end of the short straight pipe side of the bend pipe may not be considered.

### **5.2 OPEN ITEMS**

There are no open items in this Technical Report.

## **6.0 ACCEPTANCE CRITERIA**

The stress limits acceptance criteria for RCL Branch piping greater than 1 inch are specified in NB-3650 and for RCL Branch piping equal to 1 inch or smaller are specified in NC-3650 of ASME Section III. Table 6.0-1 lists the stress limits for RCL Branch piping (greater than 1 inch) and Table 6-2 lists the stress limits for RCL Branch piping (1 inch or smaller).

Table 6.0-1 RCL Branch Piping Stress Limits (greater than 1 inch piping)

| Condition     | Service Level | Category   | Loading   | Equation (NB-3650) <sup>(4)</sup>                   | Stress Limit <sup>(4)</sup>                    |
|---------------|---------------|--|---|---|--|
| Design        | -             | Primary Stress                                   | $P, DL, L_{DM}$<br>(including $L_{DFN}$ )                                 | Eq. 9<br>NB-3652                                    | $1.5 S_m$                                      |
| Normal /Upset | A/B           | Primary + Secondary Stress Intensity Range (SIR) | $P_R, TH_{MTL}, TH_{DISCON}, L_{DFN}, L_{DFU}, SSEI, SSEA$ <sup>(3)</sup> | Eq. 10<br>NB-3653.1                                 | $3 S_m$  |
|               |               | Peak SIR   | $P_R, TH_{MTL}, TH_{DISCON}, TH_{GRAD}, L_{DFN}, L_{DFU}, SSEI, SSEA$     | Eq. 11<br>NB-3653.2                                 |  |
|               |               | Thermal Bending SIR                              | $TH_{MTL}$ <sup>(2)</sup>   | Eq. 12<br>NB-3653.6(a)                              | $3 S_m$  |
|               |               | Primary + Secondary Membrane + Bending SIR       | $P_R, TH_{DISCON}, L_{DFN}, L_{DFU}, SSEI, SSEA$ <sup>(2)</sup>           | Eq. 13<br>NB-3653.6(b)                              | $3 S_m$  |
|               |               | Alternating Stress Intensity (Fatigue)           | $P_R, TH_{MTL}, TH_{DISCON}, TH_{GRAD}, L_{DFN}, L_{DFU}, SSEI, SSEA$     | NB-3653.3<br>NB-3653.4<br>NB-3653.5<br>NB-3653.6(c) |  |
|               |               | Thermal Stress Ratchet                           | $TH_{GRAD}$ (linear)  | NB3653.7  |  |
| Upset         | B             | Permissible Pressure                             | $P_M$   | NB-3654.1   | $1.1 P_a$                                      |
|               |               | Primary Stress                                   | $P_M, DL, L_{DFU}$  | NB-3654.2   | $\text{Min}(1.8 S_m, 1.5 S_y)$                 |
| Emergency     | C             | Permissible Pressure                             | $P_M$   | NB-3655.1   | $1.5 P_a$                                      |
|               |               | Primary Stress                                   | $P_M, DL, L_{DFE}$  | NB-3655.2   | $\text{Min}(2.25 S_m, 1.8 S_y)$                |
| Faulted       | D             | Permissible Pressure                             | $P_M$   | NB-3656(b)  | $2 P_a$  |
|               |               | Primary Stress                                   | $P_M, DL, L_{DFE}^{(1)}, SSEI, DBPB^{(1)}$                                | NB-3656(a)<br>NB-3656(b)                            | Appendix-F<br>or<br>$\text{Min}(3 S_m, 2 S_y)$ |
| Faulted       | D             | Secondary Stress                                 | $SSEA$  | <sup>(5)</sup>                                      | $6 S_m^{(5)}$                                  |

Notes:

- Dynamic loads are to be combined considering timing and causal relationships. SSE and DBPB are combined using the SRSS method.
- The Thermal and Primary plus Secondary Membrane plus Bending Stress Intensity Ranges (Equations 12 and 13) need only be calculated for those load sets that do not meet the Primary plus Secondary Stress Intensity Range (Equation 10) allowable.
- The earthquake inertial and anchor movement loads used in the Level B Stress Intensity Range and Alternating Stress calculations (Equations 10, 11, 13 and 14) is taken as 1/3 of the peak SSE inertial and anchor movement loads or as the peak SSE inertial and anchor movement loads. If the earthquake loads are taken as 1/3 of the peak SSE loads then the number of cycles to be considered for earthquake loading is to be 300 as derived in accordance with Appendix D of Institute of Electrical and Electronic Engineers Standard 344-2004 (Reference 6) If the earthquake loads are taken as the peak SSE loads then 20 cycles of earthquake loading is considered. Also, see Note 2.
- ASME Boiler and Pressure Vessel Code, Section III (Reference 7).

5. 
$$\frac{C_2 D_o M_{AM}}{2I} \leq 6.0 S_m \text{ and } \frac{F_{AM}}{A_M} \leq S_m$$

where

|          |   |
|----------|---|
| $D_o$    | = Pipe Outer Diameter                         |
| $I$      | = Pipe Moment of Inertia                      |
| $A_M$    | = Area of cross-section of the pipe           |
| $M_{AM}$ | = Range of resultant moment due to SSEA       |
| $F_{AM}$ | = Amplitude of longitudinal force due to SSEA |
| $S_m$    | = Allowable design stress intensity value     |

The use of  $6S_m$  limit assumes elastic behavior of the entire piping system. In the case of unbalanced systems, the design is modified to eliminate unbalance or the piping is qualified by using an allowable limit of  $3S_m$ .

Table 6.0-2 RCL Branch Piping Stress Limits (1 inch or smaller piping)

| Condition     | Service Level | Loading                        | Equation (NC-3650)      | Stress Limit <sup>(3)</sup>              |
|---------------|---------------|--------------------------------|-------------------------|--|
| Design        | -             | $P, DL$                        | Eq. 8<br>NC-3652        | $1.5 S_h$                                |
| Normal /Upset | A/B           | $P_M, DL, L_{DFN}, L_{DFU}$    | Eq. 9<br>NC-3653.1      | Min( $1.8 S_h, 1.5 S_y$ ) <sup>(1)</sup> |
|               |               | $TH_{MTL}$                     | Eq. 10<br>NC-3653.2(a)  | $S_A$                                    |
|               |               | $BS$                           | Eq. 10a<br>NC-3653.2(b) | $3S_c$                                   |
|               |               | $P_M, DL, TH_{MTL}$            | Eq. 11<br>NC-3652.2(c)  | $S_h + S_A$ <sup>(1)</sup>               |
| Emergency     | C             | $P_M, DL, L_{DFE}$             | Eq. 9<br>NC-3654        | Min( $2.25 S_h, 1.8 S_y$ )               |
| Faulted       | D             | $P_M, DL, L_{DFE}, SSEI, DBPB$ | Eq. 9<br>NC-3655        | Min( $3 S_h, 2 S_y$ )                    |
|               |               | $SSEA$                         | <sup>(5)</sup>          | $6S_h$ <sup>(5)</sup>                    |

Notes:

1. Stresses must meet the requirements of either Equation 10 or 11, not both.
2. If, during operation, the system normally carries a medium other than water (air, gas, steam), sustained loads should be checked for weight loads during hydrostatic testing as well as normal operation weight loads.
3. ASME Boiler and Pressure Vessel Code, Section III(Reference 7)
4. Dynamic loads are combined by the SRSS method.

5. 
$$\frac{C_2 D_o M_{AM}}{2I} \leq 6.0 S_h \text{ and } \frac{F_{AM}}{A_M} \leq S_h$$

where

|          |   |
|----------|---|
| $D_o$    | = Pipe Outer Diameter                         |
| $I$      | = Pipe Moment of Inertia                      |
| $A_M$    | = Area of cross-section of the pipe           |
| $M_{AM}$ | = Range of resultant moment due to SSEA       |
| $F_{AM}$ | = Amplitude of longitudinal force due to SSEA |
| $S_h$    | = Allowable stress value                      |

The use of  $6S_h$  limit assumes elastic behavior of the entire piping system. In the case of unbalanced systems, the design is modified to eliminate unbalance or the piping is qualified by using an allowable limit of  $3S_h$ .

## **7.0 DESIGN INPUT**

The piping was designed based on the design inputs described in the Design Specification (Reference 1) and the documents listed as follows:

1. N0-CF00004 Revision 0 "Piping Design Criteria" (Reference 2)
2. N0-GB00005 Revision 1 "Input Package of Stress Analysis of RCL Branch Piping and Main Steam Piping" (Reference 3)
3. N0-EE12001 Revision 2 " Class 1 Equipment Design Transients" (Reference 4)

## 8.0 LOAD AND LOAD COMBINATIONS

### 8.1 LOADINGS

#### 8.1.1 Design Temperature and Design Pressure

The Class 1 Piping Design Temperature and Design Pressure are as shown in Table 8.1-1.

**Table 8.1-1 Design Temperature and Design Pressure**

| Design Temperature (°F) | Design Pressure (psi) | Object Part  |
|-------------------------|-----------------------|--|
| 680                     | 2485                  | RC01<br>RC03<br>RC04   |
| 650                     | 2485                  | RC02<br>RH01<br>RH02<br>RH05<br>RH06<br>SI01<br>SI02<br>SI05<br>SI06<br>CS01<br>CS02 |
| 650<br>200              | 2485<br>2485          | CS04<br>CS05<br>CS06<br>CS07   |

#### 8.1.2 Sustained Loads

The weight of the piping system, its contents, any insulation and in-line equipment, and any other sustained loads identified in the Design Specification (Reference 1) were considered in the piping analysis. The mass contributed by the support was included in the analysis when it was greater than 10% of the total mass of the adjacent pipe span.

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### 8.1.3 Thermal Expansion Loads

The effect of linear thermal expansion range during various operating modes was considered along with thermal movement of terminal equipment nozzles, anchors, or restraints (thermal anchor movements) corresponding to the operating modes. The stress free temperature was taken as 70°F.

### 8.1.4 Thermal Stratification Loads (for Pressurizer surge line)

The thermal stratification stress was generated by assuming thermal stratification of the pipe fluid and switching from out-surge to in-surge or from in-surge to out-surge.

At the normal condition, thermal stratification does not occur with an initial condition of an 8 gpm out-surge.

When the transient starts with an out-surge condition, thermal stratification will not occur. When the transient starts with an in-surge condition, the thermal stratification will be formed by shifting from the initial condition of an 8 gpm out-surge to in-surge.

When the transient ceases with an in-surge condition, thermal stratification will be formed by shifting to the normal condition of the 8 gpm out-surge. When the transient ceases with an out-surge condition, thermal stratification will not be formed because the out-surge condition is maintained.

The profile of the thermal stratification used in the analysis is shown in Figure 8.1-1.



**Figure 8.1-1 Thermal Stratification Profile**



### **8.1.5 Earthquake Loads**

The effects of inertial loads and anchor movements due to an SSE are considered as Service Level D loads in the design of piping. Fatigue effects due to earthquake loads are discussed in Section 9.5.

### **8.1.6 Fluid Transient Loads**

The pressurizer safety valve and safety depressurization valve thrust loads are set in motion by the rapid actuation of valves. These loads are functions of valve opening, flow rate, flow area, and fluid properties.

### **8.1.7 Design Basis Pipe Break Loads**

US-APWR has applied the leak-before-break (LBB) methodology. As a result the main coolant piping (MCP) break and surge line break dynamic evaluations were eliminated. The postulated pipe break events that were evaluated for the reactor coolant system branch piping are as follows..

- Hot Leg Branch Line break at the 10 inch Schedule 160 Residual Heat Removal (RHR)/ Safety Injection (SI) line nozzle
- Cold Leg Branch line break at the 8 inch Schedule 160 RHR return line nozzle
- Feedwater Line break at the SG FW nozzle
- Main Steam Line break at the outside CV

The following components must be protected against mechanical loads due to a LOCA or secondary side pipe rupture (MS line break and FW line break).

#### **a. LOCA**

- (a) Intact loop including the branch piping and support components.
- (b) Intact main coolant pipe leg of the affected loop, SG support components and RCP support components in order to maintain the flow path.
- (c) Safety injection line connected to the intact leg of the affected loop in order to maintain safety injection.
- (d) MS line and main FW line in order to prevent simultaneous rupture of the secondary side.

b. All of the primary side must be protected against secondary side pipe rupture.

The information for the above types of piping is summarized in Table 8.1-2. For all of the displacements during accidents marked with “A” in Table 8.1-2, a factor of two margin was conservatively considered for dynamic effects. Forced displacements were assumed at the piping nozzle. Both translational and rotational directions were considered.

### **8.1.8 Design Transients**

The design transient conditions for each system are presented in the tables listed below.

|  |              |
|--|--------------|
| Pressurizer surge line                         | Table 8.1-3  |
| Pressurizer spray line                         | Table 8.1-4  |
| Pressurizer safety depressurization valve line | Table 8.1-5  |
| Pressurizer safety valve line                  | Table 8.1-6  |
| RHR suction loop A line                        | Table 8.1-7  |
| RHR suction loop B line                        | Table 8.1-8  |
| RHR return line                                | Table 8.1-9  |
| Accumulator line                               | Table 8.1-10 |
| DVI line                                       | Table 8.1-11 |
| CVCS charging line                             | Table 8.1-12 |
| CVCS let down line                             | Table 8.1-13 |
| CVCS seal injection line                       | Table 8.1-14 |

Table 8.1-2 Application of Accidental Load (Displacement) for each piping system

| Evaluated Line | Connected Component | Location of accident                |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
|----------------|---------------------|-------------------------------------|-------------------|------------------|----------|----------|------------------|-------------------|------------------|----------|----------|--|--|--|--|
|                |                     | Break loop (loop occurred accident) |                   |                  |          |          | Intact loop      |                   |                  |          |          |  |  |  |  |
|                |                     | Break at Hot-leg                    | Break at Cold-leg | RCP rocked rotor | MS break | FW break | Break at Hot-leg | Break at Cold-leg | RCP rocked rotor | MS break | FW break |  |  |  |  |
| RC01           | Hot-leg, PZR        |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| RC02           | Cold-leg, PZR       |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| RC03           | PZR                 |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| RC04           | PZR                 |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| RH01           | Hot-leg             |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| RH02           | Hot-leg             |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| RH05           | Cold-leg            |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| RH06           | Cold-leg            |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| SI01           | Cold-leg            |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| SI02           | Cold-leg            |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| SI05           | RV                  |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| SI06           | RV                  |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| CS01           | Cold-leg            |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| CS02           | Cross-Over-leg      |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| CS04           | RCP                 |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| CS05           | RCP                 |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| CS06           | RCP                 |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |
| CS07           | RCP                 |                                     |                   |                  |          |          |                  |                   |                  |          |          |  |  |  |  |

Table 8.1-3 Pressurizer surge line design transients (1/3)

| Level A<br>Mark | Transient   | Occurrence          | Reference |               | Remark  |
|-----------------|---|---------------------|-----------|---------------|---|
|                 |   |                     | Document  | Fig. or Table |   |
| I-a             | Plant heat-up (100F/h)  | 120                 |           | Fig. I-1      |   |
| I-b-1           | Plant cooldown (200F/h, 2235~400psig)   | 120                 |           | Fig. I-2      |   |
| I-b-2           | Plant cooldown (200F/h, lower than 400psig)   | 120                 |           | Fig. I-2      |   |
| I-c-1           | Ramp load increase between 15% and 100% of full power (5% of full power per minute) | 600                 |           | Fig. I-3      |   |
| I-c-2           | Ramp load increase between 50% and 100% of full power (5% of full power per minute) | 19, 200             |           | Fig. I-4      |   |
| I-d-1           | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) | 600                 |           | Fig. I-5      |   |
| I-d-2           | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) | 19, 200             |           | Fig. I-6      |   |
| I-e             | Step load increase of 10% of full power   | 600                 |           | Fig. I-7      |   |
| I-f             | Step load decrease of 10% of full power   | 600                 |           | Fig. I-8      |   |
| I-g             | Large step load decrease with turbine bypass  | 60                  |           | Fig. I-9      |   |
| I-h             | Steady-state fluctuation and i) Steady-state fluctuation load regulation            | 1×10 <sup>6</sup>   | Ref. 4    |               | P <sub>p</sub> ±50psi, T <sub>p</sub> ±3.1F     |
|                 | ii) Load regulation   | 1.6×10 <sup>6</sup> |           | Table 4       |   |
| I-i             | Main feedwater cycling  | 2, 100              |           | Fig. I-10     |   |
| I-j             | Refueling   | 60                  |           | Fig. I-11     | Water is replaced in 10 minutes.                |
| I-k             | Ramp load increase between 0% and 15% of full power                                 | 600                 |           | Fig. I-12     |   |
| I-l             | Ramp load decrease between 0% and 15% of full power                                 | 600                 |           | Fig. I-13     |   |
| I-m             | RCP startup   | 3, 000              |           | Fig. I-14     |   |
| I-n             | RCP shutdown  | 3, 000              |           | Fig. I-15     |   |
| I-o             | Core lifetime extension   | 60                  |           | Fig. I-16     |   |
| I-p             | Primary leakage test  | 120                 |           | Fig. I-17     |   |
| I-q             | Turbine roll test   | 10                  |           | Fig. I-18     |   |
| I-r             | Boron concentration equalization  | 39, 600             |           | —             | P <sub>p</sub> +25psi, T <sub>p</sub> +1.4F,-0F |

Table 8.1-3 Pressurizer surge line design transients (2/3)

| Level B<br>Mark | Transient   | Occurrence | Reference |               | Remark  |   |
|-----------------|---|------------|-----------|---------------|---|---|
|                 |   |            | Document  | Fig. or Table |   |   |
| II-a            | Loss of load  | 60         |           | Fig. II-1     |   |   |
| II-b            | Loss of offsite power                                   | 60         |           | Fig. II-2     |   |   |
| II-c            | Partial loss of reactor coolant flow                    | 30         |           | Fig. II-3     |   |   |
| II-d            | Reactor trip from full power                            | 30         |           | Fig. II-5     | Including the transient of Excessive feedwater flow               |   |
|                 |   |            |           |               |   | i) With no inadvertent cooldown           |
|                 |   |            |           |               |   | ii) With cooldown and no safety injection |
| II-e            | Inadvertent RCS depressurization                        | 30         |           | Fig. II-7     |   |   |
|                 |   |            |           |               |   | iii) With cooldown and safety injection   |
| II-f            | Control rod drop  | 15         | Ref. 4    | Fig. II-12    |   |   |
|                 |   |            |           |               |   | i) Umbrella case                          |
| II-g            | Inadvertent safeguards actuation                        | 30         |           | Fig. II-8     |   |   |
|                 |   |            |           |               |   | ii) Inadvertent auxiliary spray           |
| II-h            | Emergency feedwater cycling                             | 700        |           | Fig. II-9     |   |   |
| II-i            | Cold over-pressure                                      | 30         |           | Fig. II-10    |   |   |
| II-j            | Excessive feedwater flow                                | —          |           | Fig. II-11    | Be covered with the transient of Reactor trip from full power ii) |   |
| II-k            | Loss of offsite power with natural circulation cooldown | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |
| II-l            | Partial loss of emergency feedwater                     | 30         |           | —             | Please use the figure of the transient of Loss of offsite power.  |   |
| II-m            | Safe shutdown   | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |

Table 8.1-3 Pressurizer surge line design transients (3/3)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |

Table 8.1-4 Pressurizer spray line design transients (1/3)

| Level A<br>Mark | Transient   | Occurrence        | Reference |               | Remark   |
|-----------------|---|-------------------|-----------|---------------|--|
|                 |   |                   | Document  | Fig. or Table |  |
| I-a             | Plant heat-up (100F/h)  | 120               |           | Fig. I-1      |  |
| I-b-1           | Plant cooldown (200F/h, 2235~400psig)   | 120               |           | Fig. I-2      |  |
| I-b-2           | Plant cooldown (200F/h, lower than 400psig)   | 120               |           | Fig. I-2      |  |
| I-c-1           | Ramp load increase between 15% and 100% of full power (5% of full power per minute) | 600               |           | Fig. I-3      |  |
| I-c-2           | Ramp load increase between 50% and 100% of full power (5% of full power per minute) | 19, 200           |           | Fig. I-4      |  |
| I-d-1           | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) | 600               |           | Fig. I-5      |  |
| I-d-2           | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) | 19, 200           |           | Fig. I-6      |  |
| I-e             | Step load increase of 10% of full power   | 600               |           | Fig. I-7      |  |
| I-f             | Step load decrease of 10% of full power   | 600               |           | Fig. I-8      |  |
| I-g             | Large step load decrease with turbine bypass  | 60                |           | Fig. I-9      |  |
| I-h             | Steady-state fluctuation and load regulation  | 1×10 <sup>6</sup> | Ref. 4    |               | P <sub>p</sub> ±50psi, T <sub>colg</sub> ±3.1F |
|                 | i) Steady-state fluctuation<br>ii) Load regulation                                  | 8×10 <sup>5</sup> |           |               |  |
| I-i             | Main feedwater cycling  | 2, 100            | Table 4   |               |  |
| I-j             | Refueling   | 60                | Fig. I-10 |               |  |
| I-k             | Ramp load increase between 0% and 15% of full power                                 | 600               | Fig. I-11 |               | Water is replaced in 10 minutes.               |
| I-l             | Ramp load decrease between 0% and 15% of full power                                 | 600               | Fig. I-12 |               |  |
| I-m             | RCP startup   | 600               | Fig. I-13 |               |  |
| I-n             | RCP shutdown  | 3, 000            | Fig. I-14 |               |  |
| I-o             | Core lifetime extension   | 3, 000            | Fig. I-15 |               |  |
| I-p             | Primary leakage test  | 60                | Fig. I-16 |               |  |
| I-q             | Turbine roll test   | 120               | Fig. I-17 |               |  |
| I-r             | Boron concentration equalization  | 10                | Fig. I-18 |               |  |
|                 |   | 39, 600           |           | —             | P <sub>p</sub> +25psi, Tp+1.4F,-0F             |

Table 8.1-4 Pressurizer spray line design transients (2/3)

| Level B<br>Mark | Transient   | Occurrence | Reference   |               | Remark  |
|-----------------|---|------------|---|---------------|---|
|                 |   |            | Document  | Fig. or Table |   |
| II-a            | Loss of load  | 60         |   | Fig. II-1     |   |
| II-b            | Loss of offsite power                                   | 60         |   | Fig. II-2     |   |
| II-c            | Partial loss of reactor coolant flow                    | 30         |   | Fig. II-3     |   |
| II-d            | Reactor trip from full power                            | 30         | i) With no inadvertent cooldown<br>ii) With cooldown and no safety injection<br>iii) With cooldown and safety injection | Fig. II-4     |   |
|                 |   |            |   | Fig. II-5     | Including the transient of Excessive feedwater flow               |
|                 |   |            |   | Fig. II-6     |   |
| II-e            | Inadvertent RCS depressurization                        | 30         | i) Umbrella case<br>ii) Inadvertent auxiliary spray   | Fig. II-7     |   |
|                 |   |            |   | Fig. II-12    |   |
| II-f            | Control rod drop  | 15         |   | Ref. 4        |   |
| II-g            | Inadvertent safeguards actuation                        | 30         |   | Fig. II-8     |   |
| II-h            | Emergency feedwater cycling                             | 700        |   | Fig. II-9     |   |
| II-i            | Cold over-pressure                                      | 30         |   | Fig. II-10    |   |
| II-j            | Excessive feedwater flow                                | —          |   | Fig. II-11    | Be covered with the transient of Reactor trip from full power ii) |
| II-k            | Loss of offsite power with natural circulation cooldown | —          |   | —             | Be covered with the transient of Plant cooldown                   |
| II-l            | Partial loss of emergency feedwater                     | 30         |   | —             | Please use the figure of the transient of Loss of offsite power.  |
| II-m            | Safe shutdown   | —          |   | —             | Be covered with the transient of Plant cooldown                   |



Table 8.1-4 Pressurizer spray line design transients (3/3)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |

Table 8.1-5 Pressurizer safety depressurization valve line design transients (1/4)

| Level A |   | Transient         | Occurrence | Reference     |  | Remark |
|---------|---|-------------------|------------|---------------|--|--------|
| Mark    | Document  |                   |            | Fig. or Table |  |        |
| I-a     | Plant heat-up (100F/h)  | 120               | Fig. I-1   |               |  |        |
| I-b-1   | Plant cooldown (200F/h, 2235~400psig)   | 120               | Fig. I-2   |               |  |        |
| I-b-2   | Plant cooldown (200F/h, lower than 400psig)   | 120               | Fig. I-2   |               |  |        |
| I-c-1   | Ramp load increase between 15% and 100% of full power (5% of full power per minute) | 600               | Fig. I-3   |               |  |        |
| I-c-2   | Ramp load increase between 50% and 100% of full power (5% of full power per minute) | 19, 200           | Fig. I-4   |               |  |        |
| I-d-1   | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) | 600               | Fig. I-5   |               |  |        |
| I-d-2   | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) | 19, 200           | Fig. I-6   |               |  |        |
| I-e     | Step load increase of 10% of full power   | 600               | Fig. I-7   |               |  |        |
| I-f     | Step load decrease of 10% of full power   | 600               | Fig. I-8   |               |  |        |
| I-g     | Large step load decrease with turbine bypass  | 60                | Fig. I-9   |               |  |        |
| I-h     | Steady-state fluctuation and load regulation  | 1×10 <sup>6</sup> |            |               | P <sub>p</sub> ±50psi, T <sub>p</sub> ±3.1F      |        |
|         | i) Steady-state fluctuation<br>ii) Load regulation                                  | 8×10 <sup>5</sup> | Table 4    |               |  |        |
| I-i     | Main feedwater cycling  | 2, 100            | Fig. I-10  |               |  |        |
| I-m     | RCP startup   | 3, 000            | Fig. I-14  |               |  |        |
| I-n     | RCP shutdown  | 3, 000            | Fig. I-15  |               |  |        |
| I-p     | Primary leakage test  | 120               | Fig. I-17  |               |  |        |
| I-q     | Turbine roll test   | 10                | Fig. I-18  |               |  |        |
| I-r     | Boron concentration equalization  | 39, 600           | —          |               | P <sub>p</sub> +25psi, T <sub>p</sub> +1.4F, -0F |        |

Table 8.1-5 Pressurizer safety depressurization valve line design transients (2/4)

| Level B<br>Mark | Transient   | Occurrence | Reference |               | Remark  |   |
|-----------------|---|------------|-----------|---------------|---|---|
|                 |   |            | Document  | Fig. or Table |   |   |
| II-a            | Loss of load  | 60         |           | Fig. II-1     |   |   |
| II-b            | Loss of offsite power                                   | 60         |           | Fig. II-2     |   |   |
| II-c            | Partial loss of reactor coolant flow                    | 30         |           | Fig. II-3     |   |   |
| II-d            | Reactor trip from full power                            | 30         |           | Fig. II-5     | Including the transient of Excessive feedwater flow               |   |
|                 |   |            |           |               |   | i) With no inadvertent cooldown           |
|                 |   |            |           |               |   | ii) With cooldown and no safety injection |
| II-e            | Inadvertent RCS depressurization                        | 30         |           | Fig. II-7     |   |   |
|                 |   |            |           |               |   | iii) With cooldown and safety injection   |
| II-f            | Control rod drop  | 15         | Ref. 4    | Fig. II-12    |   |   |
|                 |   |            |           |               |   | i) Umbrella case                          |
| II-g            | Inadvertent safeguards actuation                        | 30         |           | Fig. II-8     |   |   |
|                 |   |            |           |               |   | ii) Inadvertent auxiliary spray           |
| II-h            | Emergency feedwater cycling                             | 700        |           | Fig. II-9     |   |   |
| II-i            | Cold over-pressure                                      | 30         |           | Fig. II-10    |   |   |
| II-j            | Excessive feedwater flow                                | -          |           | -             | Be covered with the transient of Reactor trip from full power ii) |   |
|                 |   |            |           |               |   |   |
| II-k            | Loss of offsite power with natural circulation cooldown | -          |           | -             | Be covered with the transient of Plant cooldown                   |   |
|                 |   |            |           |               |   |   |
| II-l            | Partial loss of emergency feedwater                     | 30         |           | -             | Please use the figure of the transient of Loss of offsite power.  |   |
| II-m            | Safe shutdown   | -          |           | -             | Be covered with the transient of Plant cooldown                   |   |

Table 8.1-5 Pressurizer safety depressurization valve line design transients (3/4)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |

Table 8.1-5 Pressurizer safety depressurization valve line design transients (4/4) (branch piping transients)

| Level A, B |   |            |           |               |        |
|------------|---|------------|-----------|---------------|--------|
| Mark       | Transient   | Occurrence | Reference |               | Remark |
|            |   |            | Document  | Fig. or Table |        |
| —          | Pressurizer safety depressurization valve actuation | 60         | Ref. 4    | —             |        |

Table 8.1-6 Pressurizer safety valve line design transients (1/4)

| Level A |   | Transient                   | Occurrence        | Reference     |                                    | Remark |
|---------|---|-----------------------------|-------------------|---------------|------------------------------------|--------|
| Mark    | Document  |                             |                   | Fig. or Table |                                    |        |
| I-a     | Plant heat-up (100F/h)  | 120                         | Fig. I-1          |               |                                    |        |
| I-b-1   | Plant cooldown (200F/h, 2235~400psig)   | 120                         | Fig. I-2          |               |                                    |        |
| I-b-2   | Plant cooldown (200F/h, lower than 400psig)   | 120                         | Fig. I-2          |               |                                    |        |
| I-c-1   | Ramp load increase between 15% and 100% of full power (5% of full power per minute) | 600                         | Fig. I-3          |               |                                    |        |
| I-c-2   | Ramp load increase between 50% and 100% of full power (5% of full power per minute) | 19, 200                     | Fig. I-4          |               |                                    |        |
| I-d-1   | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) | 600                         | Fig. I-5          |               |                                    |        |
| I-d-2   | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) | 19, 200                     | Fig. I-6          |               |                                    |        |
| I-e     | Step load increase of 10% of full power   | 600                         | Fig. I-7          |               |                                    |        |
| I-f     | Step load decrease of 10% of full power   | 600                         | Fig. I-8          |               |                                    |        |
| I-g     | Large step load decrease with turbine bypass  | 60                          | Fig. I-9          |               |                                    |        |
| I-h     | Steady-state fluctuation and load regulation  | i) Steady-state fluctuation |                   |               | P <sub>p</sub> ±50psi, Tp±3.1F     |        |
|         |   | ii) Load regulation         | 8×10 <sup>5</sup> | Table 4       |                                    |        |
| I-i     | Main feedwater cycling  | 2, 100                      | Fig. I-10         |               |                                    |        |
| I-m     | RCP startup   | 3, 000                      | Fig. I-14         |               |                                    |        |
| I-n     | RCP shutdown  | 3, 000                      | Fig. I-15         |               |                                    |        |
| I-p     | Primary leakage test  | 120                         | Fig. I-17         |               |                                    |        |
| I-q     | Turbine roll test   | 10                          | Fig. I-18         |               |                                    |        |
| I-r     | Boron concentration equalization  | 39, 600                     | —                 |               | P <sub>p</sub> +25psi, Tp+1.4F,-0F |        |

Table 8.1-6 Pressurizer safety valve line design transients (2/4)

| Level B<br>Mark | Transient   | Occurrence | Reference   |               | Remark  |
|-----------------|---|------------|---|---------------|---|
|                 |   |            | Document  | Fig. or Table |   |
| II-a            | Loss of load  | 60         |   | Fig. II-1     |   |
| II-b            | Loss of offsite power                                   | 60         |   | Fig. II-2     |   |
| II-c            | Partial loss of reactor coolant flow                    | 30         |   | Fig. II-3     |   |
| II-d            | Reactor trip from full power                            | 60         | i) With no inadvertent cooldown<br>ii) With cooldown and no safety injection<br>iii) With cooldown and safety injection | Fig. II-4     |   |
|                 |   |            |   | Fig. II-5     | Including the transient of Excessive feedwater flow               |
|                 |   |            |   | Fig. II-6     |   |
| II-e            | Inadvertent RCS depressurization                        | 30         | i) Umbrella case<br>ii) Inadvertent auxiliary spray   | Fig. II-7     |   |
|                 |   |            |   | Fig. II-12    |   |
| II-f            | Control rod drop  | 30         |   | Ref. 4        |   |
| II-g            | Inadvertent safeguards actuation                        | 30         |   | Fig. II-9     |   |
| II-h            | Emergency feedwater cycling                             | 700        |   | Fig. II-10    |   |
| II-i            | Cold over-pressure                                      | 30         |   | Fig. II-11    |   |
| II-j            | Excessive feedwater flow                                | —          |   | —             | Be covered with the transient of Reactor trip from full power ii) |
| II-k            | Loss of offsite power with natural circulation cooldown | —          |   | —             | Be covered with the transient of Plant cooldown                   |
| II-l            | Partial loss of emergency feedwater                     | 30         |   | —             | Please use the figure of the transient of Loss of offsite power.  |
| II-m            | Safe shutdown   | —          |   | —             | Be covered with the transient of Plant cooldown                   |

Table 8.1-6 Pressurizer safety valve line design transients (3/4)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |

Table 8.1-6 Pressurizer safety valve line design transients (4/4) (branch piping transients)

| Level A, B |                                    |            |           |               |        |
|------------|------------------------------------|------------|-----------|---------------|--------|
| Mark       | Transient                          | Occurrence | Reference |               | Remark |
|            |                                    |            | Document  | Fig. or Table |        |
| —          | Pressurizer safety valve actuation | 60         | Ref. 4    | —             |        |

Table 8.1-7 RHRS suction loop A line design transients (1/4)

| Level A |   | Transient                   | Occurrence        | Reference |               | Remark  |  |
|---------|---|-----------------------------|-------------------|-----------|---------------|---|--|
| Mark    |   |                             |                   | Document  | Fig. or Table |   |  |
| I-a     | Plant heat-up (50F/h)   |                             | 120               |           | Fig. I-1      |   |  |
| I-b     | Plant cooldown (100F/h)   |                             | 120               |           | Fig. I-2      | Including the transient of Loss of offsite power with natural circulation cooldown (10 times) and Safe shutdown (1 time). |  |
| I-c-1   | Ramp load increase between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-3      |   |  |
| I-c-2   | Ramp load increase between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-4      |   |  |
| I-d-1   | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-5      |   |  |
| I-d-2   | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-6      |   |  |
| I-e     | Step load increase of 10% of full power   |                             | 600               | Ref. 4    | Fig. I-7      |   |  |
| I-f     | Step load decrease of 10% of full power   |                             | 600               |           | Fig. I-8      |   |  |
| I-g     | Large step load decrease with turbine bypass  |                             | 60                |           | Fig. I-9      |   |  |
| I-h     | Steady-state fluctuation and load regulation  | i) Steady-state fluctuation | 1×10 <sup>6</sup> |           | —             |   | P <sub>p</sub> ±50psi, T <sub>hot</sub> , T <sub>cold</sub> , T <sub>ave</sub> ±3.1F |
|         |   | ii) Load regulation         | 8×10 <sup>5</sup> |           | Table 4       |   |  |
| I-i     | Main feedwater cycling  |                             | 2, 100            |           | Fig. I-10     |   |  |
| I-j     | Refueling   |                             | 60                |           | Fig. I-11     |   | Water is replaced in 10 minutes.   |
| I-k     | Ramp load increase between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-12     |   |  |
| I-l     | Ramp load decrease between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-13     |   |  |
| I-m     | RCP startup   |                             | 3, 000            |           | Fig. I-14     |   |  |
| I-n     | RCP shutdown  |                             | 3, 000            |           | Fig. I-15     |   |  |
| I-o     | Core lifetime extension   |                             | 60                |           | Fig. I-16     |   |  |
| I-p     | Primary leakage test  |                             | 120               |           | Fig. I-17     |   |  |
| I-q     | Turbine roll test   |                             | 10                |           | Fig. I-18     |   |  |



Table 8.1-7 RHRS suction loop A line design transients (2/4)

| Level B<br>Mark | Transient   | Occurrence | Reference |               | Remark  |   |
|-----------------|---|------------|-----------|---------------|---|---|
|                 |   |            | Document  | Fig. or Table |   |   |
| II-a            | Loss of load  | 60         |           | Fig. II-1     |   |   |
| II-b            | Loss of offsite power                                   | 60         |           | Fig. II-2     |   |   |
| II-c            | Partial loss of reactor coolant flow                    | 30         |           | Fig. II-3     |   |   |
| II-d            | Reactor trip from full power                            | 60         |           | Fig. II-4     |   |   |
|                 |   |            |           |               |   | i) With no inadvertent cooldown           |
|                 |   |            |           |               |   | ii) With cooldown and no safety injection |
|                 | iii) With cooldown and safety injection                 | 10         |           | Fig. II-5     | Including the transient of Excessive feedwater flow               |   |
| II-e            | Inadvertent RCS depressurization                        | 30         |           | Fig. II-6     |   |   |
| II-f            | Control rod drop  | 30         | Ref. 4    | Fig. II-7     |   |   |
| II-g            | Inadvertent safeguards actuation                        | 30         |           | Fig. II-8     |   |   |
| II-h            | Emergency feedwater cycling                             | 700        |           | Fig. II-9     |   |   |
| II-i            | Cold over-pressure                                      | 30         |           | Fig. II-10    |   |   |
| II-j            | Excessive feedwater flow                                | —          |           | Fig. II-11    | Be covered with the transient of Reactor trip from full power ii) |   |
| II-k            | Loss of offsite power with natural circulation cooldown | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |
| II-l            | Partial loss of emergency feedwater                     | 30         |           | —             | Please use the figure of the transient of Loss of offsite power.  |   |
| II-m            | Safe shutdown   | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |

Table 8.1-7 RHRS suction loop A line design transients (3/4)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |

Table 8.1-7 RHRS suction loop A line design transients (4/4) (branch piping transients)

| Level A, B |                |            |           |               |        |
|------------|----------------|------------|-----------|---------------|--------|
| Mark       | Transient      | Occurrence | Reference |               | Remark |
|            |                |            | Document  | Fig. or Table |        |
| a)         | Plant heat-up  | 120        | Ref. 4    | Fig. I-26     |        |
| b)         | Plant cooldown | 120        |           | Fig. I-27     |        |
| c)         | Safe shutdown  | 1          |           | Fig. II-21    |        |

Table 8.1-8 RHRS suction loop B line design transients (1/4)

| Level A |   | Transient                   | Occurrence        | Reference |               | Remark   |  |
|---------|---|-----------------------------|-------------------|-----------|---------------|--|--|
| Mark    |   |                             |                   | Document  | Fig. or Table |  |  |
| I-a     | Plant heat-up (50F/h)   |                             | 120               |           | Fig. I-1      |  |  |
| I-b     | Plant cooldown (100F/h)   |                             | 120               |           | Fig. I-2      | Including the transient of Loss of offsite power with natural circulation cooldown (10 times) and Safe shutdown (1 time) . |  |
| I-c-1   | Ramp load increase between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-3      |  |  |
| I-c-2   | Ramp load increase between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-4      |  |  |
| I-d-1   | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-5      |  |  |
| I-d-2   | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-6      |  |  |
| I-e     | Step load increase of 10% of full power   |                             | 600               | Ref. 4    | Fig. I-7      |  |  |
| I-f     | Step load decrease of 10% of full power   |                             | 600               |           | Fig. I-8      |  |  |
| I-g     | Large step load decrease with turbine bypass  |                             | 60                |           | Fig. I-9      |  |  |
| I-h     | Steady-state fluctuation and load regulation  | i) Steady-state fluctuation | 1×10 <sup>6</sup> |           | —             |  | P <sub>p</sub> ±50psi, T <sub>hot</sub> , T <sub>cold</sub> , T <sub>ave</sub> ±3.1F |
|         |   | ii) Load regulation         | 8×10 <sup>5</sup> |           | Table 4       |  |  |
| I-i     | Main feedwater cycling  |                             | 2, 100            |           | Fig. I-10     |  |  |
| I-j     | Refueling   |                             | 60                |           | Fig. I-11     |  | Water is replaced in 10 minutes.   |
| I-k     | Ramp load increase between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-12     |  |  |
| I-l     | Ramp load decrease between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-13     |  |  |
| I-m     | RCP startup   |                             | 3, 000            |           | Fig. I-14     |  |  |
| I-n     | RCP shutdown  |                             | 3, 000            |           | Fig. I-15     |  |  |
| I-o     | Core lifetime extension   |                             | 60                |           | Fig. I-16     |  |  |
| I-p     | Primary leakage test  |                             | 120               |           | Fig. I-17     |  |  |
| I-q     | Turbine roll test   |                             | 10                |           | Fig. I-18     |  |  |

Table 8.1-8 RHRS suction loop B line design transients (2/4)

| Level B<br>Mark | Transient   | Occurrence | Reference |               | Remark  |   |
|-----------------|---|------------|-----------|---------------|---|---|
|                 |   |            | Document  | Fig. or Table |   |   |
| II-a            | Loss of load  | 60         |           | Fig. II-1     |   |   |
| II-b            | Loss of offsite power                                   | 60         |           | Fig. II-2     |   |   |
| II-c            | Partial loss of reactor coolant flow                    | 30         |           | Fig. II-3     |   |   |
| II-d            | Reactor trip from full power                            | 60         |           | Fig. II-4     |   |   |
|                 |   |            |           |               |   | i) With no inadvertent cooldown           |
|                 |   |            |           |               |   | ii) With cooldown and no safety injection |
|                 | iii) With cooldown and safety injection                 | 10         |           | Fig. II-6     | Including the transient of Excessive feedwater flow               |   |
| II-e            | Inadvertent RCS depressurization                        | 30         |           | Fig. II-7     |   |   |
| II-f            | Control rod drop  | 30         | Ref. 4    | Fig. II-8     |   |   |
| II-g            | Inadvertent safeguards actuation                        | 30         |           | Fig. II-9     |   |   |
| II-h            | Emergency feedwater cycling                             | 700        |           | Fig. II-10    |   |   |
| II-i            | Cold over-pressure                                      | 30         |           | Fig. II-11    |   |   |
| II-j            | Excessive feedwater flow                                | —          |           | —             | Be covered with the transient of Reactor trip from full power ii) |   |
| II-k            | Loss of offsite power with natural circulation cooldown | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |
| II-l            | Partial loss of emergency feedwater                     | 30         |           | —             | Please use the figure of the transient of Loss of offsite power.  |   |
| II-m            | Safe shutdown   | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |

Table 8.1-8 RHRS suction loop B line design transients (3/4)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |

Table 8.1-8 RHRS suction loop B line design transients (4/4) (branch piping transients)

| Level A, B |                |            |           |               |        |
|------------|----------------|------------|-----------|---------------|--------|
| Mark       | Transient      | Occurrence | Reference |               | Remark |
|            |                |            | Document  | Fig. or Table |        |
| a)         | Plant heat-up  | 120        | Ref. 4    | Fig. I-26     |        |
| b)         | Plant cooldown | 120        |           | Fig. I-27     |        |

Table 8.1-9 RHR return line design transients (1/4)

| Level A |   | Transient                   | Occurrence        | Reference |               | Remark  |  |
|---------|---|-----------------------------|-------------------|-----------|---------------|---|--|
| Mark    |   |                             |                   | Document  | Fig. or Table |   |  |
| I-a     | Plant heat-up (50F/h)   |                             | 120               |           | Fig. I-1      |   |  |
| I-b     | Plant cooldown (100F/h)   |                             | 120               |           | Fig. I-2      | Including the transient of Loss of offsite power with natural circulation cooldown (10 times) and Safe shutdown (1 time). |  |
| I-c-1   | Ramp load increase between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-3      |   |  |
| I-c-2   | Ramp load increase between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-4      |   |  |
| I-d-1   | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-5      |   |  |
| I-d-2   | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-6      |   |  |
| I-e     | Step load increase of 10% of full power   |                             | 600               | Ref. 4    | Fig. I-7      |   |  |
| I-f     | Step load decrease of 10% of full power   |                             | 600               |           | Fig. I-8      |   |  |
| I-g     | Large step load decrease with turbine bypass  |                             | 60                |           | Fig. I-9      |   |  |
| I-h     | Steady-state fluctuation and load regulation  | i) Steady-state fluctuation | 1×10 <sup>6</sup> |           | —             |   | P <sub>p</sub> ±50psi, T <sub>hot</sub> , T <sub>cold</sub> , T <sub>ave</sub> ±3.1F |
|         |   | ii) Load regulation         | 8×10 <sup>5</sup> |           | Table 4       |   |  |
| I-i     | Main feedwater cycling  |                             | 2, 100            |           | Fig. I-10     |   |  |
| I-j     | Refueling   |                             | 60                |           | Fig. I-11     |   | Water is replaced in 10 minutes.   |
| I-k     | Ramp load increase between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-12     |   |  |
| I-l     | Ramp load decrease between 0% and 15% of full power                                 |                             | 600               | Fig. I-13 |               |   |  |
| I-m     | RCP startup   |                             | 3, 000            | Fig. I-14 |               |   |  |
| I-n     | RCP shutdown  |                             | 3, 000            | Fig. I-15 |               |   |  |
| I-o     | Core lifetime extension   |                             | 60                | Fig. I-16 |               |   |  |
| I-p     | Primary leakage test  |                             | 120               | Fig. I-17 |               |   |  |
| I-q     | Turbine roll test   |                             | 10                | Fig. I-18 |               |   |  |

Table 8.1-9 RHR return line design transients (2/4)

| Level B<br>Mark | Transient   | Occurrence | Reference |               | Remark  |   |
|-----------------|---|------------|-----------|---------------|---|---|
|                 |   |            | Document  | Fig. or Table |   |   |
| II-a            | Loss of load  | 60         |           | Fig. II-1     |   |   |
| II-b            | Loss of offsite power                                   | 60         |           | Fig. II-2     |   |   |
| II-c            | Partial loss of reactor coolant flow                    | 30         |           | Fig. II-3     |   |   |
| II-d            | Reactor trip from full power                            | 30         |           | Fig. II-5     | Including the transient of Excessive feedwater flow               |   |
|                 |   |            |           |               |   | i) With no inadvertent cooldown           |
|                 |   |            |           |               |   | ii) With cooldown and no safety injection |
|                 | iii) With cooldown and safety injection                 | 10         |           | Fig. II-6     |   |   |
| II-e            | Inadvertent RCS depressurization                        | 30         |           | Fig. II-7     |   |   |
| II-f            | Control rod drop  | 30         | Ref. 4    | Fig. II-8     |   |   |
| II-g            | Inadvertent safeguards actuation                        | 30         |           | Fig. II-9     |   |   |
| II-h            | Emergency feedwater cycling                             | 700        |           | Fig. II-10    |   |   |
| II-i            | Cold over-pressure                                      | 30         |           | Fig. II-11    |   |   |
| II-j            | Excessive feedwater flow                                | —          |           | —             | Be covered with the transient of Reactor trip from full power ii) |   |
| II-k            | Loss of offsite power with natural circulation cooldown | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |
| II-l            | Partial loss of emergency feedwater                     | 30         |           | —             | Please use the figure of the transient of Loss of offsite power.  |   |
| II-m            | Safe shutdown   | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |

Table 8.1-9 RHR return line design transients (3/4)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |

Table 8.1-9 RHR return line design transients (4/4) (branch piping transients)

| Level A, B |                |            |           |               |        |
|------------|----------------|------------|-----------|---------------|--------|
| Mark       | Transient      | Occurrence | Reference |               | Remark |
|            |                |            | Document  | Fig. or Table |        |
| a)         | Plant cooldown | 120        | Ref. 4    | Fig. I-24     |        |
| b)         | Refueling      | 60         |           | Fig. I-25     |        |
| c)         | Plant heat-up  | 120        |           | Fig. I-26     |        |



Table 8.1-10 Accumulator line design transients (1/4)

| Level A |   | Transient                   | Occurrence        | Reference |               | Remark  |  |
|---------|---|-----------------------------|-------------------|-----------|---------------|---|--|
| Mark    |   |                             |                   | Document  | Fig. or Table |   |  |
| I-a     | Plant heat-up (50F/h)   |                             | 120               |           | Fig. I-1      |   |  |
| I-b     | Plant cooldown (100F/h)   |                             | 120               |           | Fig. I-2      | Including the transient of Loss of offsite power with natural circulation cooldown (10 times) and Safe shutdown (1 time). |  |
| I-c-1   | Ramp load increase between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-3      |   |  |
| I-c-2   | Ramp load increase between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-4      |   |  |
| I-d-1   | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-5      |   |  |
| I-d-2   | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-6      |   |  |
| I-e     | Step load increase of 10% of full power   |                             | 600               | Ref. 4    | Fig. I-7      |   |  |
| I-f     | Step load decrease of 10% of full power   |                             | 600               |           | Fig. I-8      |   |  |
| I-g     | Large step load decrease with turbine bypass  |                             | 60                |           | Fig. I-9      |   |  |
| I-h     | Steady-state fluctuation and load regulation  | i) Steady-state fluctuation | 1×10 <sup>6</sup> |           | —             |   | P <sub>p</sub> ±50psi, T <sub>hot</sub> , T <sub>cold</sub> , T <sub>ave</sub> ±3.1F |
|         |   | ii) Load regulation         | 8×10 <sup>5</sup> |           | Table 4       |   |  |
| I-i     | Main feedwater cycling  |                             | 2, 100            |           | Fig. I-10     |   |  |
| I-j     | Refueling   |                             | 60                |           | Fig. I-11     |   | Water is replaced in 10 minutes.   |
| I-k     | Ramp load increase between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-12     |   |  |
| I-l     | Ramp load decrease between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-13     |   |  |
| I-m     | RCP startup   |                             | 3, 000            |           | Fig. I-14     |   |  |
| I-n     | RCP shutdown  |                             | 3, 000            |           | Fig. I-15     |   |  |
| I-o     | Core lifetime extension   |                             | 60                |           | Fig. I-16     |   |  |
| I-p     | Primary leakage test  |                             | 120               |           | Fig. I-17     |   |  |
| I-q     | Turbine roll test   |                             | 10                |           | Fig. I-18     |   |  |

Table 8.1-10 Accumulator line design transients (2/4)

| Level B<br>Mark | Transient   | Occurrence | Reference |               | Remark  |   |
|-----------------|---|------------|-----------|---------------|---|---|
|                 |   |            | Document  | Fig. or Table |   |   |
| II-a            | Loss of load  | 60         |           | Fig. II-1     |   |   |
| II-b            | Loss of offsite power                                   | 60         |           | Fig. II-2     |   |   |
| II-c            | Partial loss of reactor coolant flow                    | 30         |           | Fig. II-3     |   |   |
| II-d            | Reactor trip from full power                            | 30         |           | Fig. II-5     | Including the transient of Excessive feedwater flow               |   |
|                 |   |            |           |               |   | i) With no inadvertent cooldown           |
|                 |   |            |           |               |   | ii) With cooldown and no safety injection |
|                 | iii) With cooldown and safety injection                 | 10         |           | Fig. II-6     |   |   |
| II-e            | Inadvertent RCS depressurization                        | 30         |           | Fig. II-7     |   |   |
| II-f            | Control rod drop  | 30         | Ref. 4    | Fig. II-8     |   |   |
| II-g            | Inadvertent safeguards actuation                        | 30         |           | Fig. II-9     |   |   |
| II-h            | Emergency feedwater cycling                             | 700        |           | Fig. II-10    |   |   |
| II-i            | Cold over-pressure                                      | 30         |           | Fig. II-11    |   |   |
| II-j            | Excessive feedwater flow                                | —          |           | —             | Be covered with the transient of Reactor trip from full power ii) |   |
| II-k            | Loss of offsite power with natural circulation cooldown | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |
| II-l            | Partial loss of emergency feedwater                     | 30         |           | —             | Please use the figure of the transient of Loss of offsite power.  |   |
| II-m            | Safe shutdown   | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |

Table 8.1-10 Accumulator line design transients (3/4)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |

Table 8.1-10 Accumulator line design transients (4/4) (branch piping transients)

| Level A, B |   |            |           |               |        |
|------------|---|------------|-----------|---------------|--------|
| Mark       | Transient                                     | Occurrence | Reference |               | Remark |
|            |   |            | Document  | Fig. or Table |        |
| a)         | Inadvertent actuation of the accumulator tank | 5          | Ref. 4    | Fig. II-15    |        |
| b)         | Inadvertent RCS depressurization              | 30         |           | Fig. II-16    |        |

Table 8.1-11 DVI line design transients (1/4)

| Level A<br>Mark | Transient   | Occurrence                  | Reference |                   | Remark   |  |
|-----------------|---|-----------------------------|-----------|-------------------|--|--|
|                 |   |                             | Document  | Fig. or Table     |  |  |
| I-a             | Plant heat-up (50F/h)   | 120                         |           | Fig. I-1          |  |  |
| I-b             | Plant cooldown (100F/h)   | 120                         |           | Fig. I-2          | Including the transient of Loss of offsite power with natural circulation cooldown (10 times) and Safe shutdown (1 time) . |  |
| I-c-1           | Ramp load increase between 15% and 100% of full power (5% of full power per minute) | 600                         |           | Fig. I-3          |  |  |
| I-c-2           | Ramp load increase between 50% and 100% of full power (5% of full power per minute) | 19, 200                     |           | Fig. I-4          |  |  |
| I-d-1           | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) | 600                         |           | Fig. I-5          |  |  |
| I-d-2           | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) | 19, 200                     |           | Fig. I-6          |  |  |
| I-e             | Step load increase of 10% of full power   | 600                         | Ref. 4    | Fig. I-7          |  |  |
| I-f             | Step load decrease of 10% of full power   | 600                         |           | Fig. I-8          |  |  |
| I-g             | Large step load decrease with turbine bypass  | 60                          |           | Fig. I-9          |  |  |
| I-h             | Steady-state fluctuation and load regulation  | i) Steady-state fluctuation |           |                   |  | P <sub>p</sub> ±50psi, T <sub>hot</sub> , T <sub>cold</sub> , T <sub>ave</sub> ±3.1F |
|                 |   | ii) Load regulation         |           | 1×10 <sup>6</sup> |  |  |
| I-i             | Main feedwater cycling  | 8×10 <sup>5</sup>           |           | Table 4           |  |  |
| I-j             | Refueling   | 2, 100                      |           | Fig. I-10         |  |  |
| I-k             | Ramp load increase between 0% and 15% of full power                                 | 60                          |           | Fig. I-11         |  | Water is replaced in 10 minutes.   |
| I-l             | Ramp load decrease between 0% and 15% of full power                                 | 600                         |           | Fig. I-12         |  |  |
| I-m             | RCP startup   | 600                         |           | Fig. I-13         |  |  |
| I-n             | RCP shutdown  | 3, 000                      |           | Fig. I-14         |  |  |
| I-o             | Core lifetime extension   | 3, 000                      |           | Fig. I-15         |  |  |
| I-p             | Primary leakage test  | 60                          |           | Fig. I-16         |  |  |
| I-q             | Turbine roll test   | 120                         |           | Fig. I-17         |  |  |
|                 |   | 10                          |           | Fig. I-18         |  |  |

Table 8.1-11 DVI line design transients (2/4)

| Level B<br>Mark | Transient   | Occurrence | Reference |               | Remark  |   |
|-----------------|---|------------|-----------|---------------|---|---|
|                 |   |            | Document  | Fig. or Table |   |   |
| II-a            | Loss of load  | 60         |           | Fig. II-1     |   |   |
| II-b            | Loss of offsite power                                   | 60         |           | Fig. II-2     |   |   |
| II-c            | Partial loss of reactor coolant flow                    | 30         |           | Fig. II-3     |   |   |
| II-d            | Reactor trip from full power                            | 30         |           | Fig. II-5     | Including the transient of Excessive feedwater flow               |   |
|                 |   |            |           |               |   | i) With no inadvertent cooldown           |
|                 |   |            |           |               |   | ii) With cooldown and no safety injection |
|                 | iii) With cooldown and safety injection                 | 10         |           | Fig. II-6     |   |   |
| II-e            | Inadvertent RCS depressurization                        | 30         |           | Fig. II-7     |   |   |
| II-f            | Control rod drop  | 30         | Ref. 4    | Fig. II-8     |   |   |
| II-g            | Inadvertent safeguards actuation                        | 30         |           | Fig. II-9     |   |   |
| II-h            | Emergency feedwater cycling                             | 700        |           | Fig. II-10    |   |   |
| II-i            | Cold over-pressure                                      | 30         |           | Fig. II-11    |   |   |
| II-j            | Excessive feedwater flow                                | —          |           | —             | Be covered with the transient of Reactor trip from full power ii) |   |
| II-k            | Loss of offsite power with natural circulation cooldown | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |
| II-l            | Partial loss of emergency feedwater                     | 30         |           | —             | Please use the figure of the transient of Loss of offsite power.  |   |
| II-m            | Safe shutdown   | —          |           | —             | Be covered with the transient of Plant cooldown                   |   |

Table 8.1-11 DVI line design transients (3/4)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |

Table 8.1-11 DVI line design transients (4/4) (branch piping transients)

| Level A, B |   |            |           |               |        |
|------------|---|------------|-----------|---------------|--------|
| Mark       | Transient   | Occurrence | Reference |               | Remark |
|            |   |            | Document  | Fig. or Table |        |
| a)         | Reactor trip from full power with cooldown and safety injection | 10         | Ref. 4    | Fig. II-17    |        |
| b)         | Inadvertent RCS depressurization                                | 30         |           | Fig. II-18    |        |
| c)         | Inadvertent safeguards actuation                                | 30         |           | Fig. II-19    |        |
| d)         | Safe shutdown   | 1          |           | Fig. II-20    |        |

Table 8.1-12 CVCS charging line design transients (1/4)

| Level A |   | Transient                   | Occurrence        | Reference |               | Remark   |  |
|---------|---|-----------------------------|-------------------|-----------|---------------|--|--|
| Mark    |   |                             |                   | Document  | Fig. or Table |  |  |
| I-a     | Plant heat-up (50F/h)   |                             | 120               |           | Fig. I-1      |  |  |
| I-b     | Plant cooldown (100F/h)   |                             | 120               |           | Fig. I-2      | Including the transient of Loss of offsite power with natural circulation cooldown (10 times) and Safe shutdown (1 time) . |  |
| I-c-1   | Ramp load increase between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-3      |  |  |
| I-c-2   | Ramp load increase between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-4      |  |  |
| I-d-1   | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-5      |  |  |
| I-d-2   | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-6      |  |  |
| I-e     | Step load increase of 10% of full power   |                             | 600               | Ref. 4    | Fig. I-7      |  |  |
| I-f     | Step load decrease of 10% of full power   |                             | 600               |           | Fig. I-8      |  |  |
| I-g     | Large step load decrease with turbine bypass  |                             | 60                |           | Fig. I-9      |  |  |
| I-h     | Steady-state fluctuation and load regulation  | i) Steady-state fluctuation | 1×10 <sup>6</sup> |           | —             |  | P <sub>p</sub> ±50psi, T <sub>hot</sub> , T <sub>cold</sub> , T <sub>ave</sub> ±3.1F |
|         |   | ii) Load regulation         | 8×10 <sup>5</sup> |           | Table 4       |  |  |
| I-i     | Main feedwater cycling  |                             | 2, 100            |           | Fig. I-10     |  |  |
| I-j     | Refueling   |                             | 60                |           | Fig. I-11     |  | Water is replaced in 10 minutes.   |
| I-k     | Ramp load increase between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-12     |  |  |
| I-l     | Ramp load decrease between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-13     |  |  |
| I-m     | RCP startup   |                             | 3, 000            |           | Fig. I-14     |  |  |
| I-n     | RCP shutdown  |                             | 3, 000            |           | Fig. I-15     |  |  |
| I-o     | Core lifetime extension   |                             | 60                |           | Fig. I-16     |  |  |
| I-p     | Primary leakage test  |                             | 120               |           | Fig. I-17     |  |  |
| I-q     | Turbine roll test   |                             | 10                |           | Fig. I-18     |  |  |

Table 8.1-12 CVCS charging line design transients (2/4)

| Level B |   | Transient                                 | Occurrence | Reference     |            | Remark  |
|---------|---|---|------------|---------------|------------|---|
| Mark    | Document  |   |            | Fig. or Table |            |   |
| II-a    | Loss of load  |   | 60         |               | Fig. II-1  |   |
| II-b    | Loss of offsite power                                   |   | 60         |               | Fig. II-2  |   |
| II-c    | Partial loss of reactor coolant flow                    |   | 30         |               | Fig. II-3  |   |
| II-d    | Reactor trip from full power                            | i) With no inadvertent cooldown           | 60         |               | Fig. II-4  |   |
|         |   | ii) With cooldown and no safety injection | 30         |               | Fig. II-5  | Including the transient of Excessive feedwater flow               |
|         |   | iii) With cooldown and safety injection   | 10         |               | Fig. II-6  |   |
| II-e    | Inadvertent RCS depressurization                        |   | 30         |               | Fig. II-7  |   |
| II-f    | Control rod drop  |   | 30         |               | Fig. II-8  |   |
| II-g    | Inadvertent safeguards actuation                        |   | 30         |               | Fig. II-9  |   |
| II-h    | Emergency feedwater cycling                             |   | 700        |               | Fig. II-10 |   |
| II-i    | Cold over-pressure                                      |   | 30         |               | Fig. II-11 |   |
| II-j    | Excessive feedwater flow                                |   | —          |               | —          | Be covered with the transient of Reactor trip from full power ii) |
| II-k    | Loss of offsite power with natural circulation cooldown |   | —          |               | —          | Be covered with the transient of Plant cooldown                   |
| II-l    | Partial loss of emergency feedwater                     |   | 30         |               | —          | Please use the figure of the transient of Loss of offsite power.  |
| II-m    | Safe shutdown   |   | —          |               | —          | Be covered with the transient of Plant cooldown                   |

Ref. 4



Table 8.1-12 CVCS charging line design transients (3/4)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |

Table 8.1-12 CVCS charging line design transients (4/4) (branch piping transients)

| Level A, B |  | Transient      | Occurrence | Reference |               | Remark |
|------------|--|----------------|------------|-----------|---------------|--------|
| Mark       |  |                |            | Document  | Fig. or Table |        |
| 1.A        | Letdown shut off and re-initiated          |                | 70         |           | Fig. II-13    |        |
| 1.B        | Charging line shut off and re-initiated    | a) Maintenance | 30         |           | Fig. I-19     |        |
|            |  | b) SI          | 70         |           | Fig. II-14    |        |
| 2.A        | Charging flow 50% step decrease and return |                | 20, 400    | Ref. 4    | Fig. I-20     |        |
| 2.B        | Charging flow 50% step increase and return |                | 23, 600    |           | Fig. I-21     |        |
| 2.C        | Letdown flow 50% step decrease and return  |                | 2, 900     |           | Fig. I-22     |        |
| 2.D        | Letdown flow 100% step increase and return |                | 19, 800    |           | Fig. I-23     |        |

Table 8.1-13 CVCS let down line design transients (1/4)

| Level A |   | Transient                   | Occurrence        | Reference |               | Remark   |  |
|---------|---|-----------------------------|-------------------|-----------|---------------|--|--|
| Mark    |   |                             |                   | Document  | Fig. or Table |  |  |
| I-a     | Plant heat-up (50F/h)   |                             | 120               |           | Fig. I-1      |  |  |
| I-b     | Plant cooldown (100F/h)   |                             | 120               |           | Fig. I-2      | Including the transient of Loss of offsite power with natural circulation cooldown (10 times) and Safe shutdown (1 time) . |  |
| I-c-1   | Ramp load increase between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-3      |  |  |
| I-c-2   | Ramp load increase between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-4      |  |  |
| I-d-1   | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-5      |  |  |
| I-d-2   | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-6      |  |  |
| I-e     | Step load increase of 10% of full power   |                             | 600               | Ref. 4    | Fig. I-7      |  |  |
| I-f     | Step load decrease of 10% of full power   |                             | 600               |           | Fig. I-8      |  |  |
| I-g     | Large step load decrease with turbine bypass  |                             | 60                |           | Fig. I-9      |  |  |
| I-h     | Steady-state fluctuation and load regulation  | i) Steady-state fluctuation | 1×10 <sup>6</sup> |           | —             |  | P <sub>p</sub> ±50psi, T <sub>hot</sub> , T <sub>cold</sub> , T <sub>ave</sub> ±3.1F |
|         |   | ii) Load regulation         | 8×10 <sup>5</sup> |           | Table 4       |  |  |
| I-i     | Main feedwater cycling  |                             | 2, 100            |           | Fig. I-10     |  |  |
| I-j     | Refueling   |                             | 60                |           | Fig. I-11     |  | Water is replaced in 10 minutes.   |
| I-k     | Ramp load increase between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-12     |  |  |
| I-l     | Ramp load decrease between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-13     |  |  |
| I-m     | RCP startup   |                             | 3, 000            |           | Fig. I-14     |  |  |
| I-n     | RCP shutdown  |                             | 3, 000            |           | Fig. I-15     |  |  |
| I-o     | Core lifetime extension   |                             | 60                |           | Fig. I-16     |  |  |
| I-p     | Primary leakage test  |                             | 120               |           | Fig. I-17     |  |  |
| I-q     | Turbine roll test   |                             | 10                |           | Fig. I-18     |  |  |

Table 8.1-13 CVCS let down line design transients (2/4)

| Level B |   | Transient                                 | Occurrence | Reference     |            | Remark  |
|---------|---|---|------------|---------------|------------|---|
| Mark    | Document  |   |            | Fig. or Table |            |   |
| II-a    | Loss of load  |   | 60         |               | Fig. II-1  |   |
| II-b    | Loss of offsite power                                   |   | 60         |               | Fig. II-2  |   |
| II-c    | Partial loss of reactor coolant flow                    |   | 30         |               | Fig. II-3  |   |
| II-d    | Reactor trip from full power                            | i) With no inadvertent cooldown           | 60         |               | Fig. II-4  |   |
|         |   | ii) With cooldown and no safety injection | 30         |               | Fig. II-5  | Including the transient of Excessive feedwater flow               |
|         |   | iii) With cooldown and safety injection   | 10         |               | Fig. II-6  |   |
| II-e    | Inadvertent RCS depressurization                        |   | 30         |               | Fig. II-7  |   |
| II-f    | Control rod drop  |   | 30         |               | Fig. II-8  |   |
| II-g    | Inadvertent safeguards actuation                        |   | 30         |               | Fig. II-9  |   |
| II-h    | Emergency feedwater cycling                             |   | 700        |               | Fig. II-10 |   |
| II-i    | Cold over-pressure                                      |   | 30         |               | Fig. II-11 |   |
| II-j    | Excessive feedwater flow                                |   | —          |               | —          | Be covered with the transient of Reactor trip from full power ii) |
| II-k    | Loss of offsite power with natural circulation cooldown |   | —          |               | —          | Be covered with the transient of Plant cooldown                   |
| II-l    | Partial loss of emergency feedwater                     |   | 30         |               | —          | Please use the figure of the transient of Loss of offsite power.  |
| II-m    | Safe shutdown   |   | —          |               | —          | Be covered with the transient of Plant cooldown                   |

Ref. 4

Table 8.1-13 CVCS let down line design transients (3/4)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |

Table 8.1-13 CVCS let down line design transients (4/4) (branch piping transients)

| Level A, B |  |            |           |               |  |
|------------|--|------------|-----------|---------------|--|
| Mark       | Transient  | Occurrence | Reference |               | Remark                                     |
|            |  |            | Document  | Fig. or Table |  |
| a)         | Plant heat-up  | 120        | Ref. 4    | Fig. I-26     |  |
| b)         | Plant cooldown                                       | 120        |           | Fig. I-27     |  |
| c)         | Letdown line shut off and re-initiated (maintenance) | 30         |           | Fig. I-28     | c) and d) have the same transient diagram. |
| d)         | Letdown line shut off and re-initiated (SI)          | 70         |           | Fig. I-28     |  |
| e)         | RCS drain  | 120        |           | Fig. I-29     |  |

Table 8.1-14 CVCS seal injection line design transients (1/3)

| Level A |   | Transient                   | Occurrence        | Reference |               | Remark   |  |
|---------|---|-----------------------------|-------------------|-----------|---------------|--|--|
| Mark    |   |                             |                   | Document  | Fig. or Table |  |  |
| I-a     | Plant heat-up (50F/h)   |                             | 120               |           | Fig. I-1      |  |  |
| I-b     | Plant cooldown (100F/h)   |                             | 120               |           | Fig. I-2      | Including the transient of Loss of offsite power with natural circulation cooldown (10 times) and Safe shutdown (1 time) . |  |
| I-c-1   | Ramp load increase between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-3      |  |  |
| I-c-2   | Ramp load increase between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-4      |  |  |
| I-d-1   | Ramp load decrease between 15% and 100% of full power (5% of full power per minute) |                             | 600               |           | Fig. I-5      |  |  |
| I-d-2   | Ramp load decrease between 50% and 100% of full power (5% of full power per minute) |                             | 19, 200           |           | Fig. I-6      |  |  |
| I-e     | Step load increase of 10% of full power   |                             | 600               | Ref. 4    | Fig. I-7      |  |  |
| I-f     | Step load decrease of 10% of full power   |                             | 600               |           | Fig. I-8      |  |  |
| I-g     | Large step load decrease with turbine bypass  |                             | 60                |           | Fig. I-9      |  |  |
| I-h     | Steady-state fluctuation and load regulation  | i) Steady-state fluctuation | 1×10 <sup>6</sup> |           | —             |  | P <sub>p</sub> ±50psi, T <sub>hot</sub> , T <sub>cold</sub> , T <sub>ave</sub> ±3.1F |
|         |   | ii) Load regulation         | 8×10 <sup>5</sup> |           | Table 4       |  |  |
| I-i     | Main feedwater cycling  |                             | 2, 100            |           | Fig. I-10     |  |  |
| I-j     | Refueling   |                             | 60                |           | Fig. I-11     |  | Water is replaced in 10 minutes.   |
| I-k     | Ramp load increase between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-12     |  |  |
| I-l     | Ramp load decrease between 0% and 15% of full power                                 |                             | 600               |           | Fig. I-13     |  |  |
| I-m     | RCP startup   |                             | 3, 000            |           | Fig. I-14     |  |  |
| I-n     | RCP shutdown  |                             | 3, 000            |           | Fig. I-15     |  |  |
| I-o     | Core lifetime extension   |                             | 60                |           | Fig. I-16     |  |  |
| I-p     | Primary leakage test  |                             | 120               |           | Fig. I-17     |  |  |
| I-q     | Turbine roll test   |                             | 10                |           | Fig. I-18     |  |  |

Table 8.1-14 CVCS seal injection line design transients (2/3)

| Level B<br>Mark | Transient   | Occurrence | Reference   |               | Remark  |   |
|-----------------|---|------------|---|---------------|---|---|
|                 |   |            | Document  | Fig. or Table |   |   |
| II-a            | Loss of load  | 60         |   | Fig. II-1     |   |   |
| II-b            | Loss of offsite power                                   | 60         |   | Fig. II-2     |   |   |
| II-c            | Partial loss of reactor coolant flow                    | 30         |   | Fig. II-3     |   |   |
| II-d            | Reactor trip from full power                            | 60         | i) With no inadvertent cooldown<br>ii) With cooldown and no safety injection<br>iii) With cooldown and safety injection | Fig. II-4     |   |   |
|                 |   |            |   | Fig. II-5     | Including the transient of Excessive feedwater flow |   |
|                 |   |            |   | Fig. II-6     |   |   |
| II-e            | Inadvertent RCS depressurization                        | 30         | Ref. 4  | Fig. II-7     |   |   |
| II-f            | Control rod drop  | 30         |   | Fig. II-8     |   |   |
| II-g            | Inadvertent safeguards actuation                        | 30         |   | Fig. II-9     |   |   |
| II-h            | Emergency feedwater cycling                             | 700        |   | Fig. II-10    |   |   |
| II-i            | Cold over-pressure                                      | 30         |   | Fig. II-11    |   |   |
| II-j            | Excessive feedwater flow                                | —          |   | —             | —   | Be covered with the transient of Reactor trip from full power ii) |
| II-k            | Loss of offsite power with natural circulation cooldown | —          |   | —             | —   | Be covered with the transient of Plant cooldown                   |
| II-l            | Partial loss of emergency feedwater                     | 30         |   | —             | —   | Please use the figure of the transient of Loss of offsite power.  |
| II-m            | Safe shutdown   | —          |   | —             | —   | Be covered with the transient of Plant cooldown                   |

Table 8.1-14 CVCS seal injection line design transients (3/3)

| Level C |                                |            |           |               |        |
|---------|--------------------------------|------------|-----------|---------------|--------|
| Mark    | Transient                      | Occurrence | Reference |               | Remark |
|         |                                |            | Document  | Fig. or Table |        |
| III-a   | Small loss of coolant accident | 5          | Ref. 4    | Fig. III-1    |        |
| III-b   | Small steam line break         | 5          |           | Fig. III-2    |        |
| III-c   | Complete loss of flow          | 5          |           | Fig. III-3    |        |
| III-d   | Small feedwater line break     | 5          |           | Fig. III-4    |        |
| III-e   | SG tube rupture                | 5          |           | Fig. III-5    |        |
| Level D |                                |            |           |               |        |
| IV-a    | Large loss of coolant accident | 1          | Ref. 4    | Fig. IV-1     |        |
| IV-b    | Large steam line break         | 1          |           | Fig. IV-2     |        |
| IV-c    | RCP locked rotor               | 1          |           | Fig. IV-3     |        |
| IV-d    | Control rod ejection           | 1          |           | Fig. IV-4     |        |
| IV-e    | Large feedwater line break     | 1          |           | Fig. IV-5     |        |
| Test    |                                |            |           |               |        |
| V-a     | Primary-side hydrostatic test  | 10         | Ref. 4    | —             |        |



## 8.2 LOAD COMBINATIONS

The loading conditions consist of various combinations of pressure, thermal and external loads.

The loads combinations considered in the analysis are listed in the Table below.

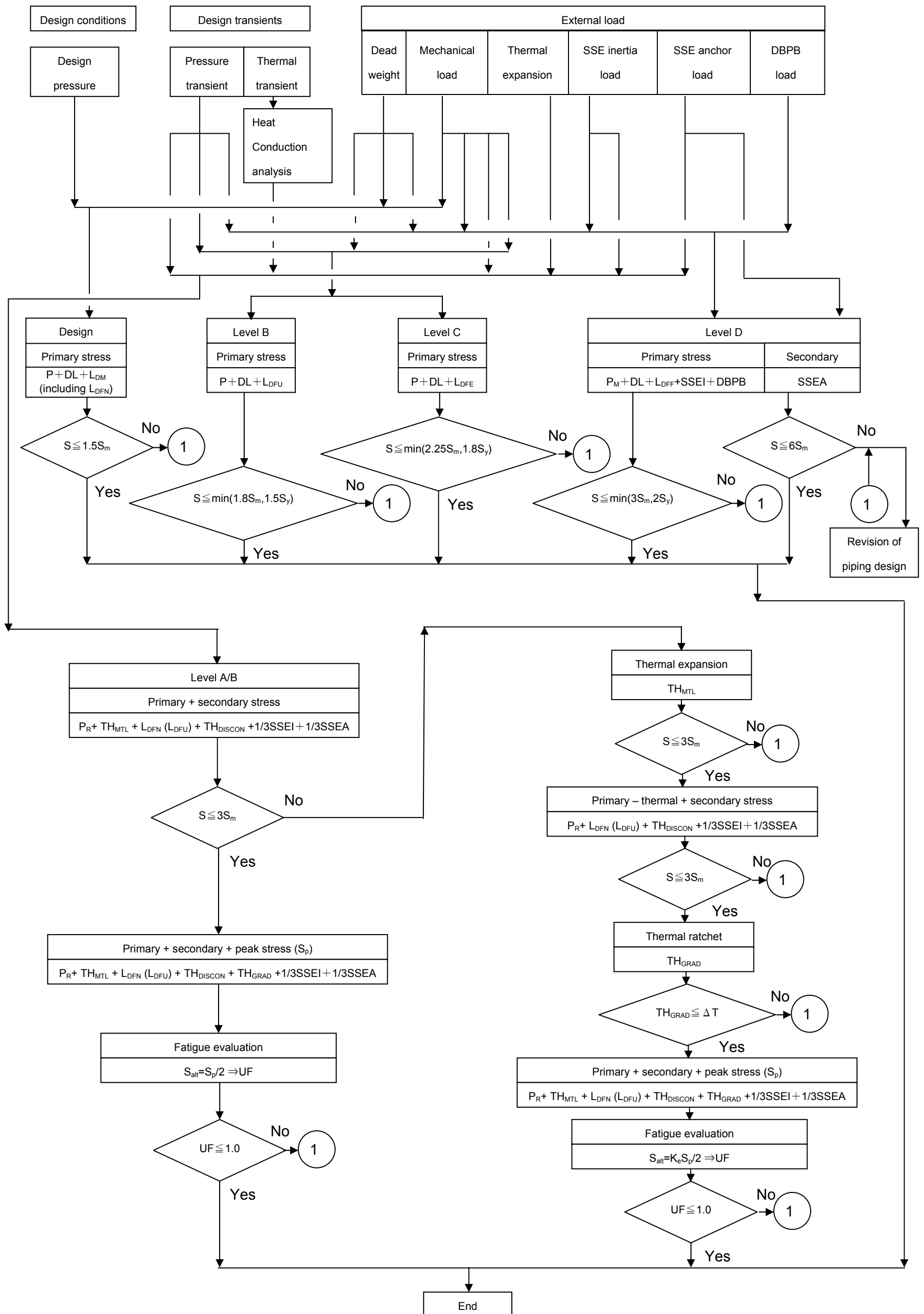
**Table 8.2-1 Loadings to be considered for various Load Condition**

| Loading Conditions   | Design | Level A/B | Level C | Level D |
|--|--------|-----------|---------|---------|
| Design Pressure  | ✓      |           |         |         |
| Maximum Operating Pressure   |        | ✓         | ✓       | ✓       |
| Dead Load  | ✓      |           | ✓       | ✓       |
| Mechanical load (pressurizer safety valve and safety depressurization valve thrust load) | ✓      | ✓         | ✓       | ✓       |
| Level A thermal, pressure transient load   |        | ✓         |         |         |
| Level B thermal, pressure transient load   |        | ✓         |         |         |
| Level C pressure transient load  |        |           | ✓       |         |
| Level D pressure transient load  |        |           |         | ✓       |
| 1/3 SSE Loads  |        | ✓         |         |         |
| SSE Loads  |        |           |         | ✓       |
| Design Basis Pipe Break  |        |           |         | ✓       |

## **9.0 METHODOLOGY**

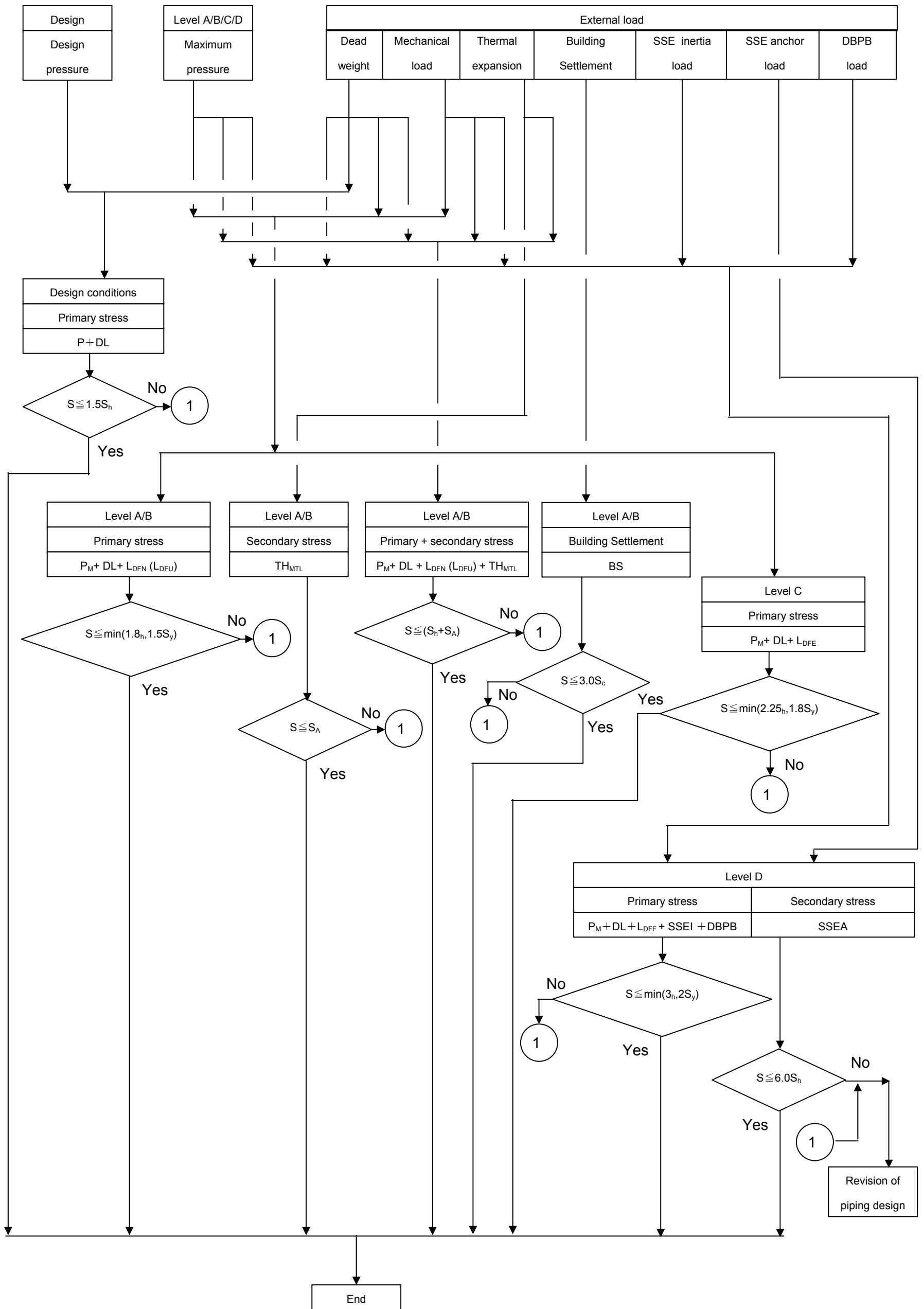
### **9.1 LOGIC DIAGRAM OF EVALUATION**

For the Reactor Coolant Branch Piping, piping that exceeds 1 inch was evaluated according to NB-3650 (Reference 7) and 1 inch or smaller than 1 inch piping was evaluated according to NC-3650 (Reference 7). The evaluation logic diagrams are shown in Figure 9.1-1 and Figure 9.1-2.



Note 1: In addition to the logic diagram shown above, permissible pressure was evaluated for Levels B, C, and D.

Figure 9.1-1 Evaluation Logic Diagram (greater than 1 inch piping)



Note 1: Either secondary stress evaluation or primary plus secondary stress evaluation may be used.

Figure 9.1-2 Evaluation Logic Diagram (1 inch or smaller piping)

## 9.2 STRUCTURAL ANALYSIS

A structural analysis was performed with the following conditions according to the Piping Design Criteria (Reference 2).

### 9.2.1 Analysis model

For dynamic analysis, the piping system is idealized as a three dimensional space frame. The analysis model consists of a sequence of nodes connected by straight pipe elements and curved pipe elements with stiffness properties representing the piping, and other in-line components.

Piping restraints and supports are idealized as zero length springs with appropriate stiffness values for the restrained degrees of freedom.

In the dynamic mathematical model, the distributed mass of the system, including pipe, contents, and insulation weight, is represented as lumped masses located at each node, which is designated as a mass point.

The following formula is used to determine the spacing between two successive mass points. The PIPESTRESS program uses this formula for mass point spacing.

$$L = \sqrt{\left[ \frac{K}{F_R} \right]} \sqrt{\frac{EI}{W}}$$

where

$$K = 0.743$$

$$L = \text{Mass point spacing (ft)}$$

$$F_R = \text{Cut-off frequency (Hz)}$$

$$E = \text{Modulus of elasticity of pipe material (psi)}$$

$$I = \text{Moment of inertia of pipe cross-section (in}^4\text{)}$$

$$W = \text{Mass per unit length of piping + insulation + contents (lbm/ft)}$$

Concentrated weights of in-line components, such as valves, flanges, and instrumentation, are also modeled as lumped masses.

Torsional effects of eccentric masses are included in the analysis.

Seismic analysis of RCL branch lines including DVI lines, decoupled from the analysis model of the RCL or RV, were performed using the applicable envelope response spectra for the RCL or RV considering the connection point as an anchor. The movements (displacements and rotations) of the RCL and RV from the thermal, SAM or pipe break analysis were applied as anchor movement with their respective load cases in the decoupled branch line analysis.

As to the seismic analysis model of the Pressurizer branch lines such as Surge line, Spray line, Safety Depressurization valve line and Safety valve line, the Pressurizer analysis model in reference 3 was coupled with these branch line analyses.

## **9.2.2 Seismic Analysis Method**

### **9.2.2.1 Damping Values**

The damping value used for the SSE was generally 4%, which is consistent with Table 3 of the RG 1.61, Rev.1. In the case when Pressurizer analysis model was coupled with piping analysis model as described above, 3% damping which was used for seismic analysis of Pressurizer was conservatively used.

### **9.2.2.2 Combination of Modal Responses**

For piping systems with no closely spaced modes, the SRSS method was applied to obtain the representative maximum response of each element, for each direction of excitation. A 10% grouping method was used for combining the responses of closely spaced modes.

### **9.2.2.3 High-Frequency Mode**

The PIPESTRESS computer program was used for analyzing the piping systems. This program uses the LOF method to calculate the effect of the high frequency rigid modes. The results obtained were treated as an additional modal result from a non-closely spaced last

mode, and were combined with other modal responses by the methods described in Subsection 9.2.2.2.

#### **9.2.2.4 Directional Combination**

The collinear responses due to each of the three spatial input components of motion were combined using the SRSS method

#### **9.2.2.5 Seismic Anchor Motion**

The effects of differential displacements of equipment or structures to which the piping system attaches during a SSE were considered.

The analysis of these seismic anchor motions (SAMs) was performed as a static analysis with all dynamic supports active. The results of this analysis were combined with the piping system seismic inertia analysis results by absolute summation.

Where supports were located within a single structure, the seismic motions were considered to be in-phase and the relative displacement between the support locations was considered in the analysis. Where supports were located within different structures, the seismic motions at these locations were assumed to move 180 degrees out-of-phase while performing the analysis.

#### **9.2.2.6 Independent Support Motion Method**

(

The supports were divided into support groups. Each support group was made up of supports that had similar time-history input. The responses caused by each support group were combined by the ABS method. The modal and directional responses were then combined as discussed above. Floor response spectrum curves used for ISM were generated using damping values identified in Section 9.2.2.1.

)

### **9.2.3 Time-History Method**

The fluid transient analysis was performed to provide the hydraulic transient input for the pressurizer safety valve and safety depressurization valve piping using RELAP5-3D (Reference 7). The time history hydraulic forces were calculated using the pressure transient, flow rate, and other fluid property obtained by the fluid transient analysis. The structural time history analysis was performed using PIPESTRESS (Reference 8) by modal superposition method.



### 9.3 THERMAL STRESS ANALYSIS

A heat conduction analysis was performed to obtain the piping temperature distribution during a thermal transient. For heat conduction analysis, the ABAQUS (Reference 10) general finite element method program was used.

In the heat conduction analysis, the temperature distribution was obtained for structural discontinuities (valves or reducers, for example) and in the piping plate thickness direction during the transient. From those results, the temperatures,  $T_a$  and  $T_b$ , of the structural discontinuity, and the temperature differences of the inner and outer pipe surfaces,  $\Delta T_1$  and  $\Delta T_2$ , were computed using our independently developed P4TEDIA program (see section 10).

Of the Level A and Level B transients, the transient that applied to each system was used. The change in the fluid temperature and heat transfer coefficient at the inner surface of the piping were used. The heat transfer coefficient used was the value obtained from the equation, described below (Gnielinski's equation), for turbulent flow within a cylindrical pipe. The outer surface of the piping was considered as a heat-retaining insulator.

$$Nu = \frac{(f/2)(Re - 1000)Pr}{1 + 12.7(f/2)^{1/2}(Pr^{2/3} - 1)}$$

$$0.5 \leq Pr \leq 2000$$

$$2300 \leq Re \leq 5 \times 10^6$$

$$1/f^{0.5} = 1.5635 \ln(Re/7) \quad (4 \times 10^3 \leq Re \leq 1 \times 10^7)$$

$$\alpha = Nu \cdot \lambda / d$$

$$Re = u \cdot d / \nu$$

$Nu$  : Nusselt number

$Re$  : Reynolds number

$Pr$  : Prandtl number

$\alpha$  : Heat transfer coefficient

$\lambda$  : Thermal conductivity of fluid

$\nu$  : Kinematic viscosity of fluid

$u$  : Flow velocity

$d$  : Inner diameter of pipe

## 9.4 STRESS EVALUATION

Stress limits for design and service loadings are as follows.

### 9.4.1 Piping that exceeds 1 inch (evaluated according to NB-3650)

(1) Design limit

(a) Primary stress evaluation (eq.9)

$$B_1 \frac{PD_0}{2t} + B_2 \frac{D_0}{2I} M_i \leq 1.5S_m$$

B<sub>1</sub>, B<sub>2</sub>: Stress indices

P: Design pressure

D<sub>0</sub>: Outside diameter

t: Wall thickness

I: Moment of inertia

M<sub>i</sub>: Dead weight, mechanical load (pressurizer safety depressurization valve and safety valve water hammer load) moment

(2) Level A/B service limits

(a) primary plus secondary stress evaluation (eq.10)

$$S_n = C_1 \frac{P_0 D_0}{2t} + C_2 \frac{D_0}{2I} M_i + C_3 E \alpha |T_a - T_b| \leq 3S_m$$

C<sub>1</sub>, C<sub>2</sub>: Stress indices

P<sub>0</sub>: Pressure range

M<sub>i</sub>: Moment ranges for following loads. Thermal expansion, mechanical load (pressurizer safety depressurization valve and safety valve water hammer load), seismic inertia load (1/3SSE), seismic anchor load (1/3SSE)

E: modulus of elasticity (room temperature)

α: Coefficient of thermal expansion (room temperature)

T<sub>a</sub>-T<sub>b</sub>: Structural discontinuity temperature difference range

(b) Primary plus secondary plus peak stress evaluation (Eq.11)

$$S_p = K_1 C_1 \frac{P_0 D_0}{2t} + K_2 C_2 \frac{D_0}{2I} M_i + \frac{1}{2(1-\nu)} K_3 E \alpha |\Delta T_1| + K_3 C_3 E \alpha |T_a - T_b| + \frac{1}{1-\nu} E \alpha |\Delta T_2|$$

K<sub>1</sub>, K<sub>2</sub>, K<sub>3</sub>: Stress indices

ΔT<sub>1</sub>: Absolute value of the range of the temperature difference between the temperature of the outside T<sub>0</sub> and the temperature of the inside surface T<sub>1</sub> of the piping product assuming moment generating equivalent linear temperature distribution

ΔT<sub>2</sub>: Absolute value of the range for that portion of the nonlinear thermal gradient through the wall thickness not included in ΔT<sub>1</sub>.

ν: Poisson's ratio (=0.3)

Sp is computed to obtain the stress intensity Salt for the fatigue analysis described later. Sp does not have any allowable stress.

(c) Fatigue evaluation

For  $S_n \leq 3S_m$

$$S_{alt} = \frac{S_p}{2},$$

$$UF \leq 1.0$$

(d) Simplified elastic-plastic discontinuity analysis

For  $S_n > 3 S_m$

1)

$$S_e = C_2 \frac{D_0}{2I} M_i^* \leq 3S_m \text{ (eq.12)}$$

$M_i^*$ : Thermal expansion (including anchor movements) moment range

2)

$$C_1 \frac{P_0 D_0}{2t} + C_2 \frac{D_0}{2I} M_i + C_3 E \alpha |T_a - T_b| \leq 3S_m \text{ (eq.13)}$$

$M_i$ : Moment ranges for following loads. Dead weight, mechanical load (pressurizer safety depressurization valve and safety valve water hammer load), seismic inertial load (1/3SSE), seismic anchor load (1/3SSE)

$C_3$ : Stress index

3) Fatigue Evaluation

$$S_{alt} = K_e \frac{S_p}{2} \text{ (eq.14)}$$

$$UF \leq 1.0$$

where

$$K_e = 1.0 \cdots S_n \leq 3S_m$$

$$K_e = 1.0 + \frac{1-n}{n(m-1)} \left( \frac{S_n}{3S_m} - 1 \right) \cdots 3S_m < S_n < 3mS_m$$

$$K_e = \frac{1}{n} \cdots S_n \geq 3mS_m$$

$n=0.3, m=1.7 \cdots$  for austenitic stainless steel

4)

$$\Delta T_{1range} \leq_e \frac{y'S_y}{0.7E\alpha} C_4$$

|    |      |      |      |      |
|----|------|------|------|------|
| x  | 0.3  | 0.5  | 0.7  | 0.8  |
| y' | 3.33 | 2.00 | 1.20 | 0.80 |

$$x = \frac{PD_0}{2t} \frac{1}{S_y}$$

P: maximum pressure for the set of conditions under consideration

C4: 1.3 (austenitic stainless steel)

Sy: Yield point at the average fluid temperature of the load set

(3) Level B service limits

(a) Permissible pressure

$$P_M \leq 1.1P_a$$

$$P_a = \frac{2S_m t}{D_0 - 2yt}$$

P<sub>m</sub>: maximum pressure for Level B

y:0.4

(b) Primary stress evaluation (eq.9)

$$B_1 \frac{PD_0}{2t} + B_2 \frac{D_0}{2I} M_i \leq \min(1.8S_m, 1.5S_y)$$

P: Maximum pressure for Level B

M<sub>i</sub>: Moment ranges for following loads. Dead weight, mechanical load (pressurizer safety depressurization valve and safety valve water hammer load)

(4) Level C service limits

(a) Permissible pressure

$$P_M \leq 1.5P_a$$

$$P_a = \frac{2S_m t}{D_0 - 2yt}$$

P<sub>m</sub>: maximum pressure for Level C

y:0.4

(b) Primary stress evaluation (eq.9)

$$B_1 \frac{PD_0}{2t} + B_2 \frac{D_0}{2I} M_i \leq \min(2.25S_m, 1.8S_y)$$

P: Level C maximum pressure

M<sub>i</sub>: Moment ranges for following loads. Dead weight, mechanical load (pressurizer safety depressurization valve and safety valve water hammer load)

(5) Level D service limits

(a) Permissible pressure

$$P_M \leq 2.0P_a$$

$$P_a = \frac{2S_m t}{D_0 - 2yt}$$

P<sub>m</sub>: maximum pressure for Level D  
y:0.4

(b) Primary stress evaluation (eq.9)

$$B_1 \frac{PD_0}{2t} + B_2 \frac{D_0}{2I} M_i \leq \min(3S_m, 2S_y)$$

P: maximum pressure for Level D

M<sub>i</sub>: Moment ranges for following loads. Dead weight, mechanical load (pressurizer safety depressurization valve and safety valve water hammer load), SSE seismic inertia load, DBPB load

Note that the SSE seismic inertia load and DBPB load were combined using the SRSS method.

(c) Secondary stress evaluation

$$\frac{C_2 D_0 M_{AM}}{2I} \leq 6.0S_m$$

$$\frac{F_{AM}}{A_M} \leq S_m$$

M<sub>AM</sub>: Range of resultant moment due to SSEA

F<sub>AM</sub>: Amplitude of longitudinal force due to SSEA

A<sub>M</sub>: Piping cross-sectional area

9.4.2 1 inch or smaller piping (evaluated according to NC-3650)

(1) Design limit

(a) Primary stress evaluation (eq.8)

$$S_{SL} = B_1 \frac{PD_0}{2t_n} + B_2 \frac{M_A}{Z} \leq 1.5S_h$$

B<sub>1</sub>, B<sub>2</sub>: Stress indices

P: Design pressure

D<sub>0</sub>: Outside diameter

t<sub>n</sub>: Wall thickness

Z: Section modulus

M<sub>A</sub>: Dead weight (no sustained mechanical load other than dead weight)

(2) Level A/B service limits

(a) Primary stress evaluation (eq.9)

$$S_{OL} = B_1 \frac{P_{\max} D_0}{2t_n} + B_2 \left( \frac{M_A + M_B}{Z} \right) \leq \min(1.8S_h, 1.5S_y)$$

P<sub>max</sub>: Peak pressure

M<sub>B</sub>: mechanical load (this load was not applied in NPS 1 and less piping)

(b) secondary stress evaluation (eq.10)

$$S_E = \frac{iM_c}{Z} \leq S_A$$

$$S_A = f(1.25S_c + 0.25S_h)$$

i: Stress intensification factor

M<sub>c</sub>: Thermal expansion

(c) primary plus secondary stress evaluation (eq.11)

$$S_{TE} = \frac{PD_0}{4t_n} + 0.75i \left( \frac{M_A}{Z} \right) + i \left( \frac{M_c}{Z} \right) \leq (S_h + S_A)$$

Evaluation may use either (b) or (c).

(d) Building Settlement evaluation (eq.10a)

$$\frac{iM_D}{Z} \leq 3.0S_c$$

M<sub>D</sub>: Building Settlement load

(3) Level C service limit

(a) primary stress evaluation (eq.9)

$$S_{OL} = B_1 \frac{P_{\max} D_0}{2t_n} + B_2 \left( \frac{M_A + M_B}{Z} \right) \leq \min(2.25S_h, 1.8S_y)$$

$P_{max}$ : Peak pressure

$M_B$ : Mechanical load (this load was not applied in NPS 1 and less piping),

(4) Level D service limits

(a) Primary stress evaluation (eq.9)

$$S_{OL} = B_1 \frac{P_{max} D_0}{2t_n} + B_2 \left( \frac{M_A + M_B}{Z} \right) \leq \min(3S_h, 2S_y)$$

$P_{max}$ : Peak pressure

$M_B$ : Mechanical load (this load was not applied in NPS 1 and less piping), SSE seismic inertia load, DBPB load

Here, SSE seismic inertia load and DBPB load were combined by the SRSS method.

(b) Secondary stress evaluation

$$\frac{C_2 D_0 M_{AM}}{2I} \leq 6.0 S_h$$

$$\frac{F_{AM}}{A_M} \leq S_h$$

$M_{AM}$ : Range of resultant moment due to SSEA

$F_{AM}$ : Amplitude of longitudinal force due to SSEA

$A_M$ : Piping cross-sectional area

## 9.5 FATIGUE EVALUATION

The fatigue analysis was based on the rules of NB-3653 of ASME Section III. These rules require calculation of the total stress, including the peak stress, to determine the allowable number of stress cycles for the specified Service Loadings.

The design transients for ASME Level A and B service conditions (Table 8.1-3 to Table 8.1-14) were used in the evaluation of cyclic fatigue. The effect of 300 cycles of a 1/3 SSE seismic event was included in the evaluation of cyclic fatigue, treated as a Level B service condition. The number of cycles was based on equivalent fatigue usage for 20 cycles of a single SSE event.

**10.0 COMPUTER PROGRAMS USED**

The Table below provides a brief description of each of the computer programs used.

**Table 10.0-1 Computer Program Description**

| No. | Program Name | Version  | Description   |
|-----|--------------|----------|---|
| 1   | PIPESTRESS   | 3.6.0    | PIPESTRESS is a computer program for the analysis of piping systems. This program is used for the analysis of ASME Code, Section III, Class 1, 2, 3 and ASME B31.1 piping systems under various load conditions.  |
| 2   | ABAQUS       | 6.7.1    | ABAQUS is a general-purpose finite element computer program that performs a wide range of linear and nonlinear engineering simulations. This program is used for temperature distribution analysis and thermal stress analysis according to piping geometries and design transients such as fluid temperature and coefficient of heat transfer. |
| 3   | RELAP5-3D    | 2.4.2    | RELAP5-3D is a computer program for the fluid transient analysis. This program is used for the analysis of a behavior, such as water hammer, by modeling flow volume and flow path.   |
| 4   | P4TEDIA      | 1.3      | P4TEDIA is an in-house program to obtain temperature difference between in-side and out-side of pipe $\Delta T_1$ , $\Delta T_2$ and temperature difference at structural discontinuous point $T_a$ - $T_b$ . This program uses the thermal distribution analysis results generated by ABAQUS.  |
| 5   | PICEP        | 06/30/87 | PICEP is a program developed by the Electric Power Research Institute. This program is used for predicting leakage rate from assumed through-wall cracks in the leak-before-break evaluation of piping.   |

All these computer programs were verified and validated in compliance with the MHI quality assurance program. The computer programs were validated using one of the methods described below. Verification tests demonstrate the capability of the computer program to produce valid results for the test problems encompassing the range of permitted usage defined by the program documentation.

- Hand calculations
- Known solution for similar or standard problem
- Acceptable experimental test results
- Published analytical results
- Results from other similar verified programs



## **11.0 ANALYSIS RESULTS**

The calculated stress-to-allowable ratio (calculated stress divided by allowable value), the cumulative fatigue usage factor, and the thermal stress ratchet results for the most limiting locations are summarized in the Table 11.0-1 and 11.0-2. The ASME Code allowable limits were satisfied in all cases.

The detailed analysis models and results for each piping system are described in the Appendix 1.

LBB evaluation was applied for Pressurizer Surge Line , Accumulator Loop A and B Lines and it was confirmed that these lines satisfy the LBB criteria using BAC as described in Appendix 2.

Table 11.0-1 RCL Branch Piping Result Summary (greater than 1 inch piping)

| Condition     | Service Level | Category   | Loading   | Equation (NB-3650)                                  | Stress Limit                                   | Stress-to-Allowable Ratio |
|---------------|---------------|--|---|---|--|---------------------------|
| Design        | -             | Primary Stress                                   | $P, DL, L_{DM}$<br>(including $L_{DFN}$ )                             | Eq. 9<br>NB-3652                                    | $1.5 S_m$                                      |                           |
| Normal /Upset | A/B           | Primary + Secondary Stress Intensity Range (SIR) | $P_R, TH_{MTL}, TH_{DISCON}, L_{DFN}, L_{DFU}, SSEI, SSEA$            | Eq. 10<br>NB-3653.1                                 | $3 S_m$  |                           |
|               |               | Thermal Bending SIR                              | $TH_{MTL}$  | Eq. 12<br>NB-3653.6(a)                              | $3 S_m$  |                           |
|               |               | Primary + Secondary Membrane + Bending SIR       | $P_R, TH_{DISCON}, L_{DFN}, L_{DFU}, SSEI, SSEA$                      | Eq. 13<br>NB-3653.6(b)                              | $3 S_m$  |                           |
|               |               | Alternating Stress Intensity (Fatigue)           | $P_R, TH_{MTL}, TH_{DISCON}, TH_{GRAD}, L_{DFN}, L_{DFU}, SSEI, SSEA$ | NB-3653.3<br>NB-3653.4<br>NB-3653.5<br>NB-3653.6(c) | Allowable Value<br>1                           |                           |
|               |               | Thermal Stress Ratchet                           | $TH_{GRAD}$ (linear)  | NB3653.7  | Allowable Temperature                          |                           |
| Upset         | B             | Permissible Pressure                             | $P_M$   | NB-3654.1   | $1.1 P_a$                                      |                           |
|               |               | Primary Stress                                   | $P_M, DL, L_{DFU}$  | NB-3654.2   | $\text{Min}(1.8 S_m, 1.5 S_y)$                 |                           |
| Emergency     | C             | Permissible Pressure                             | $P_M$   | NB-3655.1   | $1.5 P_a$                                      |                           |
|               |               | Primary Stress                                   | $P_M, DL, L_{DFE}$  | NB-3655.2   | $\text{Min}(2.25 S_m, 1.8 S_y)$                |                           |
| Faulted       | D             | Permissible Pressure                             | $P_M$   | NB-3656(b)  | 2 Pa   |                           |
|               |               | Primary Stress                                   | $P_M, DL, L_{DFE}, SSEI, DBPB$  | NB-3656(a)<br>NB-3656(b)                            | Appendix-F<br>or<br>$\text{Min}(3 S_m, 2 S_y)$ |                           |
| Faulted       | D             | Secondary Stress                                 | $SSEA$  |   | $6 S_m$  |                           |

Note:

- Eq.10 was not satisfied and then NB-3653.6 and 7 were evaluated.

Table 11.0-2 RCL Branch Piping Result Summary (1 inch or smaller piping)

| Condition     | Service Level | Loading                        | Equation (NC-3650)      | Stress Limit                    | Stress-to-Allowable Ratio |
|---------------|---------------|--------------------------------|-------------------------|---------------------------------|---------------------------|
| Design        | -             | $P, DL$                        | Eq. 8<br>NC-3652        | $1.5 S_h$                       |                           |
| Normal /Upset | A/B           | $P_M, DL, L_{DFN}, L_{DFU}$    | Eq. 9<br>NC-3653.1      | $\text{Min}(1.8 S_h, 1.5 S_y)$  |                           |
|               |               | $TH_{MTL}$                     | Eq. 10<br>NC-3653.2(a)  | $S_A$                           |                           |
|               |               | $BS$                           | Eq. 10a<br>NC-3653.2(b) | $3S_c$                          |                           |
|               |               | $P_M, DL, TH_{MTL}$            | Eq. 11<br>NC-3652.2(c)  | $S_h + S_A$                     |                           |
| Emergency     | C             | $P_M, DL, L_{DFE}$             | Eq. 9<br>NC-3654        | $\text{Min}(2.25 S_h, 1.8 S_y)$ |                           |
| Faulted       | D             | $P_M, DL, L_{DFE}, SSEI, DBPB$ | Eq. 9<br>NC-3655        | $\text{Min}(3 S_h, 2 S_y)$      |                           |
|               |               | $SSEA$                         |                         | $6S_h$                          |                           |

## **12.0 REFERENCES**

1. N0-GB00002 Revision 1 "Class 1 Piping ASME Design Specification (excluding Reactor Coolant Loop Piping)"
2. N0-CF00004 Revision 0 "Piping Design Criteria"
3. N0-GB00005 Revision 1 "Input Package of Stress Analysis of RCL Branch Piping and Main Steam Piping"
4. N0-EE12001 Revision 2 "Class 1 Equipment Design Transients"
5. ASME Boiler and Pressure Vessel Code, Section II, Division 1, 2001 Edition through 2003 Addenda
6. IEEE Recommended Practice for Seismic Qualification of Class 1E Equipment for Nuclear Power Generating Stations, IEEE Std 344-2004, Appendix D, Institute of Electrical and Electronic Engineers Power Engineering Society, New York, New York, June 2005.
7. ASME Boiler and Pressure Vessel Code, Section III, Division 1, 1992 Edition through 1992 Addenda
8. INEL, "RELAP5-3D Code manual", 2001
9. DST Computer Services. S.A., "PIPESTRESS User's Manual", Version 3.6.0, 2007
10. SIMURIA, "ABAQUS Analysis User's Manual", Version 6.7, 2007
11. "Additional Information for Design Completion Plan of US-APWR Piping and Components" UAP-HF-08123, July, 2008.

## Appendix 1-1

### RC01 Pressurizer Surge Line Piping Analysis Results

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|  |                 |
|--|-----------------|
| 1. INPUT   |                 |
| 1.1 Used for creating the pipe structural model                            |                 |
| 1.1.1 Block division and piping specifications                             | Table A1-1-1-1  |
| 1.1.2 Piping isometrics  | Figure A1-1-1-1 |
| 1.1.3 Concentrated mass  | Table A1-1-1-2  |
| 1.1.4 Support point rigidity   | Table A1-1-1-3  |
| 1.1.5 Valve rigidity   | Table A1-1-1-4  |
| 1.2 Used for creating load conditions                                      |                 |
| 1.2.1 Level A/B design transient   | see main text   |
| 1.2.2 Level A/B thermal displacement input data                            | Table A1-1-1-5  |
| 1.2.3 Level A, B temperature and pressure input data                       | Table A1-1-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data               | Table A1-1-1-7  |
| 1.2.5 Floor response curve   | Figure A1-1-1-2 |
| 1.2.6 Seismic anchor displacement input data                               | Table A1-1-1-8  |
| 1.2.7 DBPB displacement input data   | Table A1-1-1-9  |
| 2. OUTPUT  |                 |
| 2.1 PIPESTRESS analysis model diagram                                      | Figure A1-1-2-1 |
| 2.2 Natural frequency analysis results                                     | Table A1-1-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                           | Figure A1-1-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a - T_b$ ) | Table A1-1-2-2  |
| 2.5 Piping stress and fatigue evaluation results                           | Table A1-1-2-3  |
| 2.6 LBB evaluation results   | Figure A1-1-2-3 |

Table A1-1-1-1 Block division and piping specifications



Figure A1-1-1-1 Piping isometrics



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Table A1-1-1-2 Concentrated mass

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Table A1-1-1-3 Support point rigidity

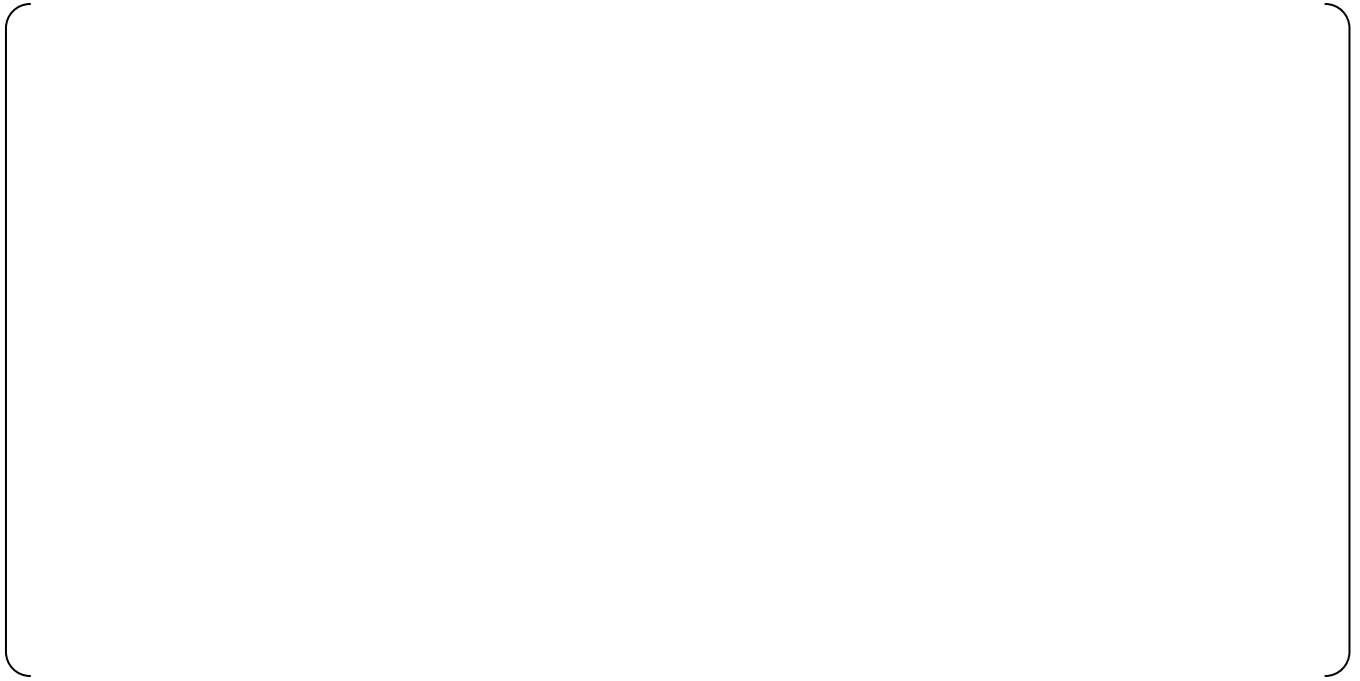


Table A1-1-1-4 Valve rigidity

Table A1-1-1-5 Level A/B thermal displacement input data (1/2)  
(Point: 9010)

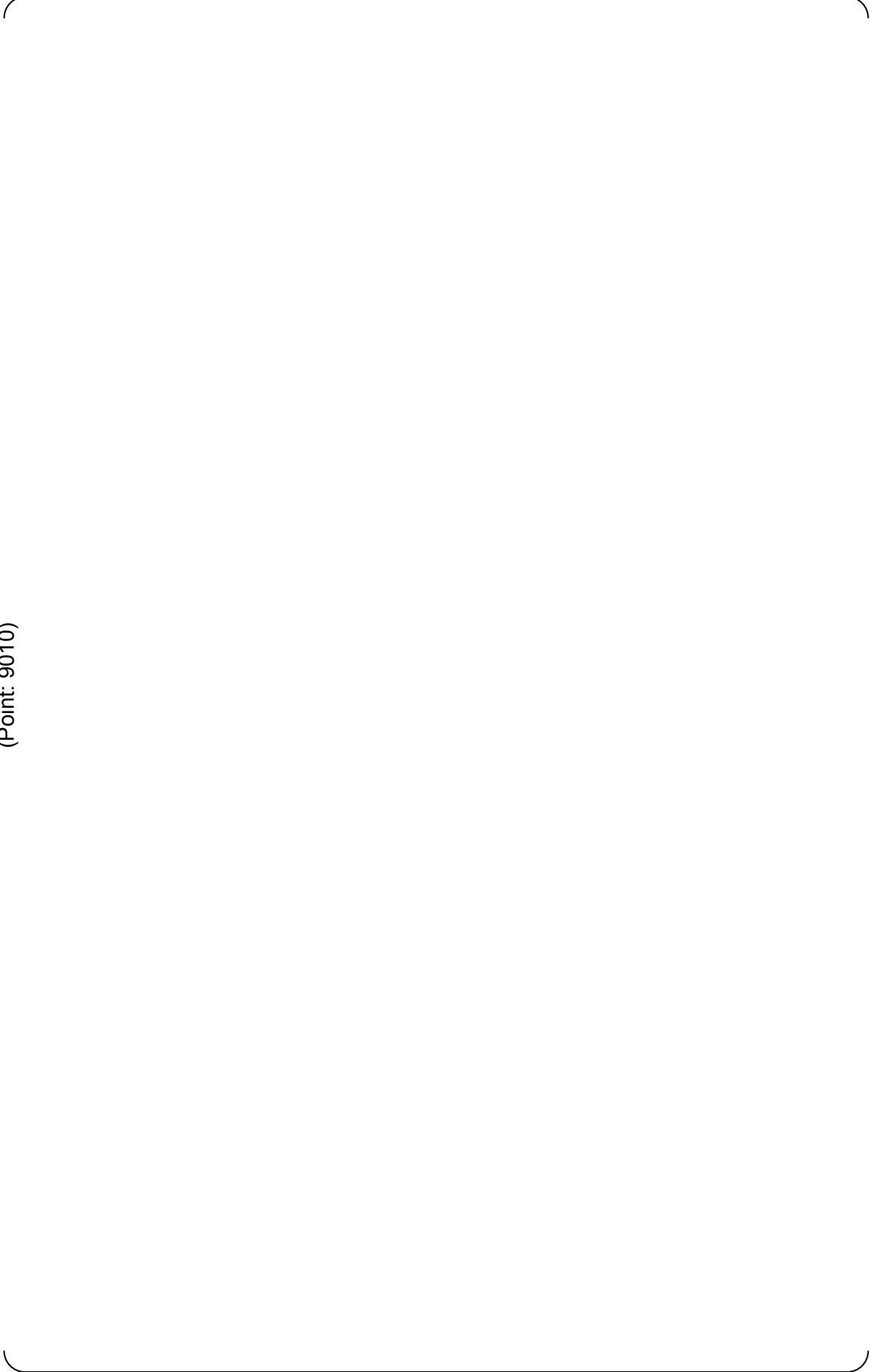






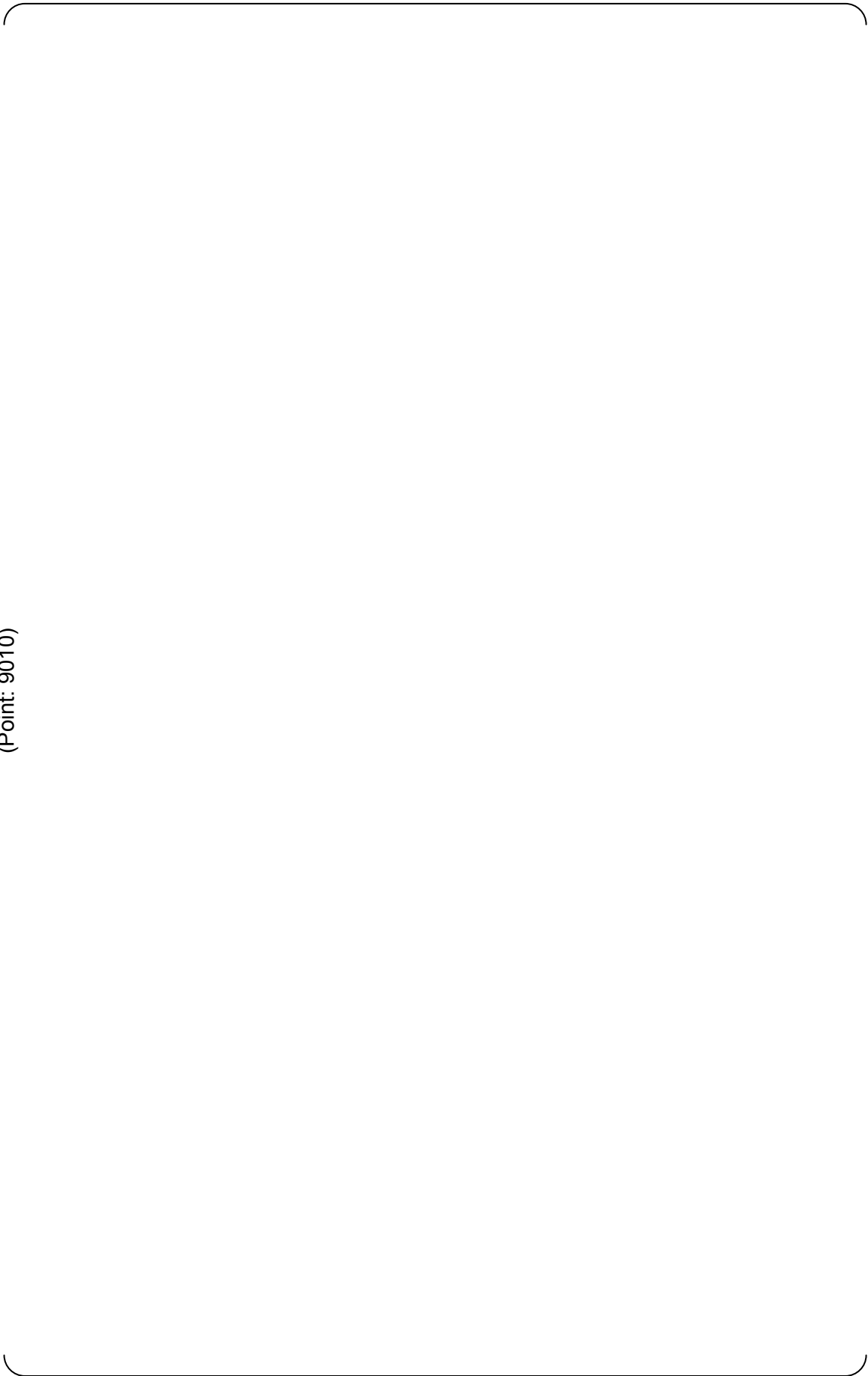








Table A1-1-1-5 Level A/B thermal displacement input data (2/2)  
(Point: 9010)

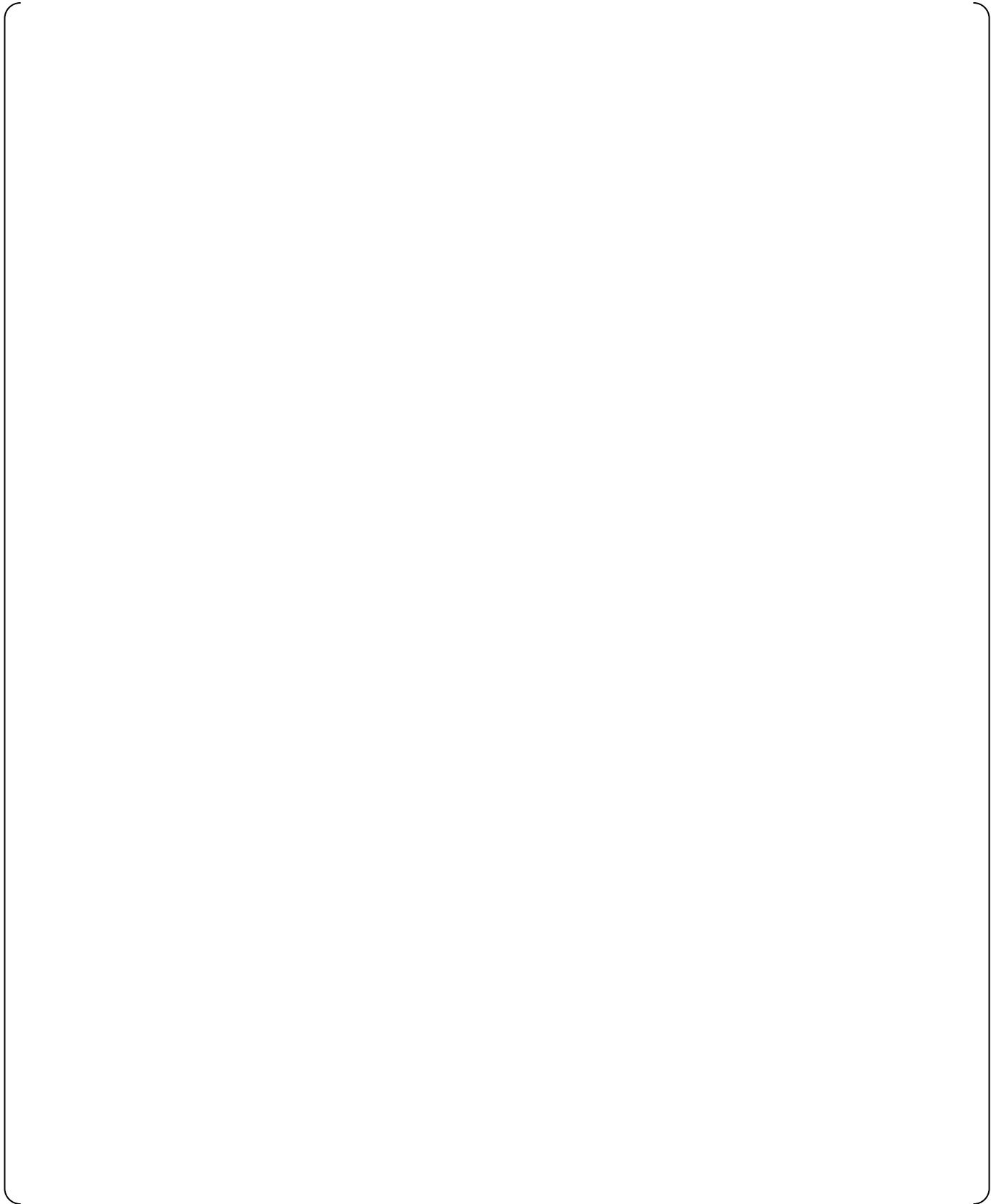


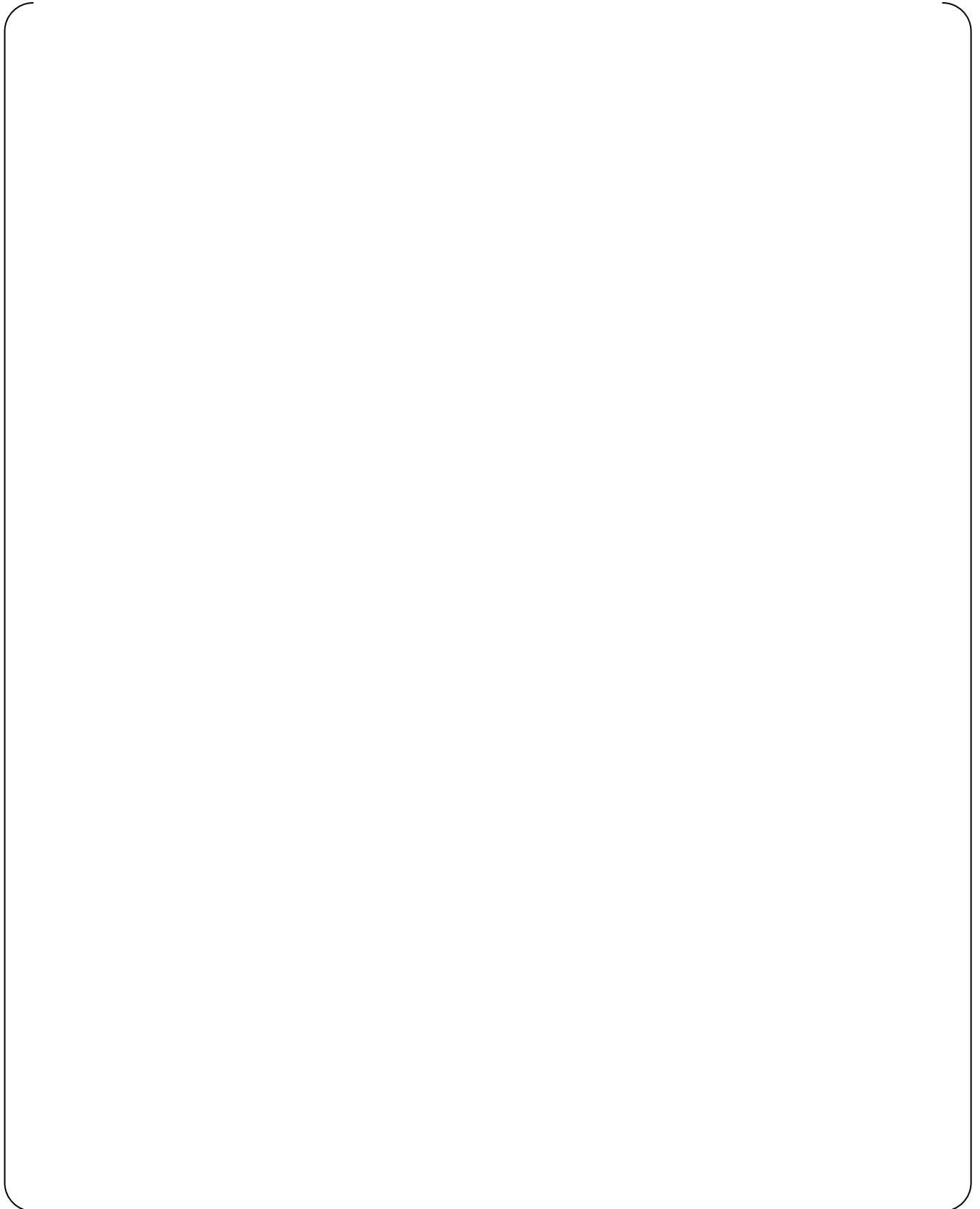




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Table A1-1-1-6 Level A, B temperature and pressure input data (1/3)  
(Section I)







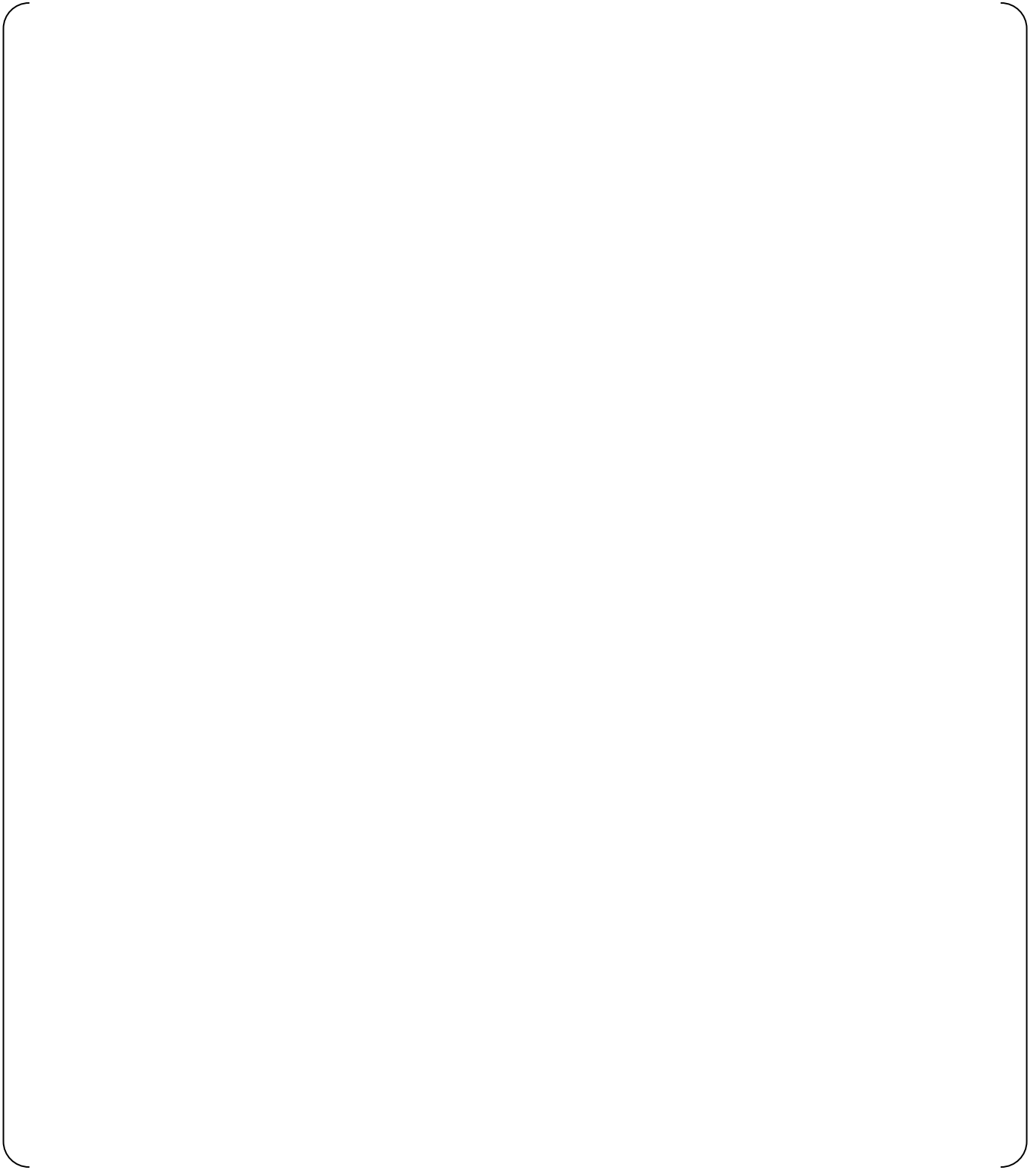


Table A1-1-1-6 Level A, B temperature and pressure input data (2/3)  
(Section I)



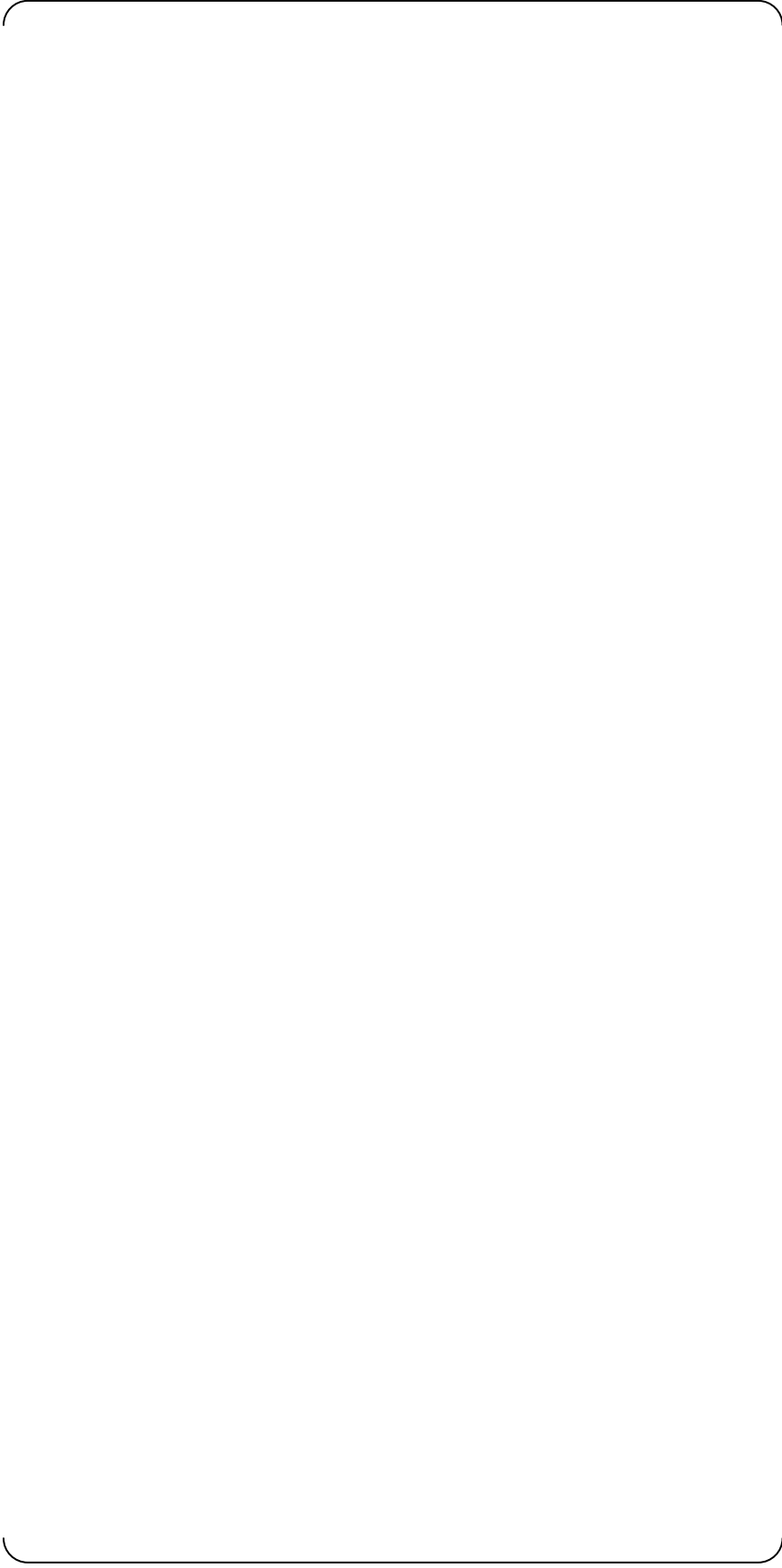
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Table A1-1-1-6 Level A, B temperature and pressure input data (3/3)  
(Section I)

Table A1-1-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-1-1-2 Floor response curve (1/12)**  
Pressurizer Surge Line (RC01) FRS for Piping  
X(EW) direction (damping 3.0%)



**Figure A1-1-1-2 Floor response curve (2/12)**  
Pressurizer Surge Line (RC01) FRS for Piping  
Y (NS) direction (damping 3.0%)



**Figure A1-1-1-2 Floor response curve (3/12)**  
Pressurizer Surge Line (RC01) FRS for Piping  
Z (Vert.) direction (damping 3.0%)

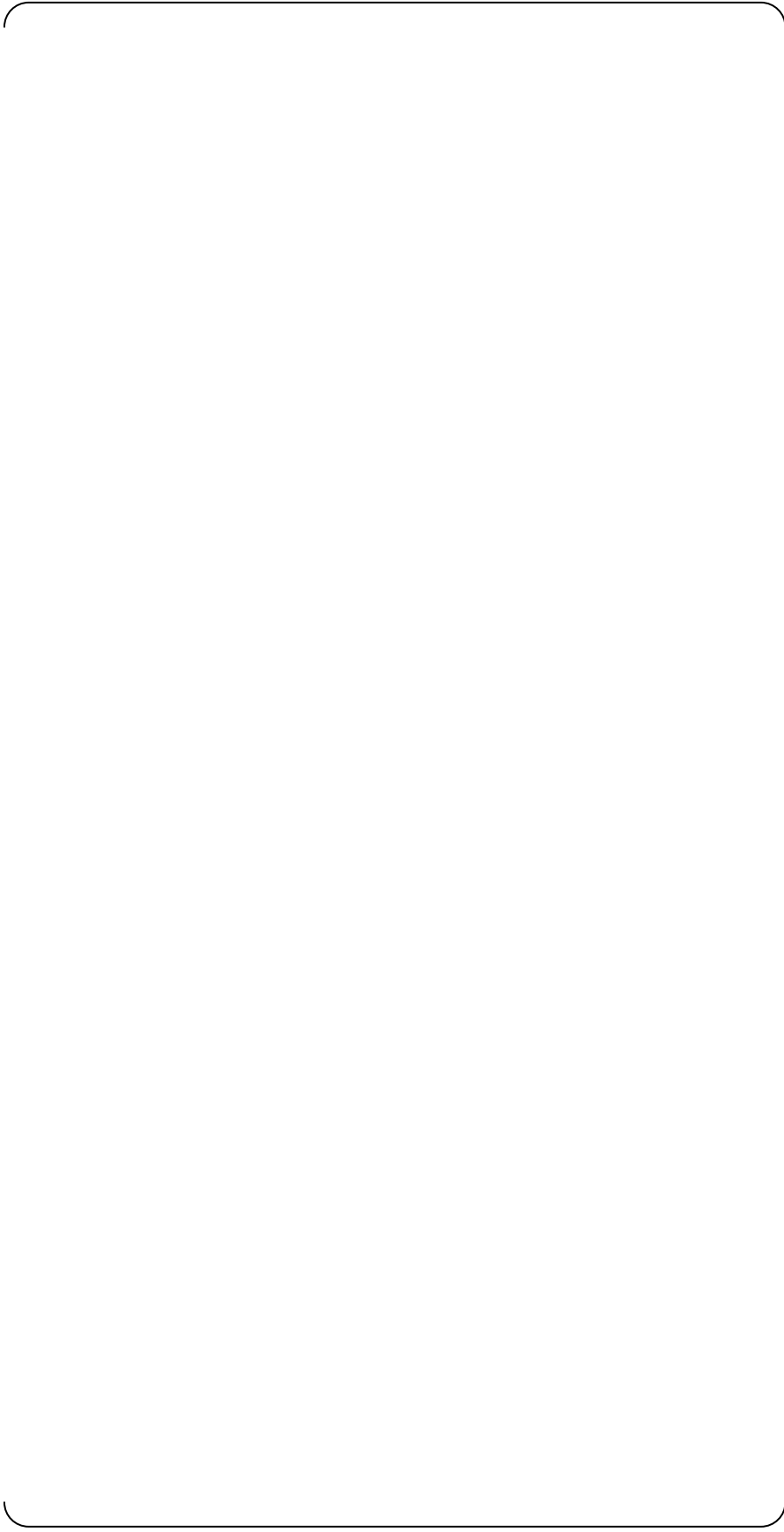




**Figure A1-1-1-2 Floor response curve (4/12)**  
Pressurizer Surge Line (RC01) FRS for MCP Nozzle  
X (EW) direction (damping 3.0%)



**Figure A1-1-1-2 Floor response curve (5/12)**  
Pressurizer Surge Line (RC01) FRS for MCP Nozzle  
Y (NS) direction (damping 3.0%)



**Figure A1-1-1-2 Floor response curve (6/12)**  
Pressurizer Surge Line (RC01) FRS for MCP Nozzle  
Z (Vert.) direction (damping 3.0%)



**Figure A1-1-1-2 Floor response curve (7/12)**  
Pressurizer Surge Line (RC01) FRS for Pressurizer Base Plate  
X (EW) direction (damping 3.0%)



**Figure A1-1-1-2 Floor response curve (8/12)**  
Pressurizer Surge Line (RC01) FRS for Pressurizer Base Plate  
Y (NS) direction (damping 3.0%)



**Figure A1-1-1-2 Floor response curve (9/12)**  
Pressurizer Surge Line (RC01) FRS for Pressurizer Base Plate  
Z (Vert.) direction (damping 3.0%)



**Figure A1-1-1-2 Floor response curve (10/12)**  
Pressurizer Surge Line (RC01) FRS for Pressurizer Support  
X (EW) direction (damping 3.0%)



**Figure A1-1-1-2 Floor response curve (11/12)**  
Pressurizer Surge Line (RC01) FRS for Pressurizer Support  
Y (NS) direction (damping 3.0%)





**Figure A1-1-1-2 Floor response curve (12/12)**  
Pressurizer Surge Line (RC01) FRS for Pressurizer Support  
Z (Vert.) direction (damping 3.0%)

Table A1-1-1-8 Seismic anchor displacement input data

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Table A1-1-1-9 DBPB displacement input data

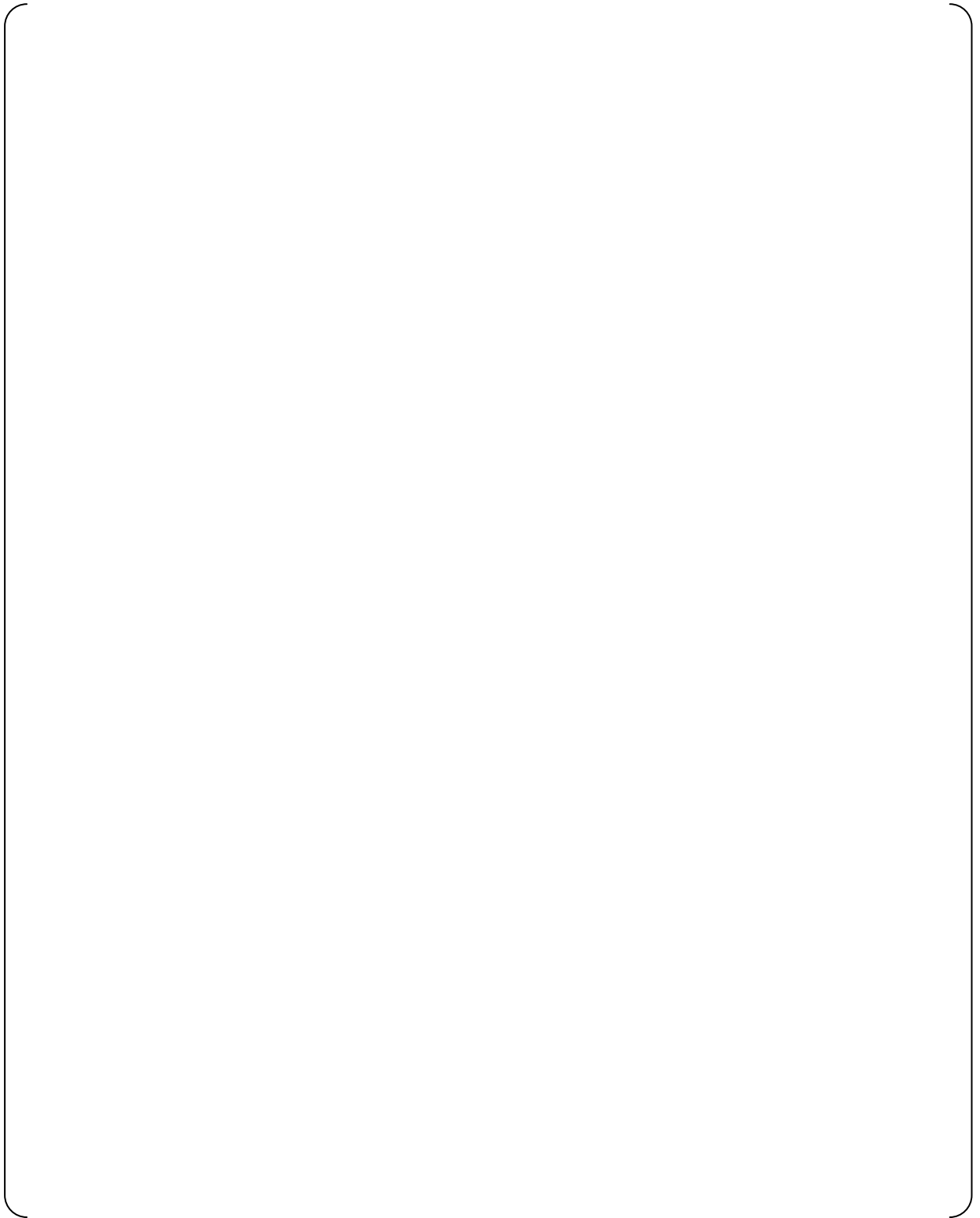
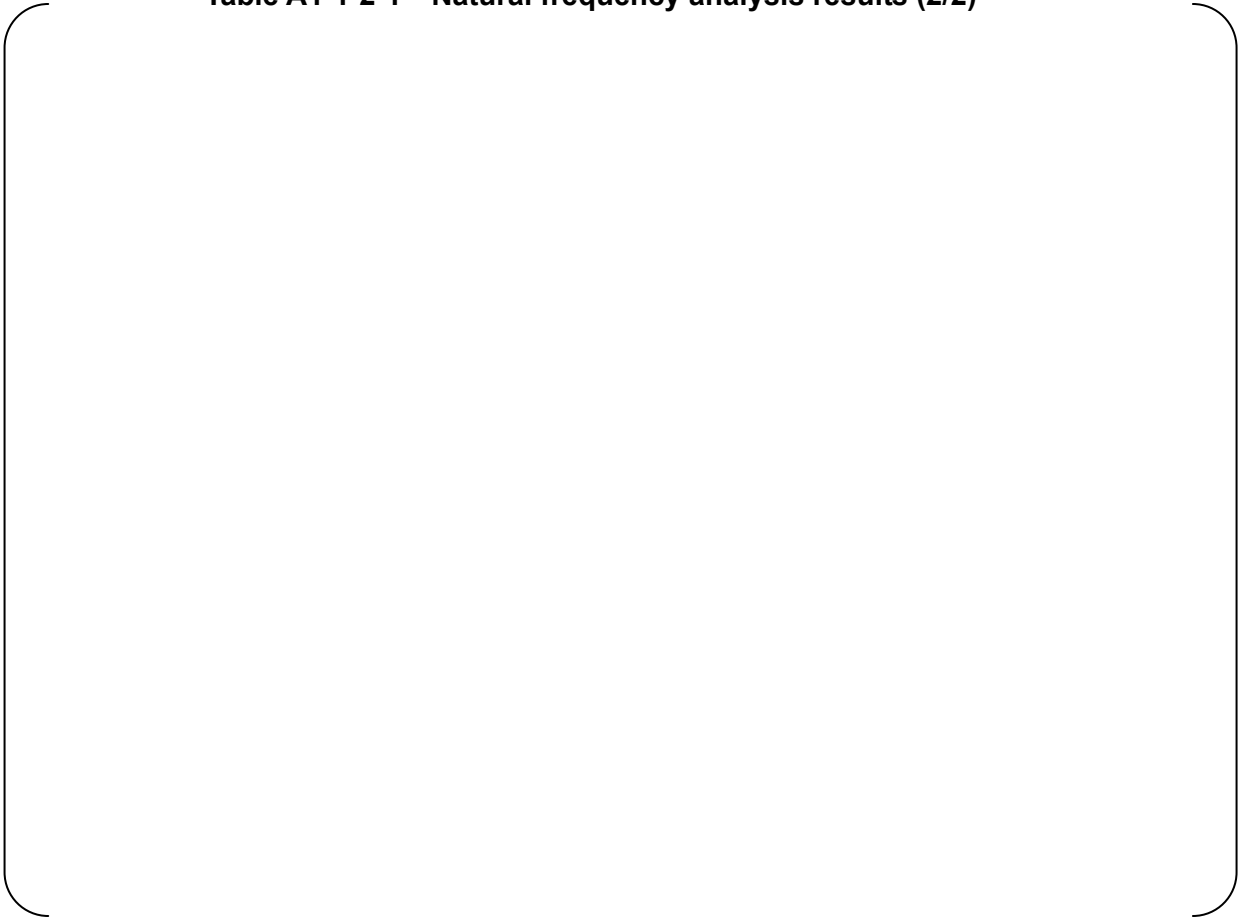


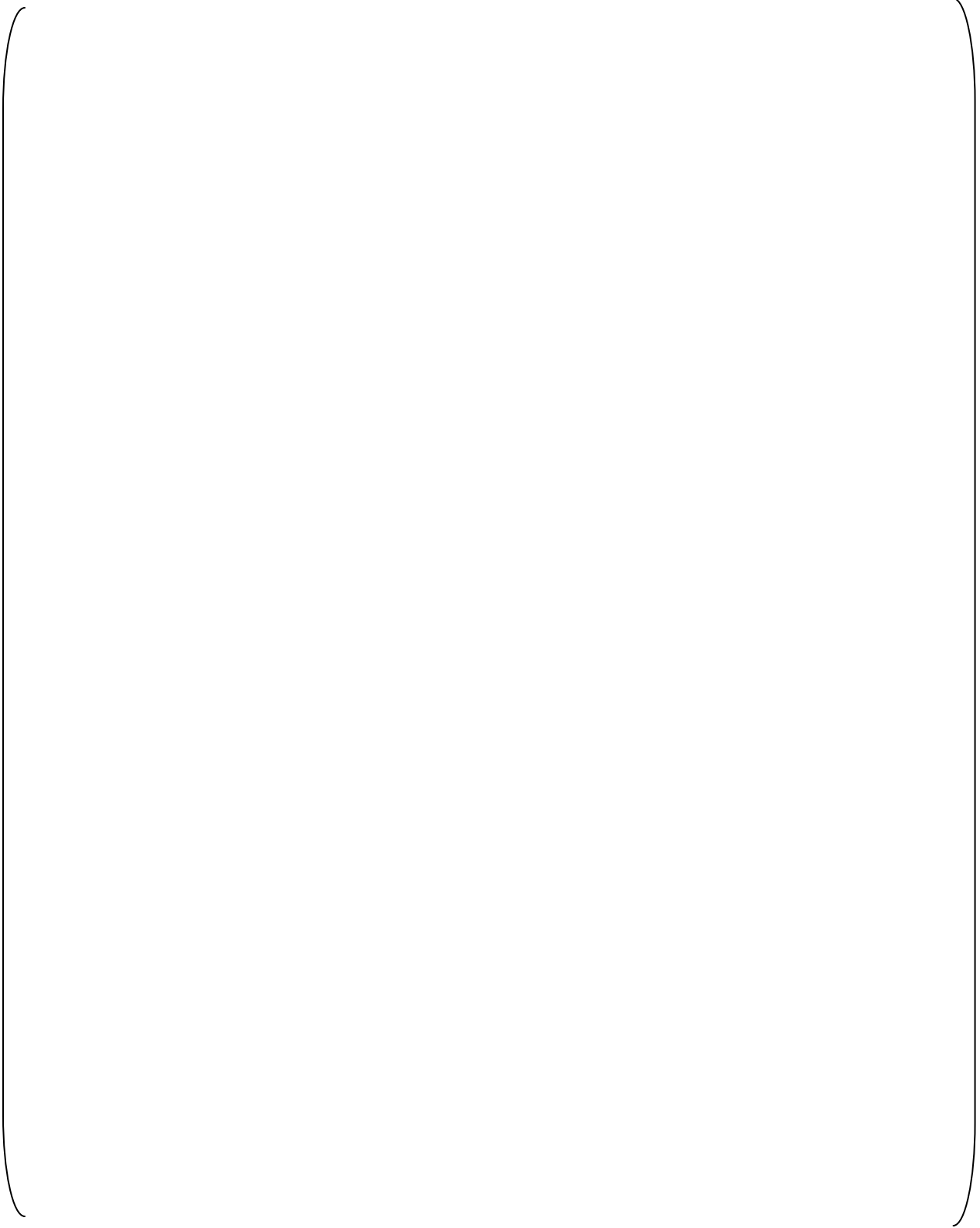
Figure A1-1-2-1 PIPESTRESS analysis model diagram

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Table A1-1-2-1 Natural frequency analysis results (1/2)

Table A1-1-2-1 Natural frequency analysis results (2/2)





**Figure A1-1-2-2 Frequency mode diagram (primary)**

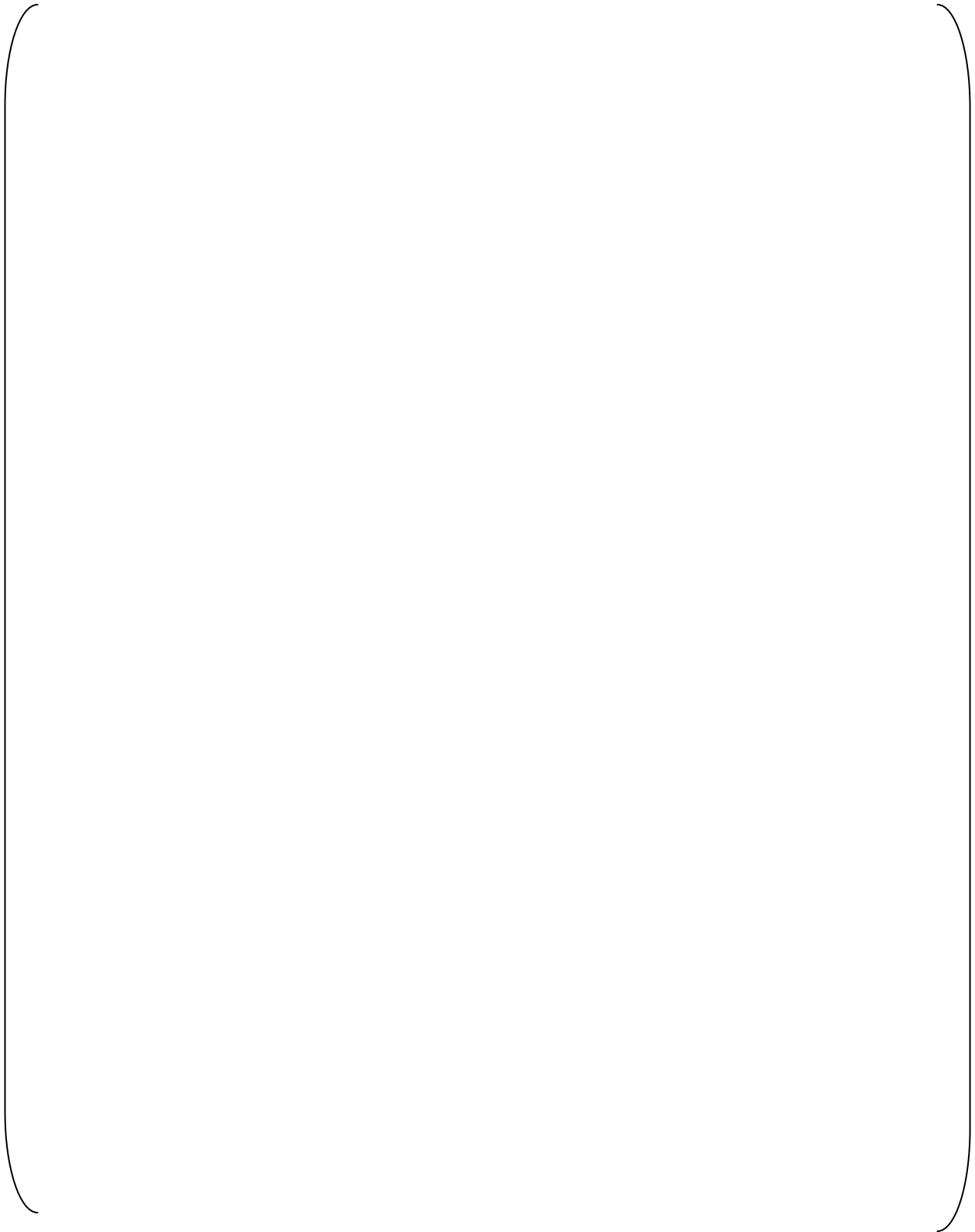


Figure A1-1-2-2 Frequency mode diagram (secondary)



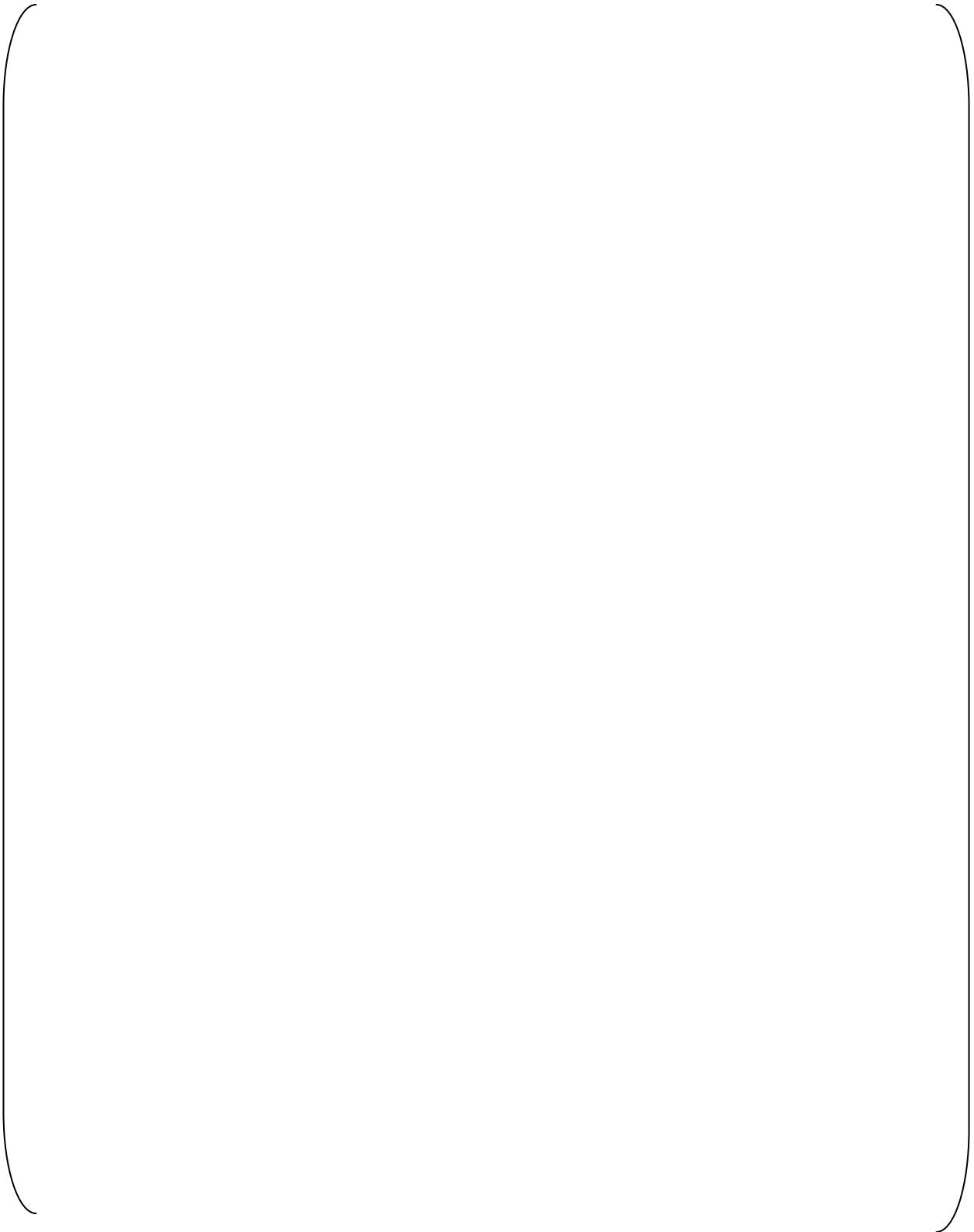
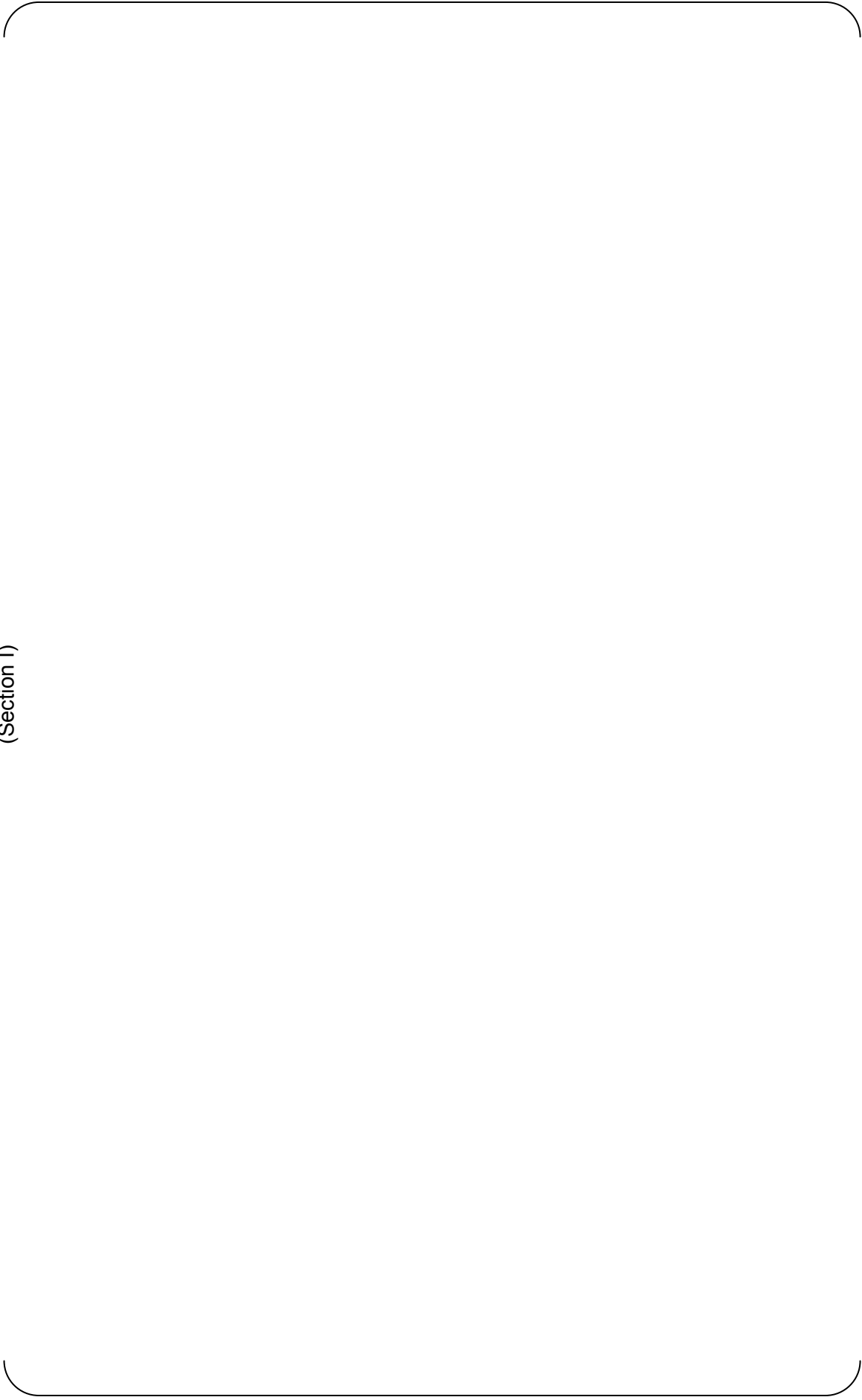


Figure A1-1-2-2 Frequency mode diagram (tertiary)

Table A1-1-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (1/3)  
(Section I)







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





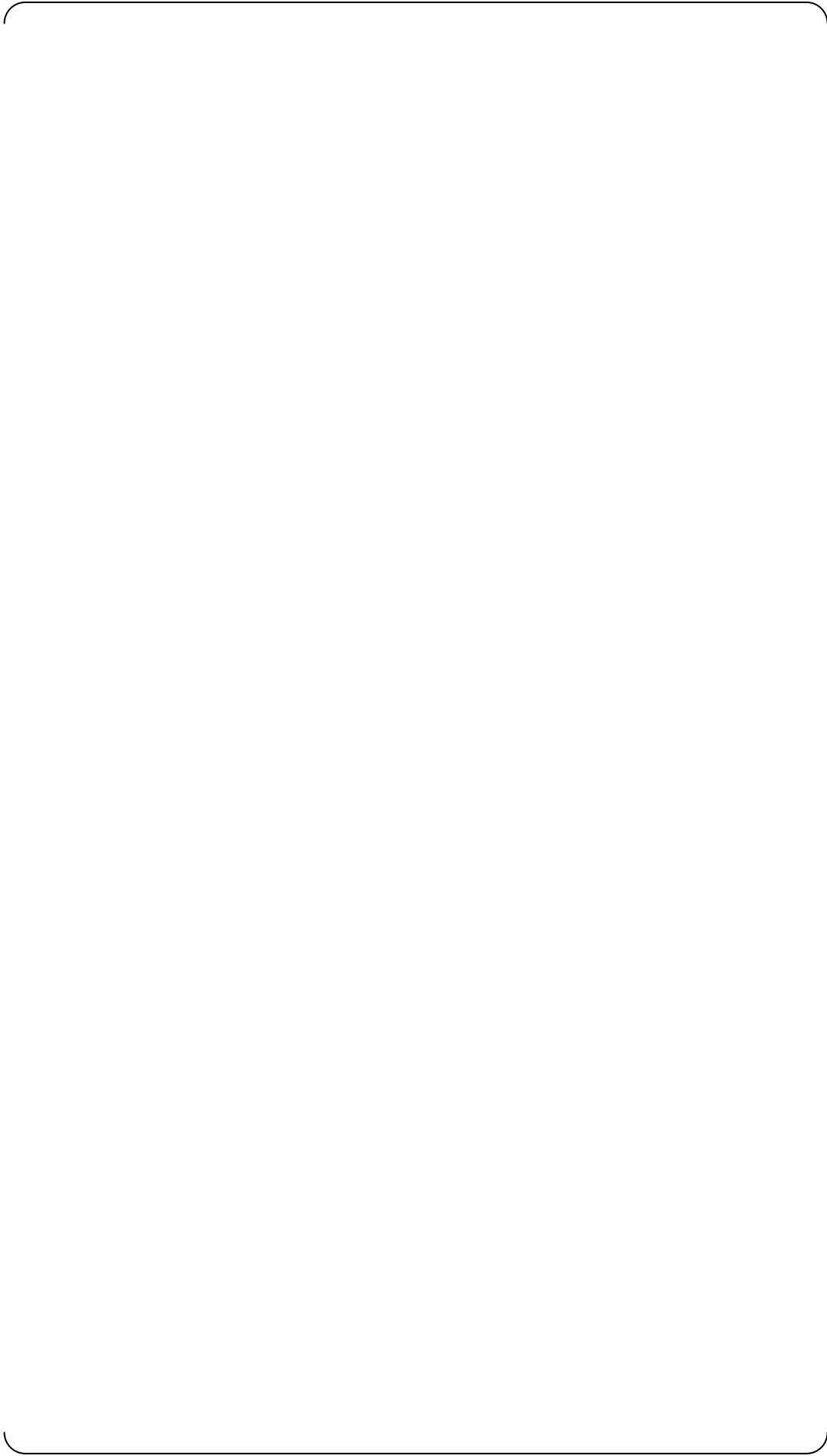
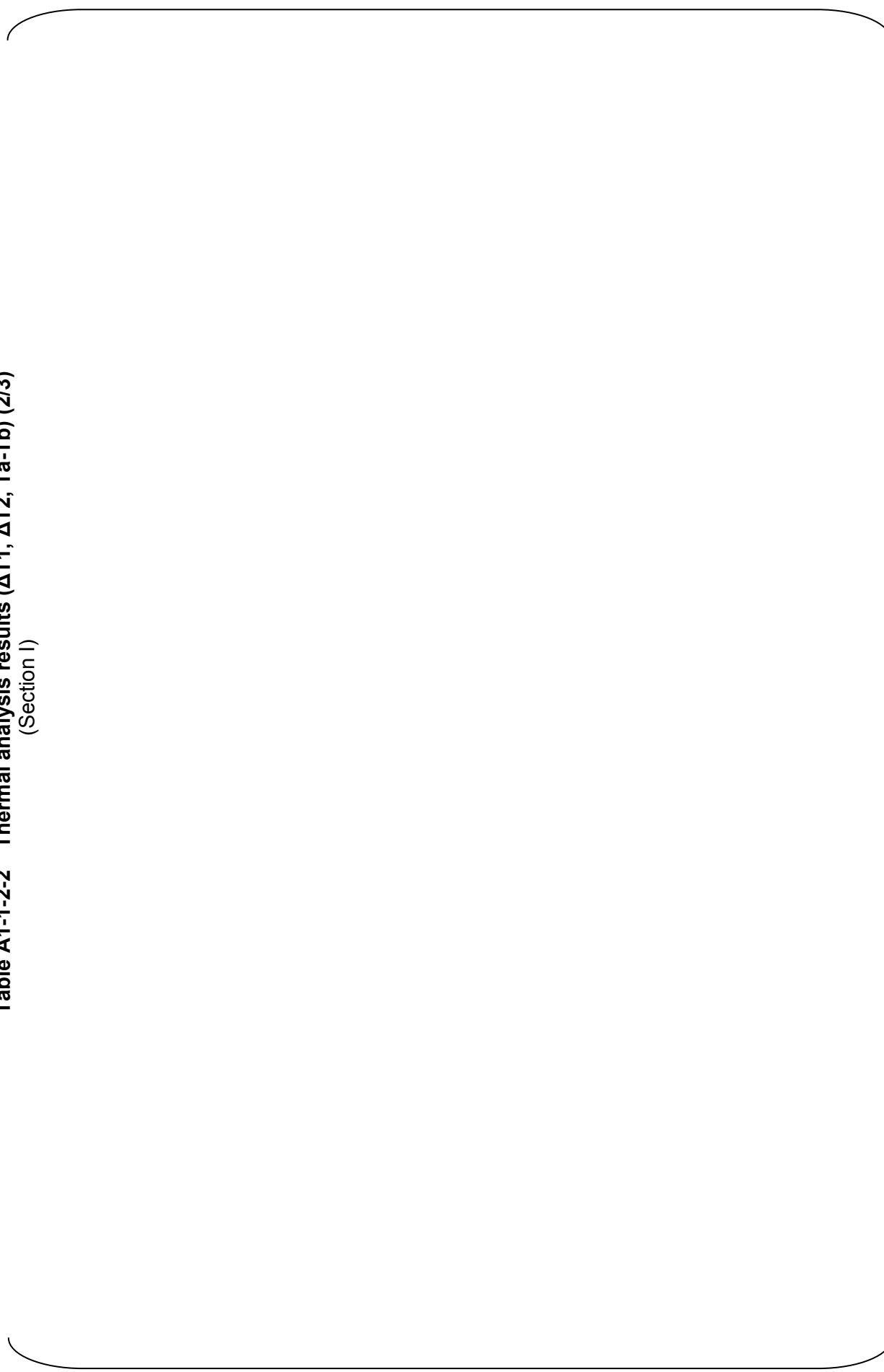


Table A1-1-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (2/3)  
(Section I)





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**

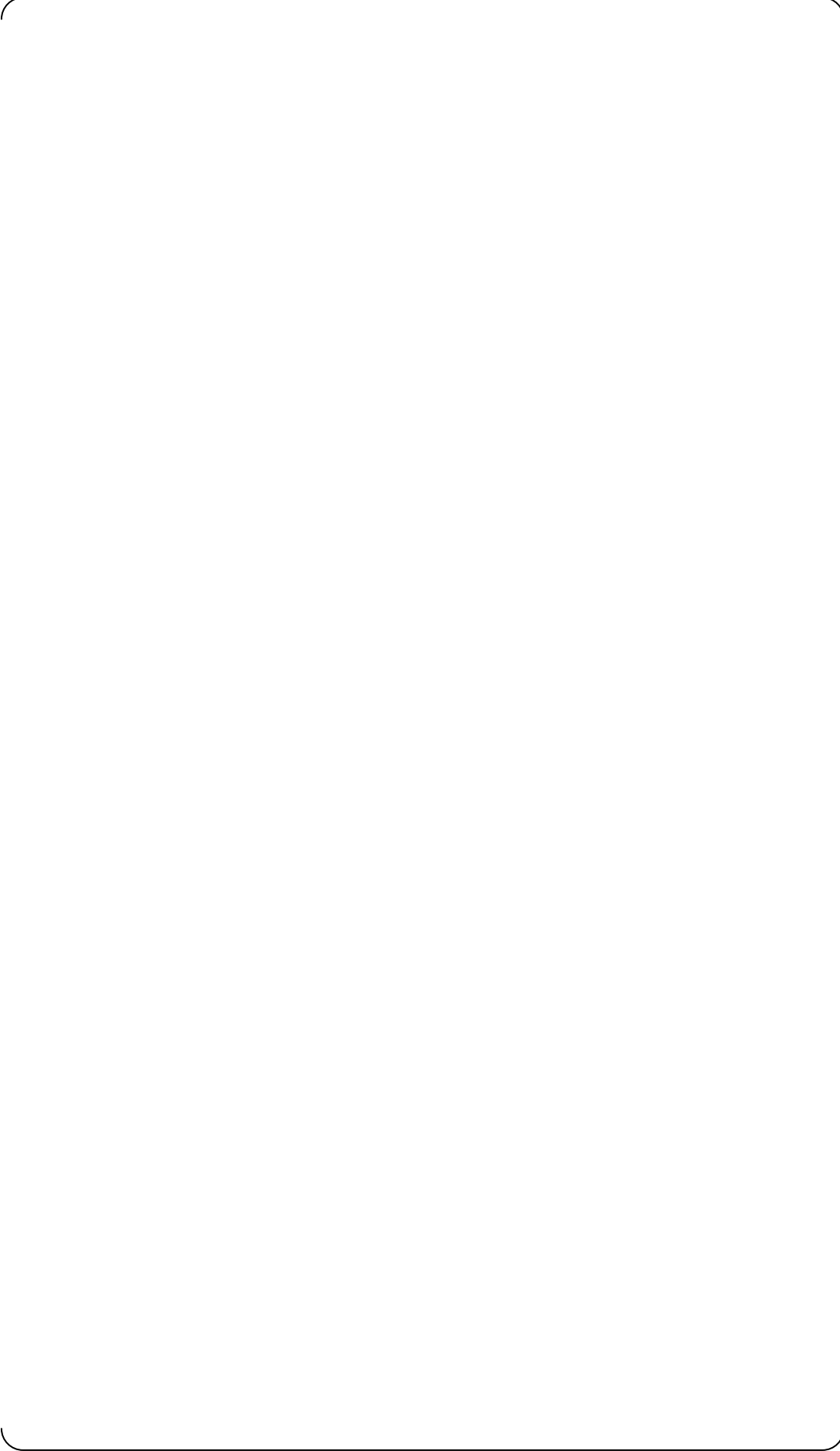
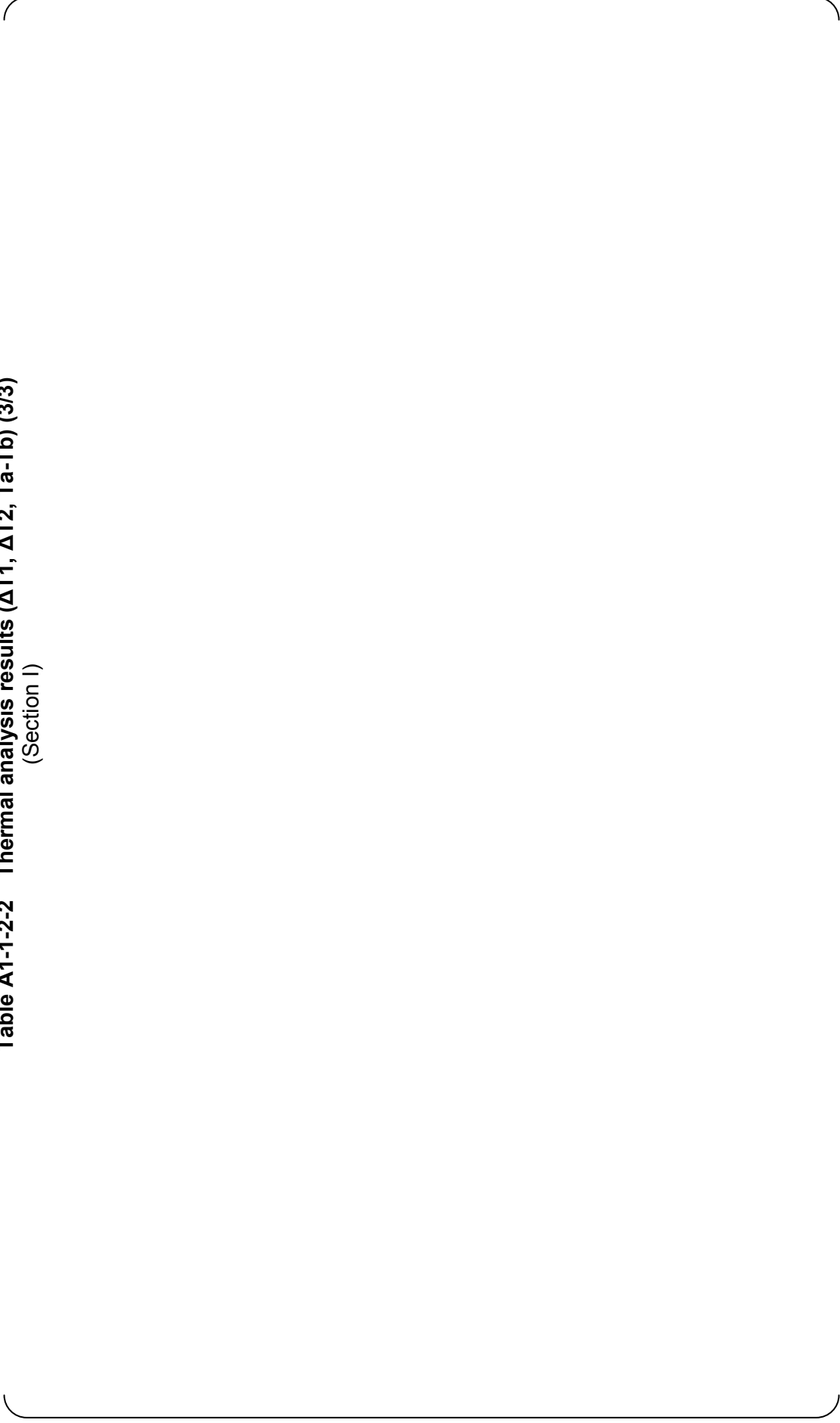




Table A1-1-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (3/3)  
(Section I)



**Table A1-1-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-1-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)



Figure A1-1-2-3 LBB evaluation results

## Appendix 1-2

### RC02 Pressurizer Spray Line Piping Analysis Results

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|  |                 |
|--|-----------------|
| 1. INPUT   |                 |
| 1.1 Used for creating the pipe structural model                            |                 |
| 1.1.1 Block division and piping specifications                             | Table A1-2-1-1  |
| 1.1.2 Piping isometrics  | Figure A1-2-1-1 |
| 1.1.3 Concentrated mass  | Table A1-2-1-2  |
| 1.1.4 Support point rigidity   | Table A1-2-1-3  |
| 1.1.5 Valve rigidity   | Table A1-2-1-4  |
| 1.2 Used for creating load conditions                                      |                 |
| 1.2.1 Level A/B design transient   | see main text   |
| 1.2.2 Level A/B thermal displacement input data                            | Table A1-2-1-5  |
| 1.2.3 Level A, B temperature and pressure input data                       | Table A1-2-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data               | Table A1-2-1-7  |
| 1.2.5 Floor response curve   | Figure A1-2-1-2 |
| 1.2.6 Seismic relative displacement input data                             | Table A1-2-1-8  |
| 1.2.7 DBPB displacement input data   | Table A1-2-1-9  |
| 2. OUTPUT  |                 |
| 2.1 PIPESTRESS analysis Model diagram                                      | Figure A1-2-2-1 |
| 2.2 Natural frequency analysis results                                     | Table A1-2-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                           | Figure A1-2-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a - T_b$ ) | Table A1-2-2-2  |
| 2.5 Piping stress and fatigue evaluation results                           | Table A1-2-2-3  |



Table A1-2-1-1 Block division and piping specifications (1/2)

Table A1-2-1-1 Block division and piping specifications (2/2)

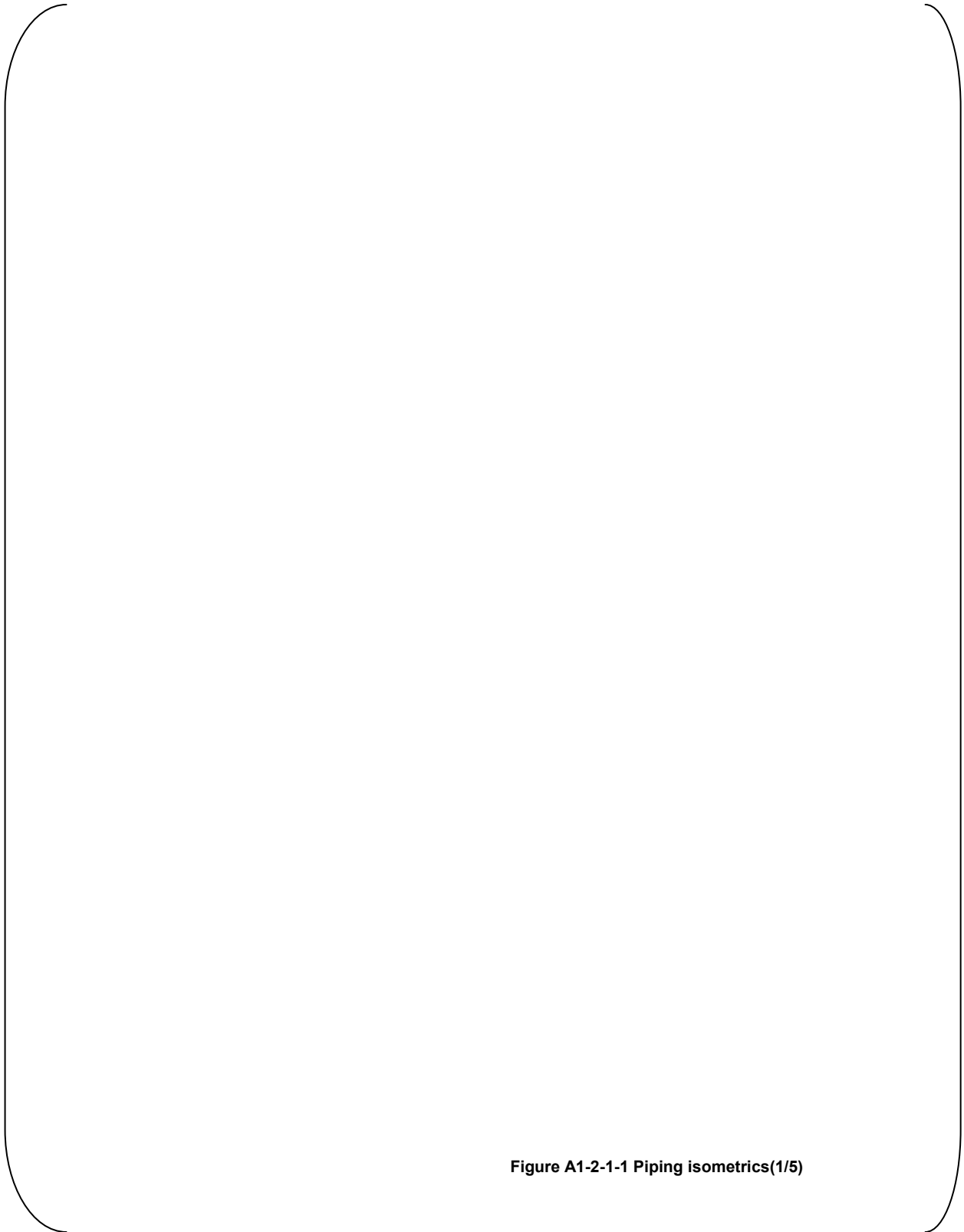


Figure A1-2-1-1 Piping isometrics(1/5)

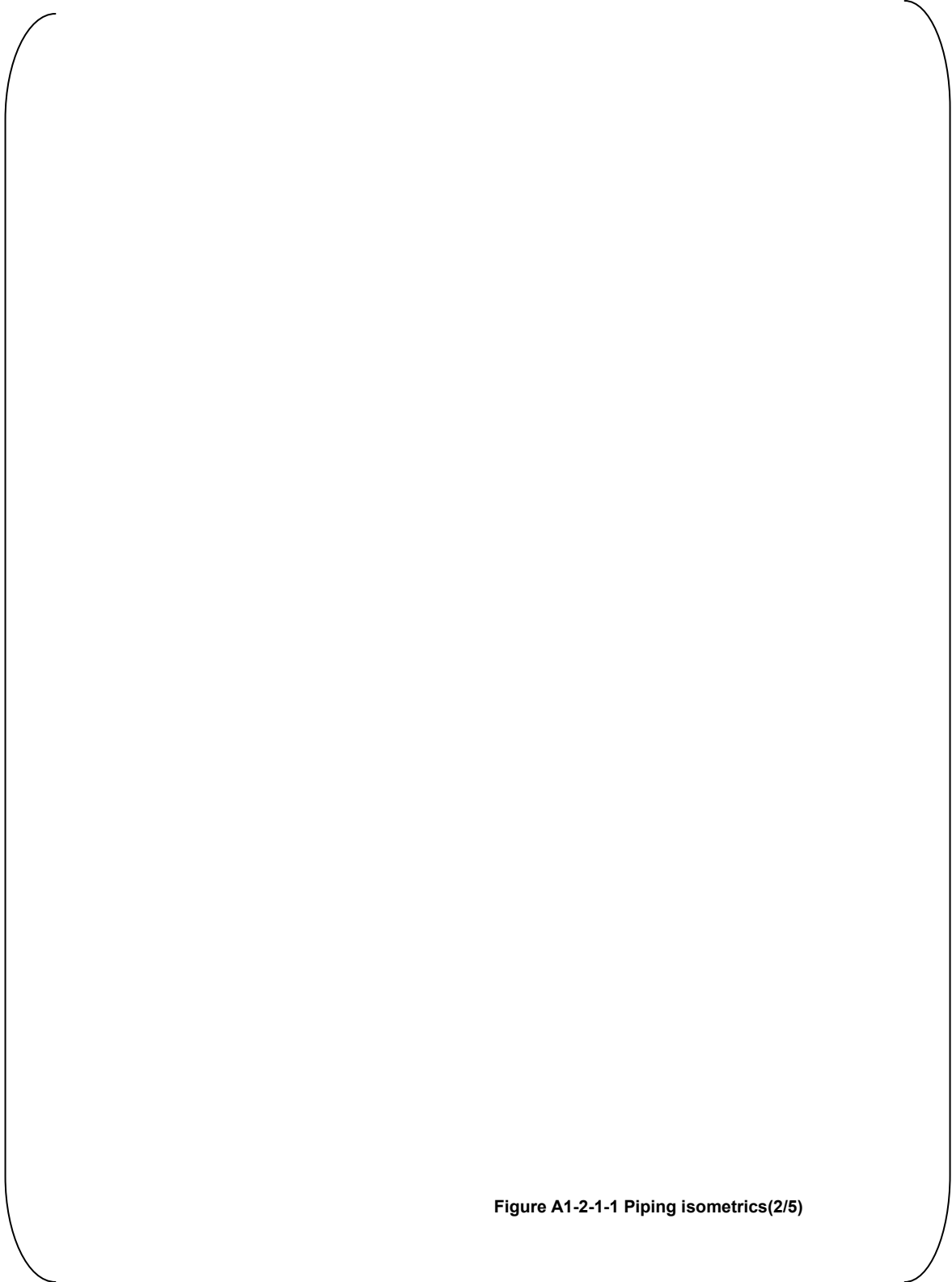


Figure A1-2-1-1 Piping isometrics(2/5)

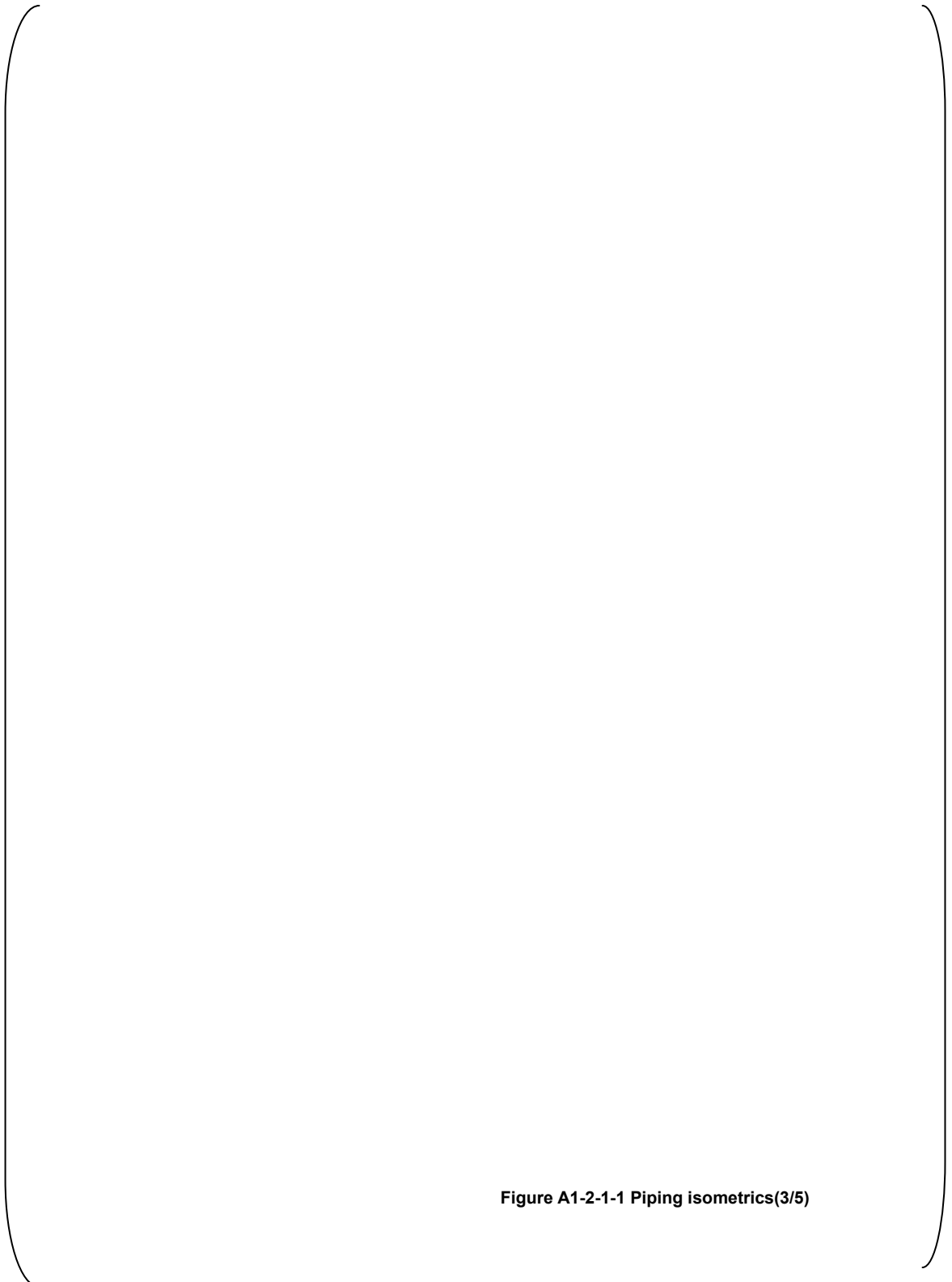


Figure A1-2-1-1 Piping isometrics(3/5)

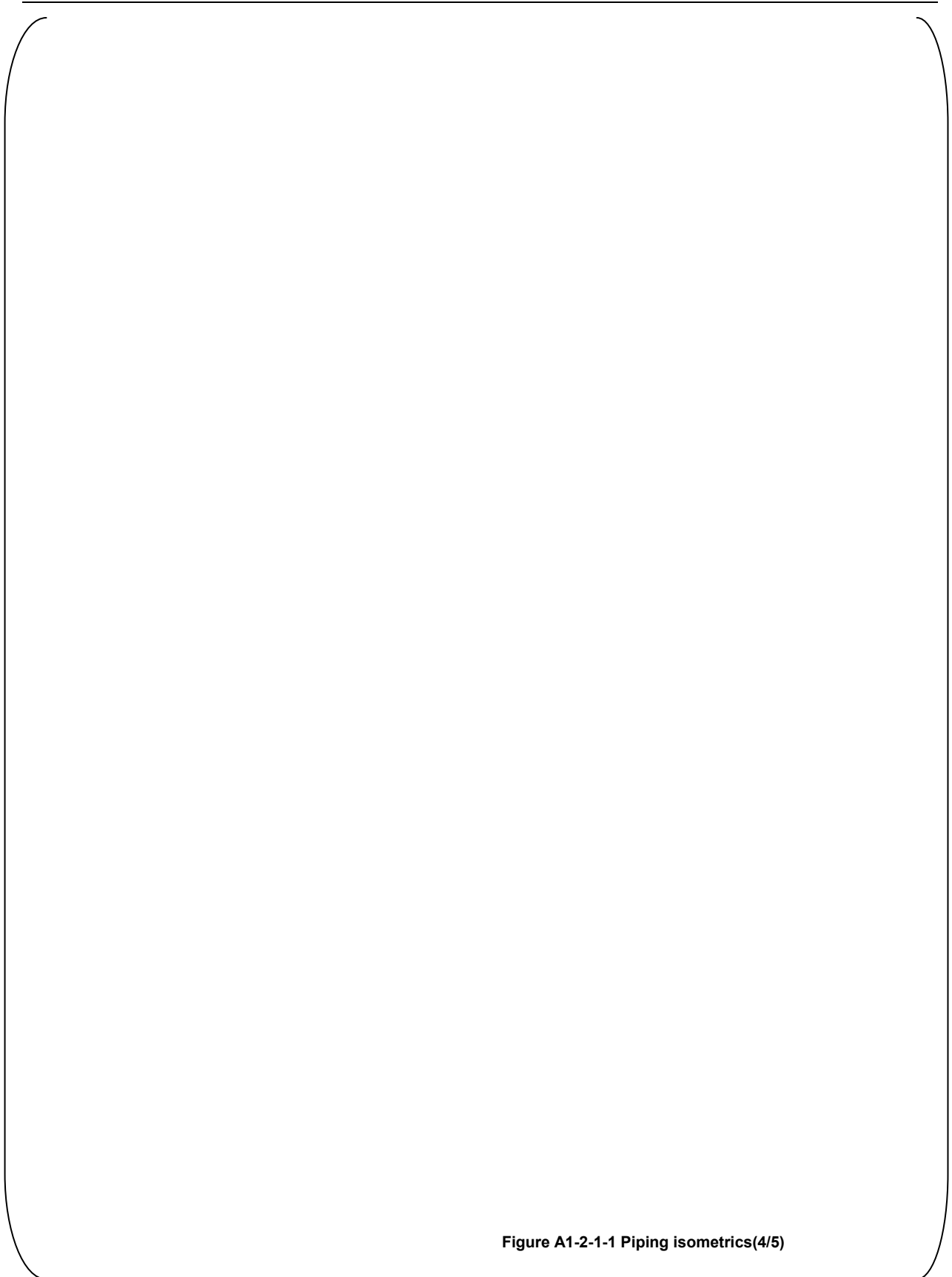


Figure A1-2-1-1 Piping isometrics(4/5)

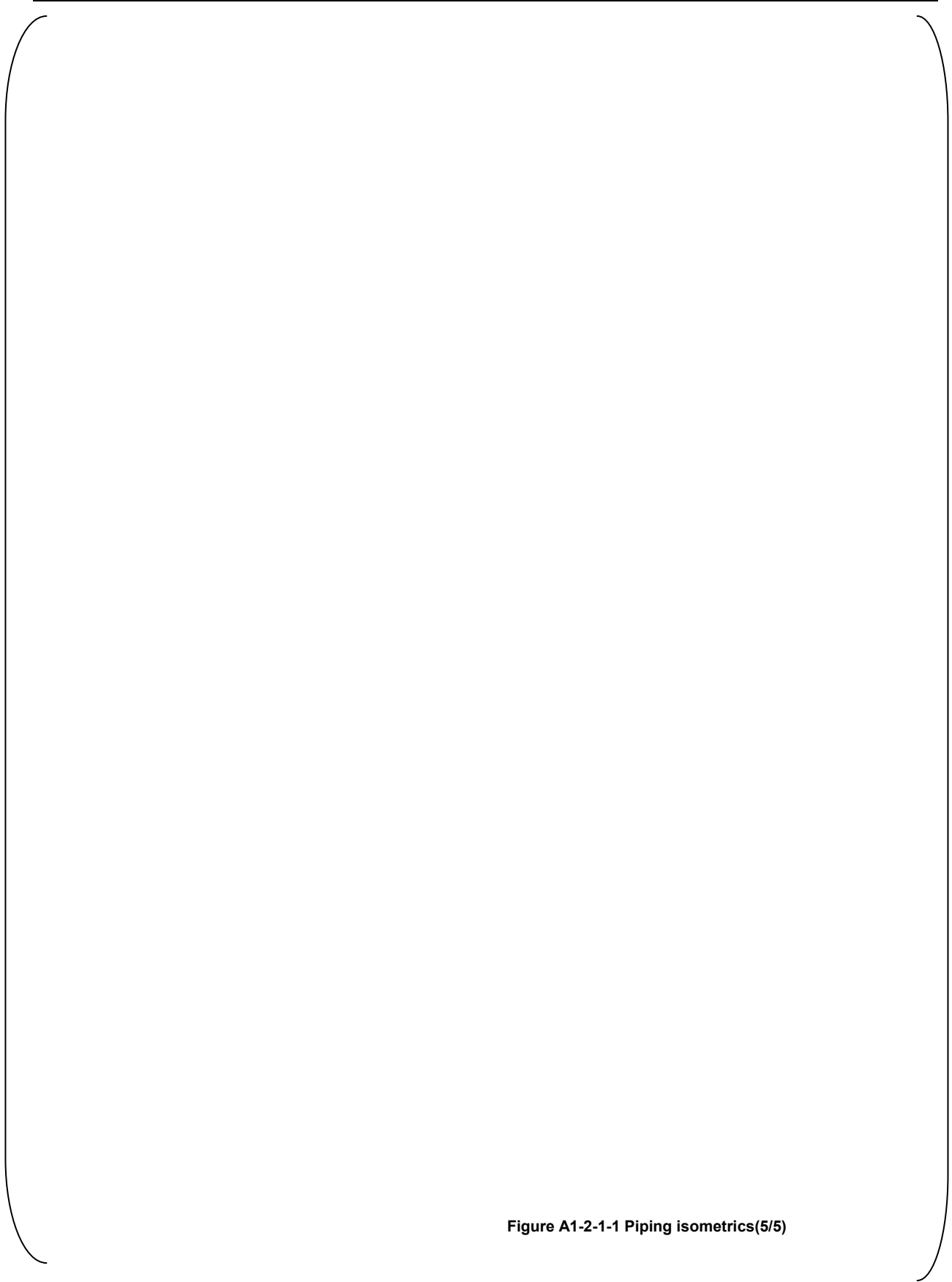
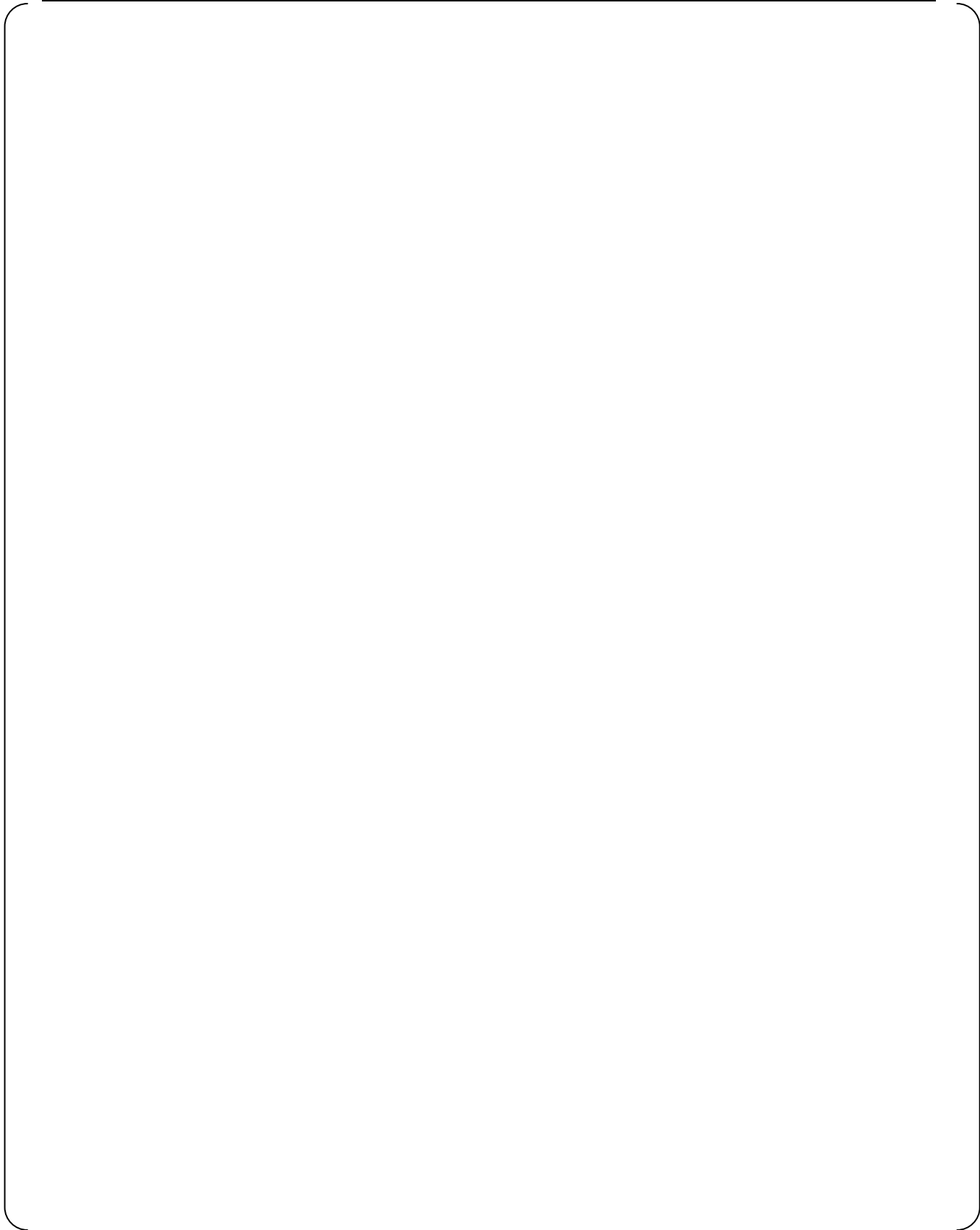


Figure A1-2-1-1 Piping isometrics(5/5)

Table A1-2-1-2 Concentrated mass





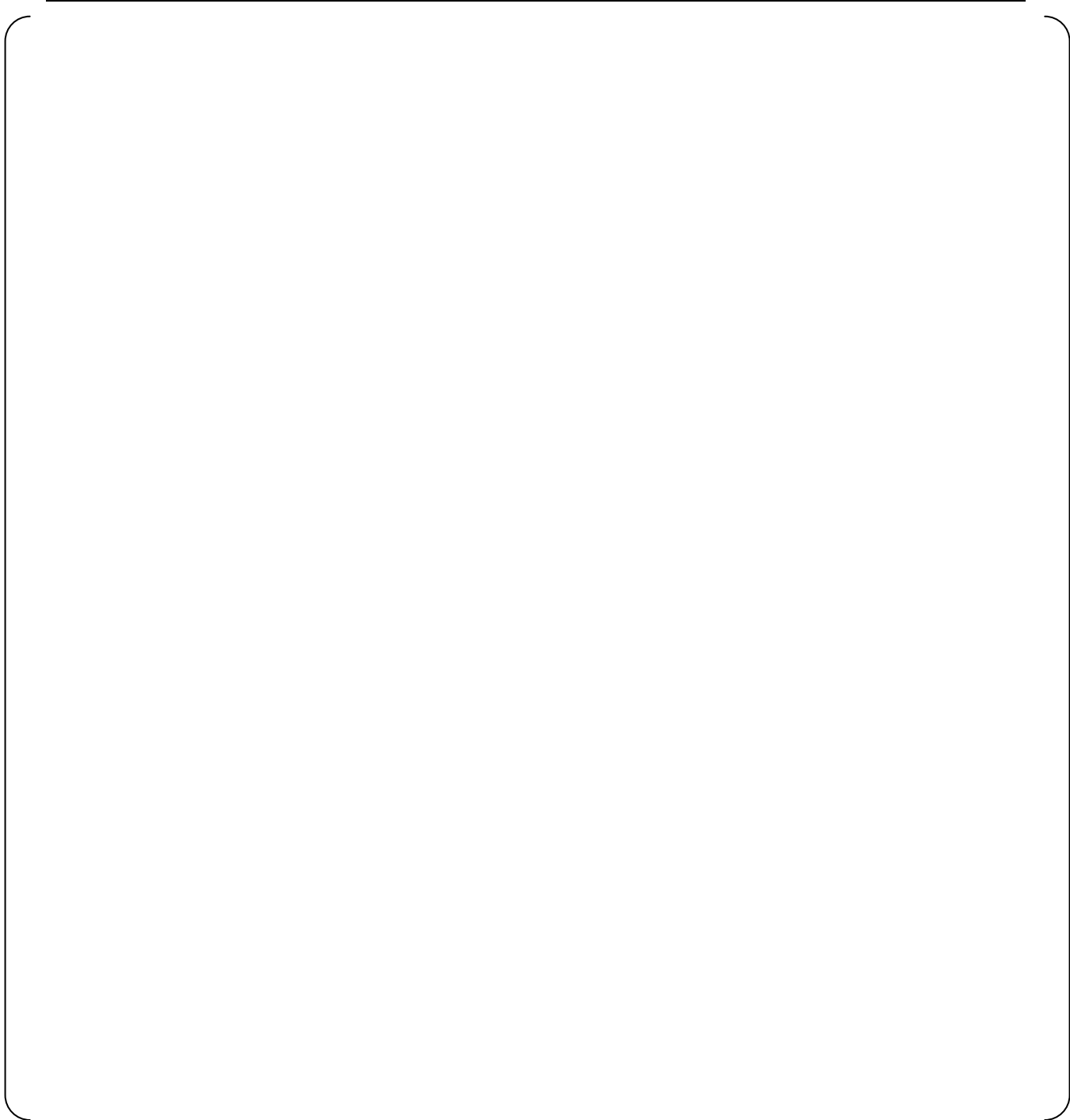
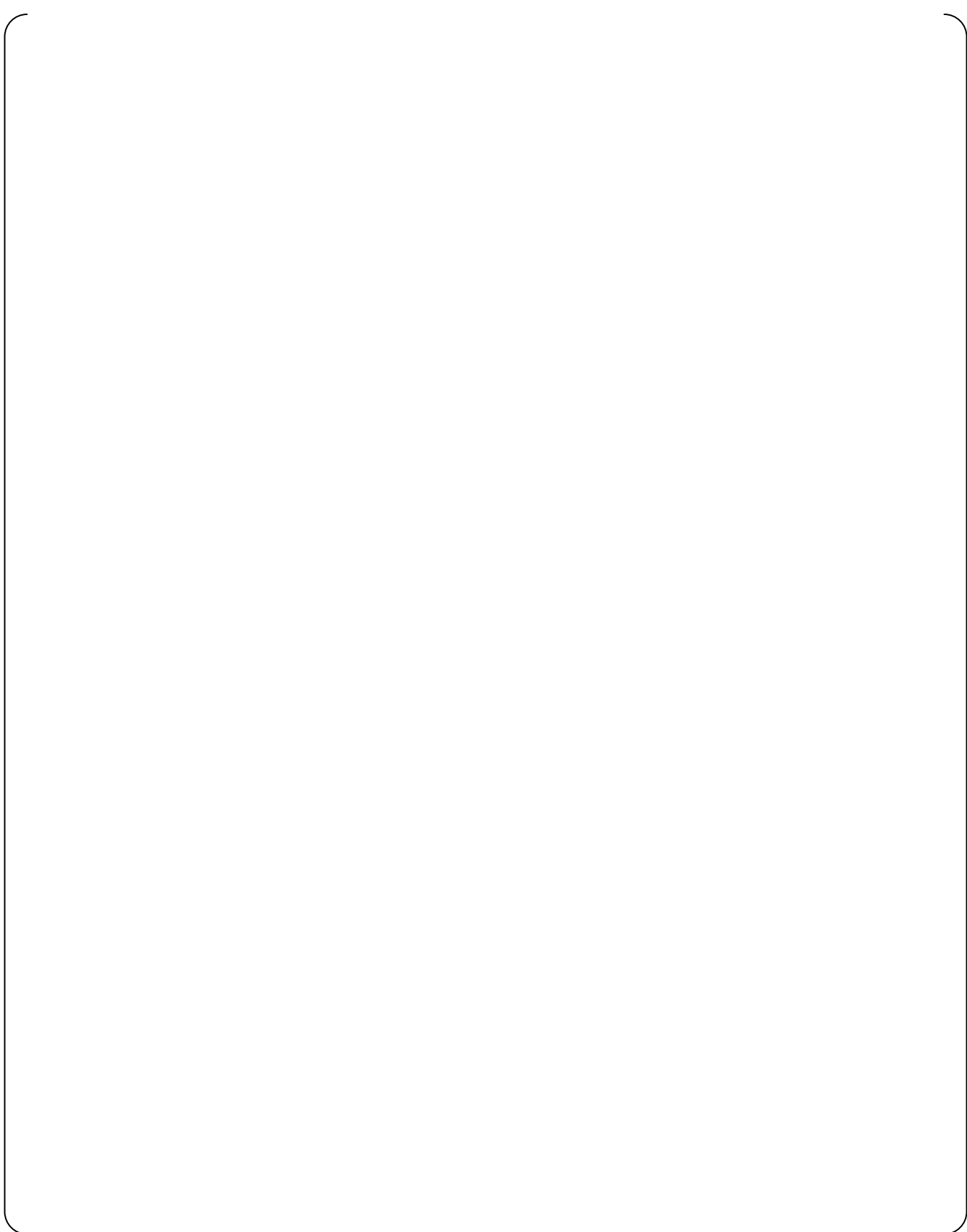
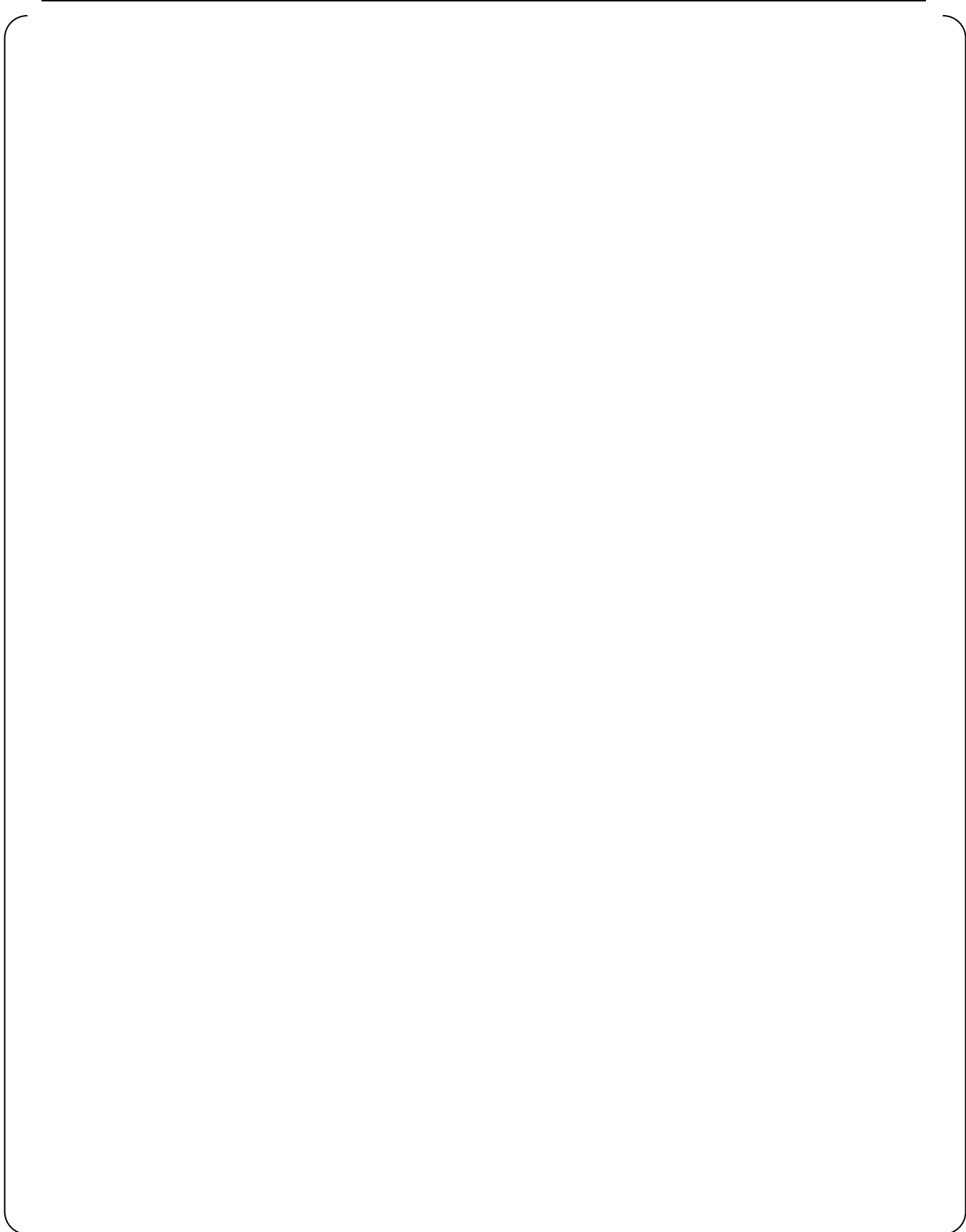
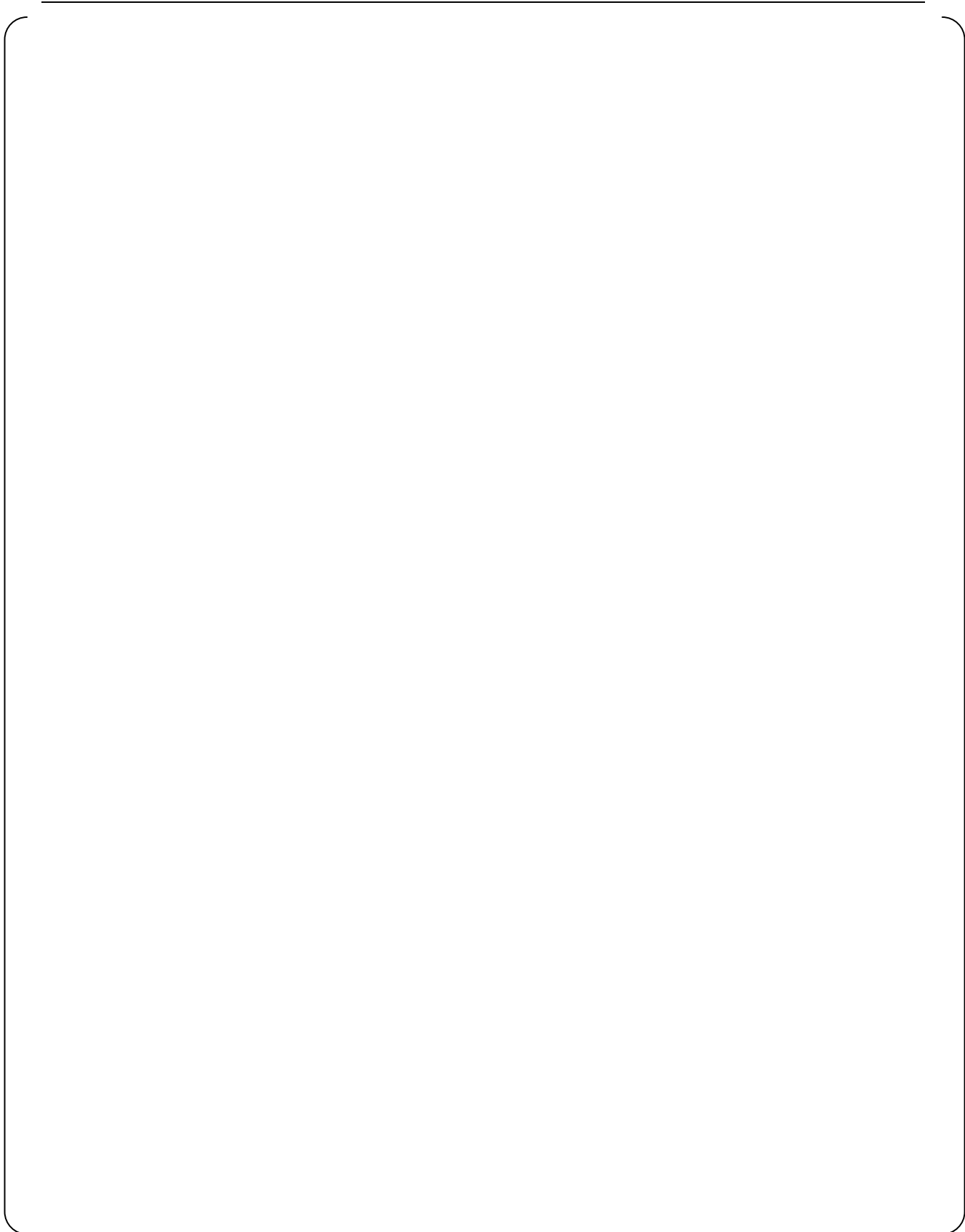


Table A1-2-1-3 Support point rigidity









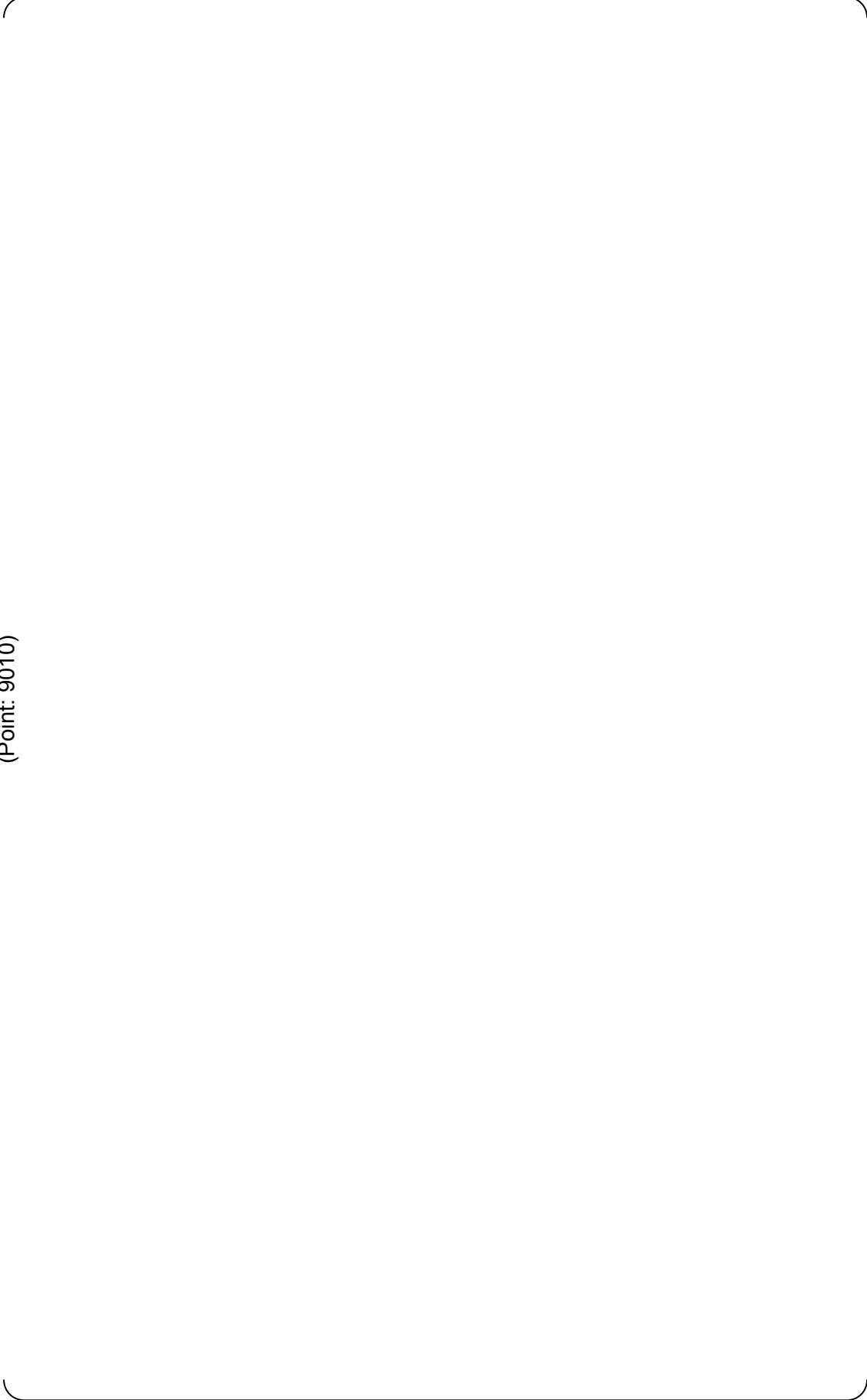




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Table A1-2-1-4 Valve rigidity

Table A1-2-1-5 Level A/B thermal displacement input data (1/4)  
(Point: 9010)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-1-5 Level A/B thermal displacement input data (2/4)  
(Point: 9010)


| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-1-5 Level A/B thermal displacement input data (3/4)  
(Point: 9020)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-1-5 Level A/B thermal displacement input data (4/4)  
(Point: 9020)





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

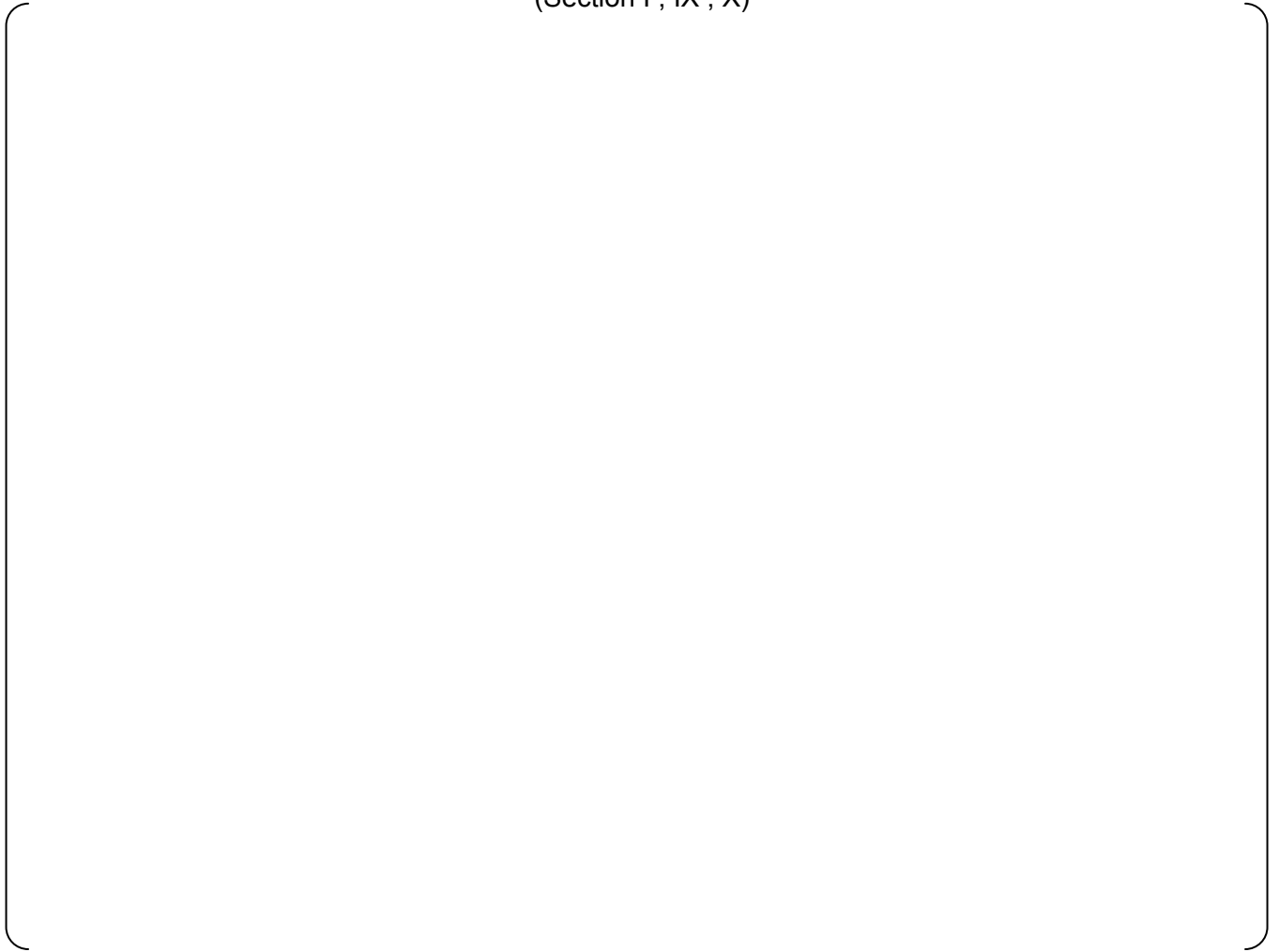
**MUAP-09011-NP (R2)**



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**Table A1-2-1-6 Level A, B temperature and pressure input data(1/18)**  
(Section I , IX , X)

Table A1-2-1-6 Level A, B temperature and pressure input data(2/18)  
(Section I , IX , X)



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**Table A1-2-1-6 Level A, B temperature and pressure input data(3/18)**  
(Section I , IX , X)

Table A1-2-1-6 Level A, B temperature and pressure input data(4/18)  
(Section II , III)

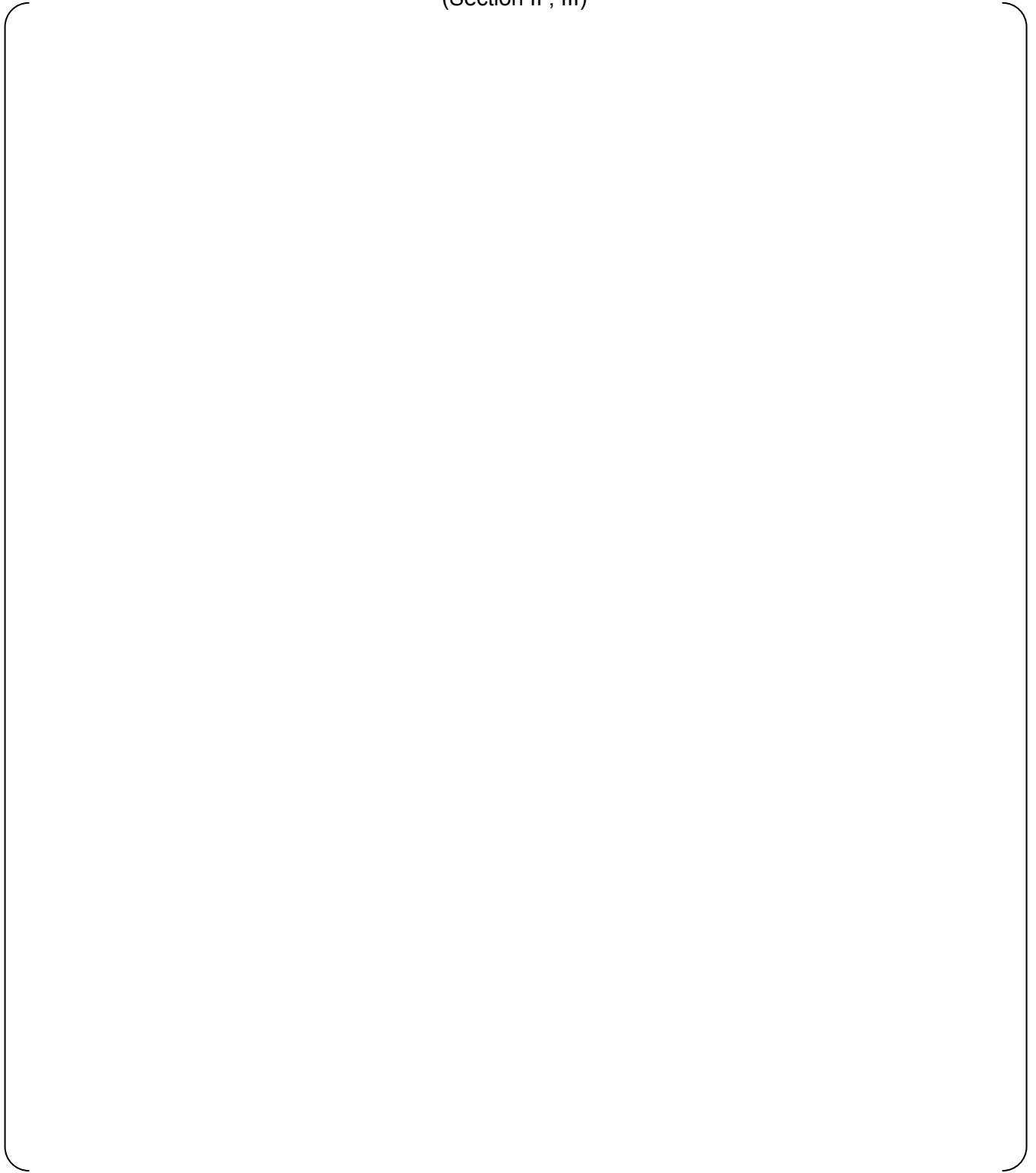


Table A1-2-1-6 Level A, B temperature and pressure input data(5/18)  
(Section II , III)

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**Table A1-2-1-6 Level A, B temperature and pressure input data(6/18)**  
(Section II , III)

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**Table A1-2-1-6 Level A, B temperature and pressure input data(7/18)**  
(Section IV , V , VI , VII , VIII,XII)

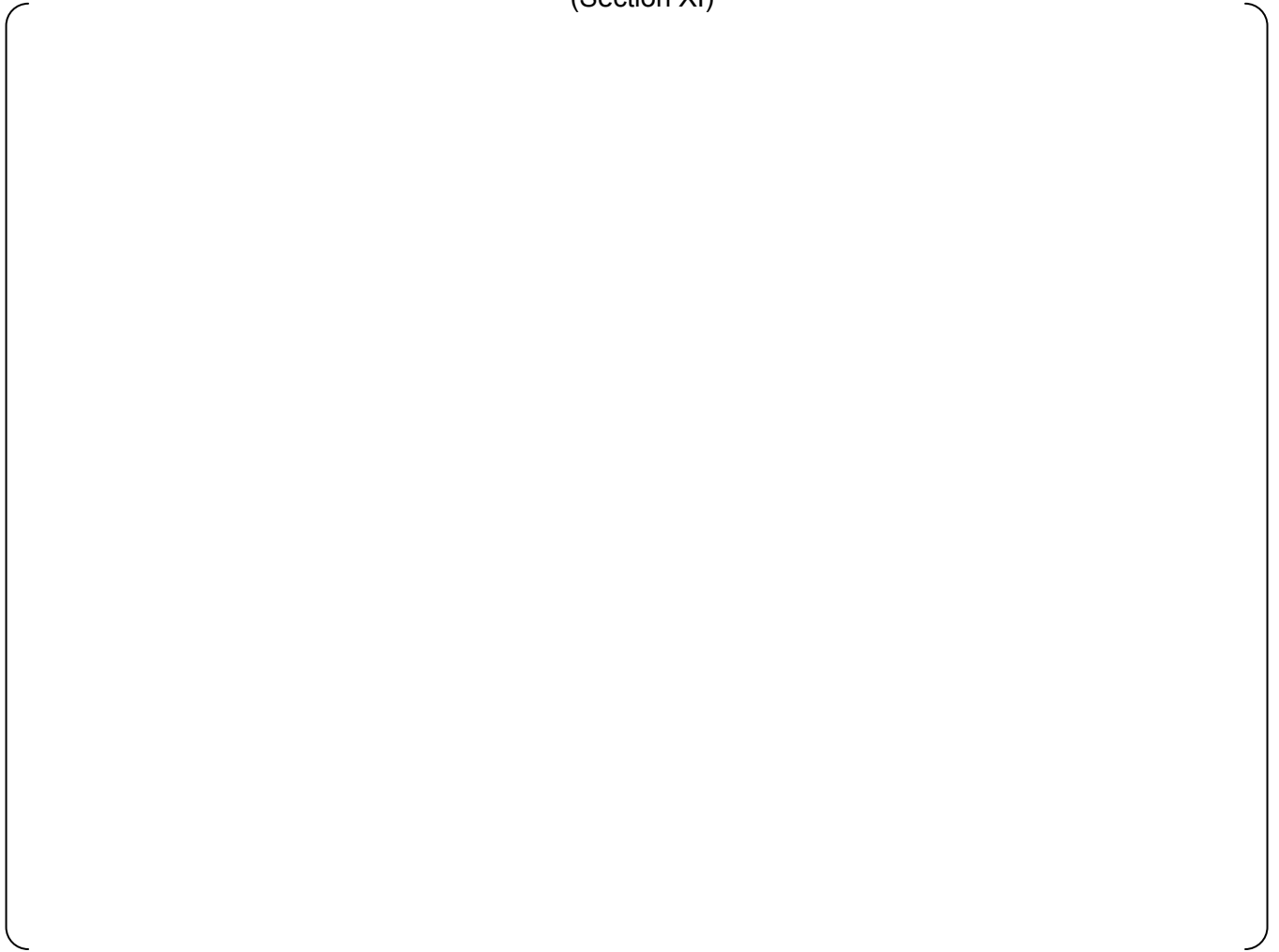


**Table A1-2-1-6 Level A, B temperature and pressure input data(8/18)**  
(Section IV , V , VI , VII , VIII ,XII)

**Table A1-2-1-6 Level A, B temperature and pressure input data(9/18)**  
(Section IV , V , VI , VII , VIII ,XII)

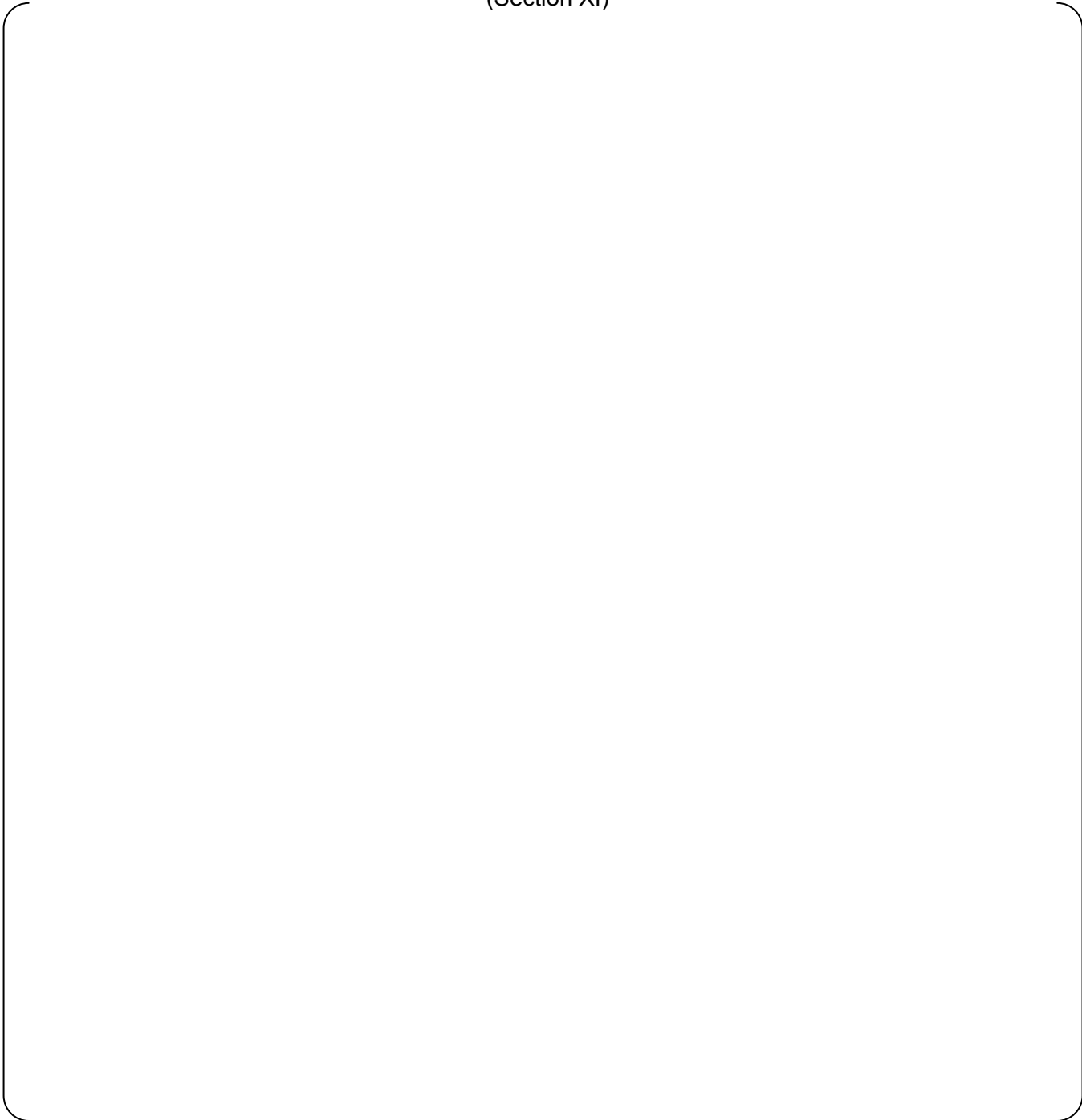
Table A1-2-1-6 Level A, B temperature and pressure input data(10/18)  
(Section XI)

Table A1-2-1-6 Level A, B temperature and pressure input data(11/18)  
(Section XI)



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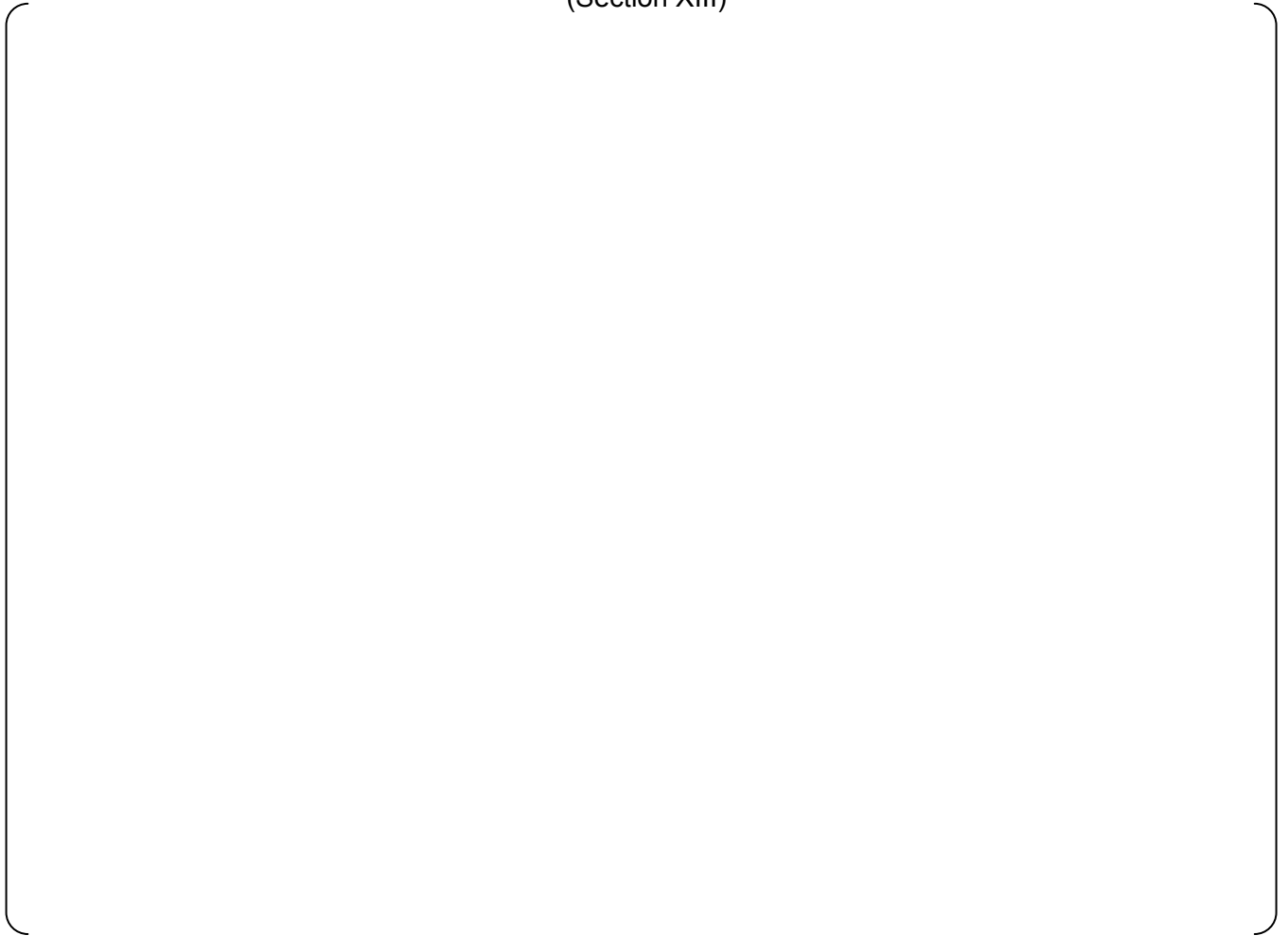
Table A1-2-1-6 Level A, B temperature and pressure input data(12/18)  
(Section XI)



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Table A1-2-1-6 Level A, B temperature and pressure input data(13/18)  
(Section XIII)

Table A1-2-1-6 Level A, B temperature and pressure input data(14/18)  
(Section XIII)



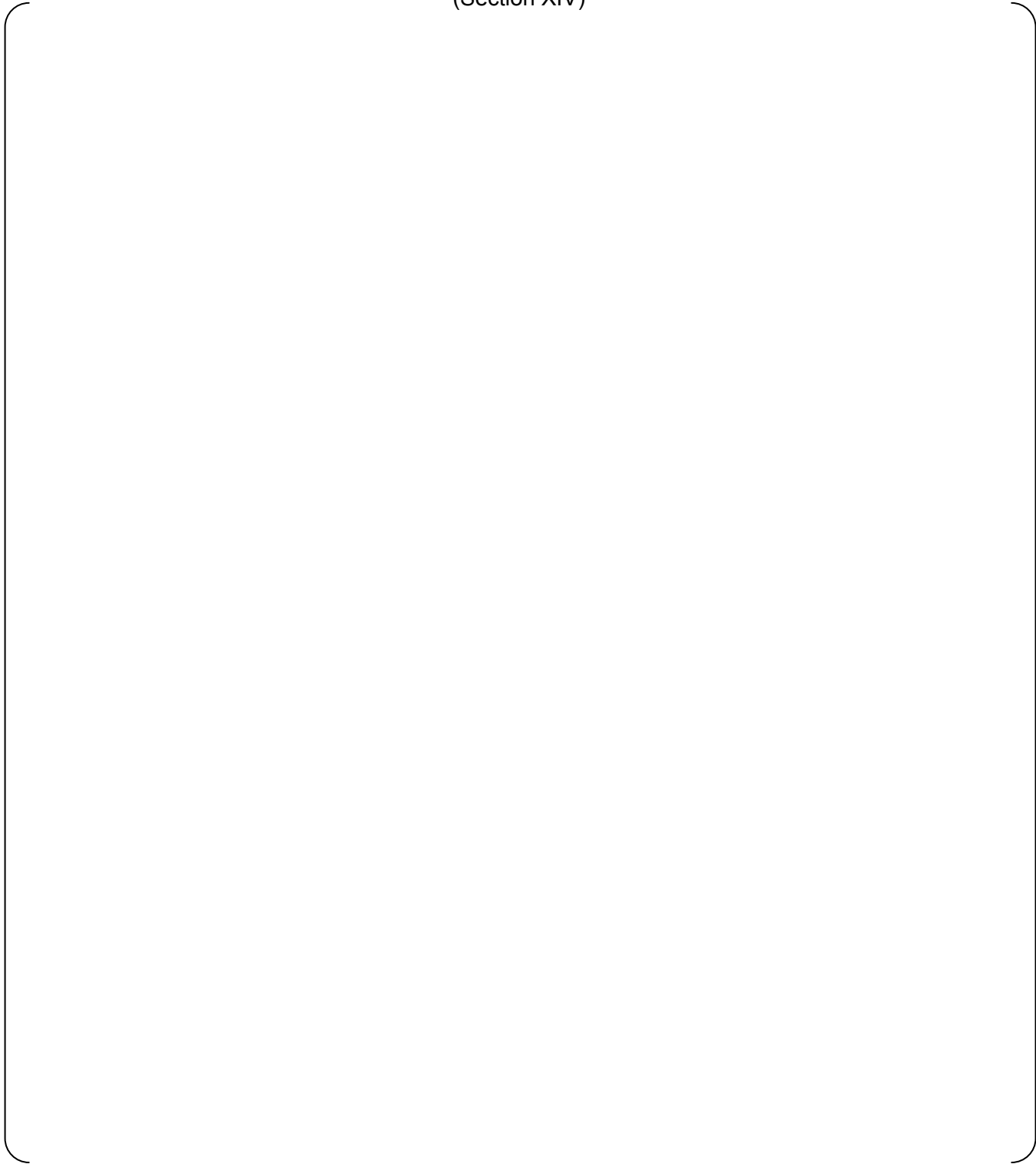
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Table A1-2-1-6 Level A, B temperature and pressure input data(15/18)  
(Section XIII)



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Table A1-2-1-6 Level A, B temperature and pressure input data(16/18)  
(Section XIV)



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Table A1-2-1-6 Level A, B temperature and pressure input data(17/18)  
(Section XIV)

Table A1-2-1-6 Level A, B temperature and pressure input data(18/18)  
(Section XIV)

Table A1-2-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-2-1-2 Floor response curve (1/12)**  
Pressurizer Spray Line (RC02) FRS for piping  
X (EW) direction (damping 3.0%)



**Figure A1-2-1-2 Floor response curve (2/12)**  
Pressurizer Spray Line (RC02) FRS for piping  
Y (NS) direction (damping 3.0%)



**Figure A1-2-1-2 Floor response curve (3/12)**  
Pressurizer Spray Line (RC02) FRS for piping  
Z (Vert.) direction (damping 3.0%)



**Figure A1-2-1-2 Floor response curve (4/12)**  
Pressurizer Spray Line (RC02) FRS for MCP nozzle  
X (EW) direction (damping 3.0%)





**Figure A1-2-1-2 Floor response curve (5/12)**  
Pressurizer Spray Line (RC02) FRS for MCP nozzle  
Y (NS) direction (damping 3.0%)



**Figure A1-2-1-2 Floor response curve (6/12)**  
Pressurizer Spray Line (RC02) FRS for MCP nozzle  
Z (Vert.) direction (damping 3.0%)



**Figure A1-2-1-2 Floor response curve (7/12)**  
Pressurizer Spray Line (RC02) FRS for Pressurizer base plate  
X (EW) direction (damping 3.0%)



**Figure A1-2-1-2 Floor response curve (8/12)**  
Pressurizer Spray Line (RC02) FRS for Pressurizer base plate  
Y (NS) direction (damping 3.0%)



**Figure A1-2-1-2 Floor response curve (9/12)**  
Pressurizer Spray Line (RC02) FRS for Pressurizer base plate  
Z (Vert.) direction (damping 3.0%)



**Figure A1-2-1-2 Floor response curve (10/12)**  
Pressurizer Spray Line (RC02) FRS for Pressurizer support  
X (EW) direction (damping 3.0%)



**Figure A1-2-1-2 Floor response curve (11/12)**  
Pressurizer Spray Line (RC02) FRS for Pressurizer support  
Y (NS) direction (damping 3.0%)



**Figure A1-2-1-2 Floor response curve (12/12)**  
Pressurizer Spray Line (RC02) FRS for Pressurizer support  
Z (Vert.) direction (damping 3.0%)



Table A1-2-1-8 Seismic relative displacement input data









Table A1-2-1-9 DBPB displacement input data

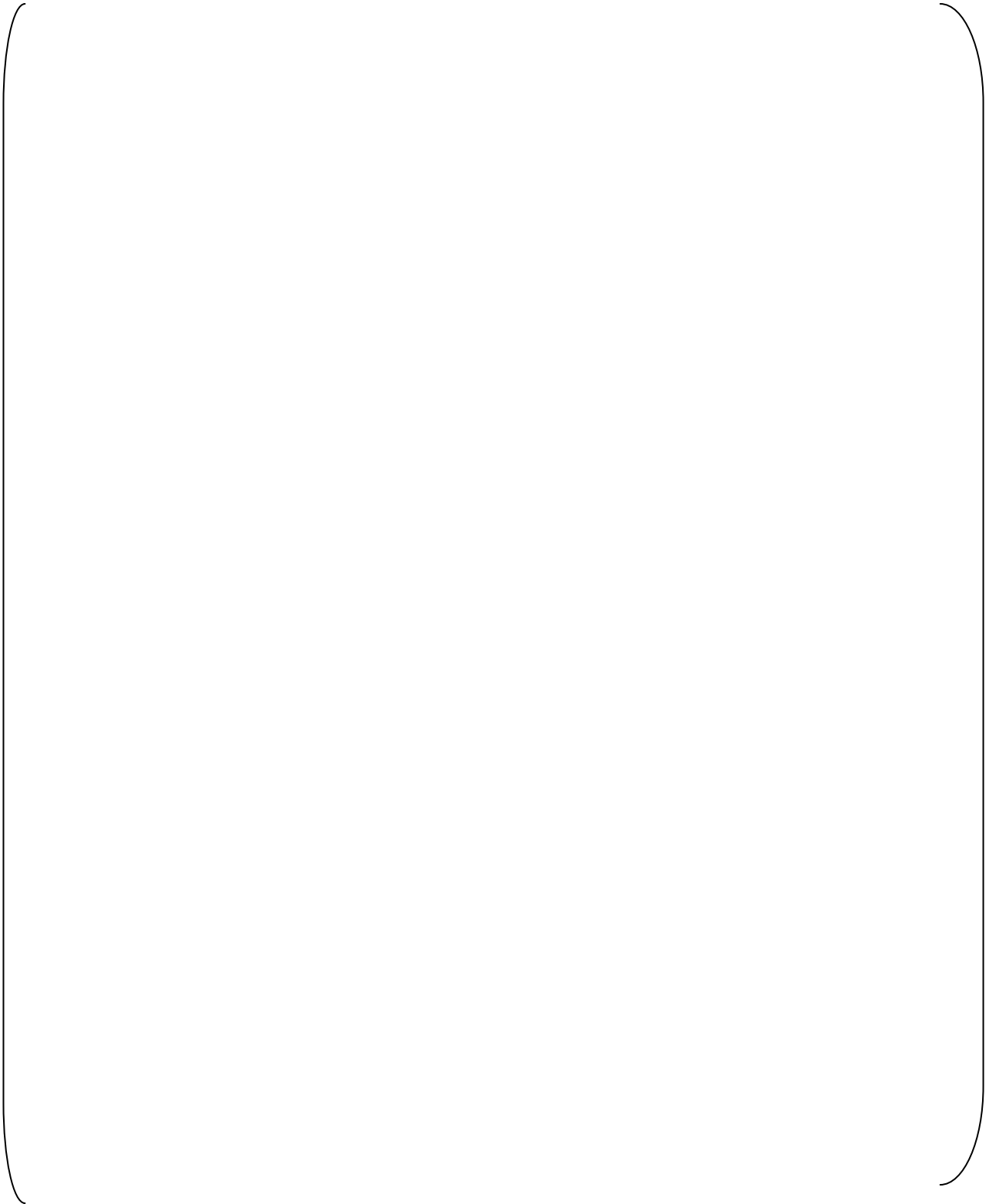
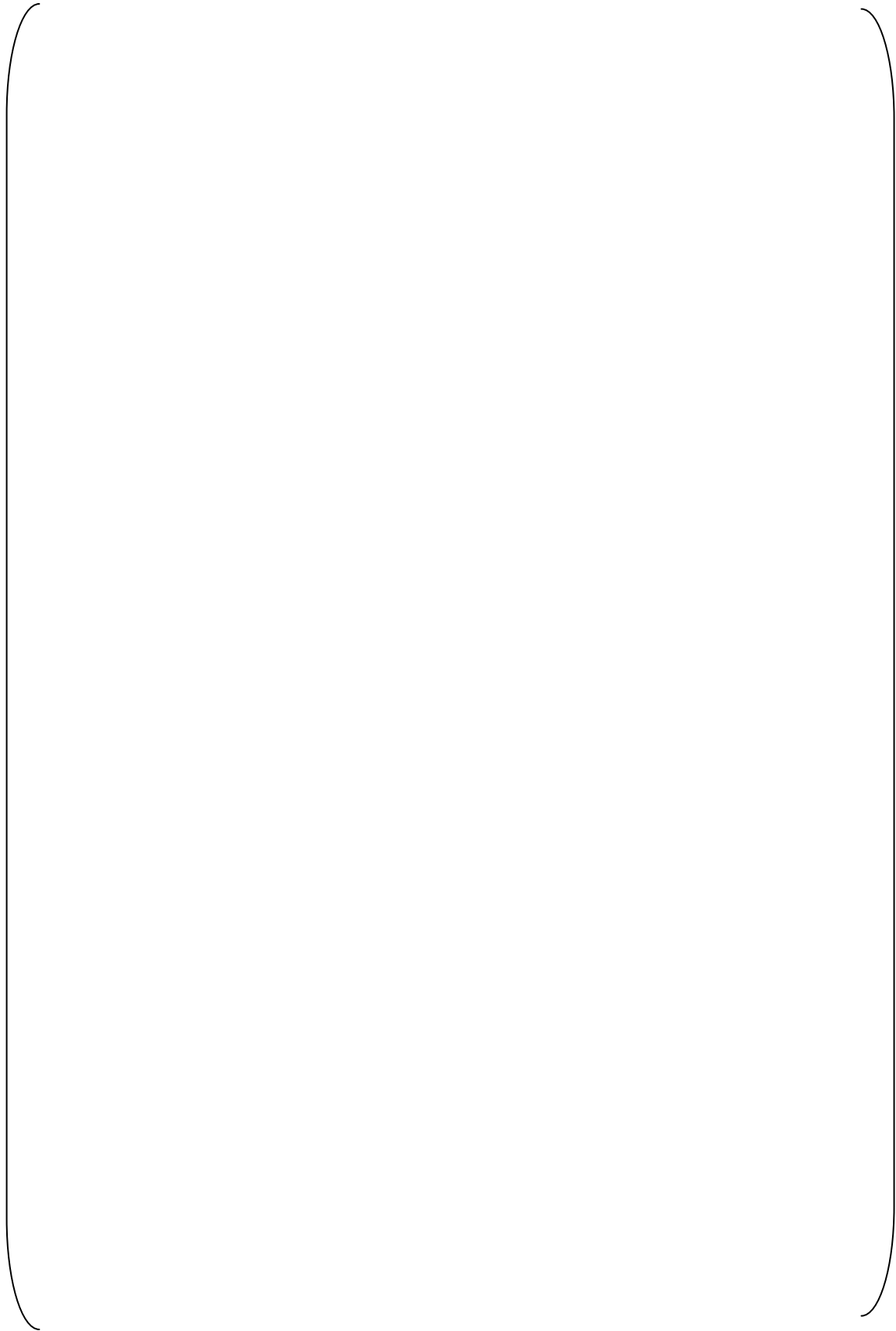


Figure A1-2-2-1 PIPESTRESS analysis model diagram

Table A1-2-2-1 Natural frequency analysis results (1/2)





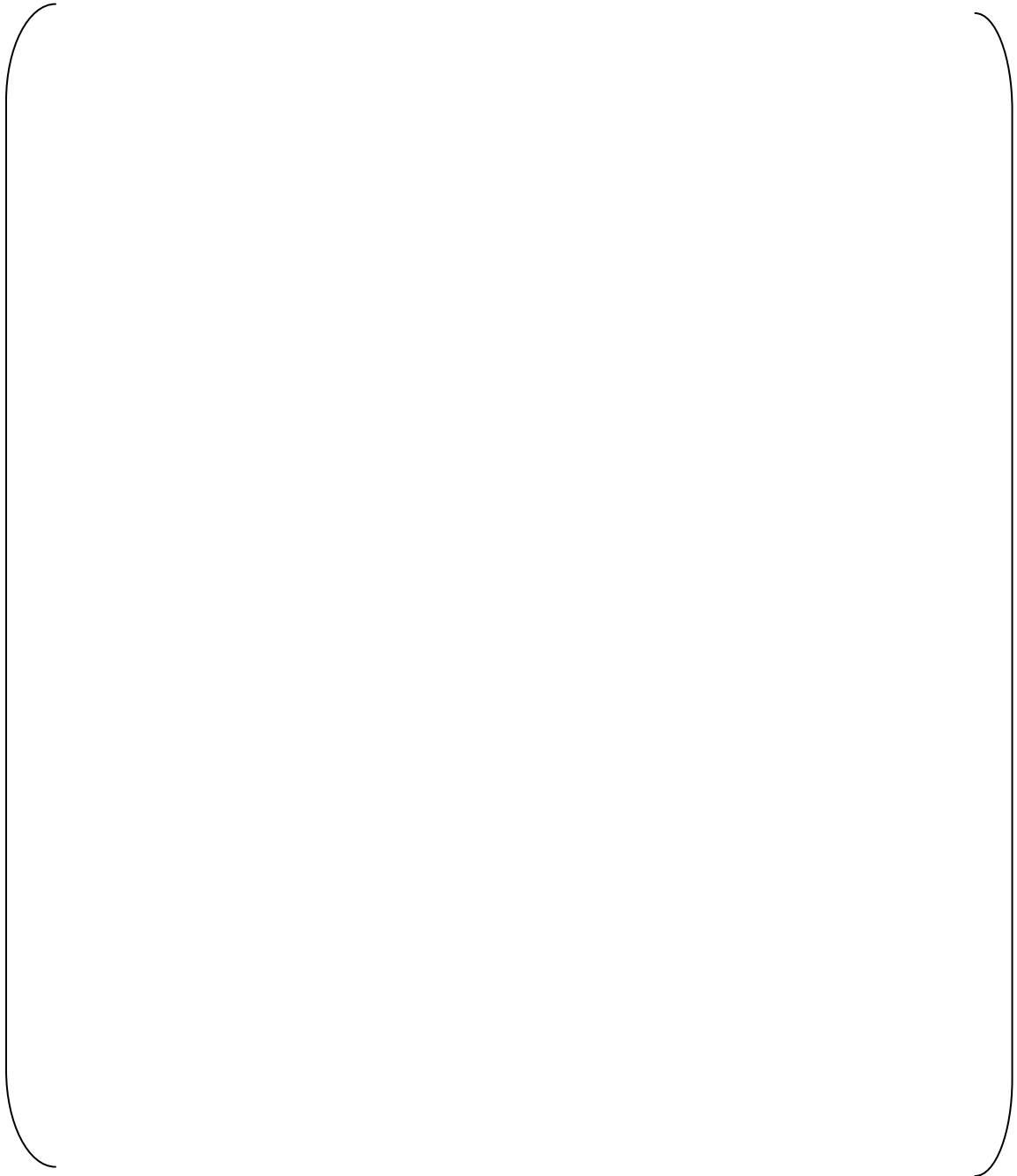
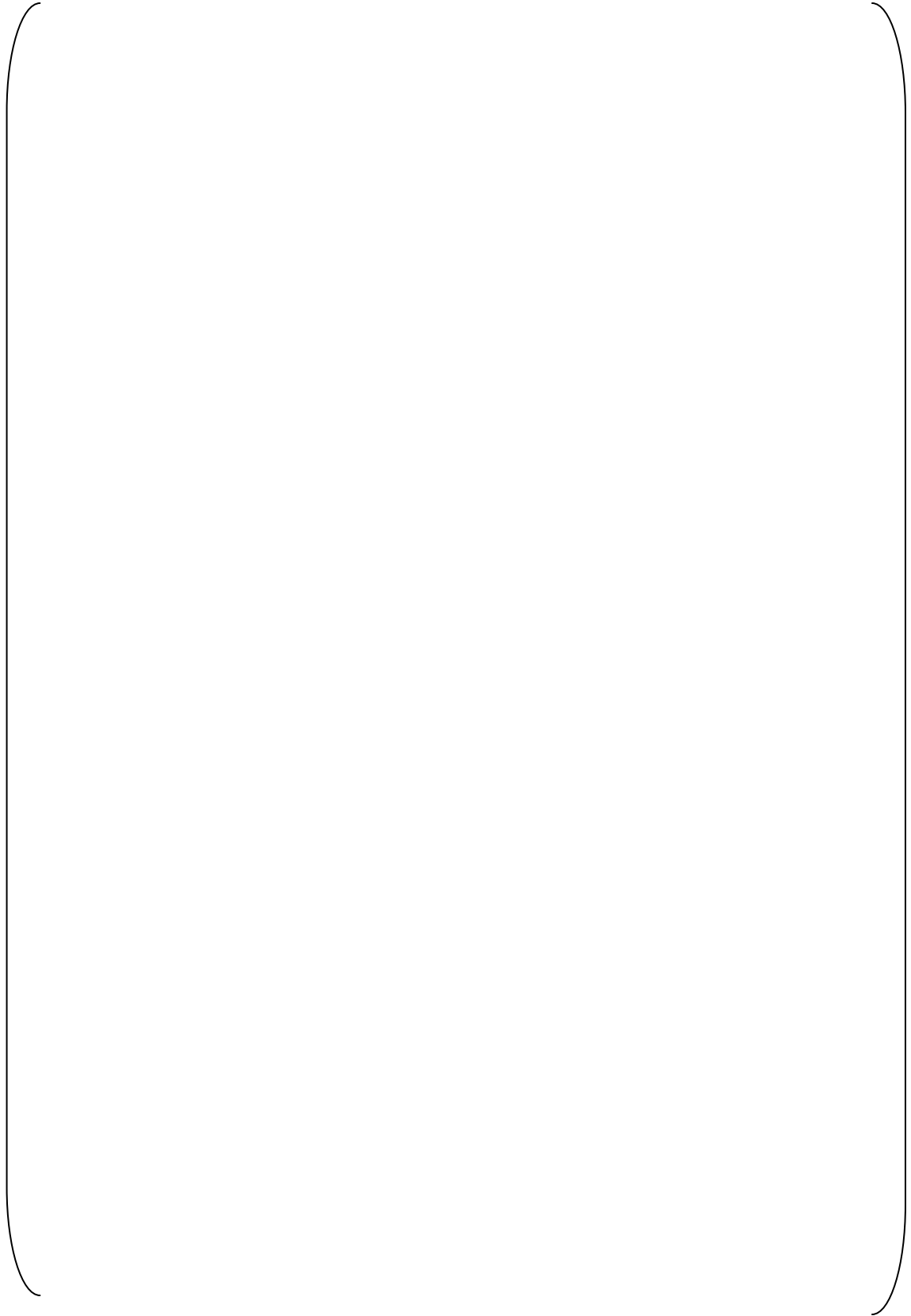
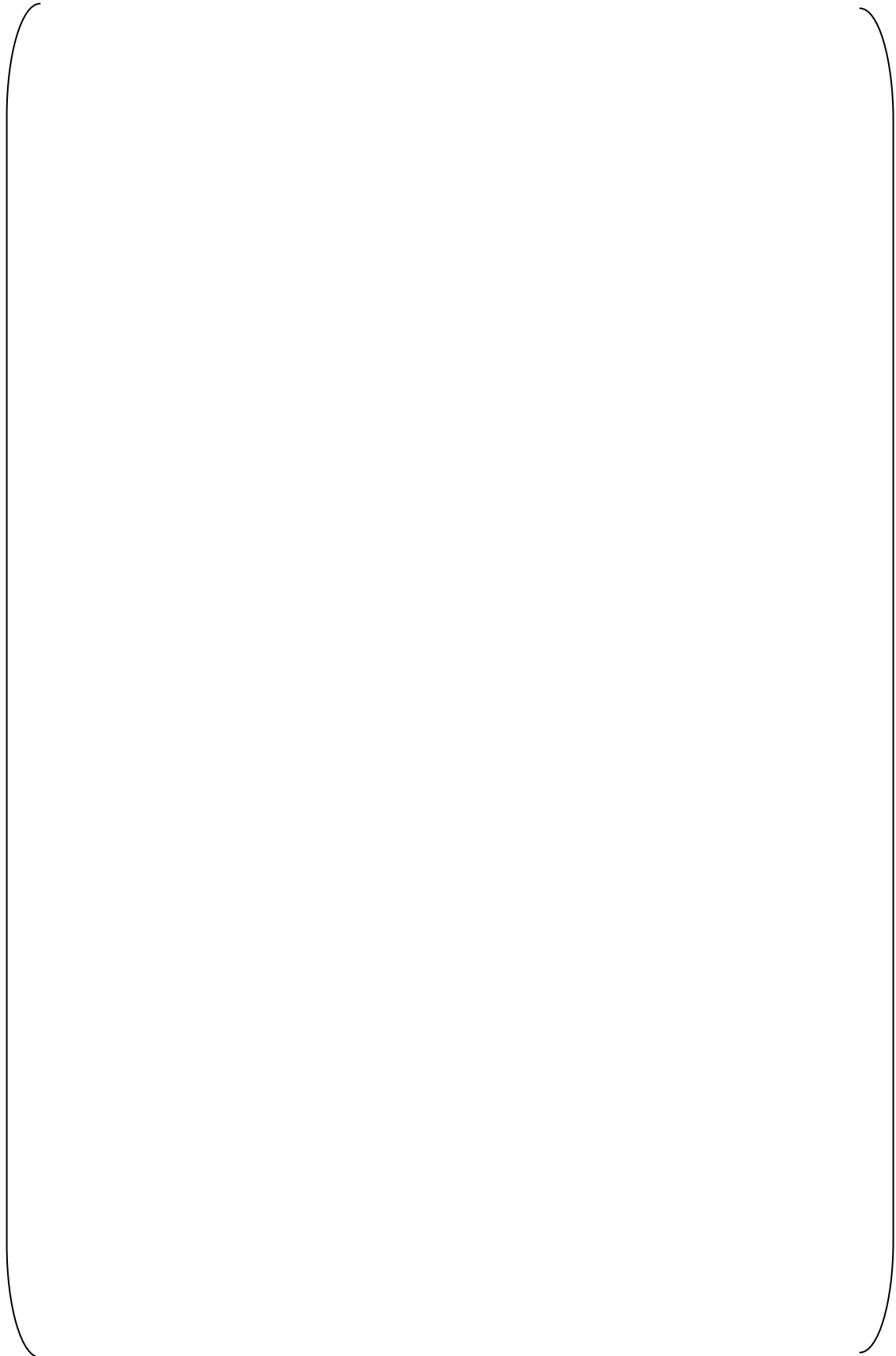
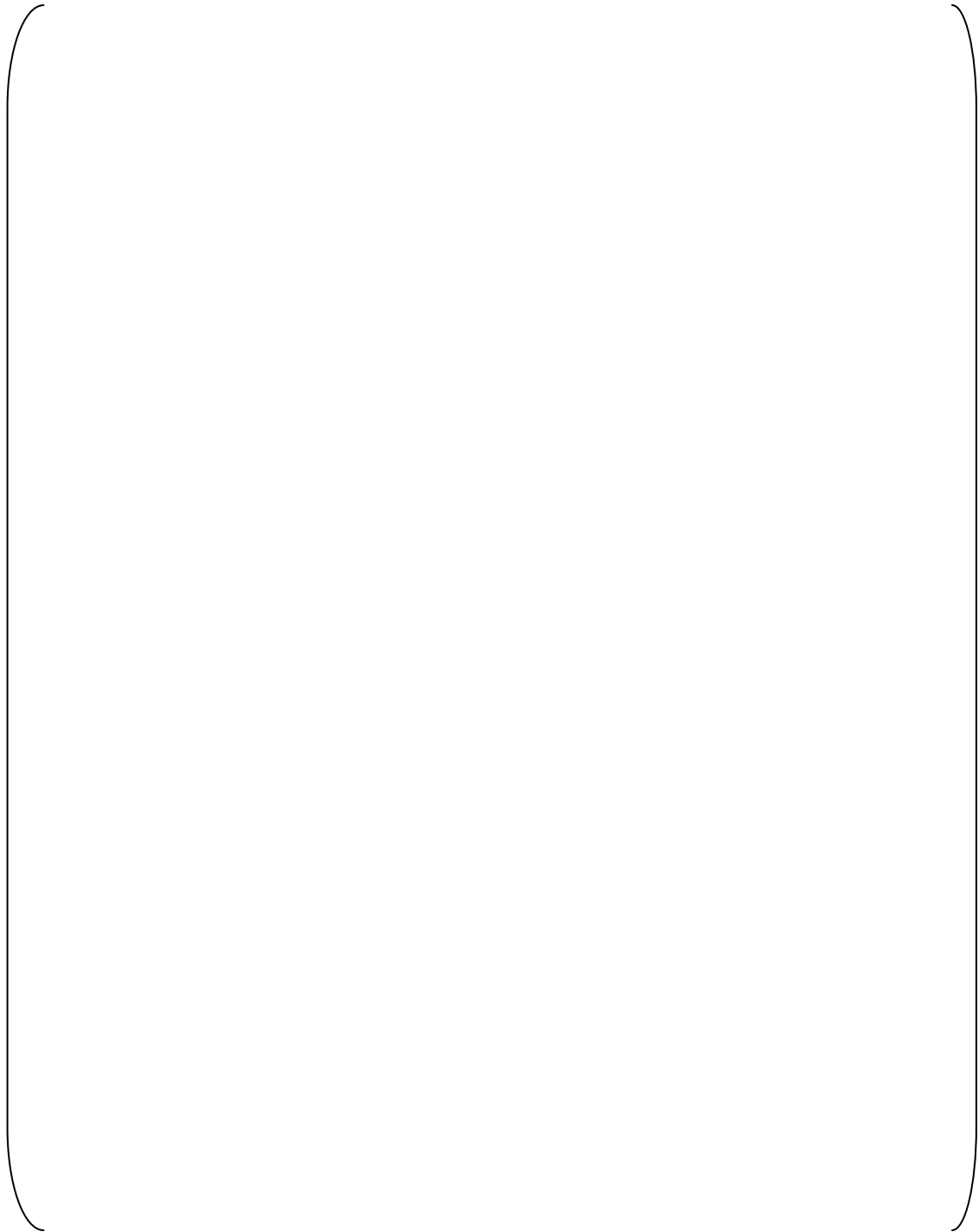


Table A1-2-2-1 Natural frequency analysis results(2/2)

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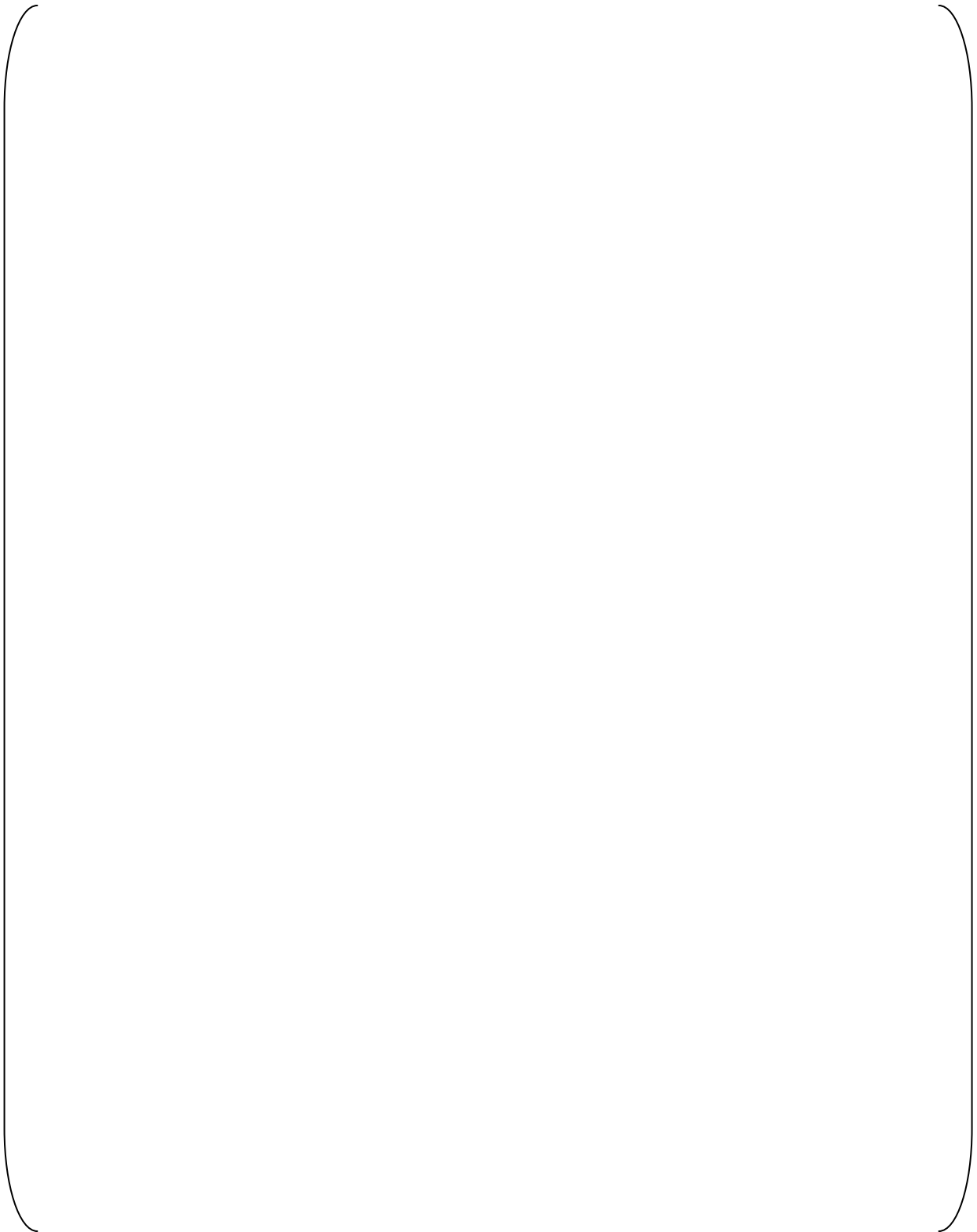


Figure A1-2-2-2 Frequency mode diagram (primary)

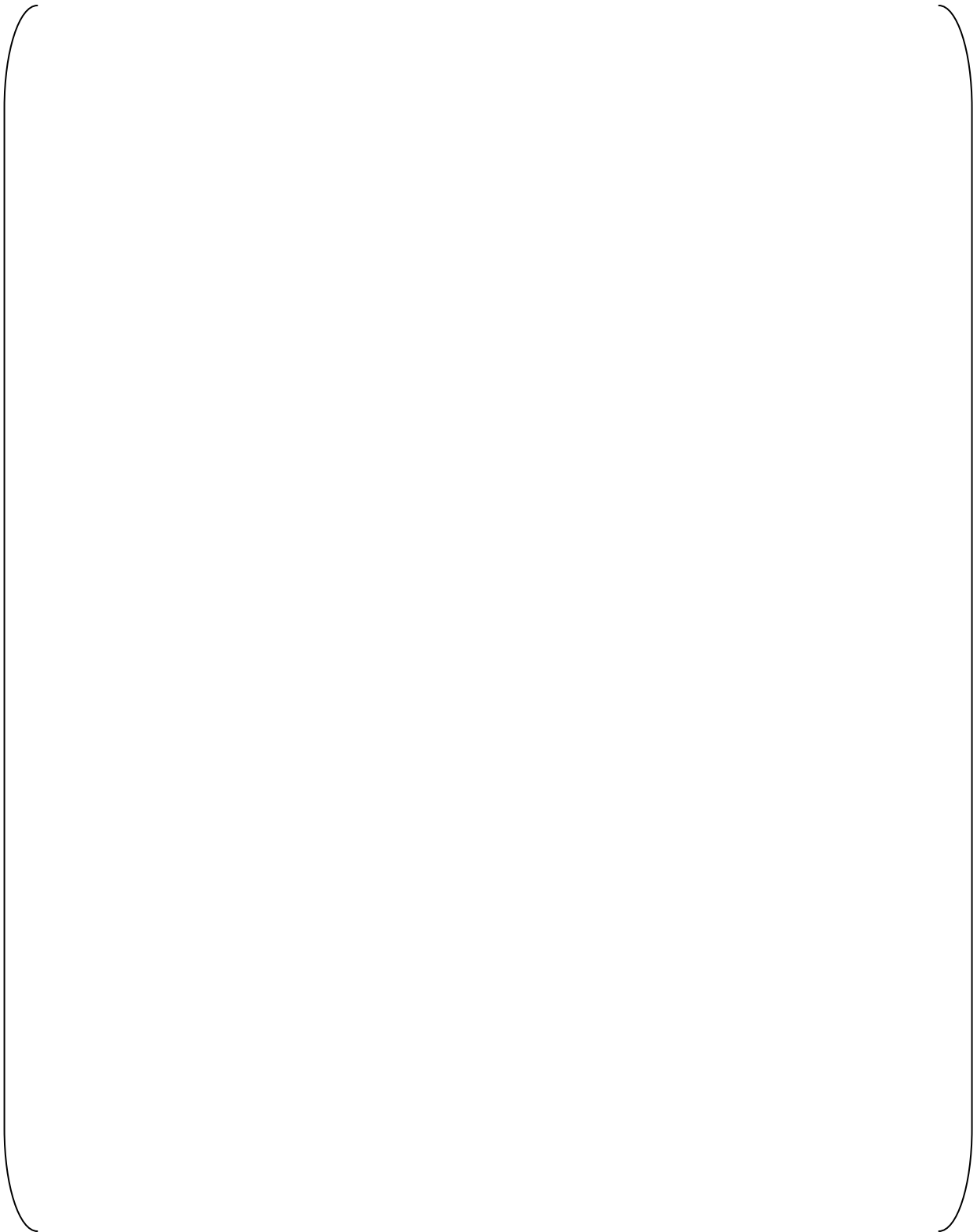


Figure A1-2-2-2 Frequency mode diagram (secondary)

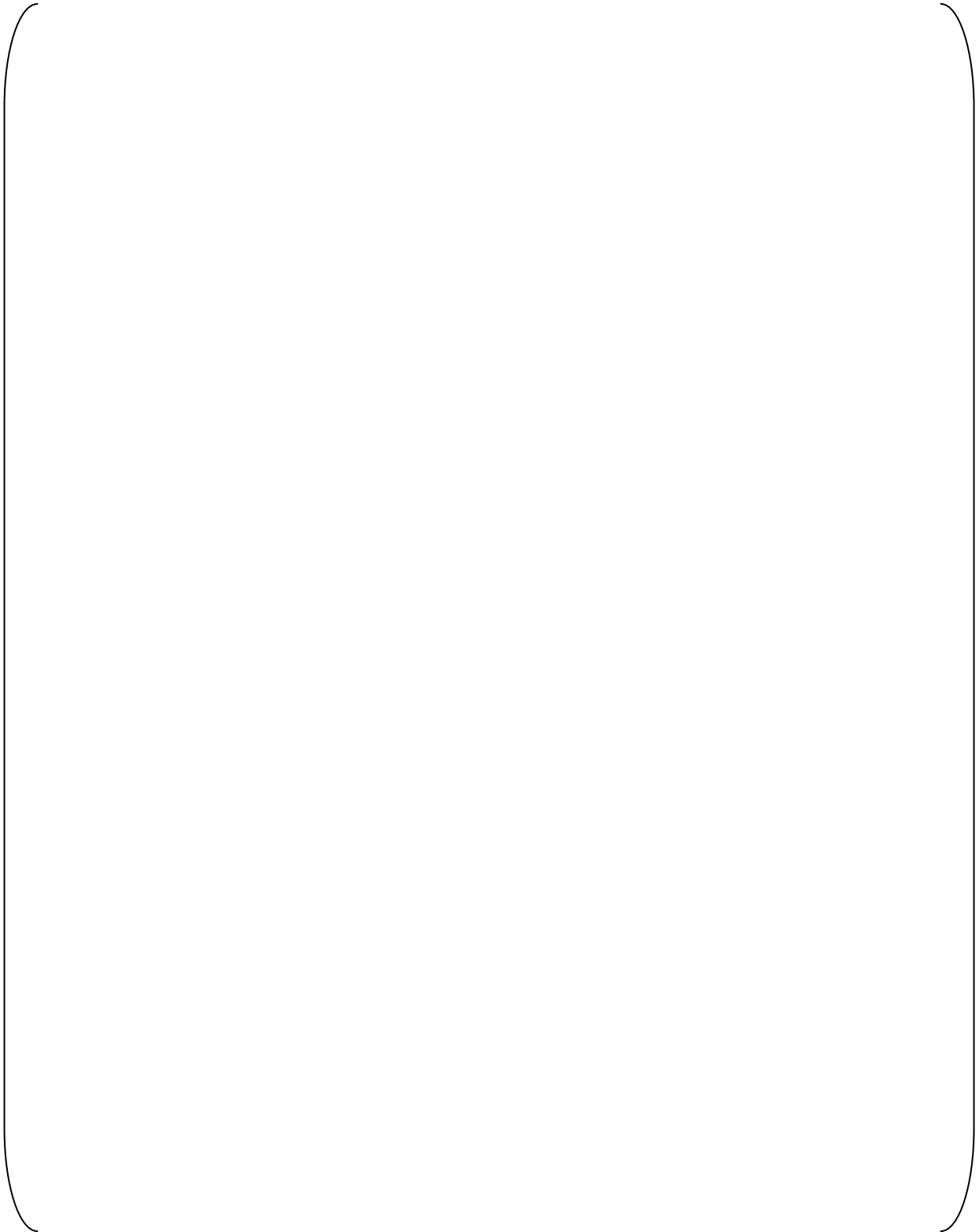
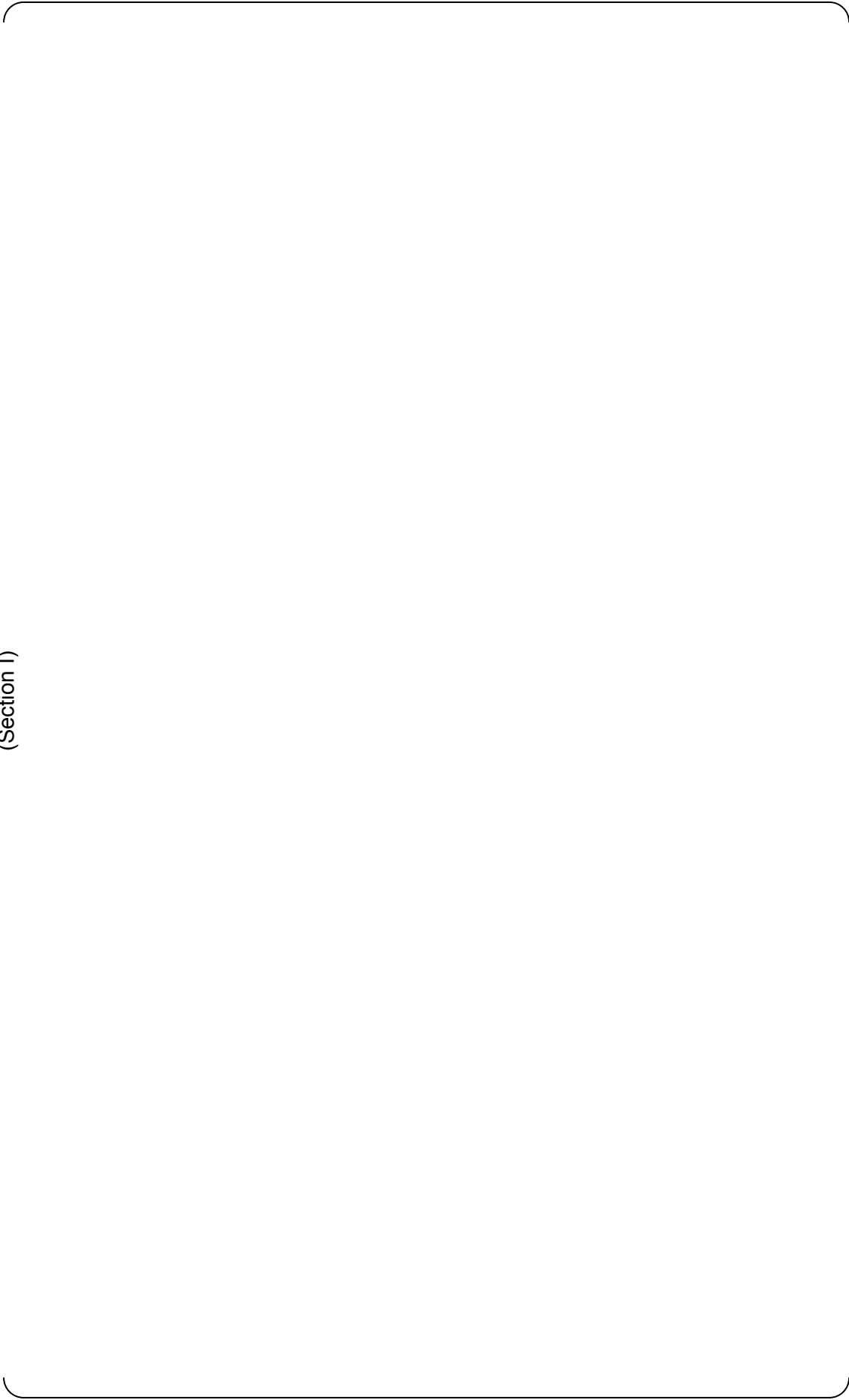


Figure A1-2-2-2 Frequency mode diagram (tertiary)



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (1/39)  
(Section I)




**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (2/39)  
(Section I)

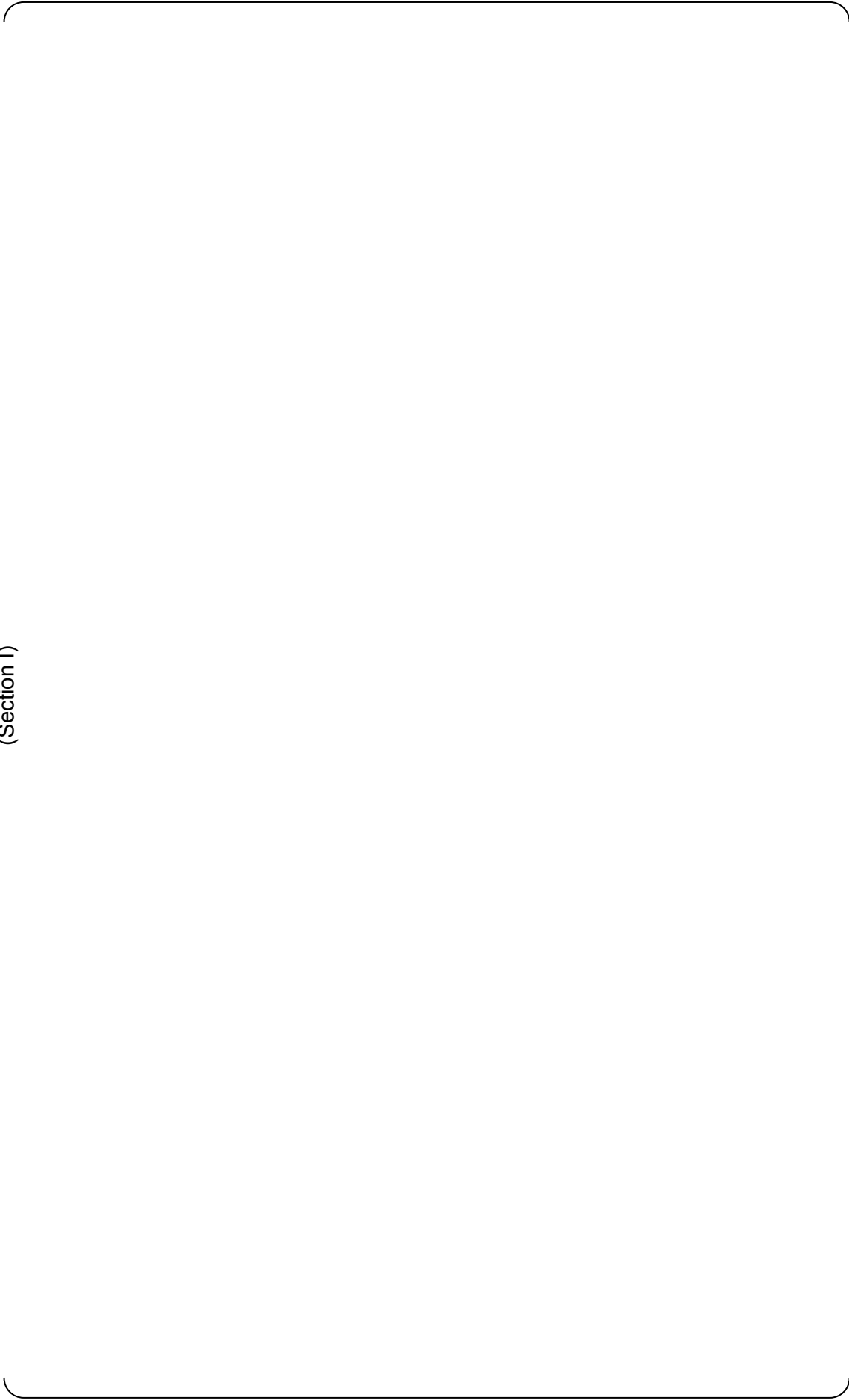


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (3/39)  
(Section I)

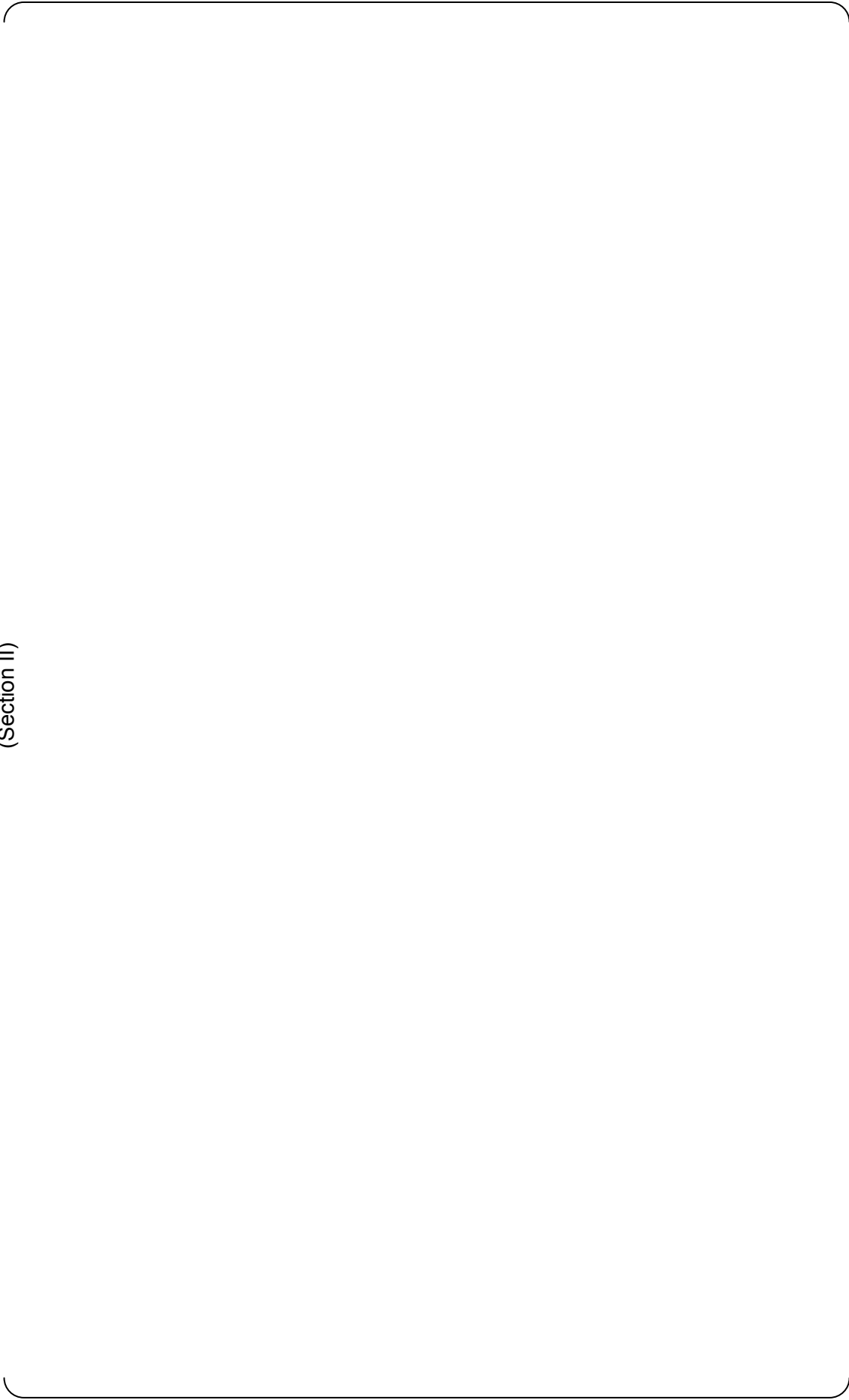


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (4/39)  
(Section II)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (9/39)  
(Section III)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**

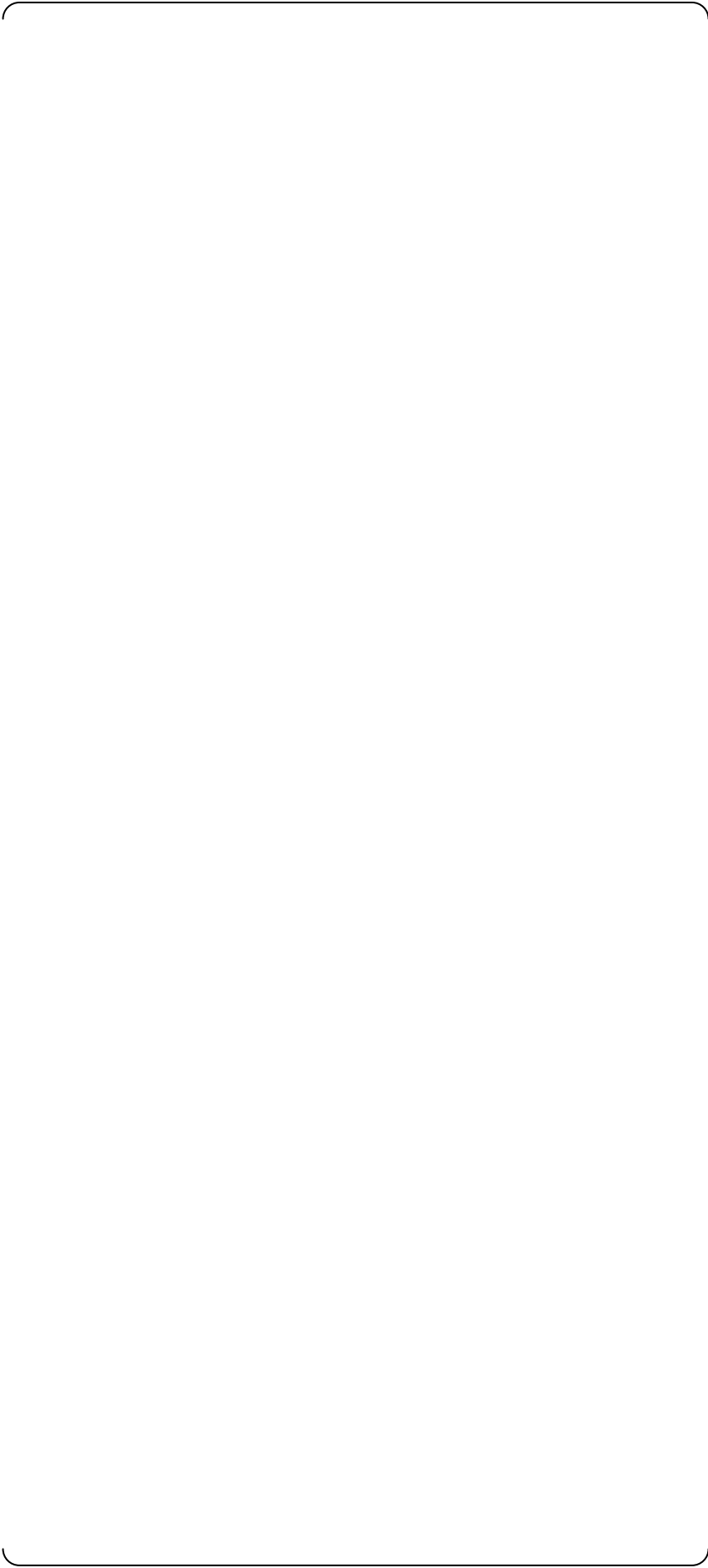


Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb)(10/39)  
(Section IV)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**

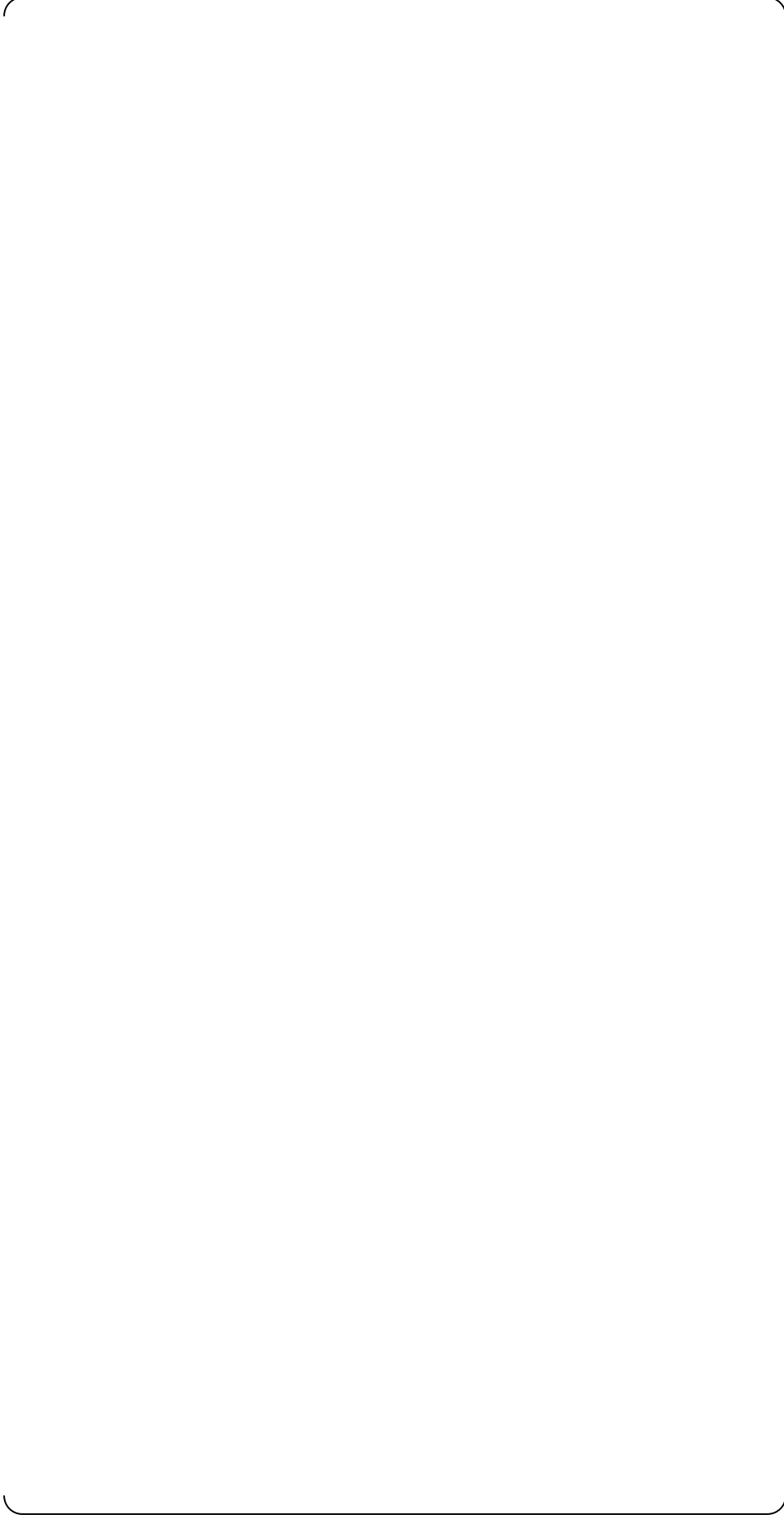


Table A1-2-2-2 Thermal analysis results ( $\Delta T_1$ ,  $\Delta T_2$ , Ta-Tb)(11/39)  
(Section IV)

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**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (12/39)  
(Section IV)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (13/39)  
(Section V)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb)(14/39)  
(Section V)

| Location | $\Delta T1$ | $\Delta T2$ | Ta-Tb |
|----------|-------------|-------------|-------|
|----------|-------------|-------------|-------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (15/39)  
(Section V)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (16/39)  
(Section VI)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (18/39)  
(Section VI)





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (20/39)  
(Section VII)

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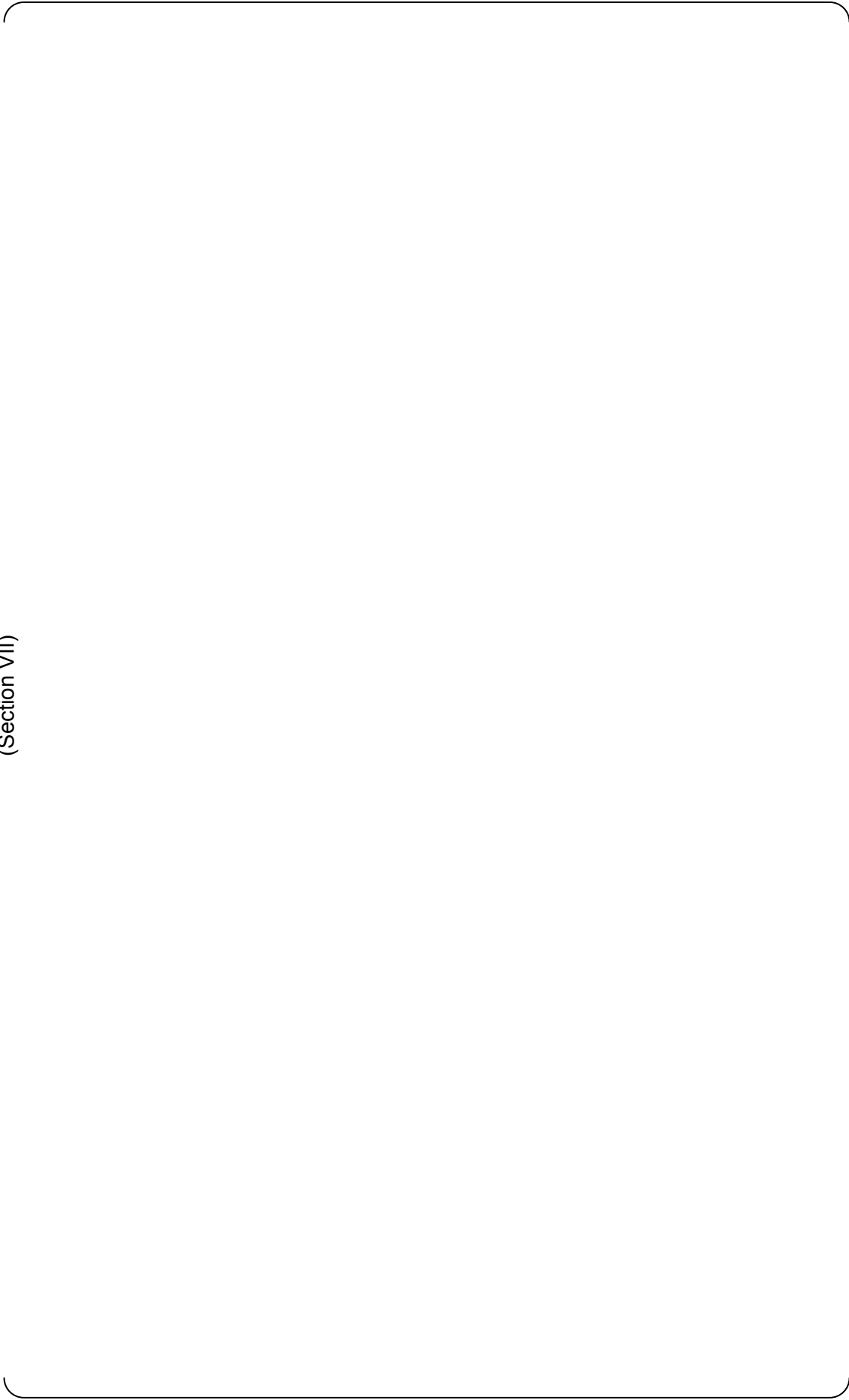
**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (21/39)  
(Section VII)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (22/39)  
(Section VIII)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



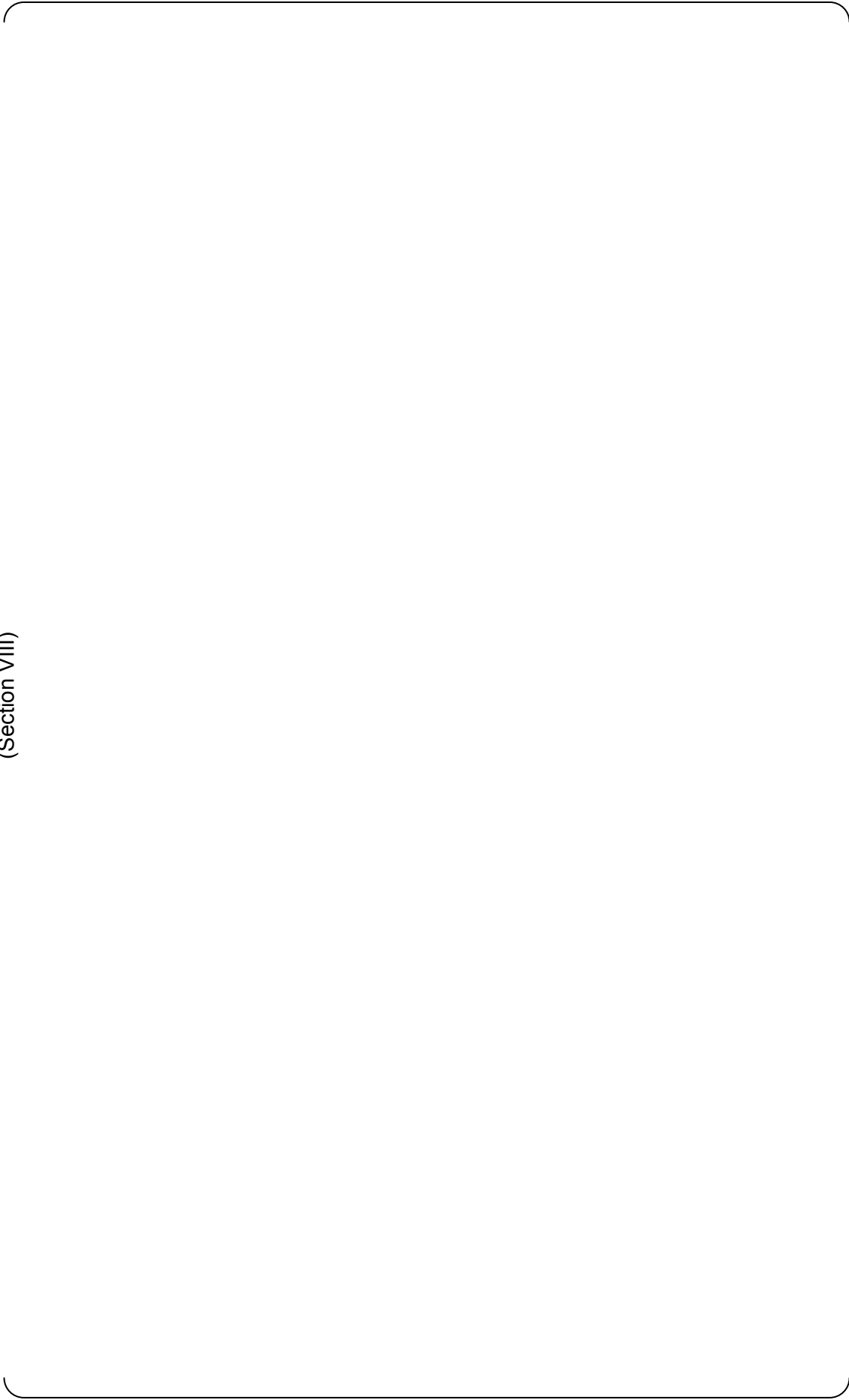


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (24/39)  
(Section VIII)




**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (25/39)  
(Section IX)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (26/39)  
(Section IX)

| Location | $\Delta T1$ | $\Delta T2$ | $Ta-Tb$ |
|----------|-------------|-------------|---------|
|----------|-------------|-------------|---------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (27/39)  
(Section IX)

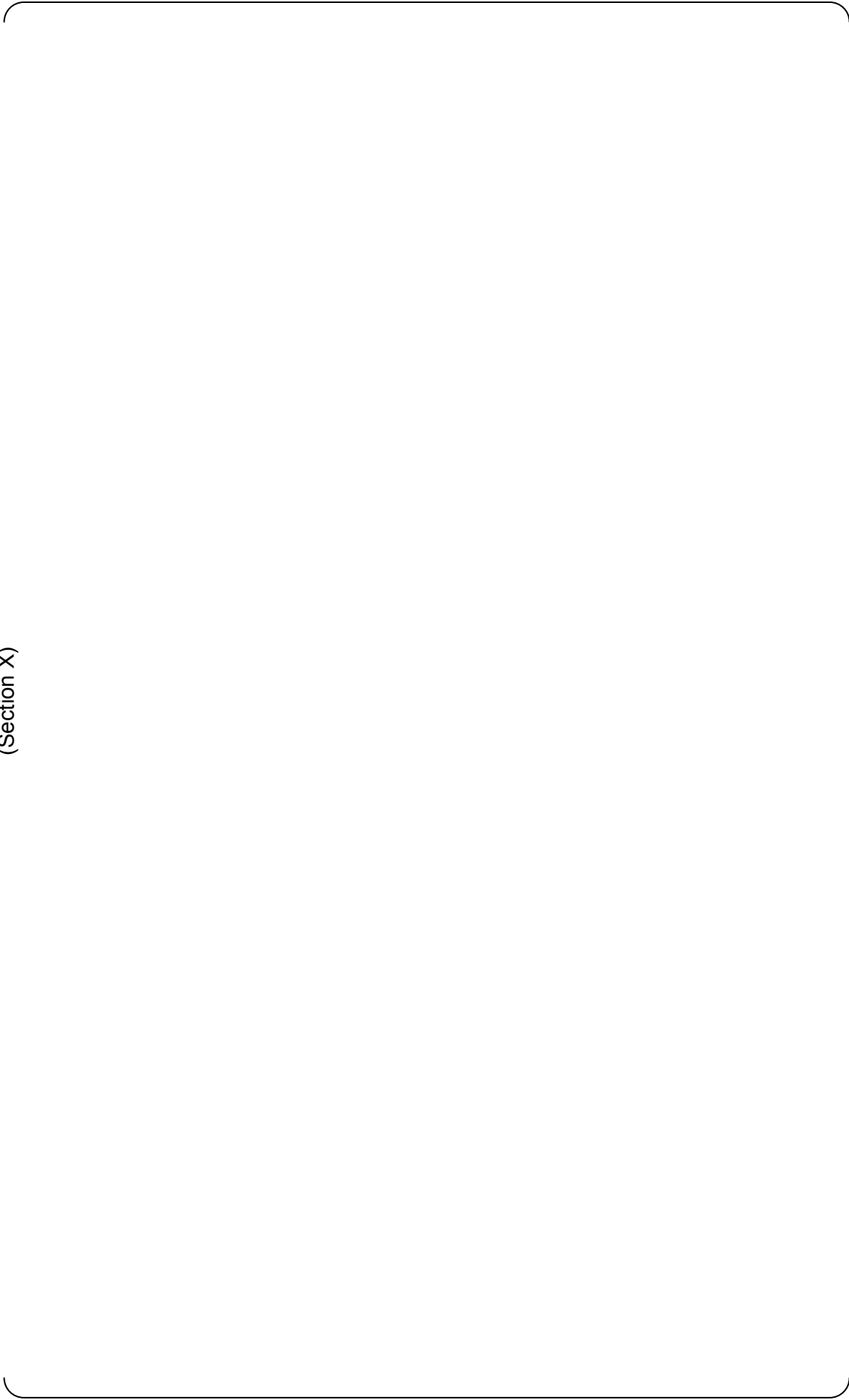


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (28/39)  
(Section X)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





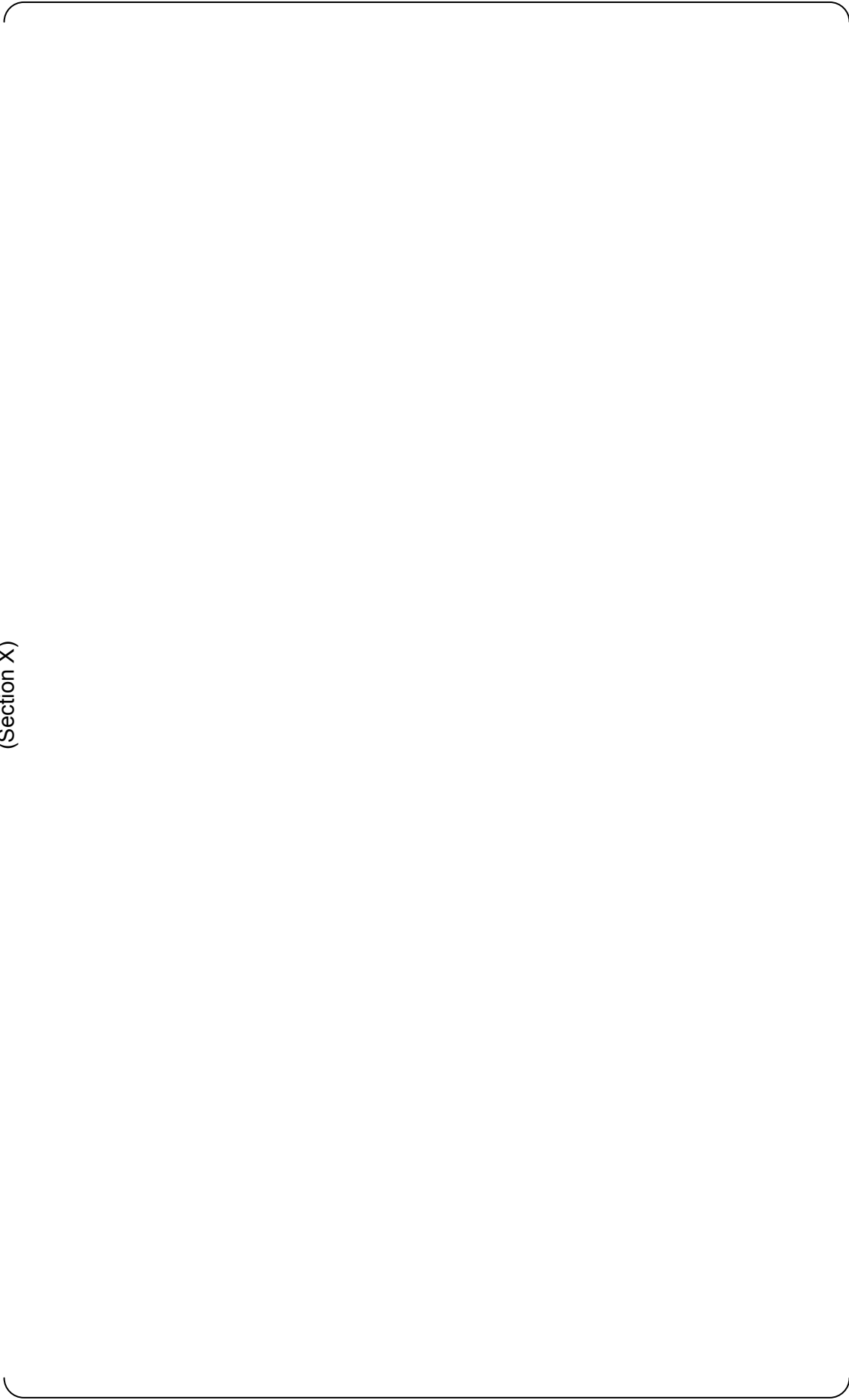


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (30/39)  
(Section X)

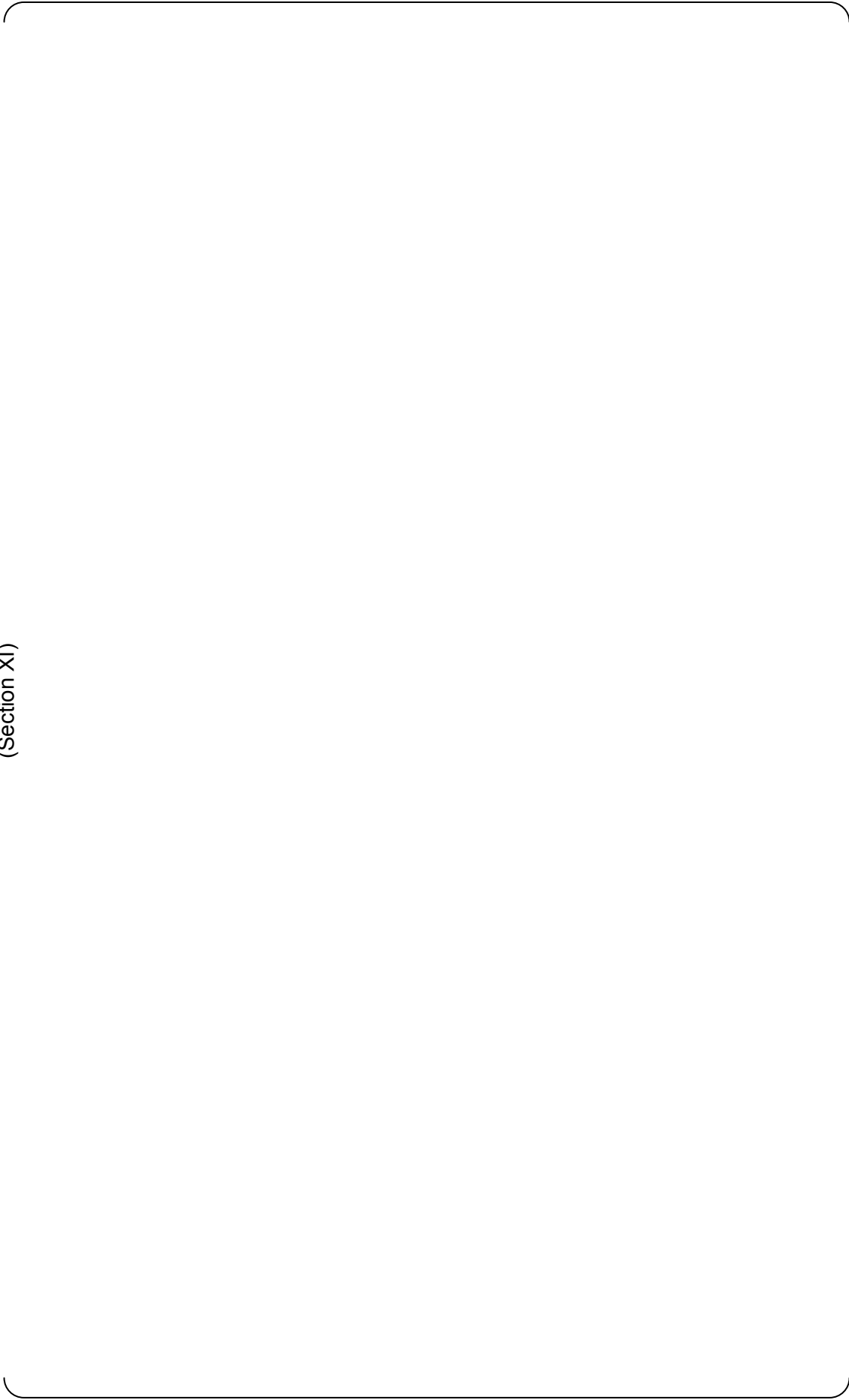


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (31/39)  
(Section XI)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (33/39)  
(Section XI)

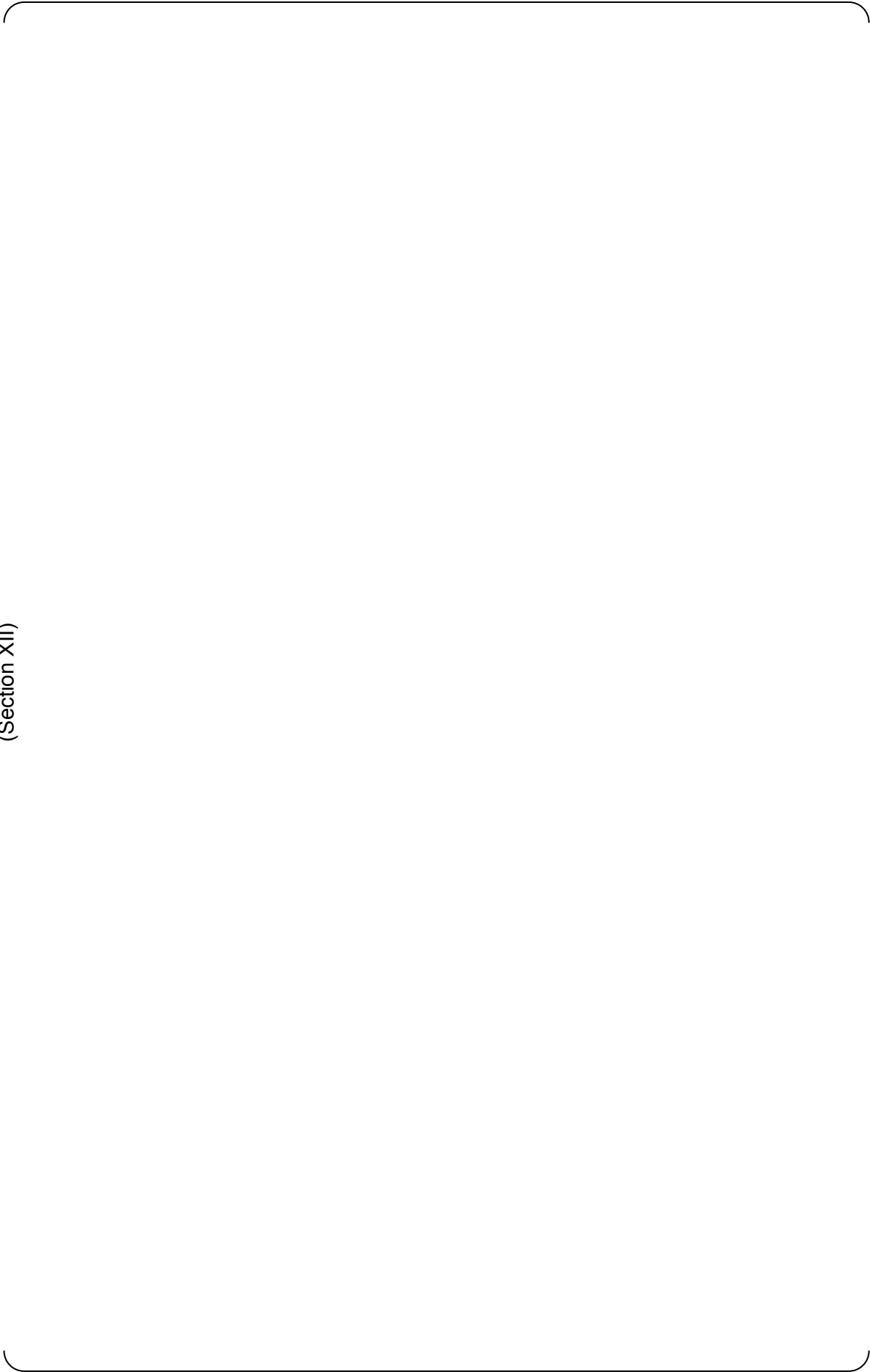


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb)(34/39)  
(Section XII)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb)(35/39)  
(Section XII)

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**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (36/39)  
(Section XI)

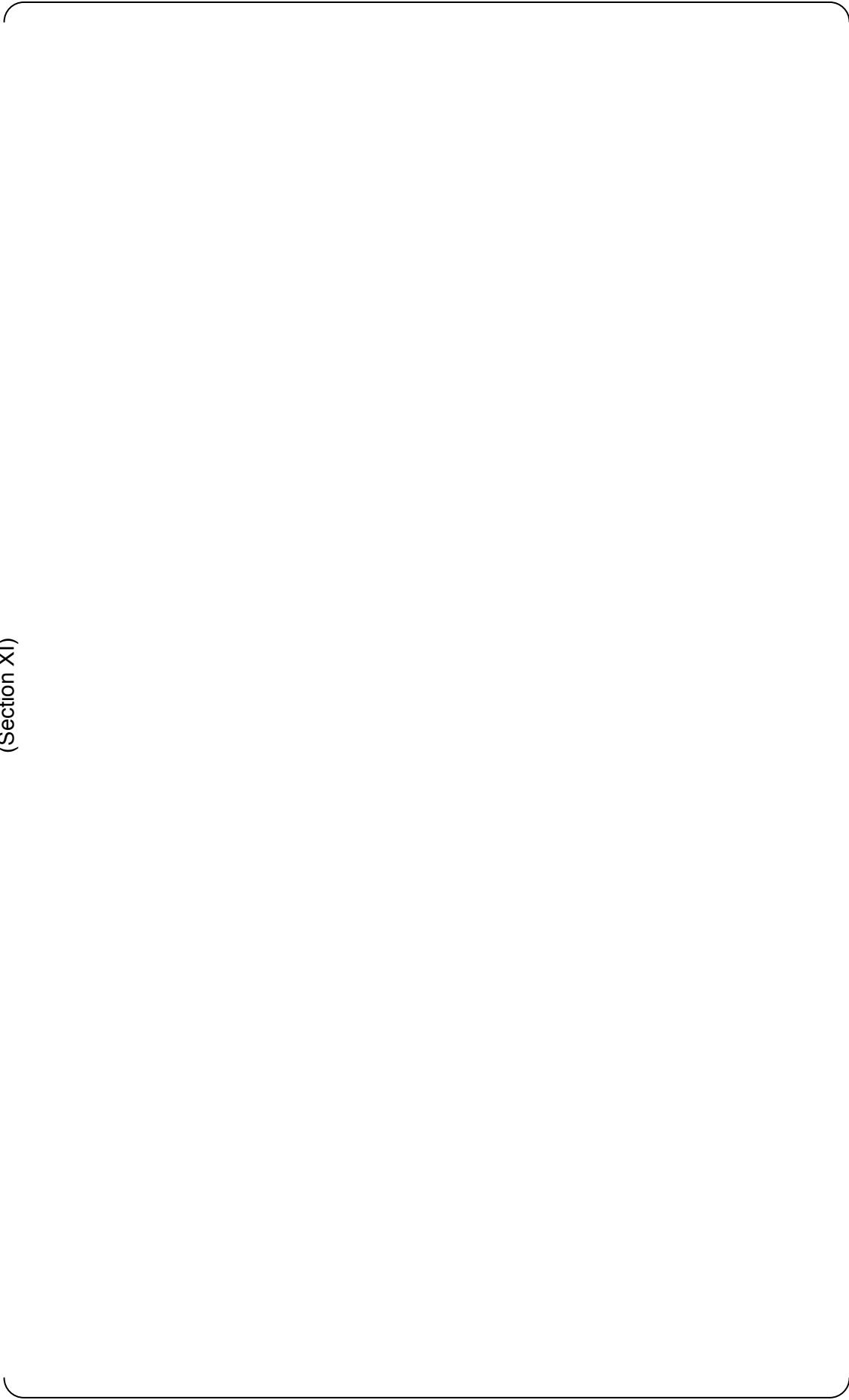
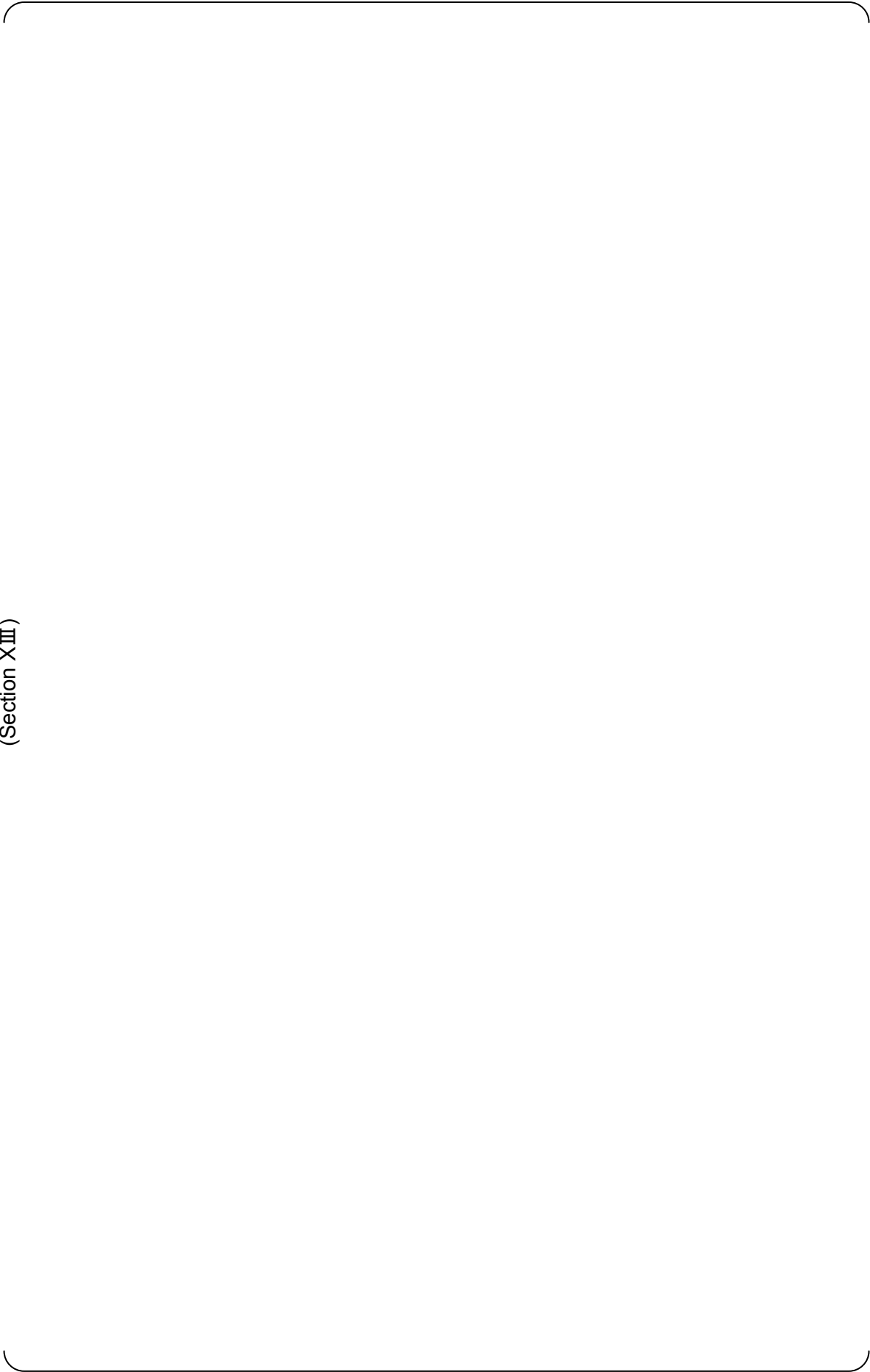






Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (37/39)  
(Section XIII)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (38/39)  
(Section XIII)

| Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ , $\Delta T2$ , Ta-Tb) (38/39)<br>(Section XIII) |
|--|
|--|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-2-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (39/39)  
(Section XI)





**Table A1-2-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-2-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)



## Appendix 1-3

### RC03 Pressurizer Safety Depressurization Valve Line Piping Analysis Results

1. INPUT

|   |                 |
|---|-----------------|
| 1.1 Used for creating the pipe structural model                       |                 |
| 1.1.1 Block division and piping specifications                        | Table A1-3-1-1  |
| 1.1.2 Piping isometrics   | Figure A1-3-1-1 |
| 1.1.3 Concentrated mass   | Table A1-3-1-2  |
| 1.1.4 Support point rigidity  | Table A1-3-1-3  |
| 1.1.5 Valve rigidity  | Table A1-3-1-4  |
| 1.2 Used for creating load conditions                                 |                 |
| 1.2.1 Level A/B design transient                                      | see main text   |
| 1.2.2 Level A/B thermal displacement input data                       | Table A1-3-1-5  |
| 1.2.3 Level A, B temperature and pressure input data                  | Table A1-3-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data          | Table A1-3-1-7  |
| 1.2.5 Floor response curve  | Figure A1-3-1-2 |
| 1.2.6 Seismic anchor displacement input data                          | Table A1-3-1-8  |
| 1.2.7 DBPB displacement input data                                    | Table A1-3-1-9  |
| 1.2.8 Initial condition and valve open characteristics (Water hammer) | Table A1-3-1-10 |

2. OUTPUT

|  |                 |
|--|-----------------|
| 2.1 PIPESTRESS analysis model diagram                                | Figure A1-3-2-1 |
| 2.2 Water hammer analysis model diagram                              | Figure A1-3-2-2 |
| 2.3 Natural frequency analysis results                               | Table A1-3-2-1  |
| 2.4 Frequency mode diagram (primary to tertiary)                     | Figure A1-3-2-3 |
| 2.5 Thermal analysis results ( $\Delta T1$ , $\Delta T2$ , $Ta-Tb$ ) | Table A1-3-2-2  |
| 2.6 Piping stress and fatigue evaluation results                     | Table A1-3-2-3  |

Table A1-3-1-1 Block division and piping specifications (1/3)

Table A1-3-1-1 Block division and piping specifications (2/3)

**Table A1-3-1-1 Block division and piping specifications (3/3)**

\*1 CIL: Capsule insulation containing lead , RMI: Reflective metal insulation , FI: Fibrous insulation

\*2 Cladding weight is included in the insulation weight



Figure A1-3-1-1 Piping isometrics(1/3)

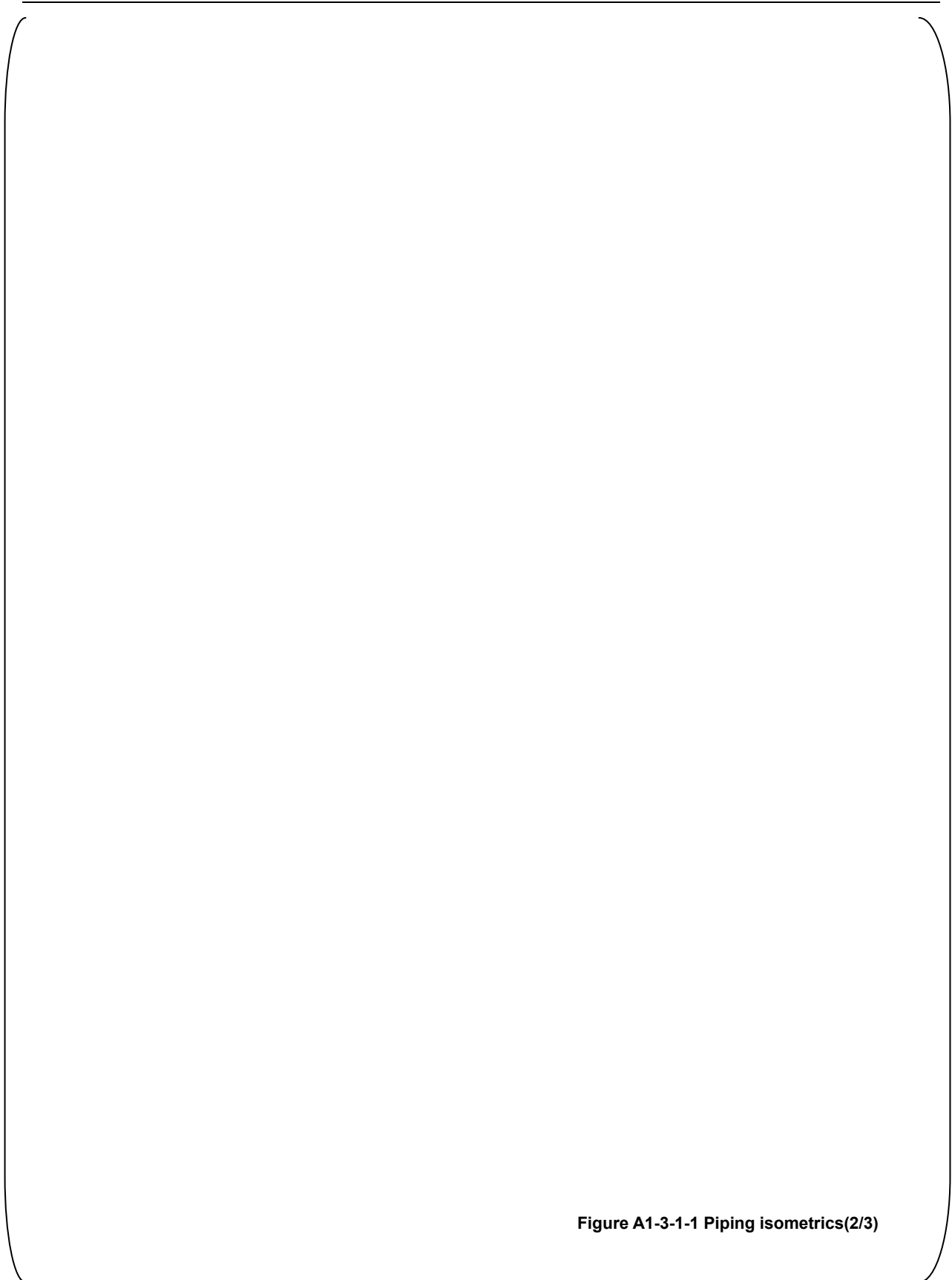


Figure A1-3-1-1 Piping isometrics(2/3)

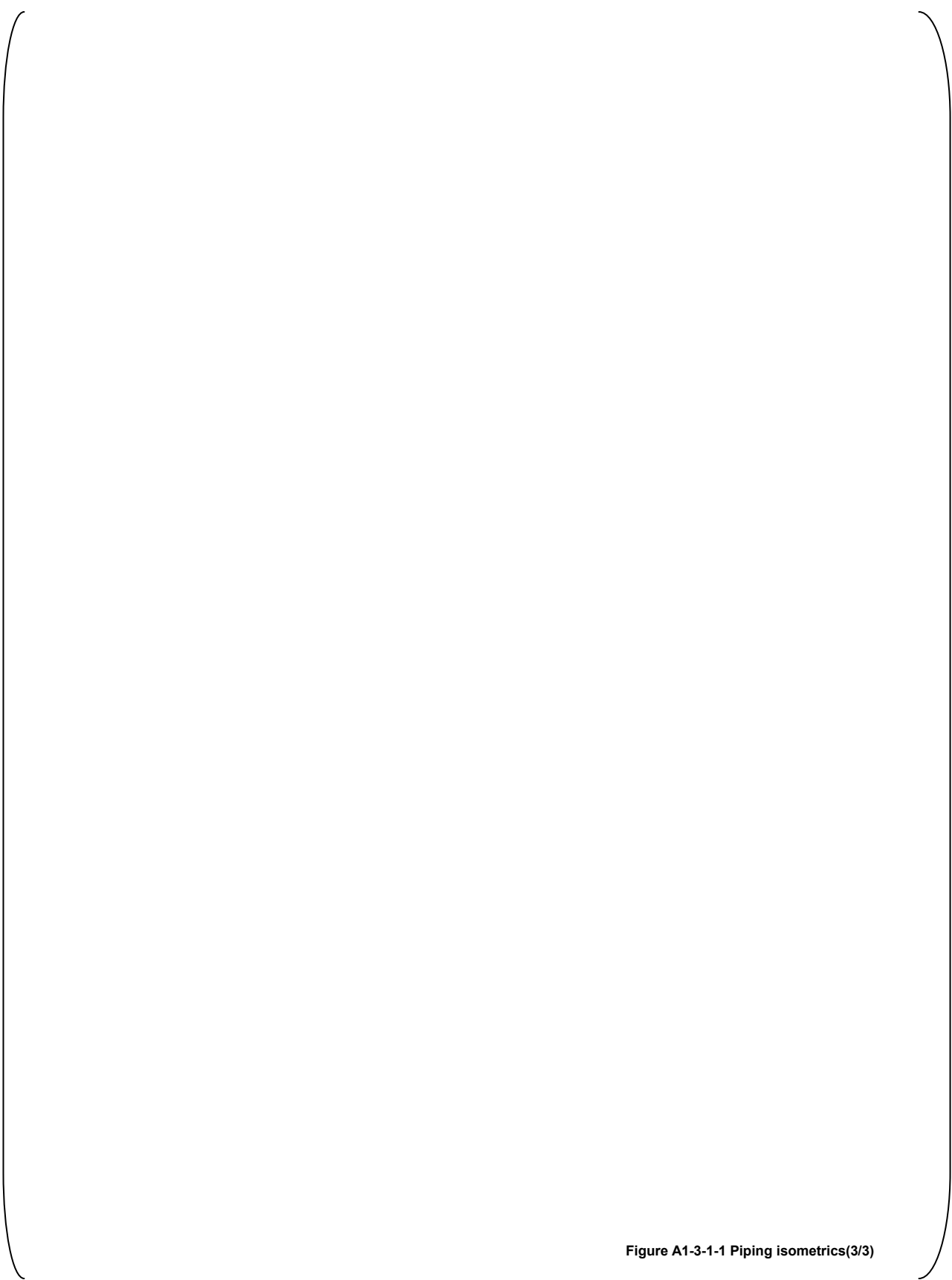


Figure A1-3-1-1 Piping isometrics(3/3)



Table A1-3-1-2 Concentrated mass

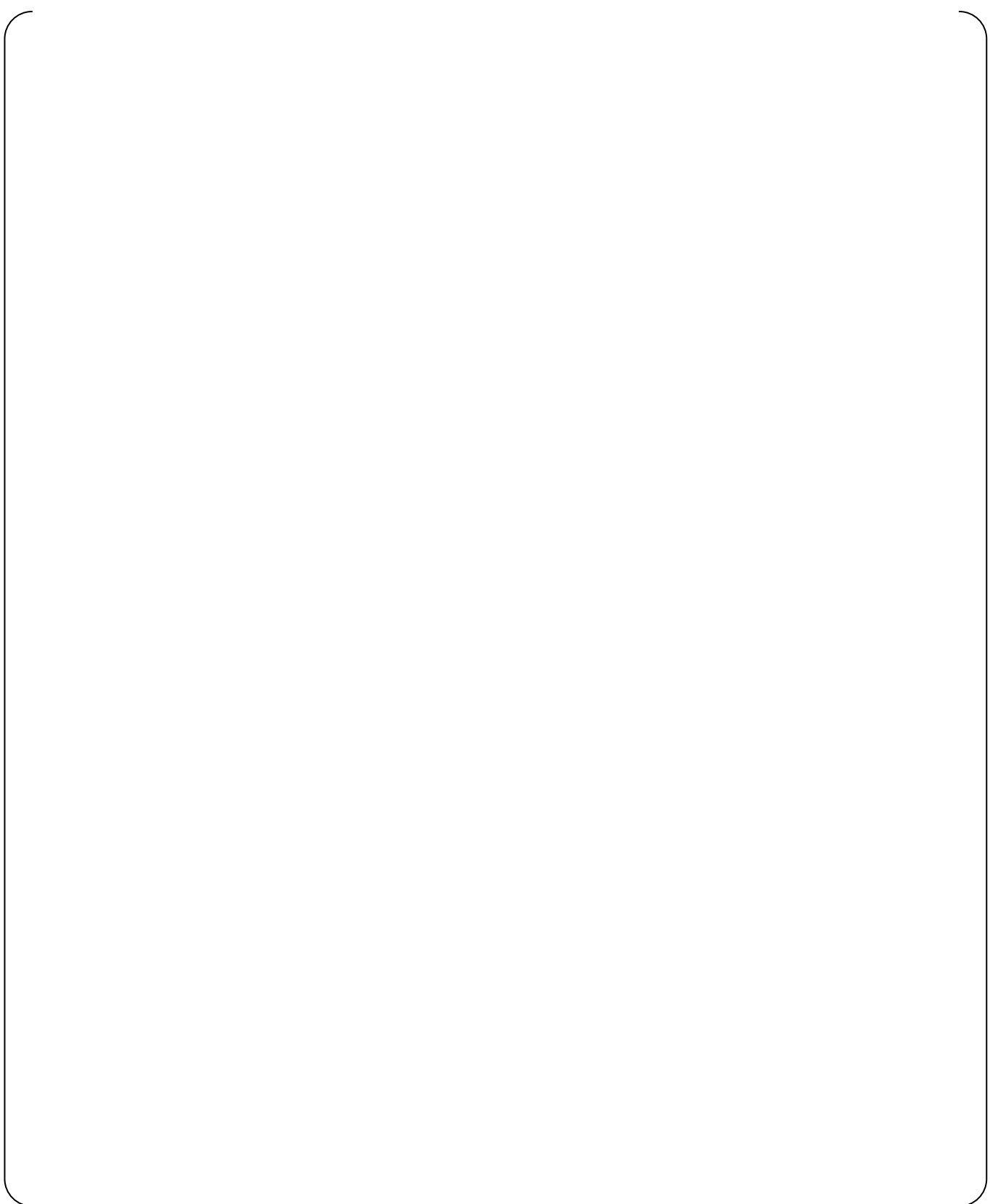








Table A1-3-1-3 Support point rigidity







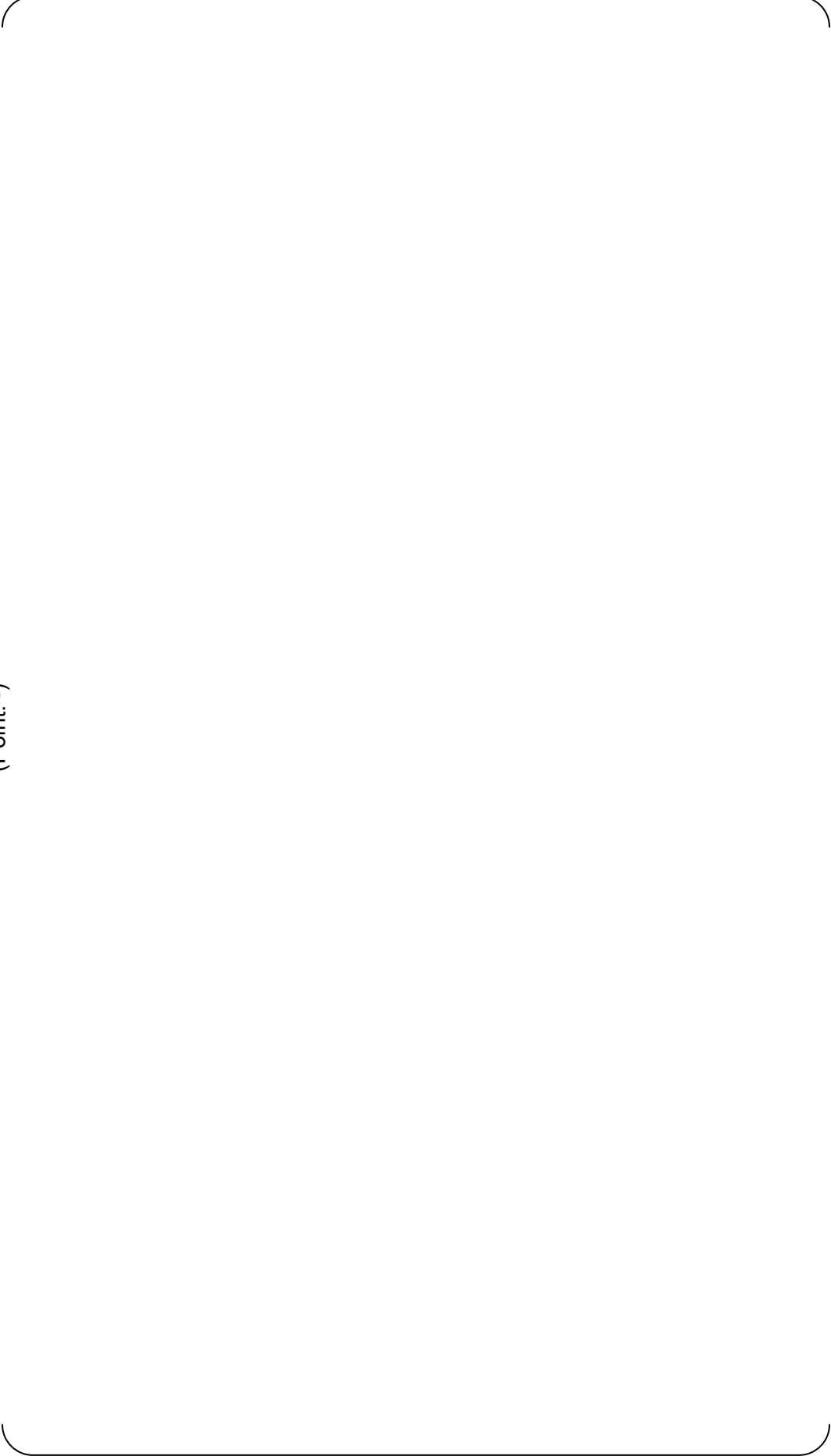






Table A1-3-1-4 Valve rigidity

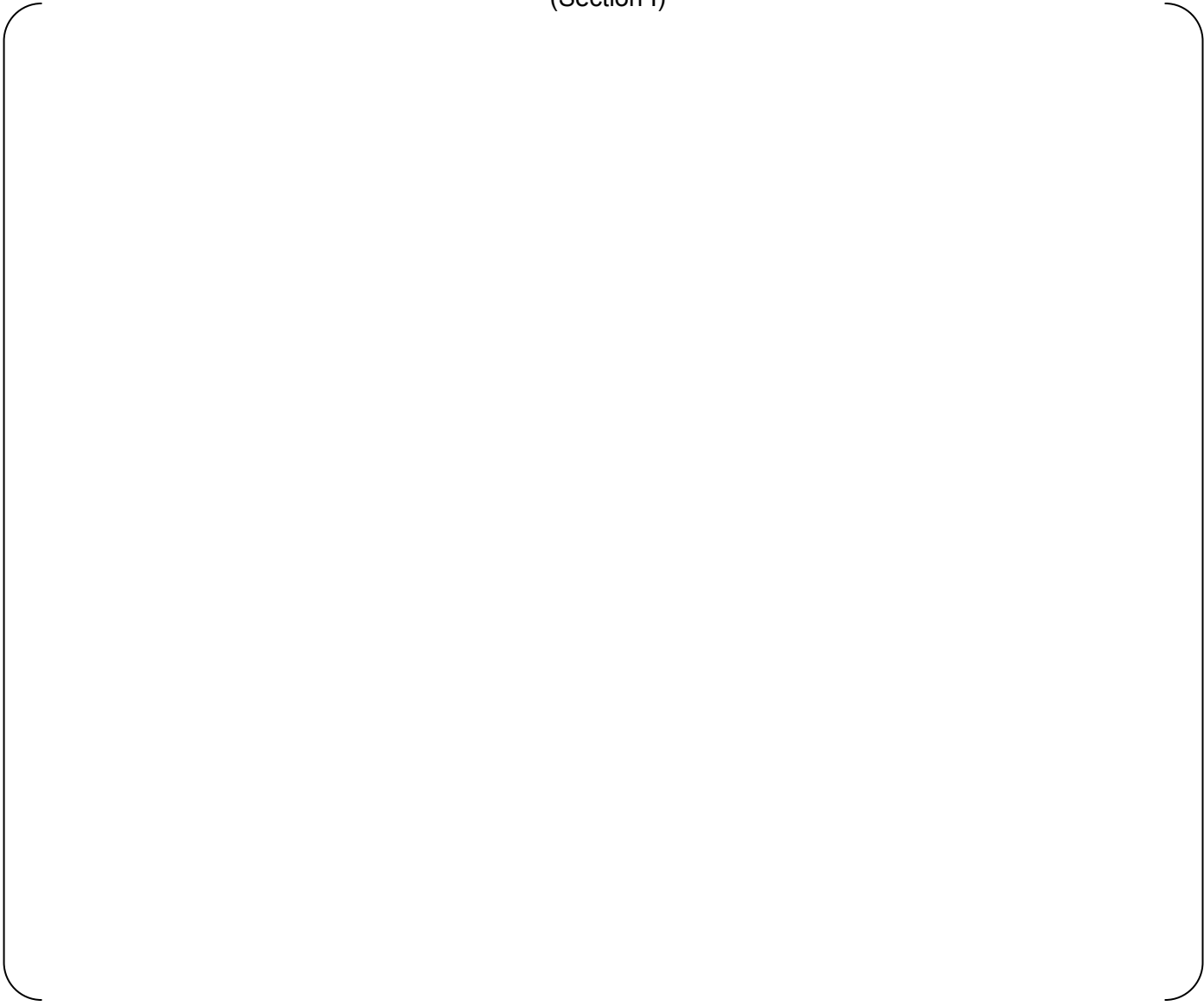
Table A1-3-1-5 Level A/B thermal displacement input data  
(Point: -)



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**Table A1-3-1-6 Level A, B temperature and pressure input data (1/21)**  
(Section I)

Table A1-3-1-6 Level A, B temperature and pressure input data (2/21)  
(Section I)



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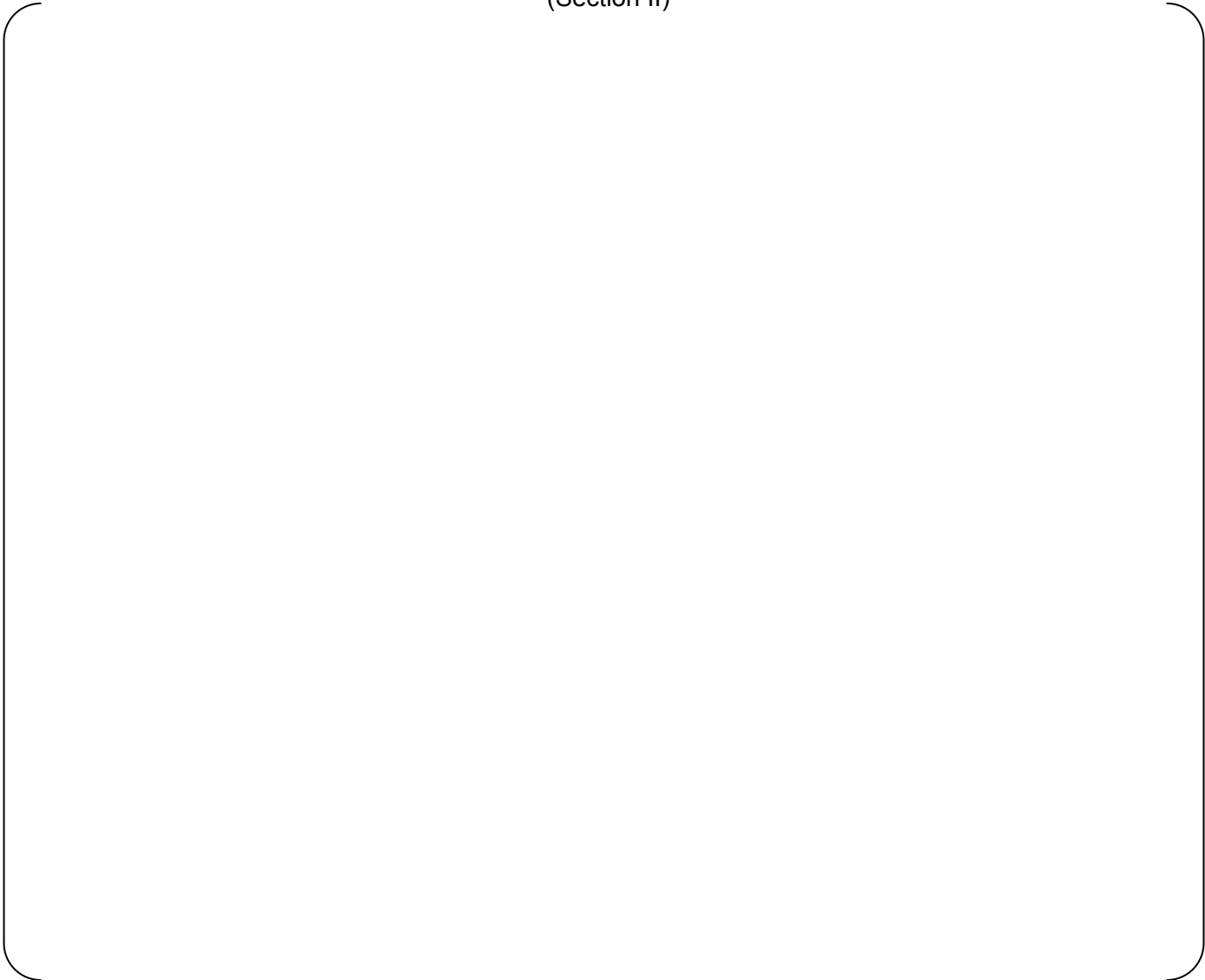
Table A1-3-1-6 Level A, B temperature and pressure input data (3/21)  
(Section I)



Table A1-3-1-6 Level A, B temperature and pressure input data (4/21)  
(Section II)



Table A1-3-1-6 Level A, B temperature and pressure input data (5/21)  
(Section II)



**Table A1-3-1-6 Level A, B temperature and pressure input data (6/21)  
(Section II)**

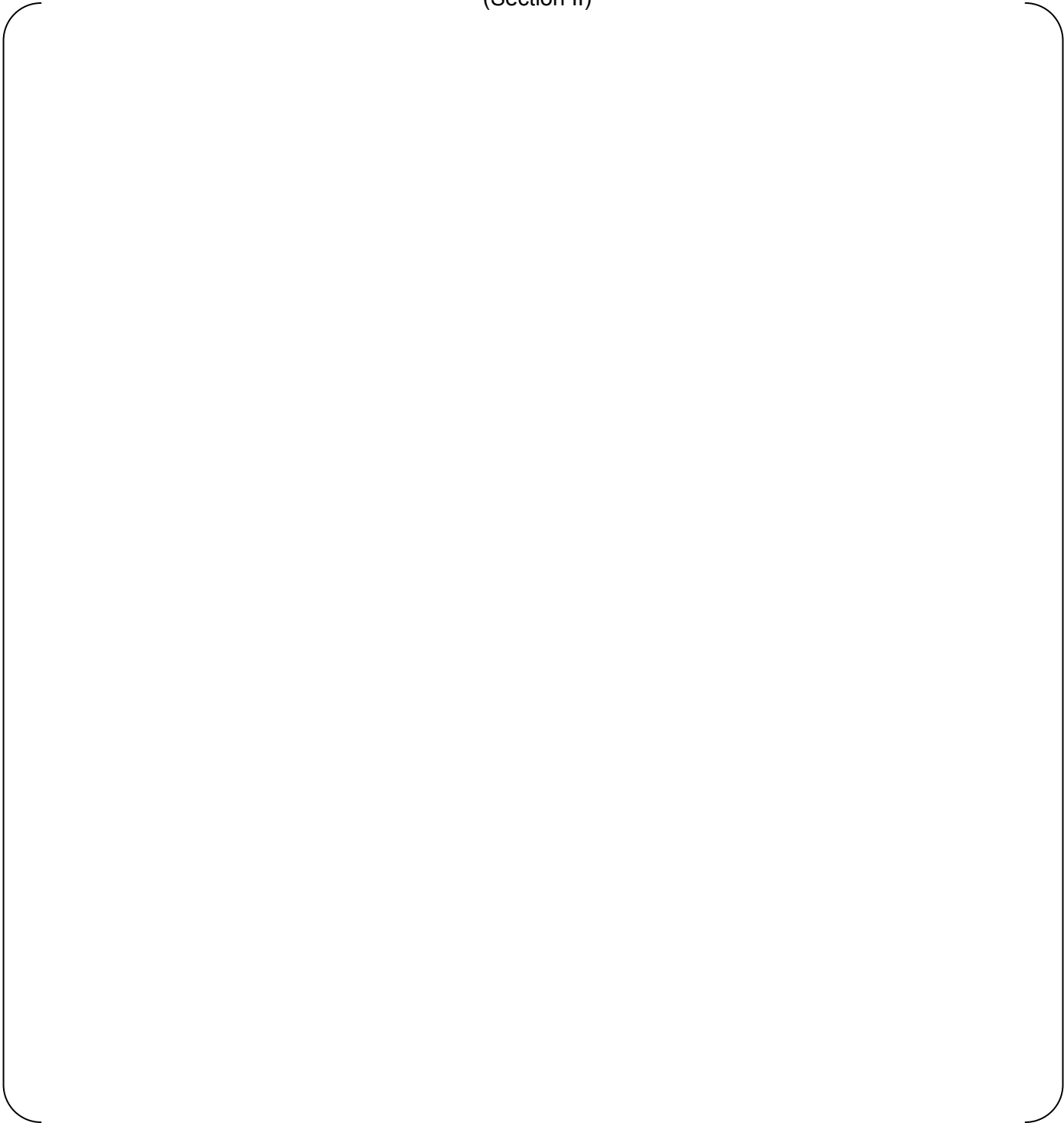
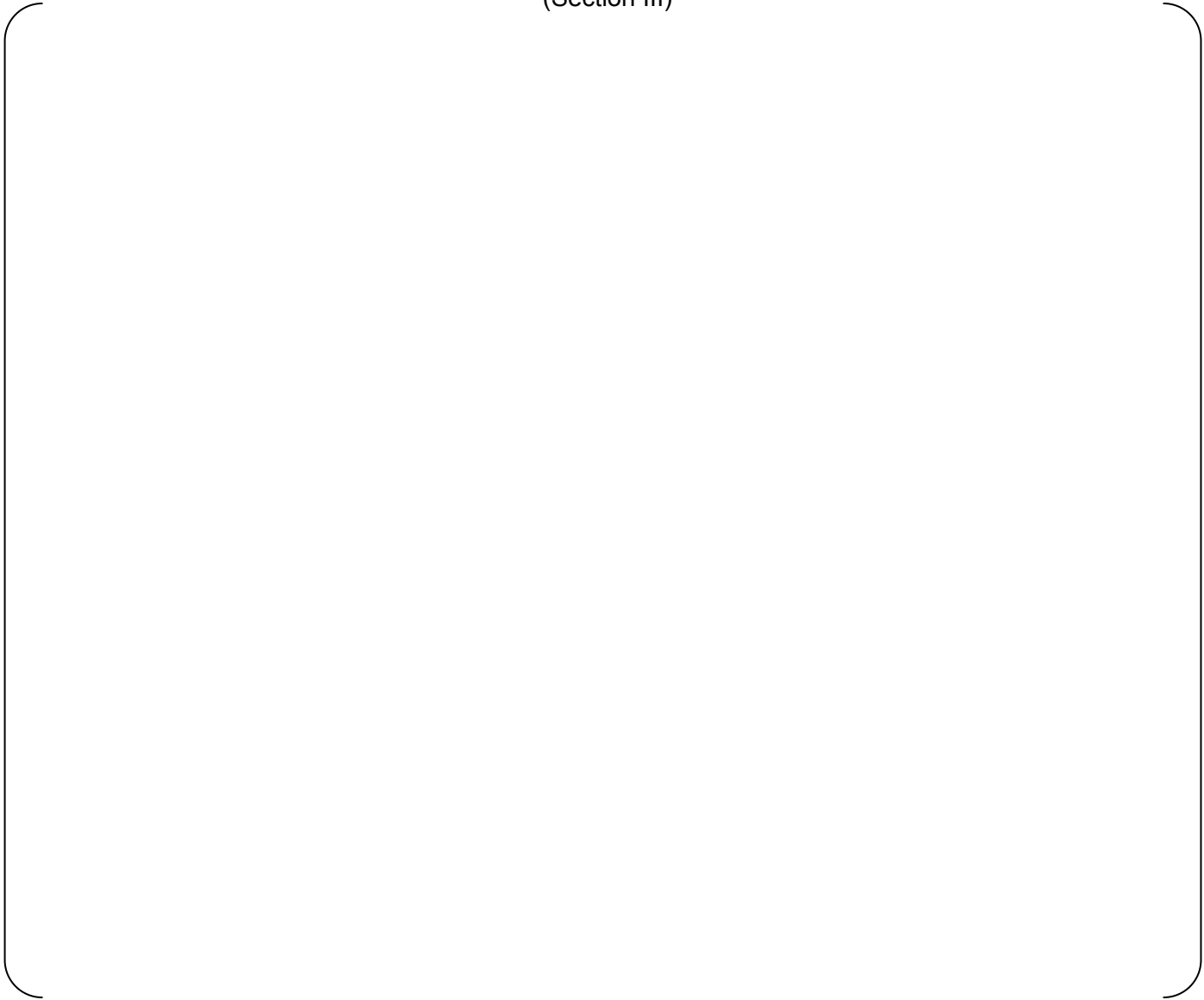


Table A1-3-1-6 Level A, B temperature and pressure input data (7/21)  
(Section III)



Table A1-3-1-6 Level A, B temperature and pressure input data (8/21)  
(Section III)



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Table A1-3-1-6 Level A, B temperature and pressure input data (9/21)  
(Section III)

Table A1-3-1-6 Level A, B temperature and pressure input data (10/21)  
(Section IV)



Table A1-3-1-6 Level A, B temperature and pressure input data (11/21)  
(Section IV)

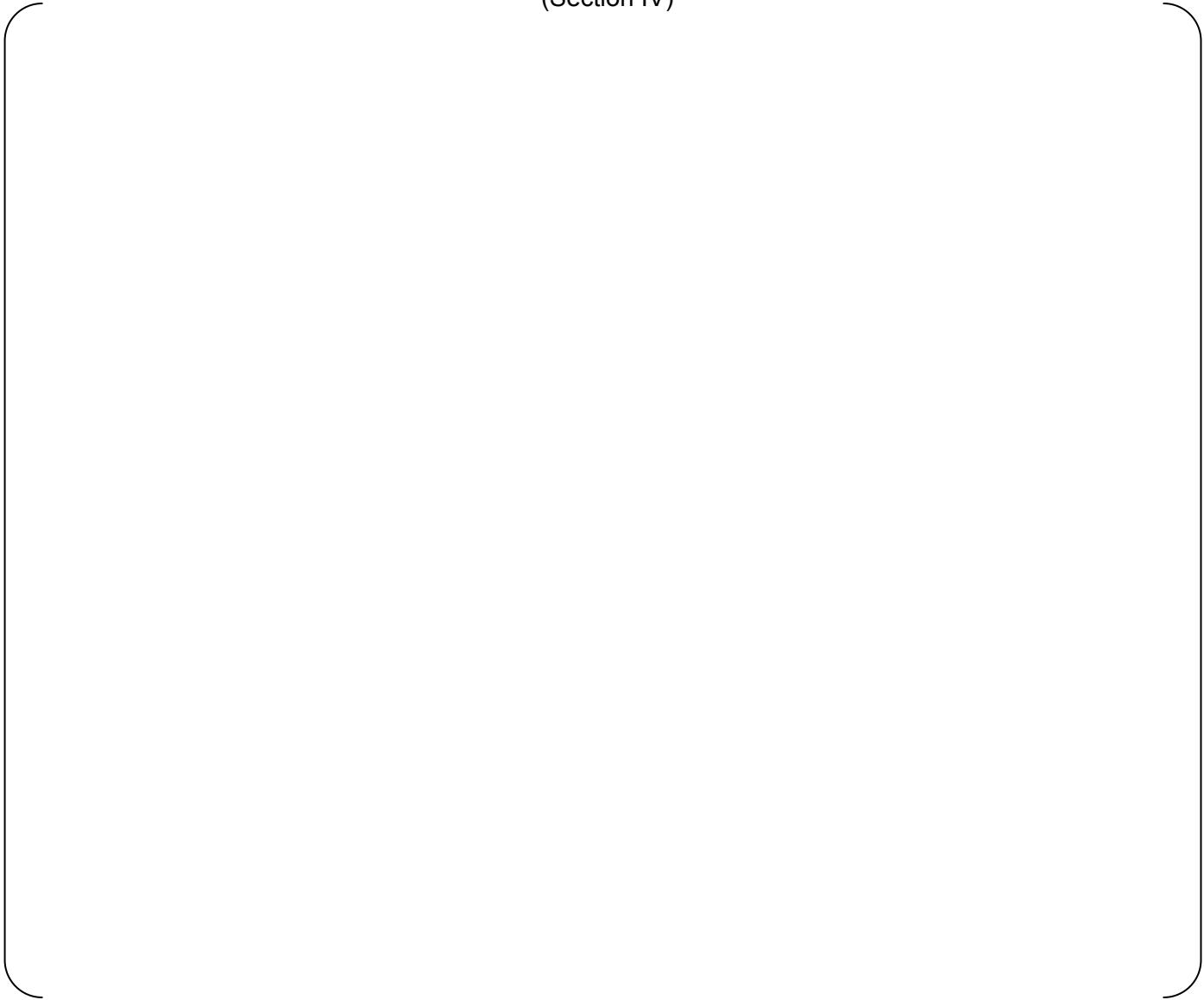




Table A1-3-1-6 Level A, B temperature and pressure input data (12/21)  
(Section IV)

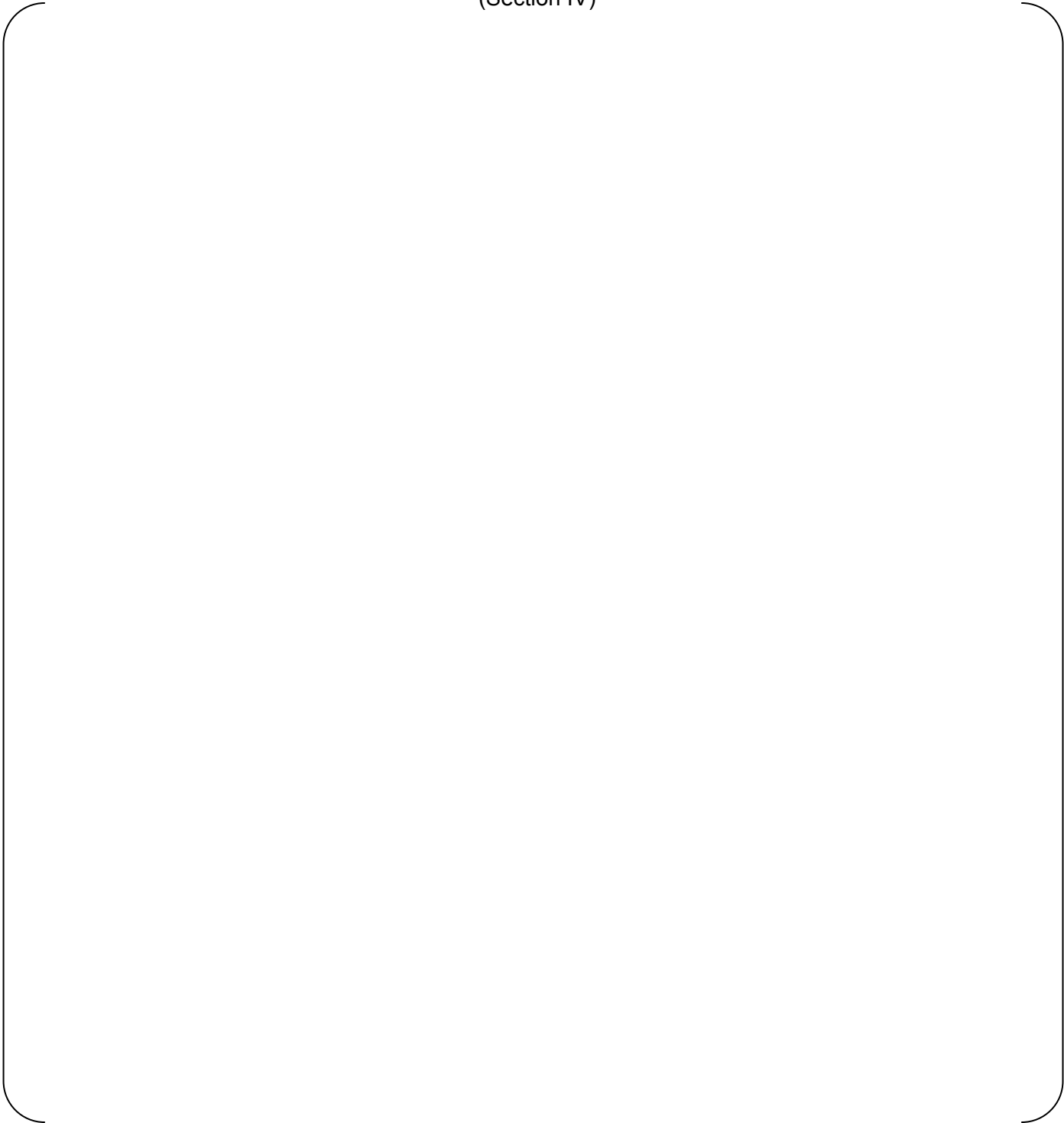
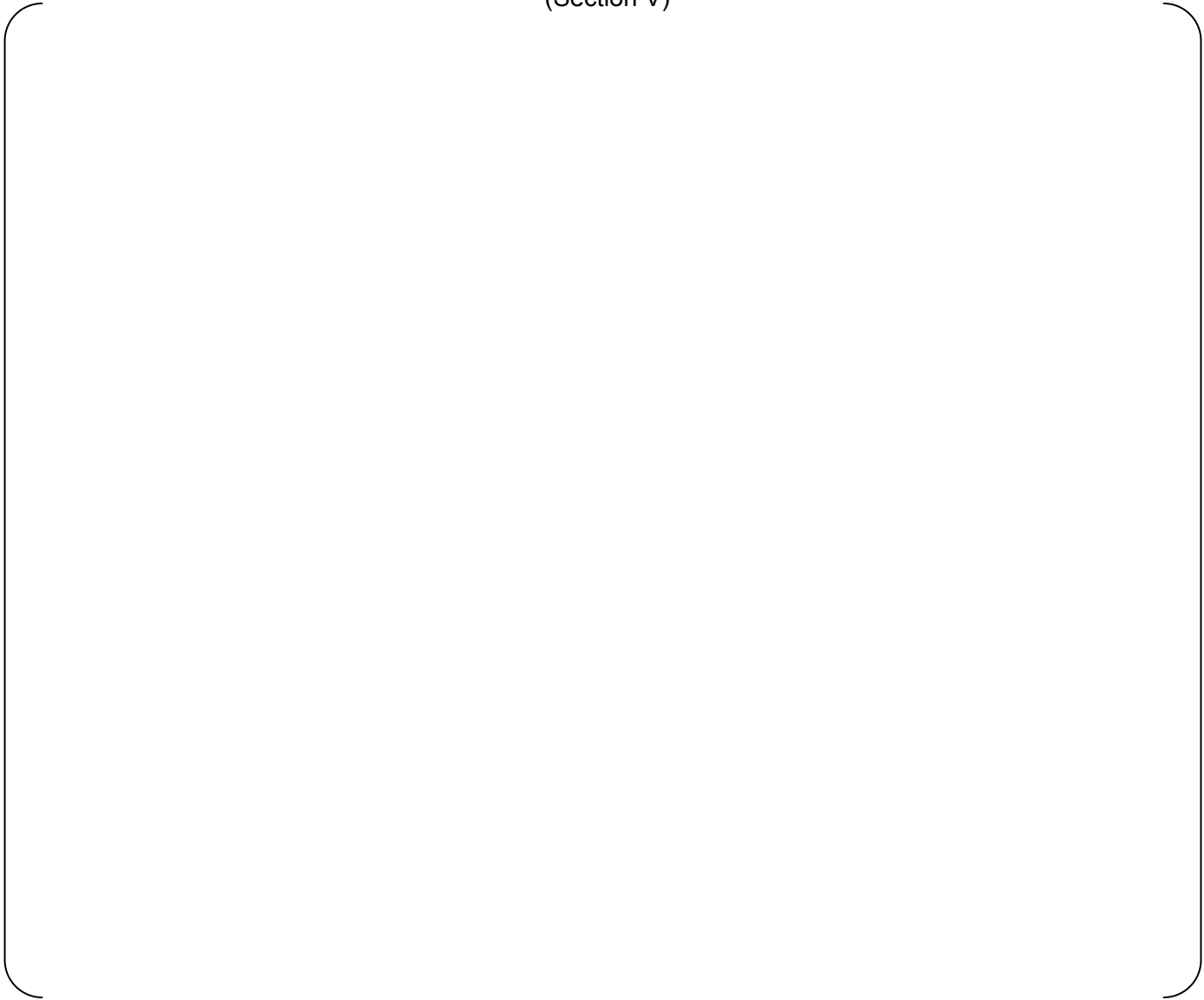


Table A1-3-1-6 Level A, B temperature and pressure input data (13/21)  
(Section V)

Table A1-3-1-6 Level A, B temperature and pressure input data (14/21)  
(Section V)



**Table A1-3-1-6 Level A, B temperature and pressure input data (15/21)  
(Section V)**

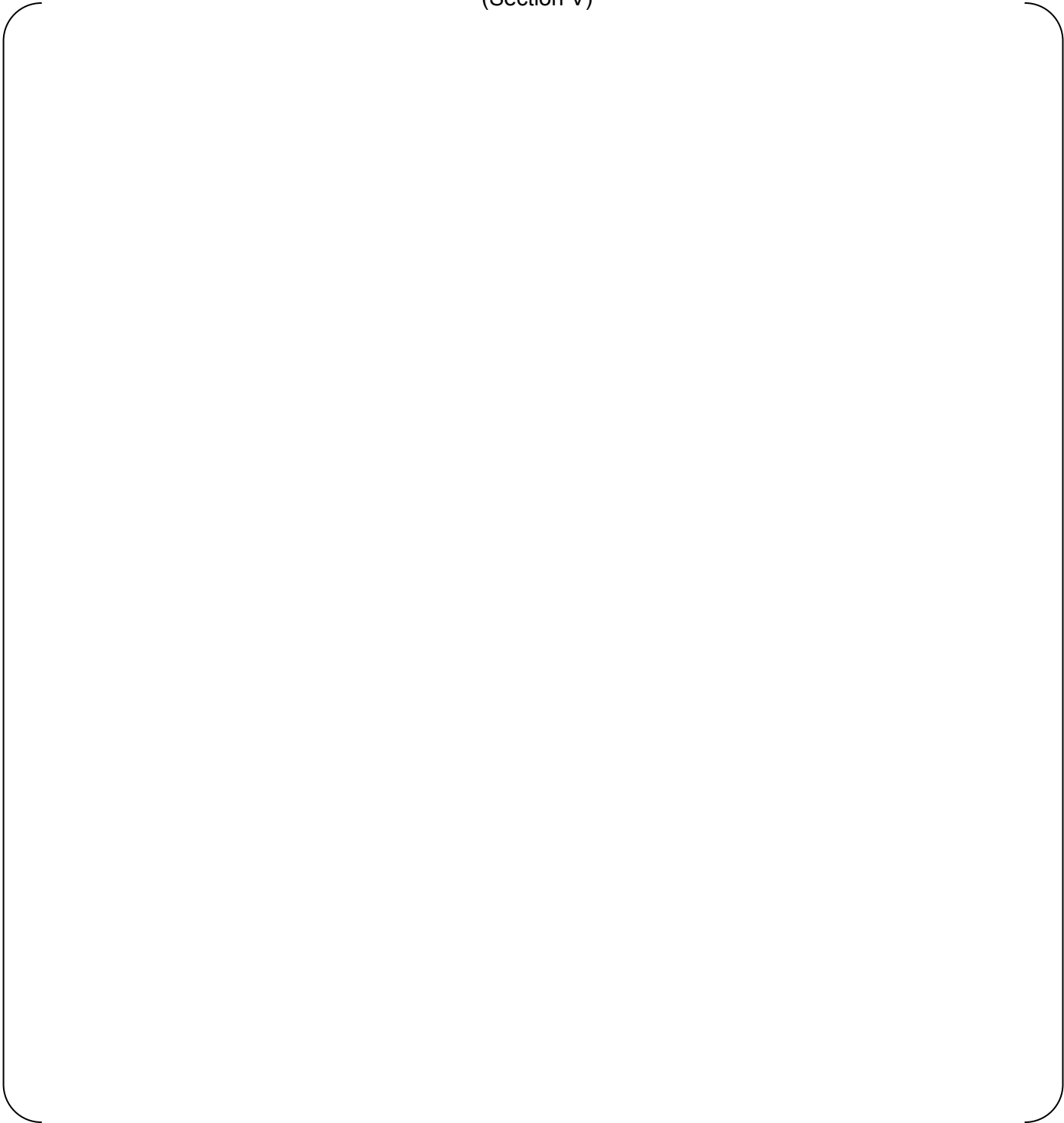


Table A1-3-1-6 Level A, B temperature and pressure input data (16/21)  
(Section VI)

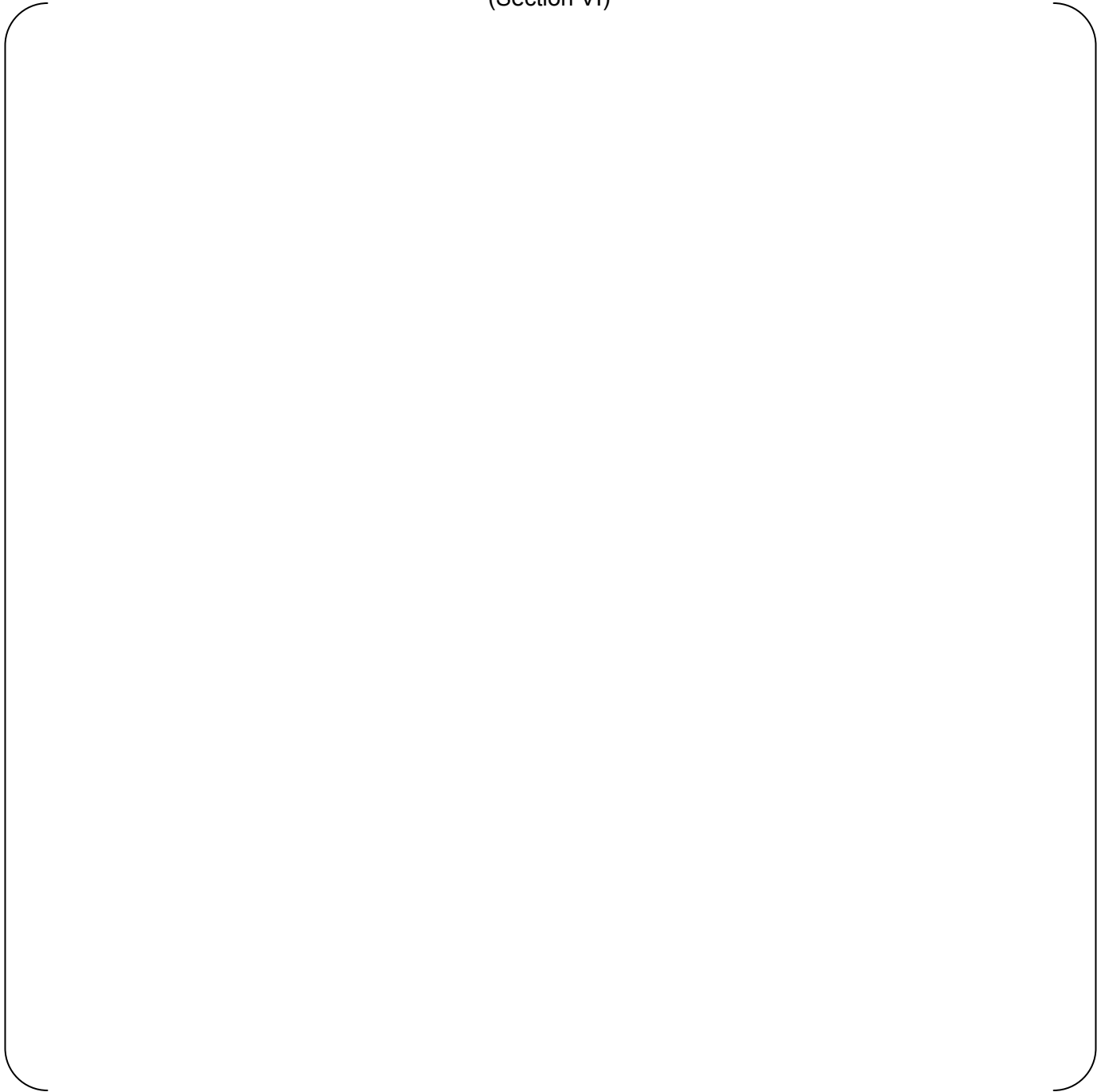
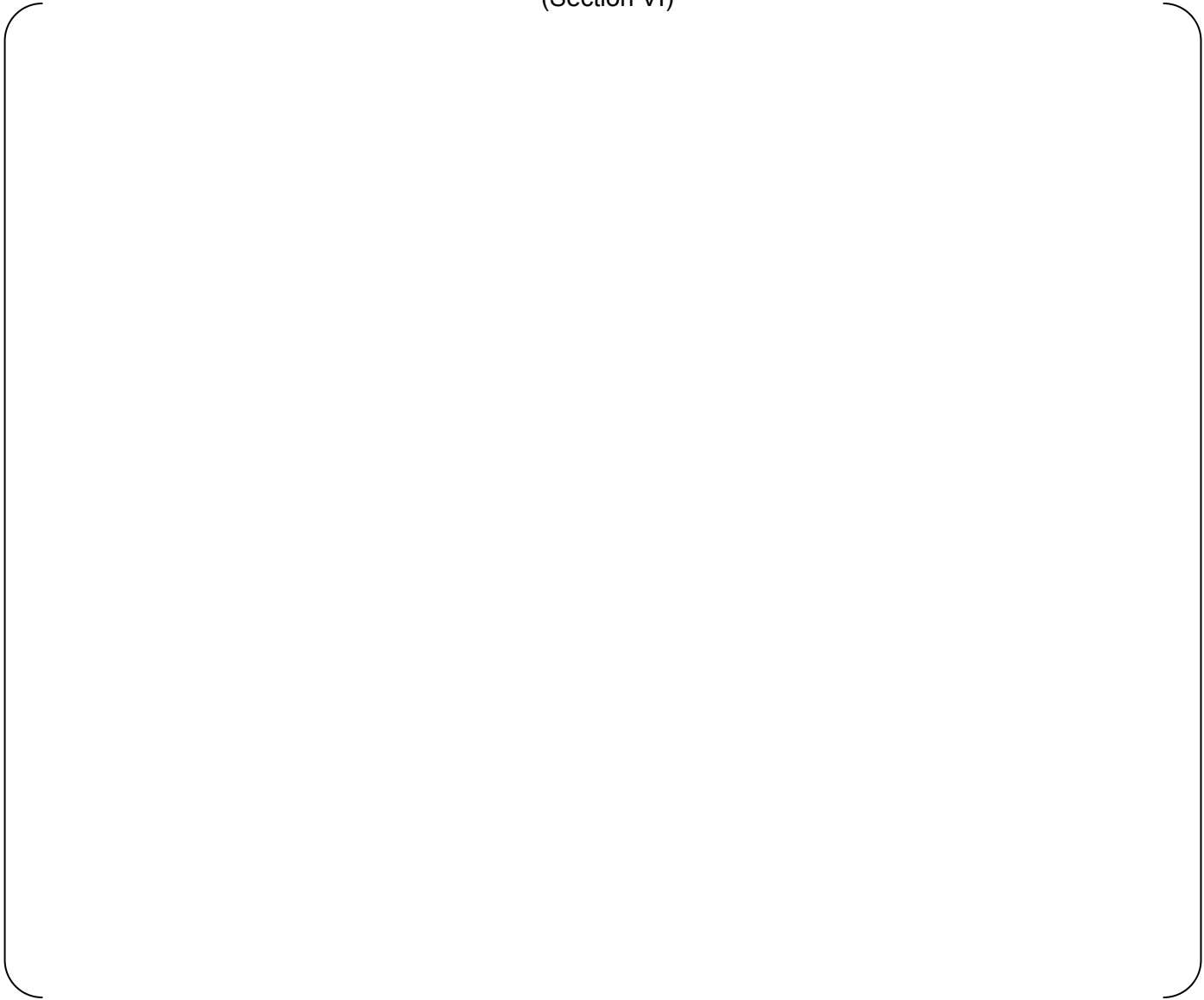


Table A1-3-1-6 Level A, B temperature and pressure input data (17/21)  
(Section VI)



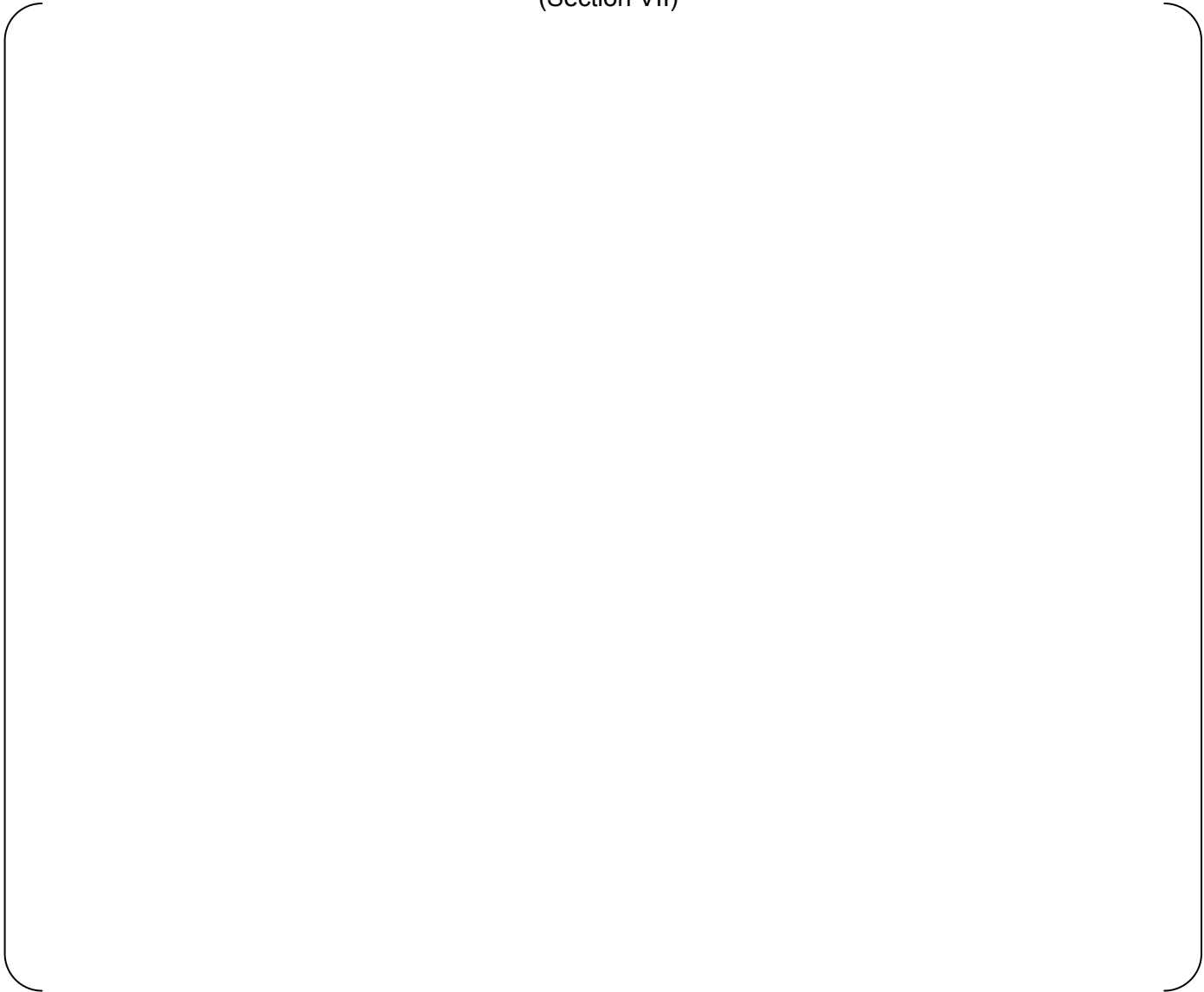
**Table A1-3-1-6 Level A, B temperature and pressure input data (18/21)  
(Section VI)**

**Table A1-3-1-6 Level A, B temperature and pressure input data (19/21)  
(Section VII)**





Table A1-3-1-6 Level A, B temperature and pressure input data (20/21)  
(Section VII)



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Table A1-3-1-6 Level A, B temperature and pressure input data (21/21)  
(Section VII)

Table A1-3-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-3-1-2 Floor response curve (1/9)**  
Pressurizer Safety Depressurization Valve Line (RC03) FRS for Pressurizer base plate  
X (EW) direction (damping 3.0%)



**Figure A1-3-1-2 Floor response curve (2/6)**  
Pressurizer Safety Depressurization Valve Line (RC03) FRS for Pressurizer base plate  
Y (NS) direction (damping 3.0%)



**Figure A1-3-1-2 Floor response curve (3/9)**  
Pressurizer Safety Depressurization Valve Line (RC03) FRS for Pressurizer base plate  
Z (Vert.) direction (damping 3.0%)



**Figure A1-3-1-2 Floor response curve (4/9)**  
Pressurizer Safety Depressurization Valve Line (RC03) FRS for Pressurizer support  
X (EW) direction (damping 3.0%)



**Figure A1-3-1-2 Floor response curve (5/9)**  
Pressurizer Safety Depressurization Valve Line (RC03) FRS for Pressurizer support  
Y (NS) direction (damping 3.0%)





**Figure A1-3-1-2 Floor response curve (6/9)**  
Pressurizer Safety Depressurization Valve Line (RC03) FRS for Pressurizer support  
Z (Vert.) direction (damping 3.0%)



**Figure A1-3-1-2 Floor response curve (7/9)**  
Pressurizer Safety Depressurization Valve Line (RC03) FRS for Piping  
X (EW) direction (damping 3.0%)



**Figure A1-3-1-2 Floor response curve (8/9)**  
Pressurizer Safety Depressurization Valve Line (RC03) FRS for Piping  
Y (NS) direction (damping 3.0%)



**Figure A1-3-1-2 Floor response curve (9/9)**  
Pressurizer Safety Depressurization Valve Line (RC03) FRS for Piping  
Z (Vert.) direction (damping 3.0%)

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Table A1-3-1-8 Seismic anchor displacement input data



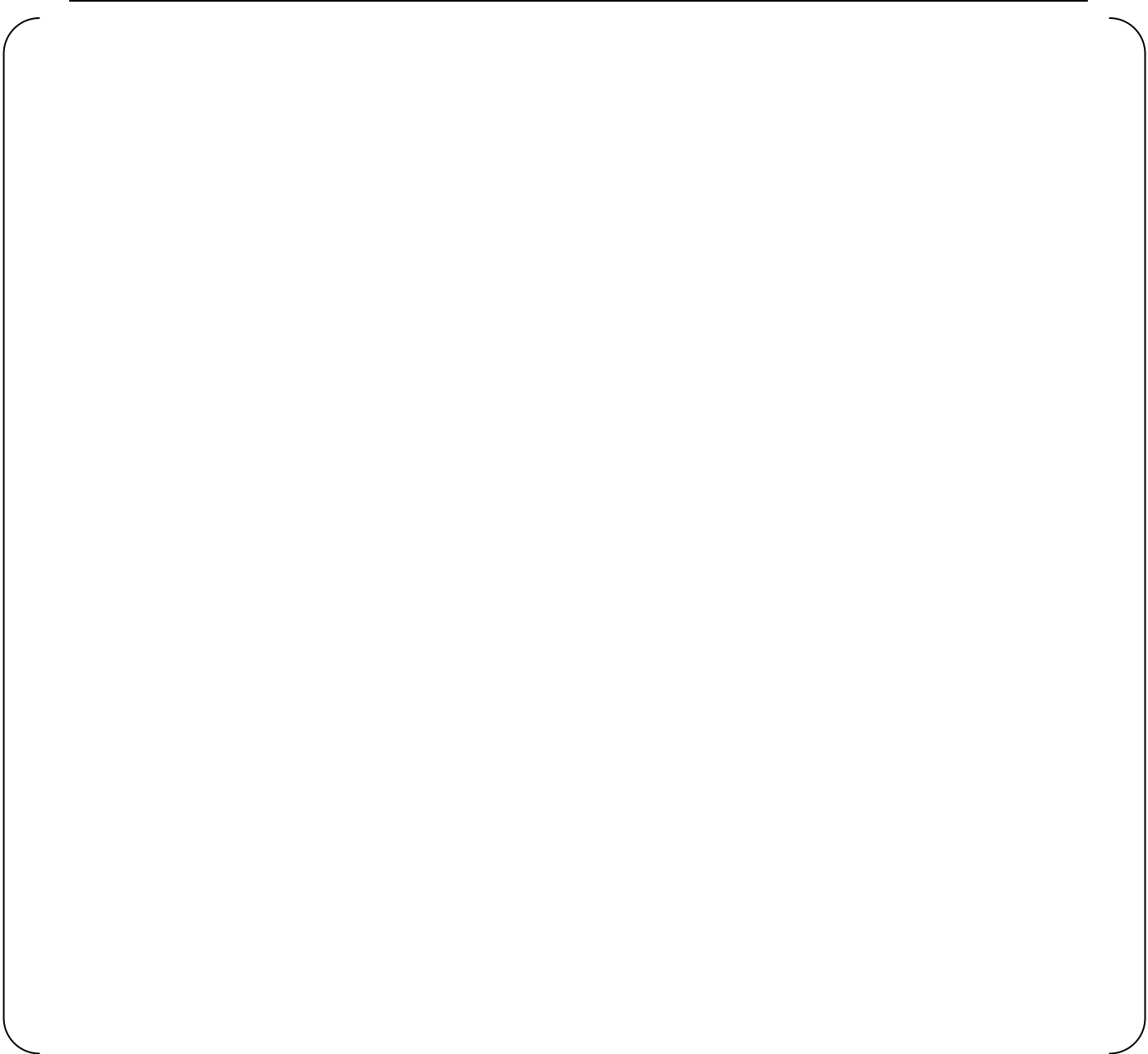


Table A1-3-1-9 DBPB displacement input data



Table A1-3-1-10 Initial condition and valve open characteristics (Water hammer)



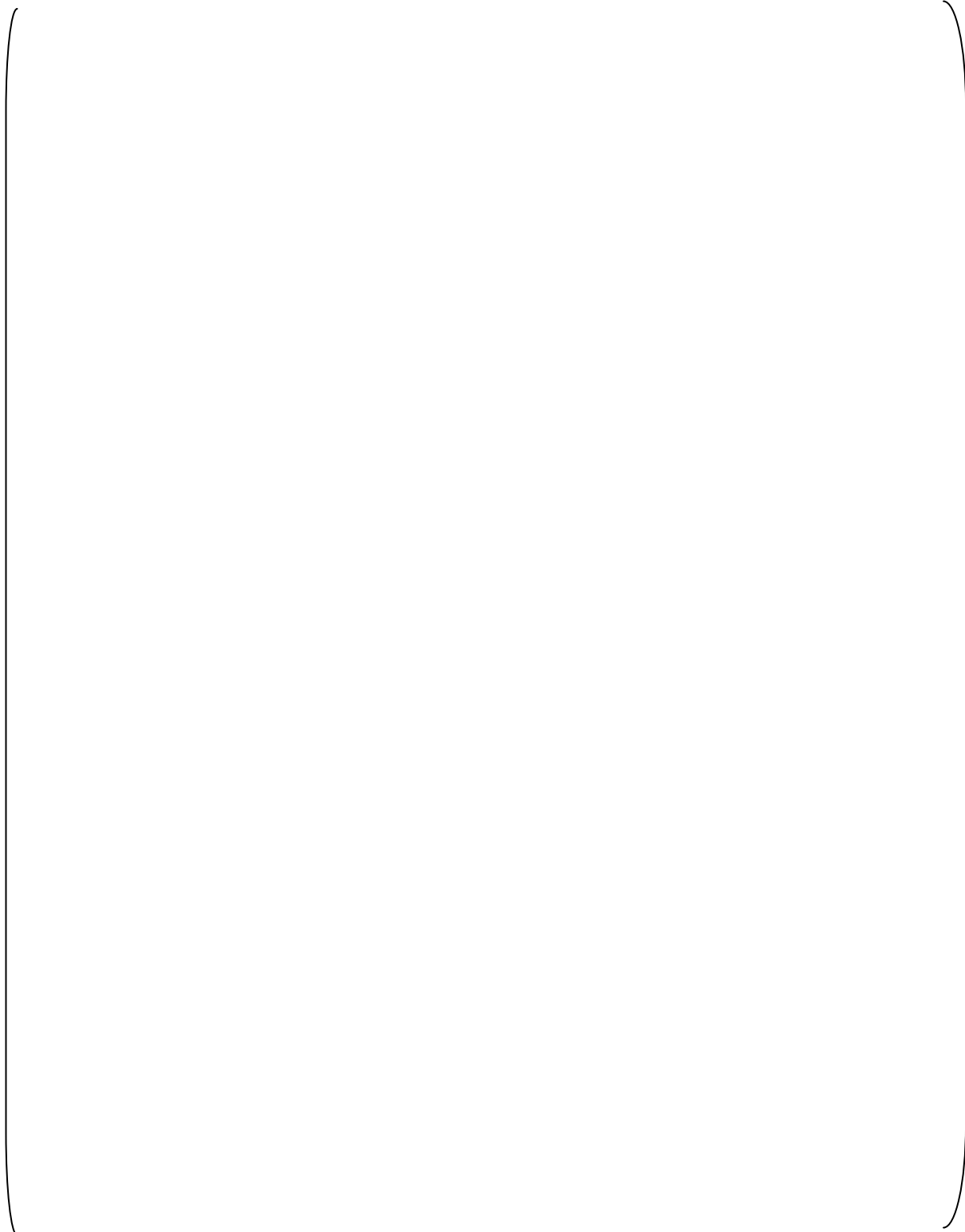
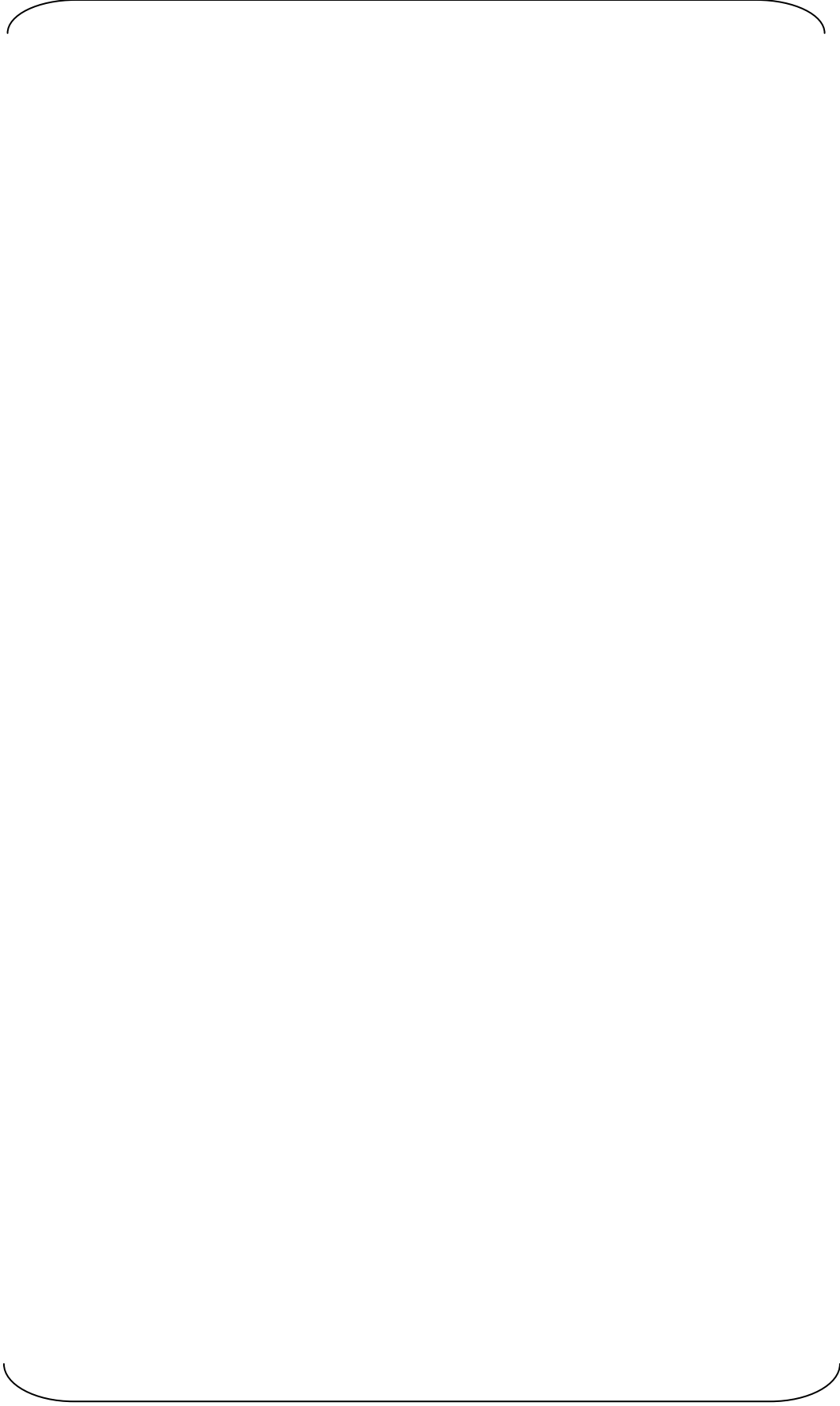


Figure A1-3-2-1 PIPESTRESS analysis model diagram

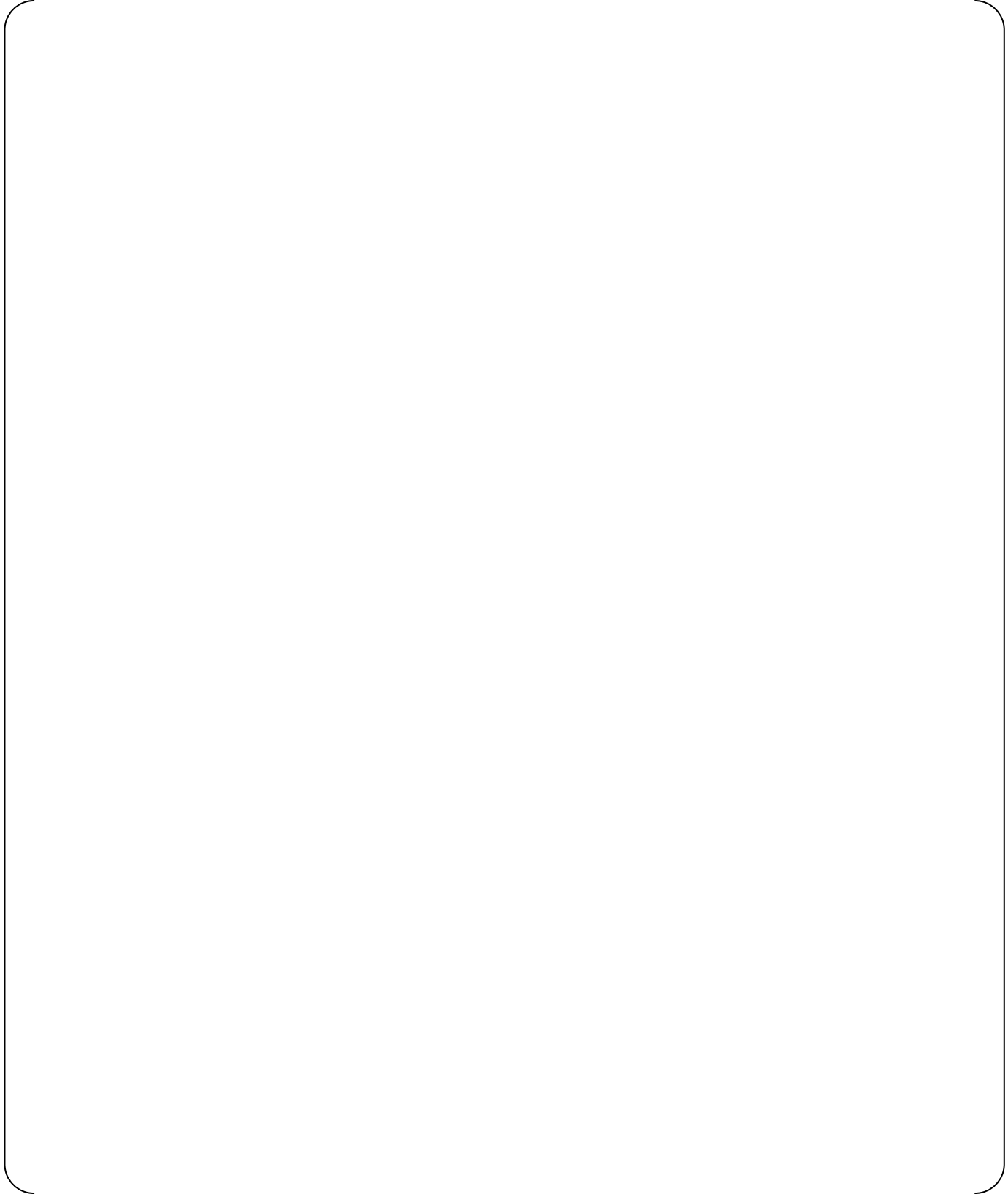


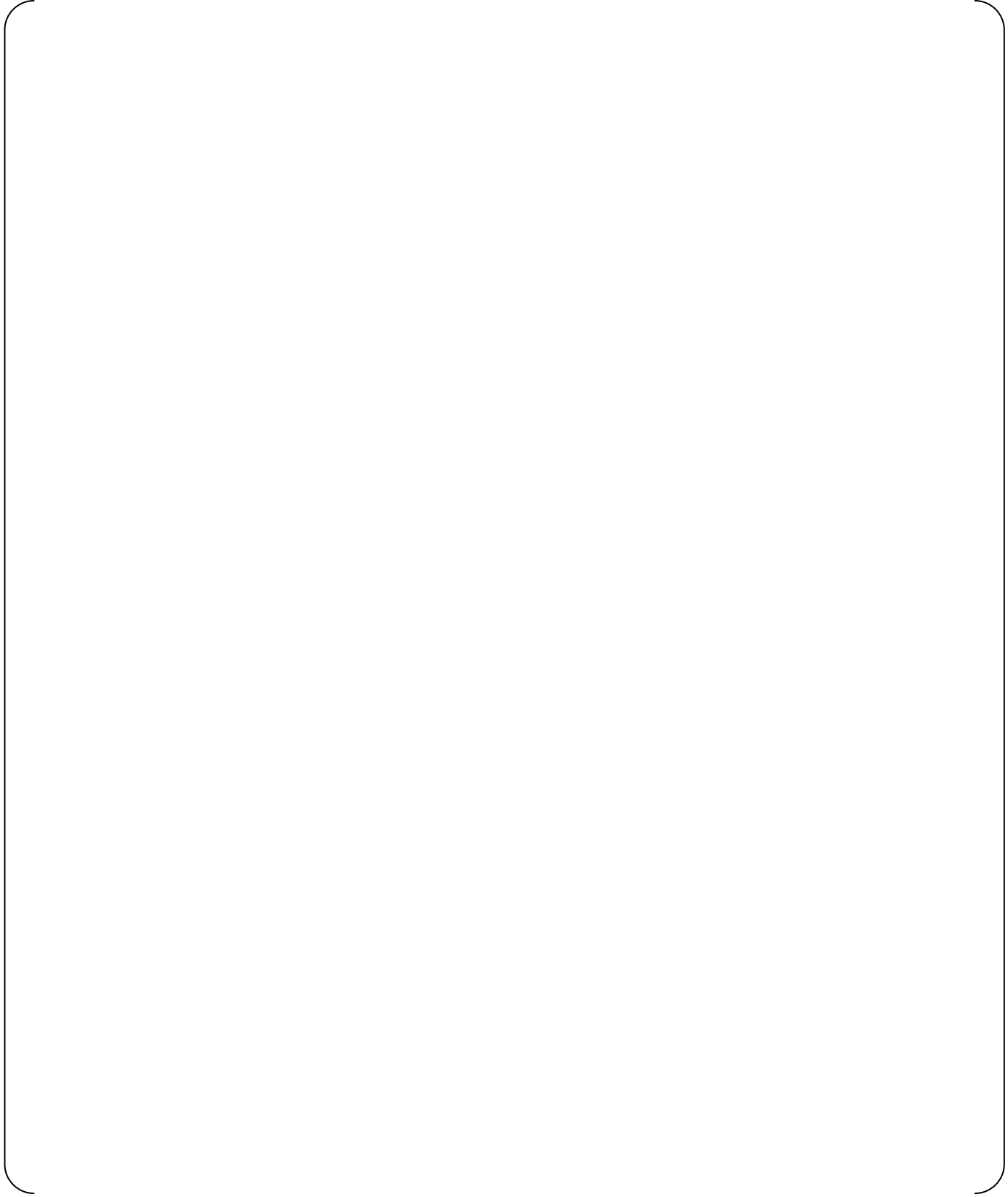
**Figure A1-3-2-2 Water hammer analysis model diagram (1/2)**  
Analysis model for Pressurizer safety depressurization valve water hammer calculation

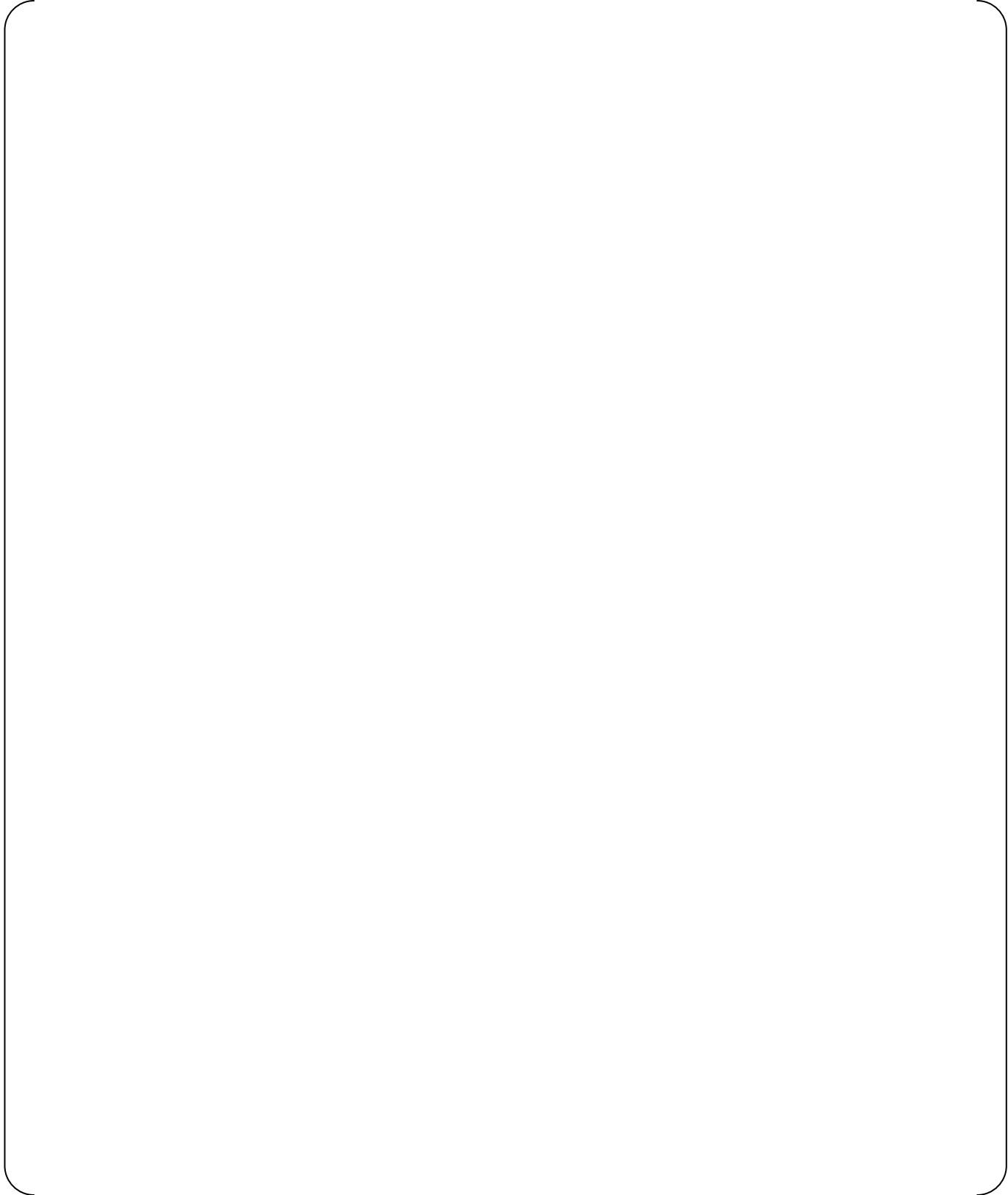


**Figure A1-3-2-2 Water hammer analysis model diagram (2/2)**  
Analysis model for Pressurizer safety valve water hammer calculation

Table A1-3-2-1 Natural frequency analysis results











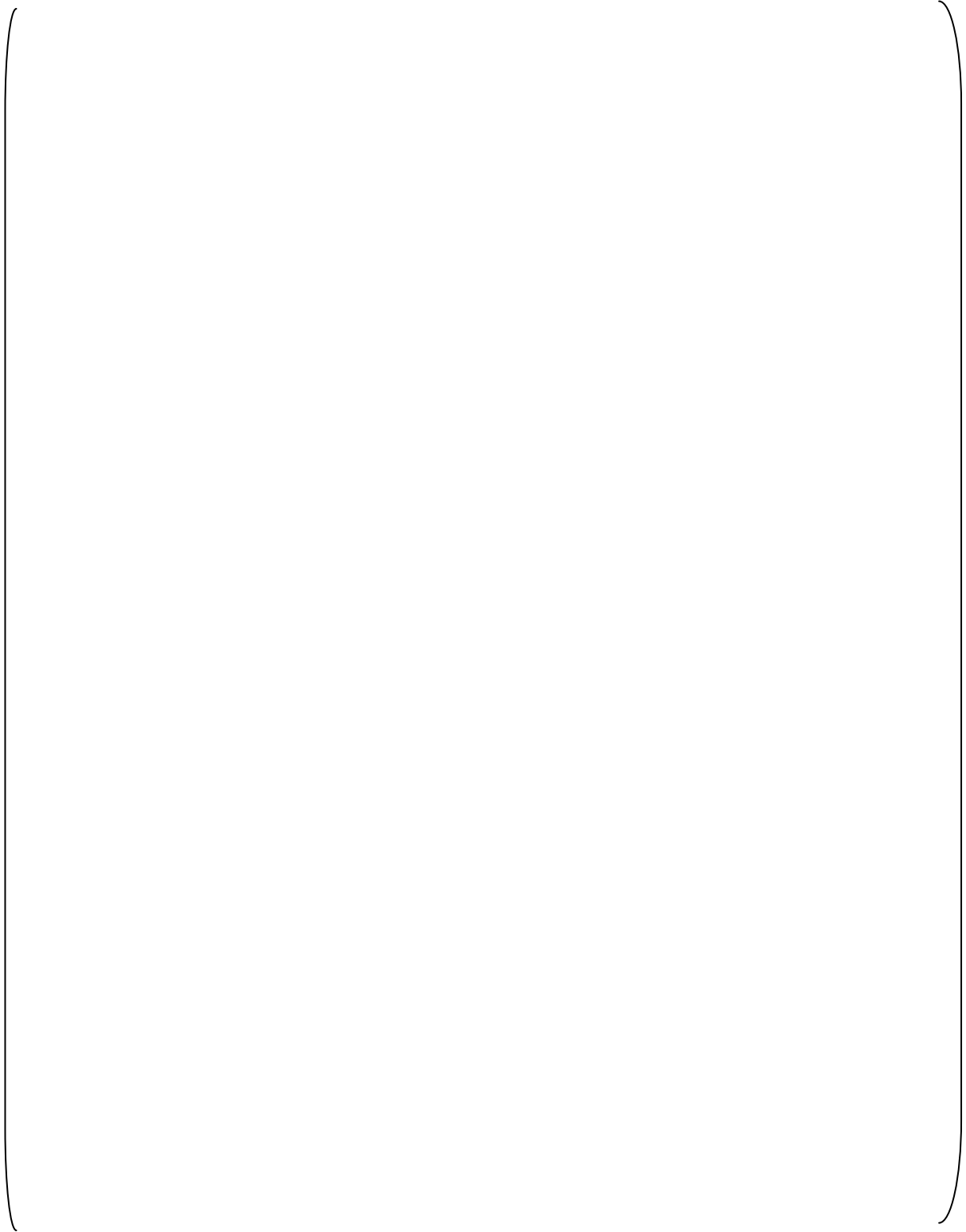


Figure A1-3-2-3 Frequency mode diagram (primary)

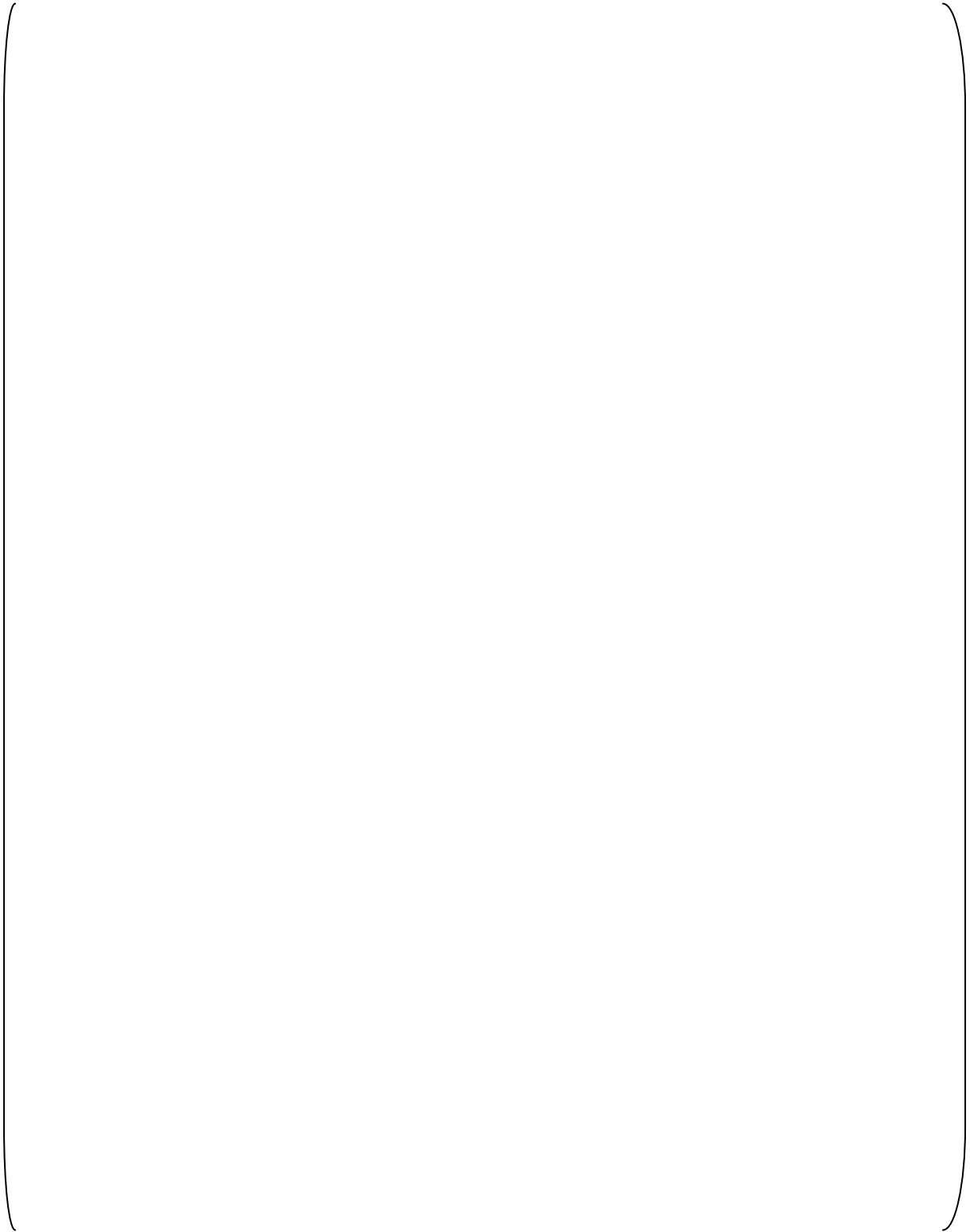


Figure A1-3-2-3 Frequency mode diagram (secondary)

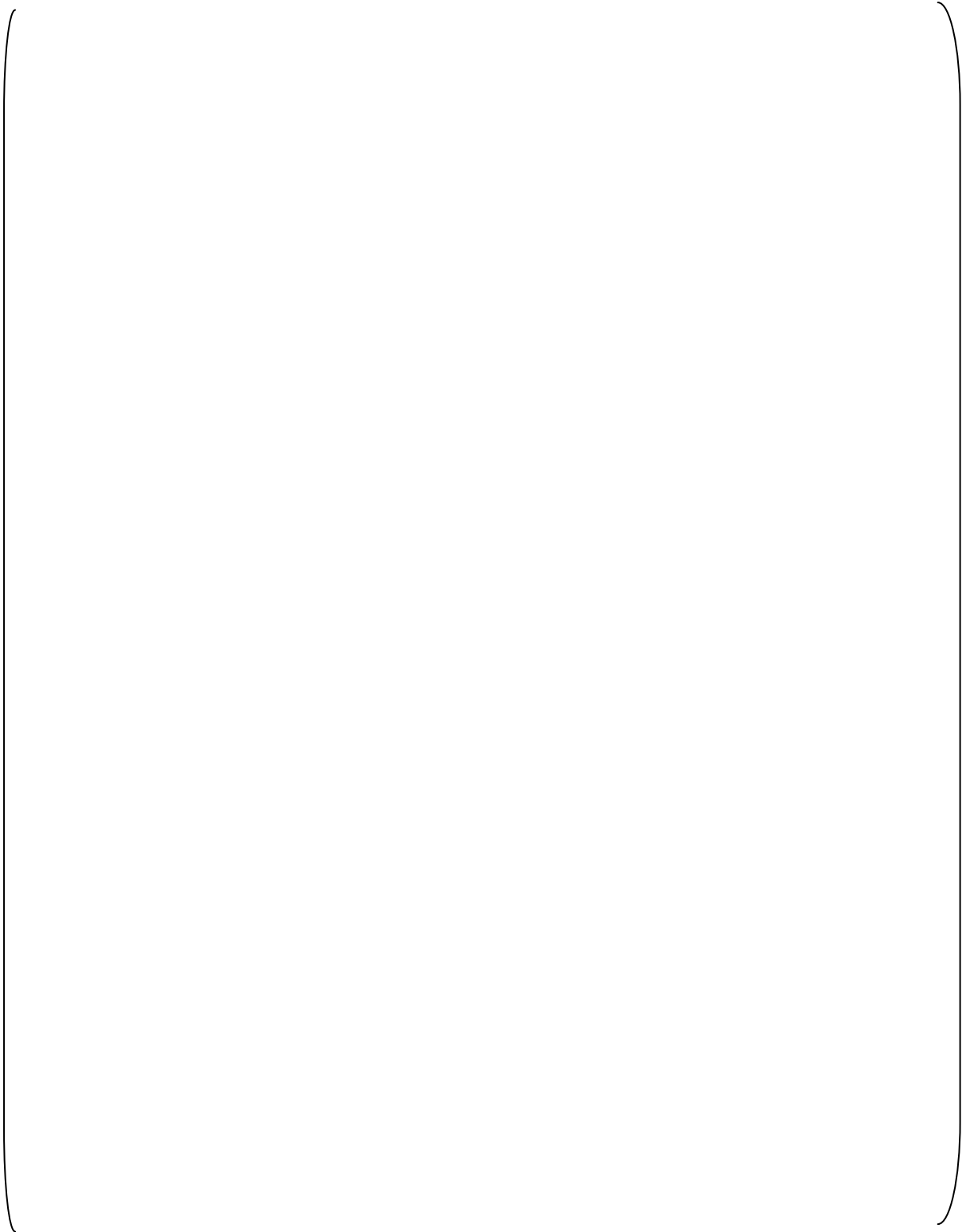


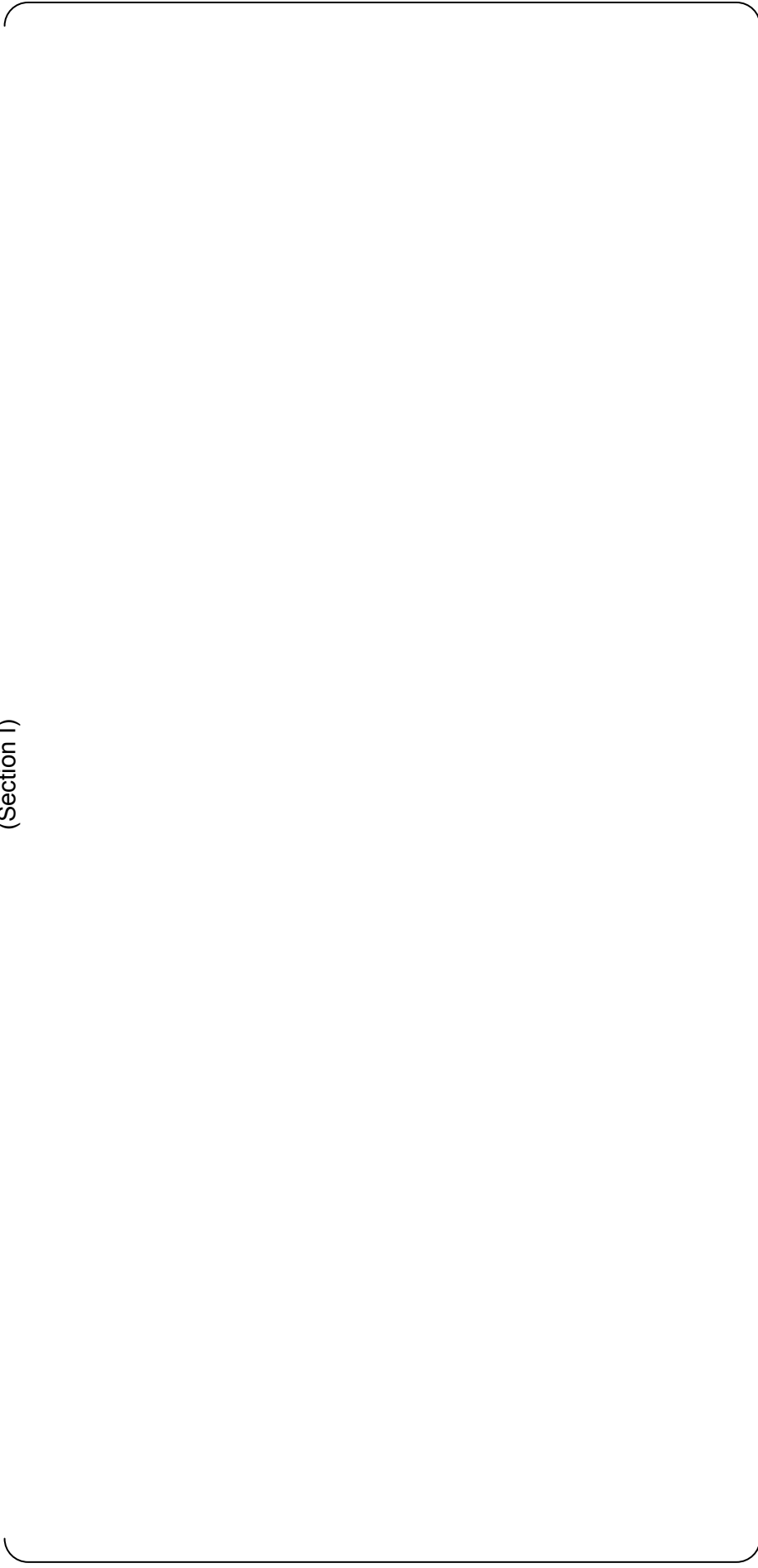
Figure A1-3-2-3 Frequency mode diagram (tertiary)

Table A1-3-2-2 Thermal analysis results ( $\Delta T_1$ ,  $\Delta T_2$ ,  $T_a$ - $T_b$ ) (1/15)  
(Section I)





Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (2/15)  
(Section I)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**

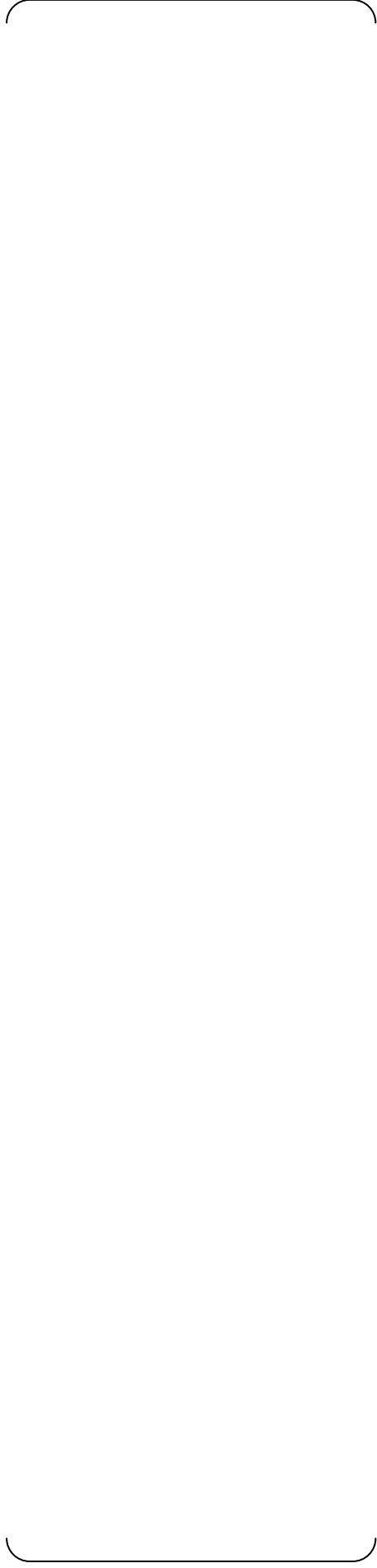
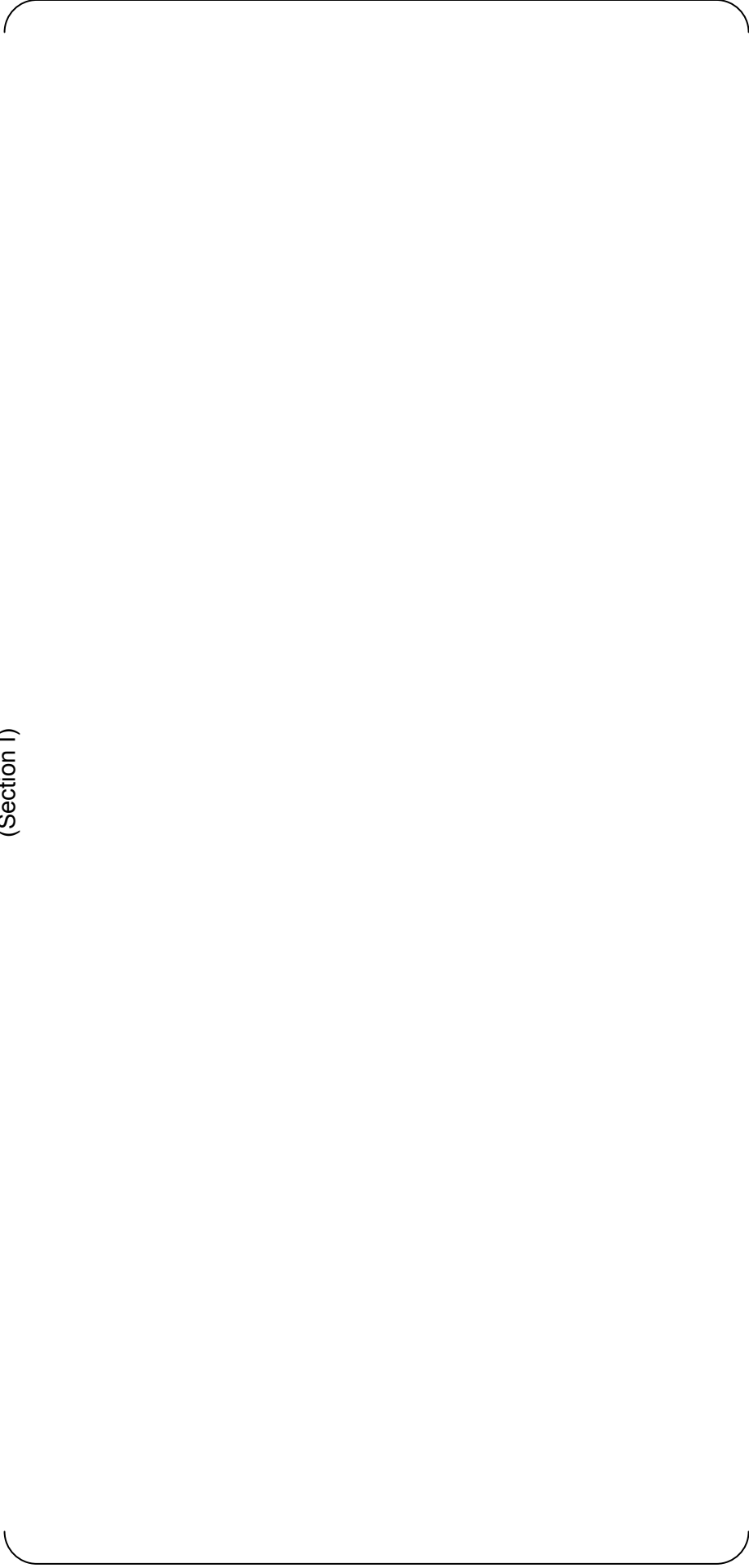




Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (3/15)  
(Section I)



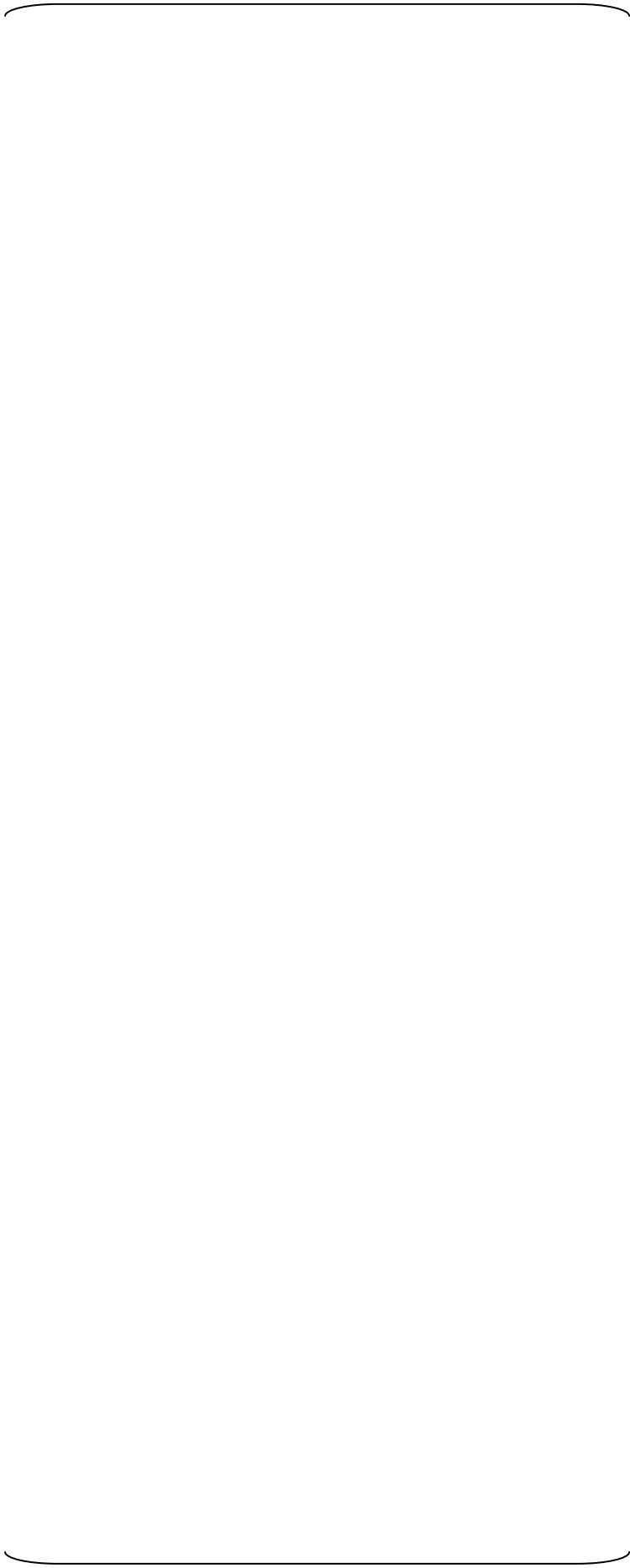


Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (4/15)  
(Section II)






Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (5/15)  
(Section II)

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (6/15)  
(Section II)







Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (7/15)  
(Section III)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**

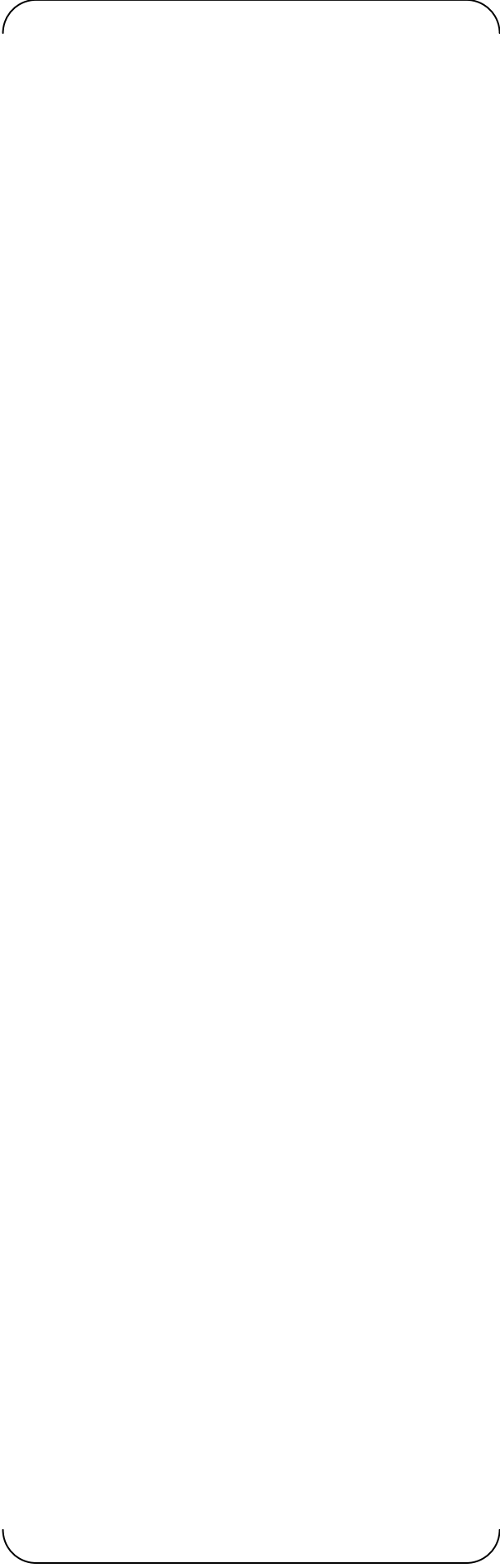


Table A1-3-2-2 Thermal analysis results ( $\Delta T_1$ ,  $\Delta T_2$ , Ta-Tb) (8/15)  
(Section III)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (9/15)  
(Section III)

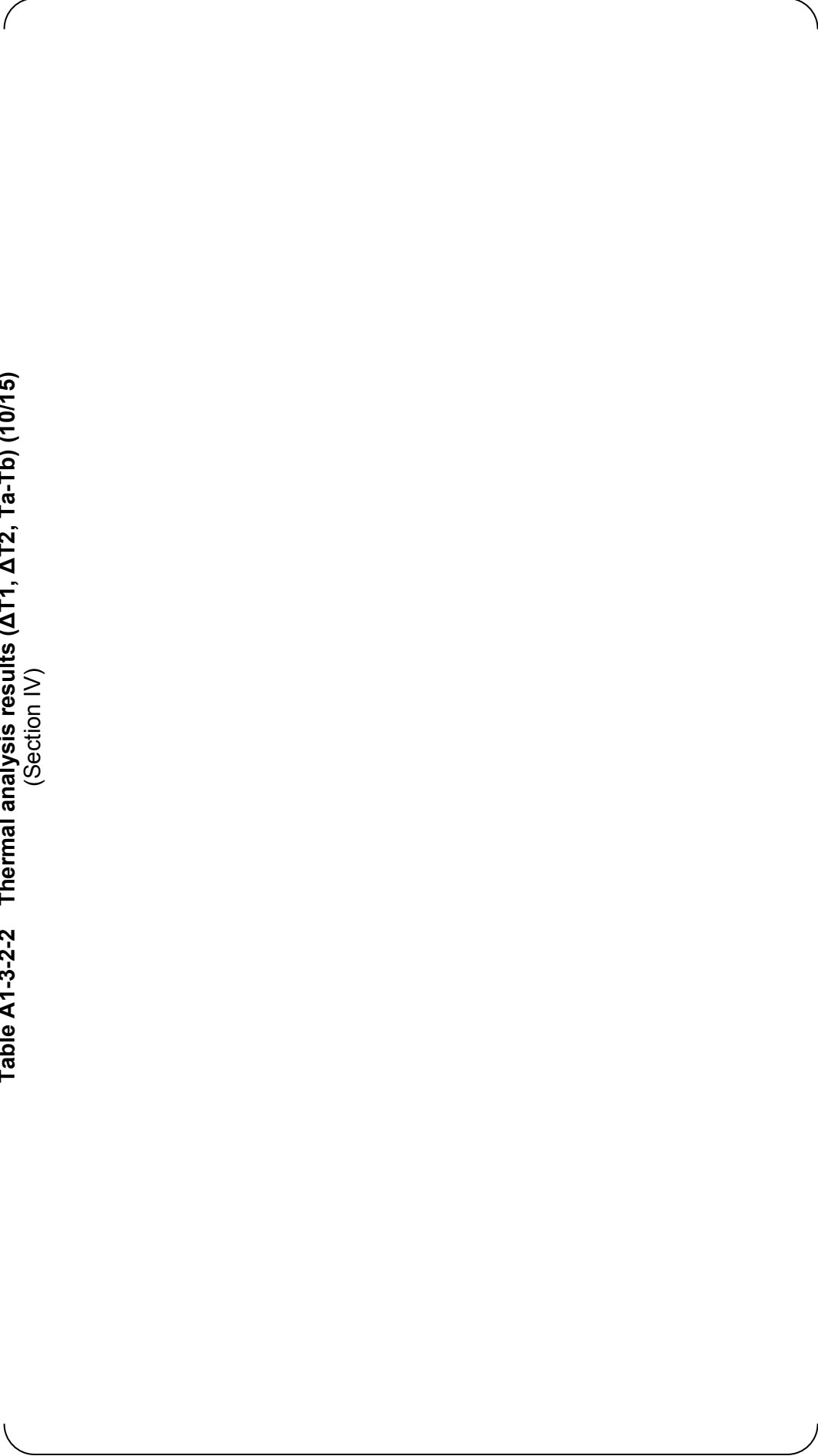


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (10/15)  
(Section IV)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (11/15)  
(Section IV)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (12/15)  
(Section IV)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (13/15)  
(Section V)

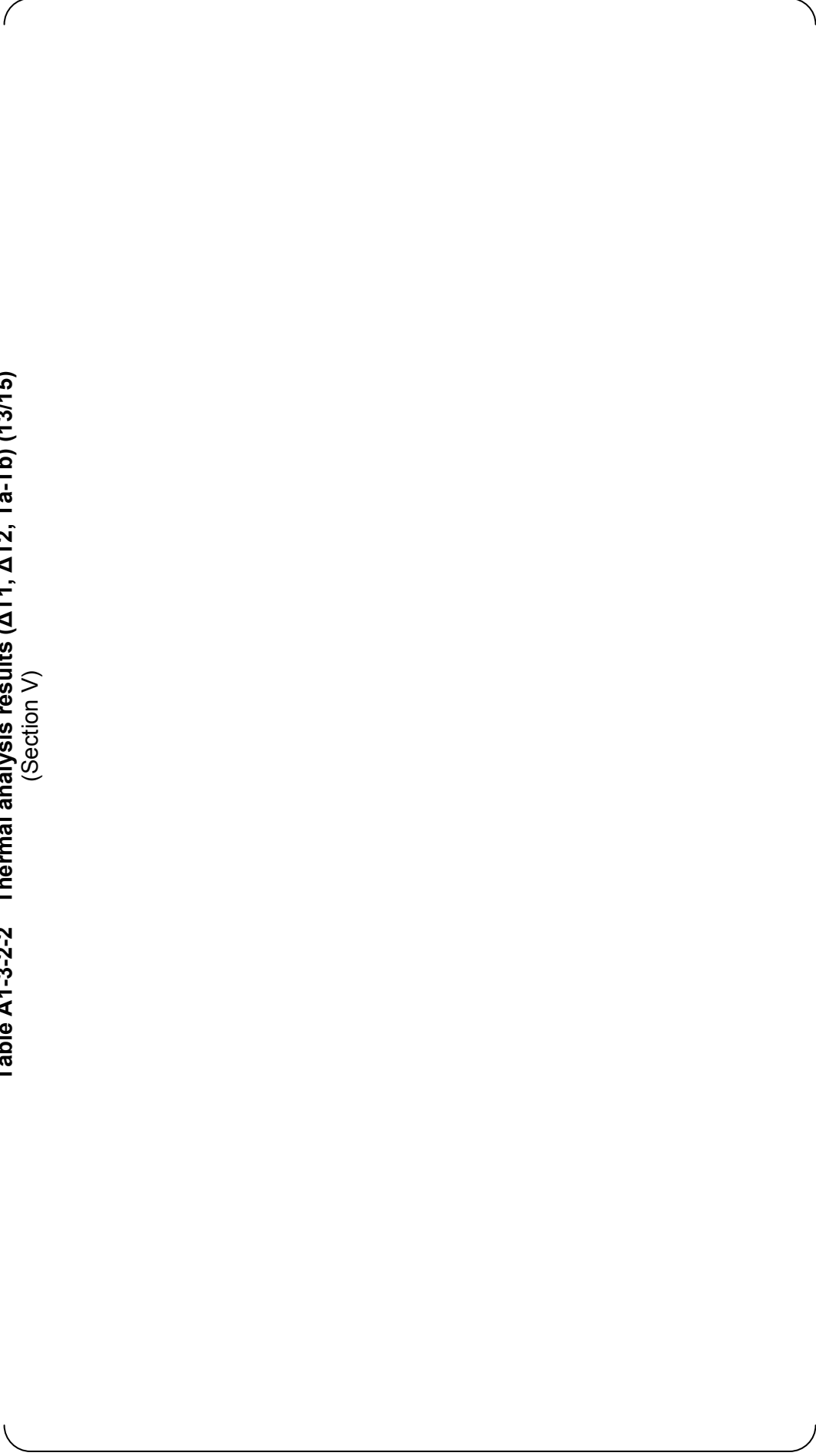
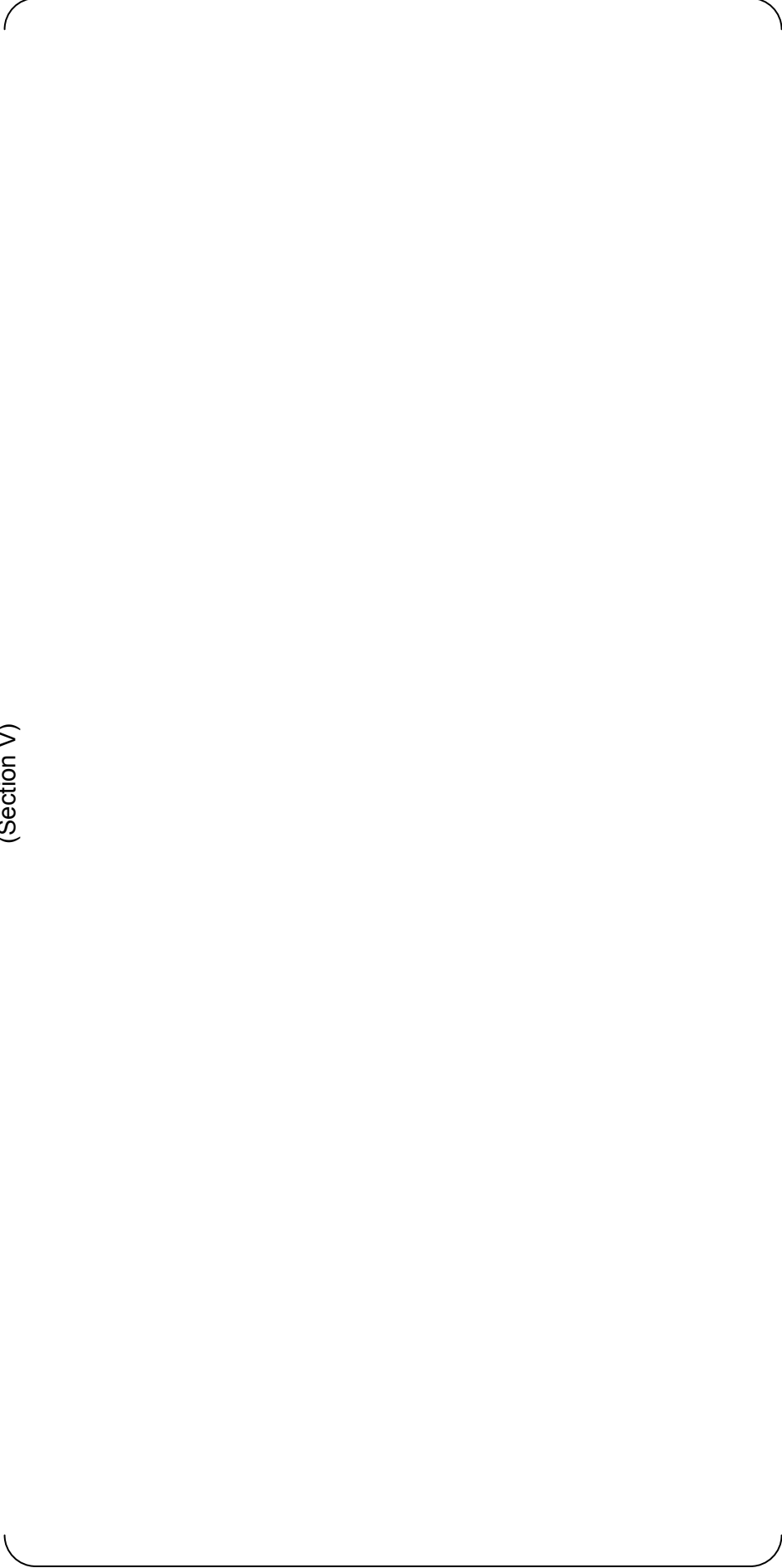




Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (14/15)  
(Section V)



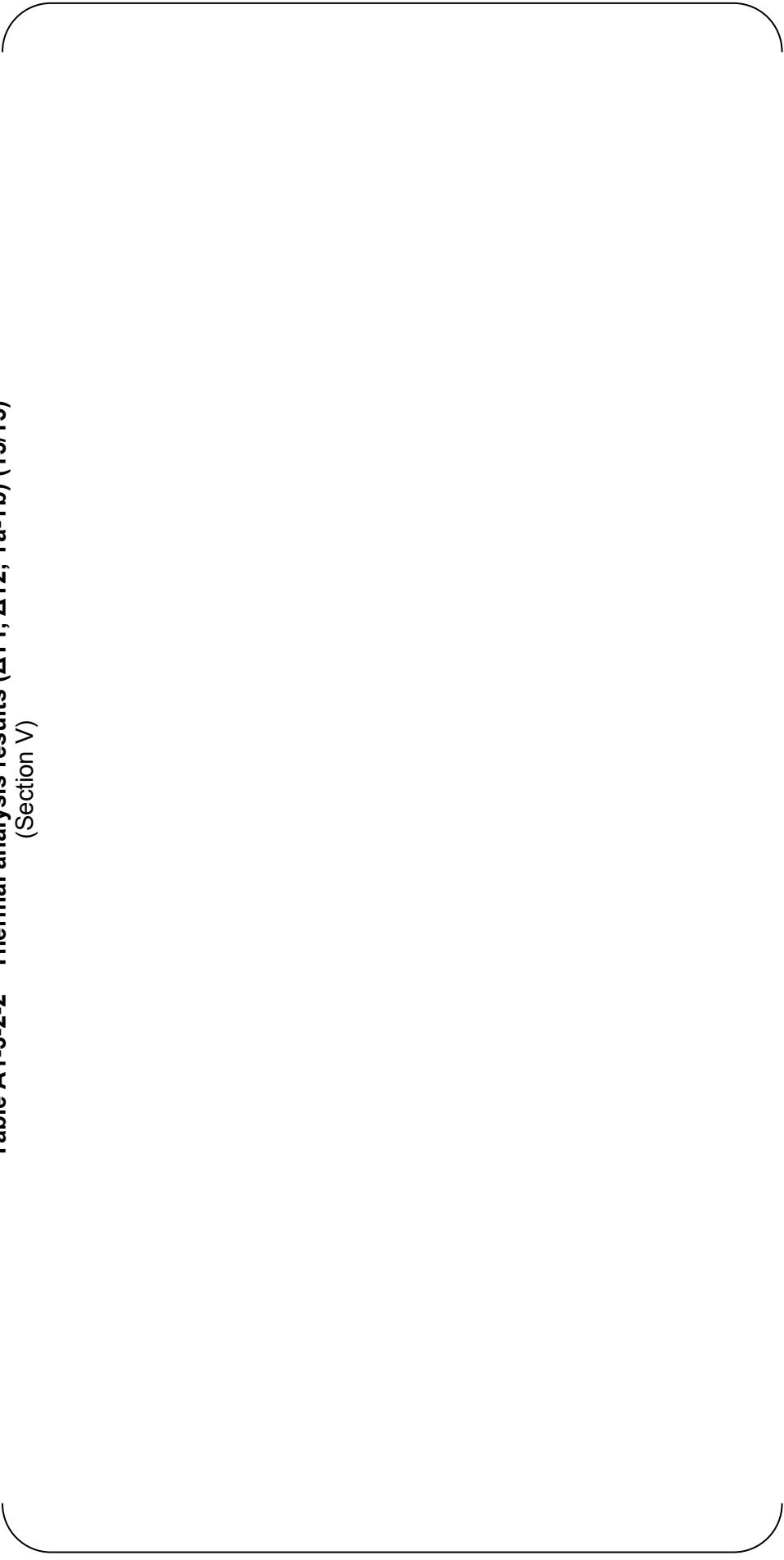
**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-3-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (15/15)  
(Section V)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



**Table A1-3-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-3-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

**Appendix 1-4**

**RC04  
Pressurizer Safety Valve Line  
Piping Analysis Results**

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|  |                 |
|--|-----------------|
| 1. INPUT   |                 |
| 1.1 Used for creating the pipe structural model                              |                 |
| 1.1.1 Block division and piping specifications                               | Table A1-4-1-1  |
| 1.1.2 Piping isometrics  | Figure A1-4-1-1 |
| 1.1.3 Concentrated mass  | Table A1-4-1-2  |
| 1.1.4 Support point rigidity   | Table A1-4-1-3  |
| 1.1.5 Valve rigidity   | Table A1-4-1-4  |
| 1.2 Used for creating load conditions  |                 |
| 1.2.1 Level A/B design transient   | see main text   |
| 1.2.2 Level A/B thermal displacement input data                              | Table A1-4-1-5  |
| 1.2.3 Level A, B temperature and pressure input data                         | Table A1-4-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data                 | Table A1-4-1-7  |
| 1.2.5 Floor response curve   | Figure A1-4-1-2 |
| 1.2.6 Seismic anchor displacement input data                                 | Table A1-4-1-8  |
| 1.2.7 DBPB displacement input data   | Table A1-4-1-9  |
| 1.2.8 Initial condition and valve open characteristics (Water hammer)        | Table A1-4-1-10 |
| 2. OUTPUT  |                 |
| 2.1 PIPESTRESS analysis model diagram  | Figure A1-4-2-1 |
| 2.2 Water hammer analysis model diagram                                      | Figure A1-4-2-2 |
| 2.3 Natural frequency analysis results                                       | Table A1-4-2-1  |
| 2.4 Frequency mode diagram (primary to tertiary)                             | Figure A1-4-2-3 |
| 2.5 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a$ - $T_b$ ) | Table A1-4-2-2  |
| 2.6 Piping stress and fatigue evaluation results                             | Table A1-4-2-3  |

**Table A1-4-1-1 Block division and piping specifications**

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-4-1-2 Concentrated mass

Table A1-4-1-3 Support point rigidity

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Table A1-4-1-4 Valve rigidity

Table A1-4-1-5 Level A/B thermal displacement input data (1/1)  
(Point: -)

| Point | Level A/B thermal displacement input data (1/1) |
|-------|---|
|-------|---|

Table A1-4-1-6 Level A, B temperature and pressure input data (1/6)  
(Section I)

Table A1-4-1-6 Level A, B temperature and pressure input data (2/6)  
(Section I)

Table A1-4-1-6 Level A, B temperature and pressure input data (3/6)  
(Section I)

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Table A1-4-1-6 Level A, B temperature and pressure input data (4/6)  
(Section II)



Table A1-4-1-6 Level A, B temperature and pressure input data (5/6)  
(Section II)

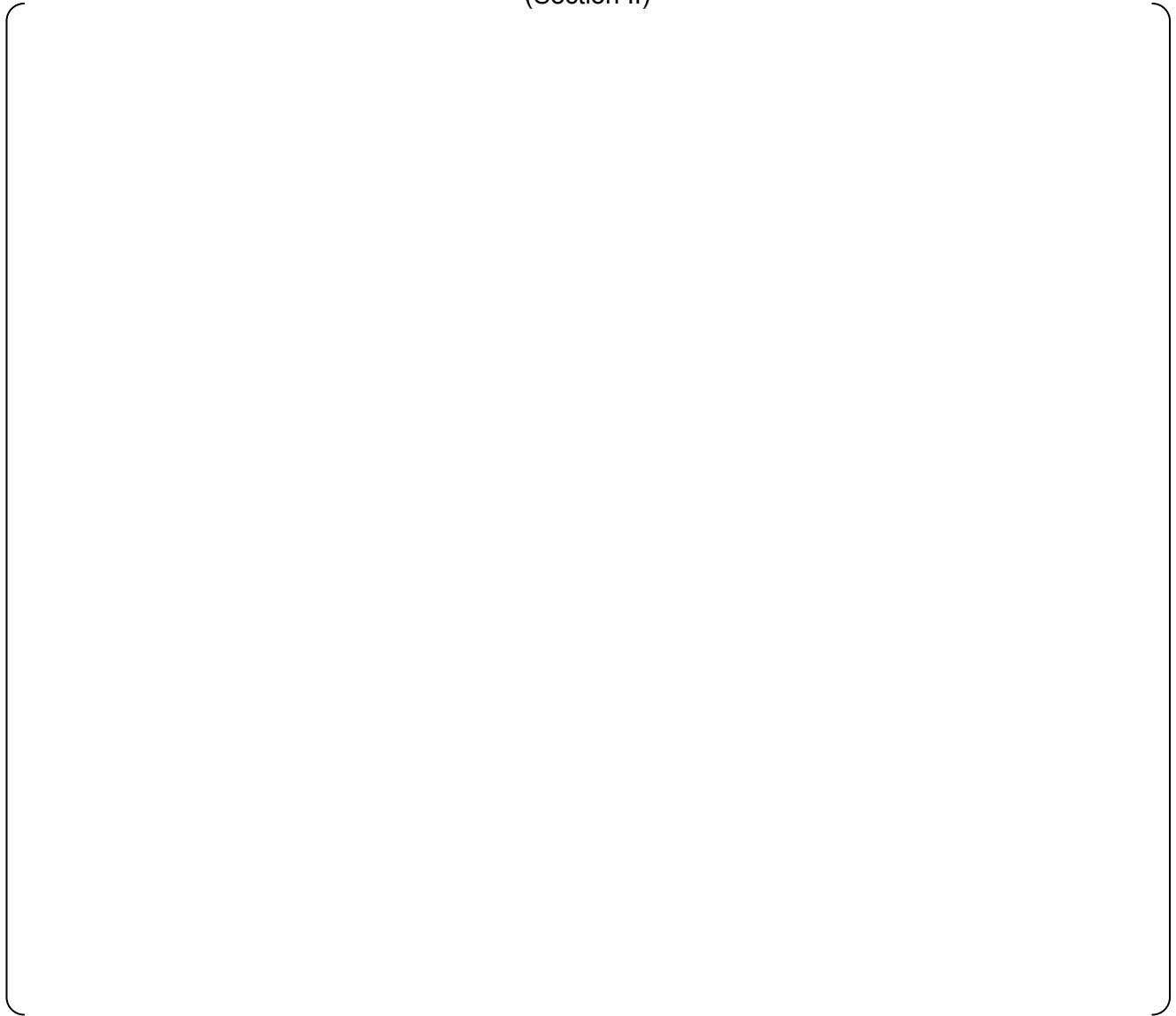


Table A1-4-1-6 Level A, B temperature and pressure input data (6/6)  
(Section II)

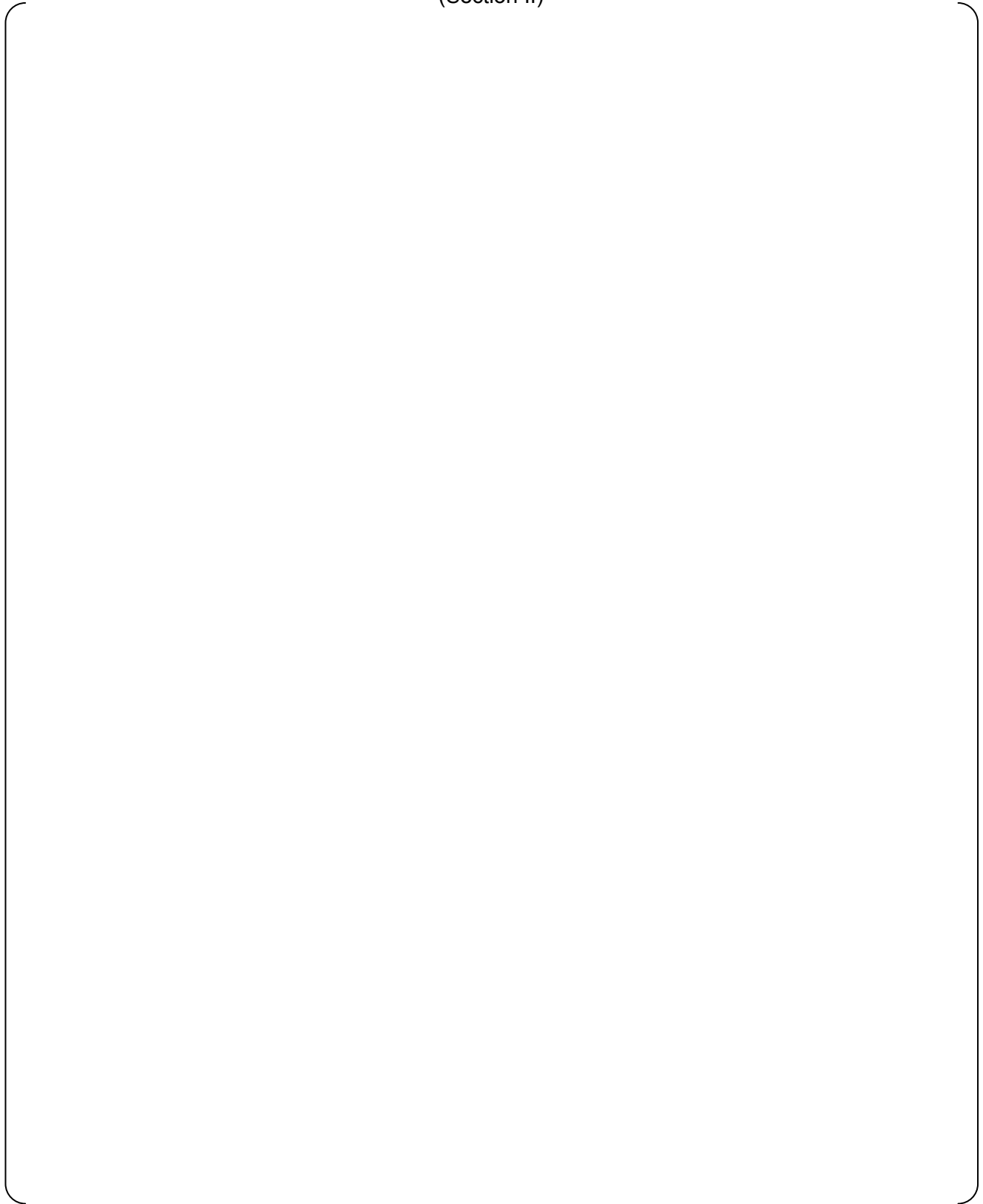


Table A1-4-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-4-1-2 Floor response curve (1/4)**  
Pressurizer Safety Valve Line (RC04-07) FRS for PZR Base Plate  
X-Y direction envelope (damping 3.0%)



**Figure A1-4-1-2 Floor response curve (2/4)**  
Pressurizer Safety Valve Line (RC04-07) FRS for PZR Base Plate  
Z (Vert.) direction (damping 3.0%)



**Figure A1-4-1-2 Floor response curve (3/4)**  
Pressurizer Safety Valve Line (RC04-07) FRS for PZR Support  
X-Y direction envelope (damping 3.0%)



**Figure A1-4-1-2 Floor response curve (4/4)**  
Pressurizer Safety Valve Line (RC04-07) FRS for PZR Support  
Z (Vert.) direction (damping 3.0%)

Table A1-4-1-8 Seismic anchor displacement input data



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Table A1-4-1-9 DBPB displacement input data

Table A1-4-1-10 Initial condition and valve open characteristics (Water hammer)



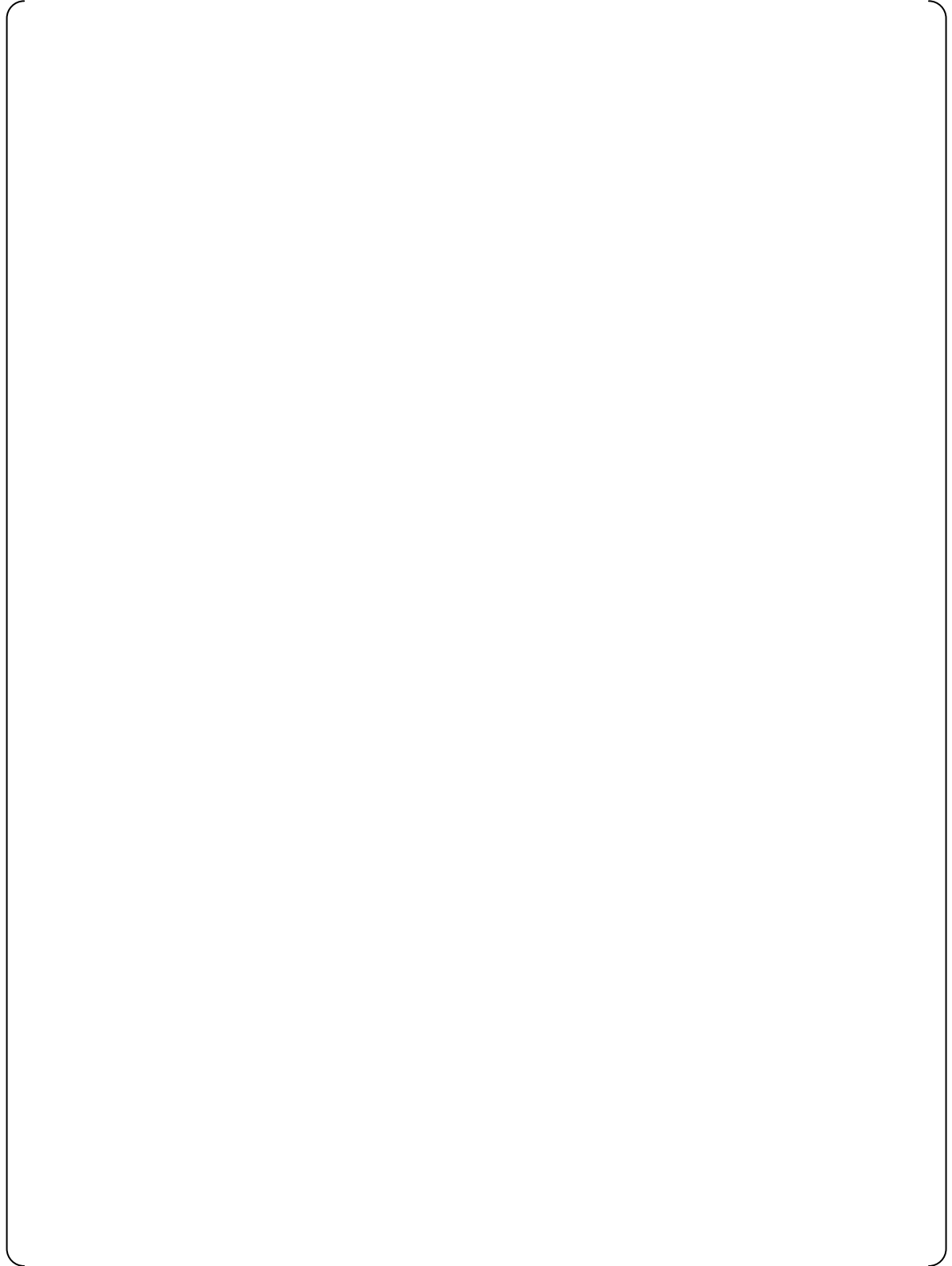


Figure A1-4-2-1 PIPESTRESS analysis model diagram (1/2)

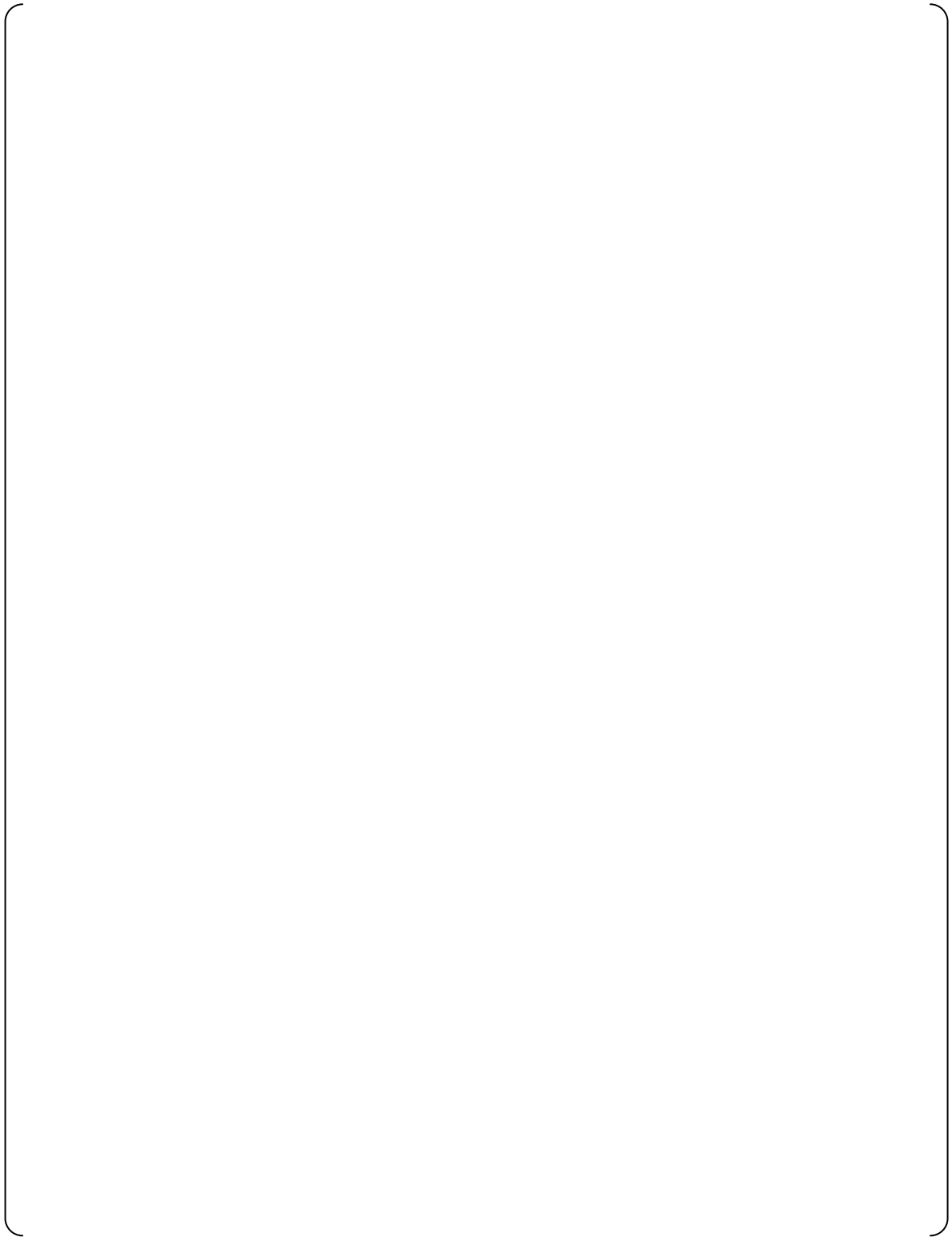
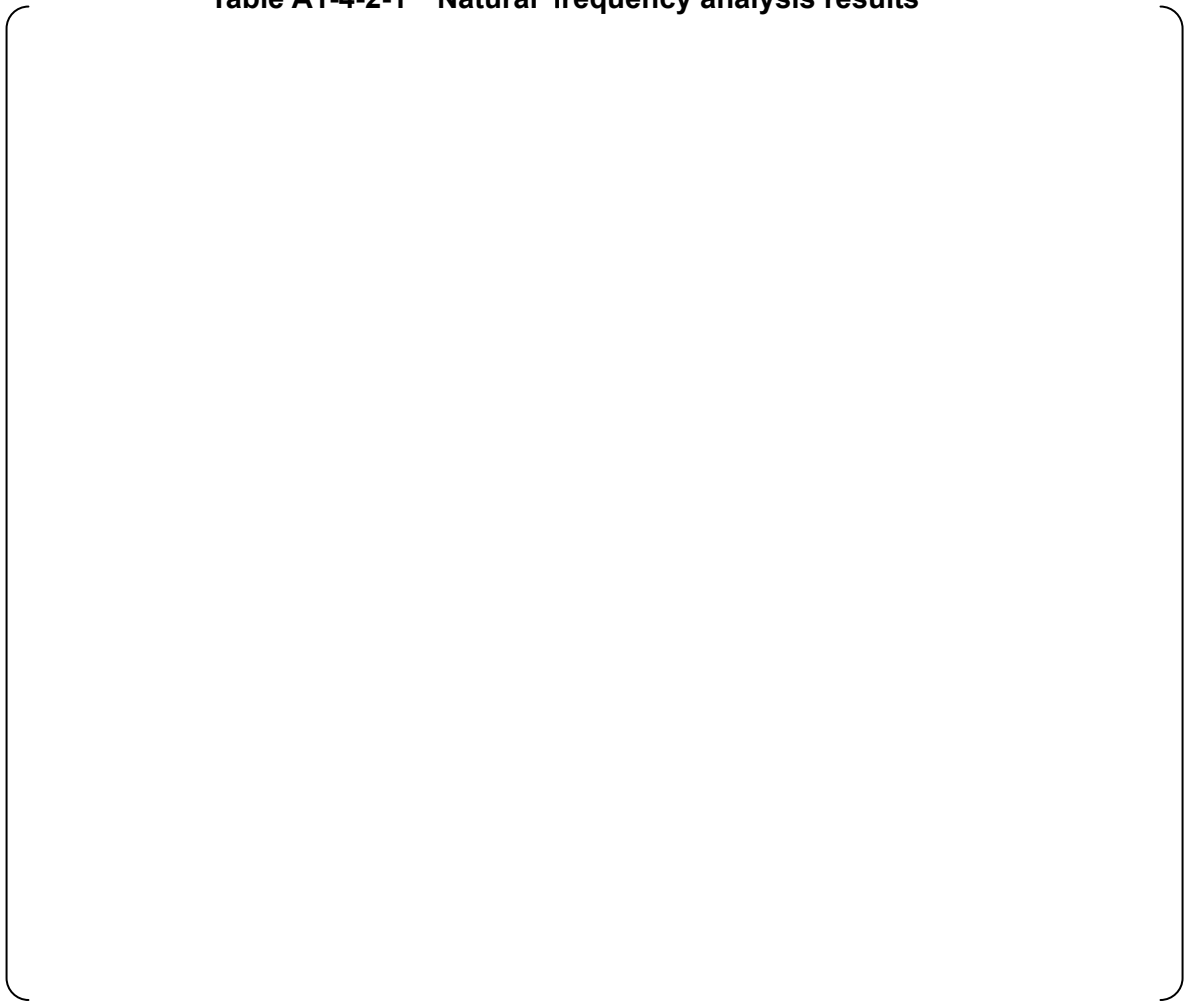


Figure A1-4-2-1 PIPESTRESS analysis model diagram (2/2)



**Figure A1-4-2-2 Water hammer analysis model diagram**  
Analysis model for Pressurizer safety valve water hammer calculation

Table A1-4-2-1 Natural frequency analysis results



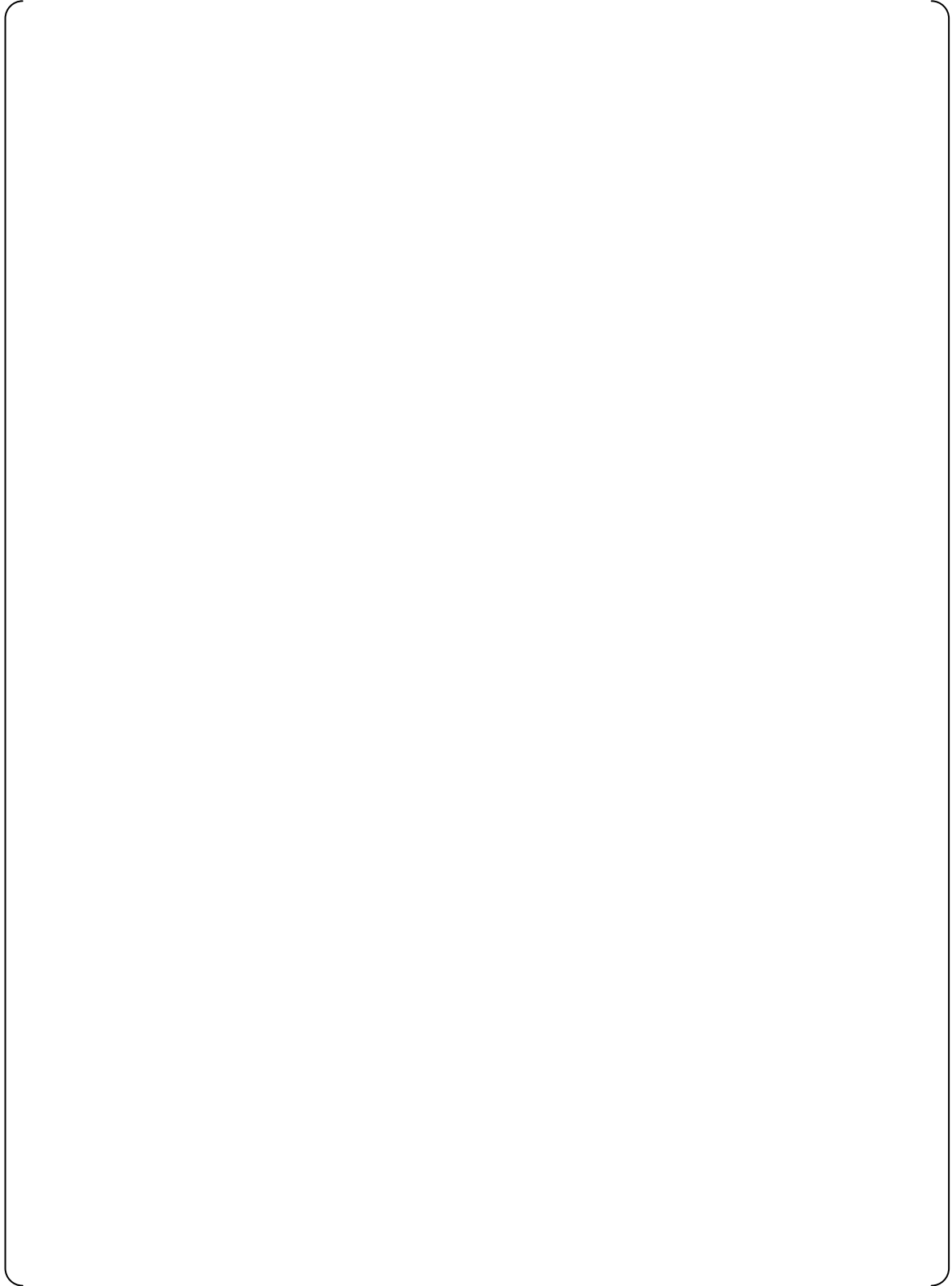


Figure A1-4-2-3 Frequency mode diagram (primary)

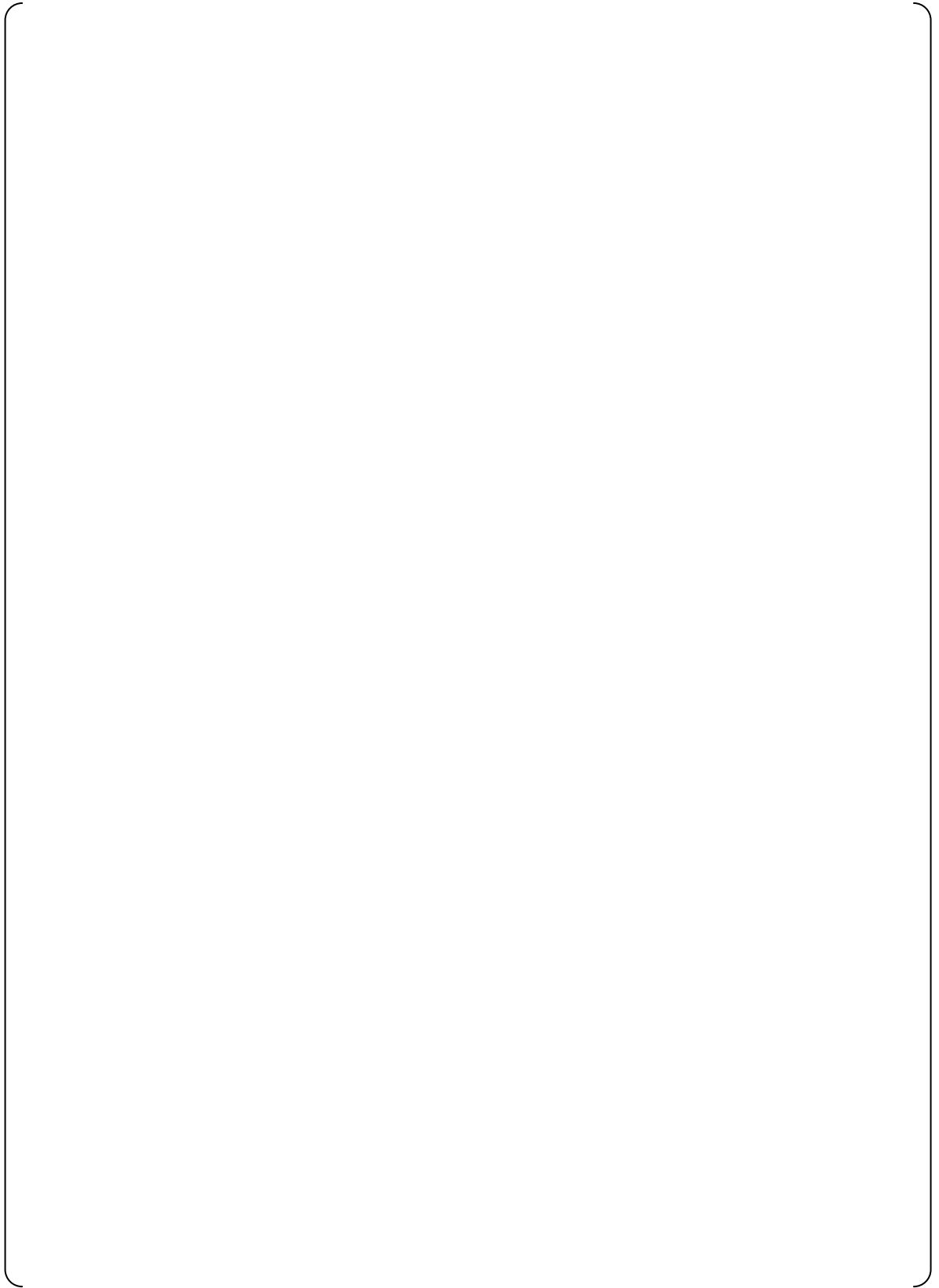


Figure A1-4-2-3 Frequency mode diagram (secondary)



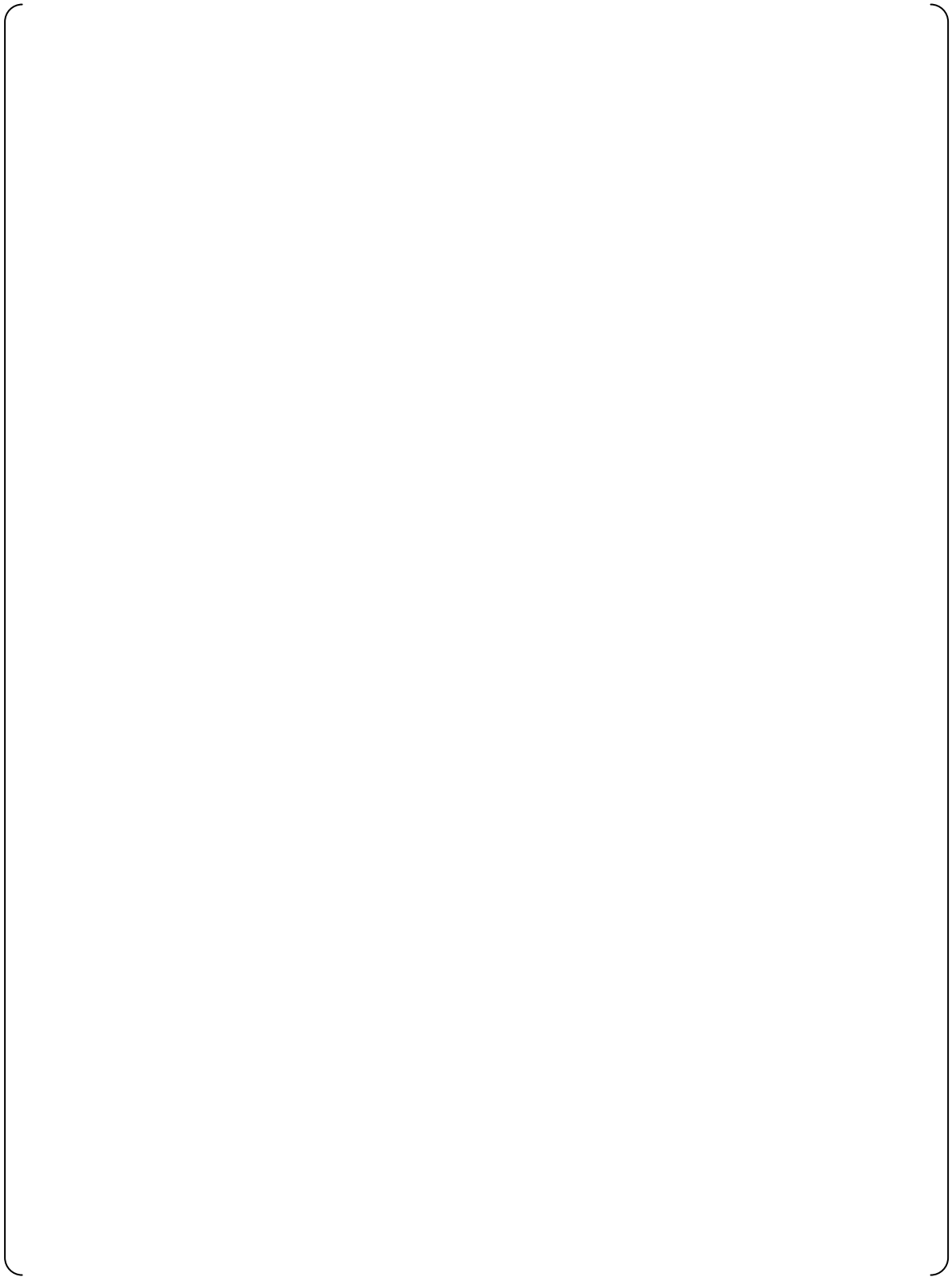
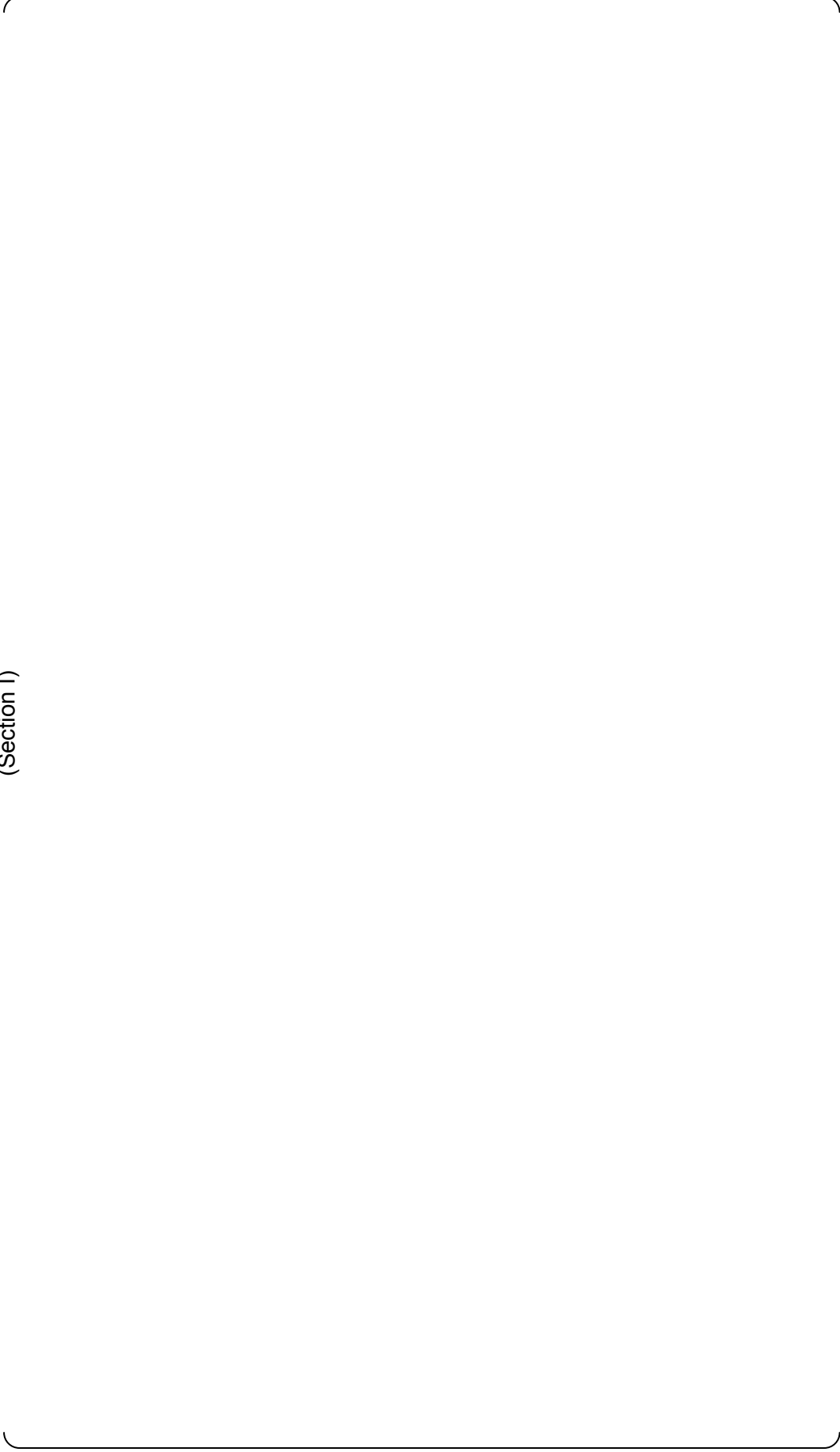


Figure A1-4-2-3 Frequency mode diagram (tertiary)

Table A1-4-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (1/6)  
(Section I)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

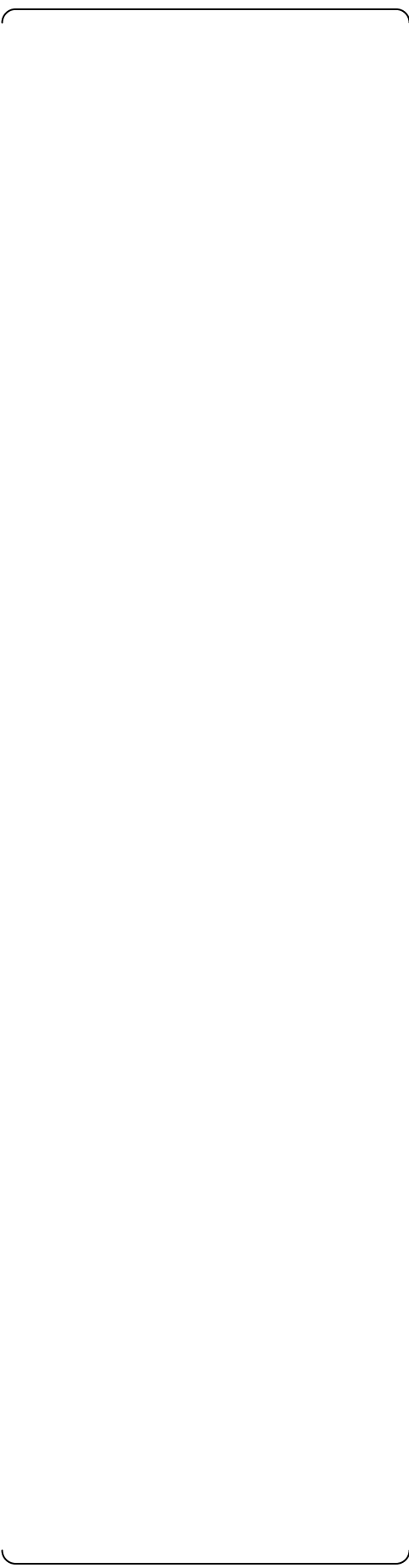
**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**











**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-4-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (5/6)  
(Section II)

|  |
|--|
|  |
|--|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**

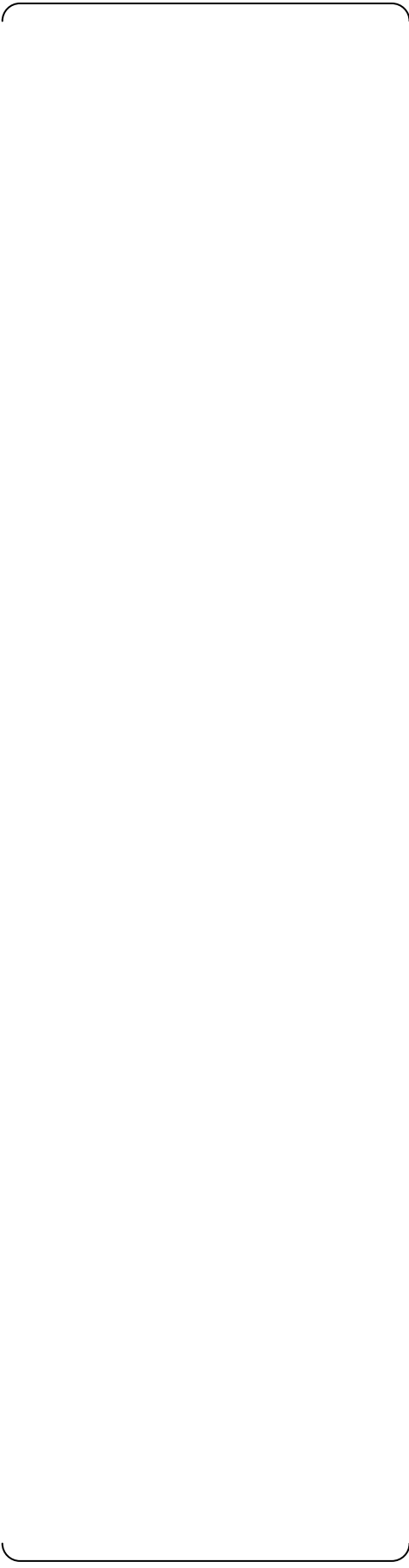
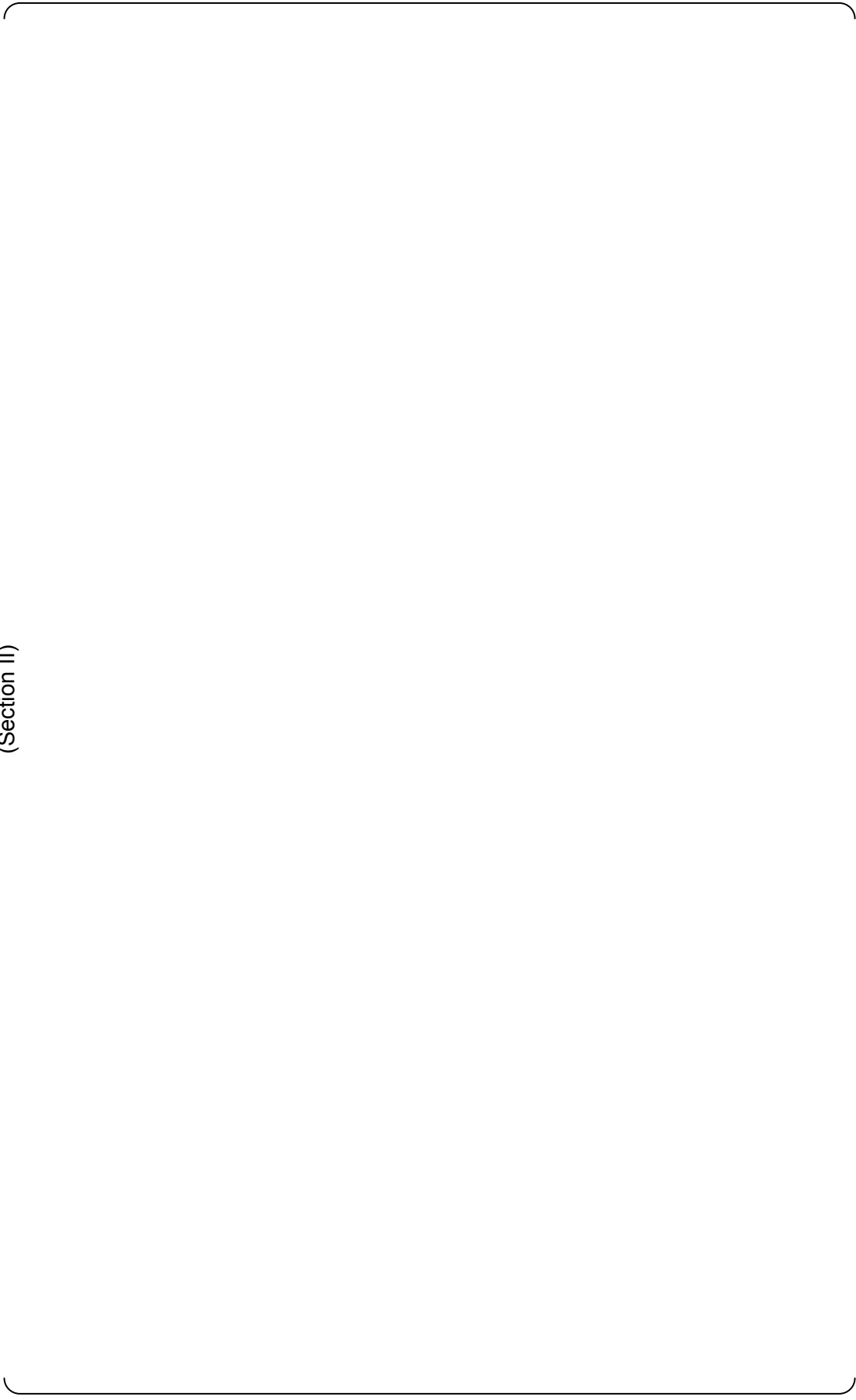
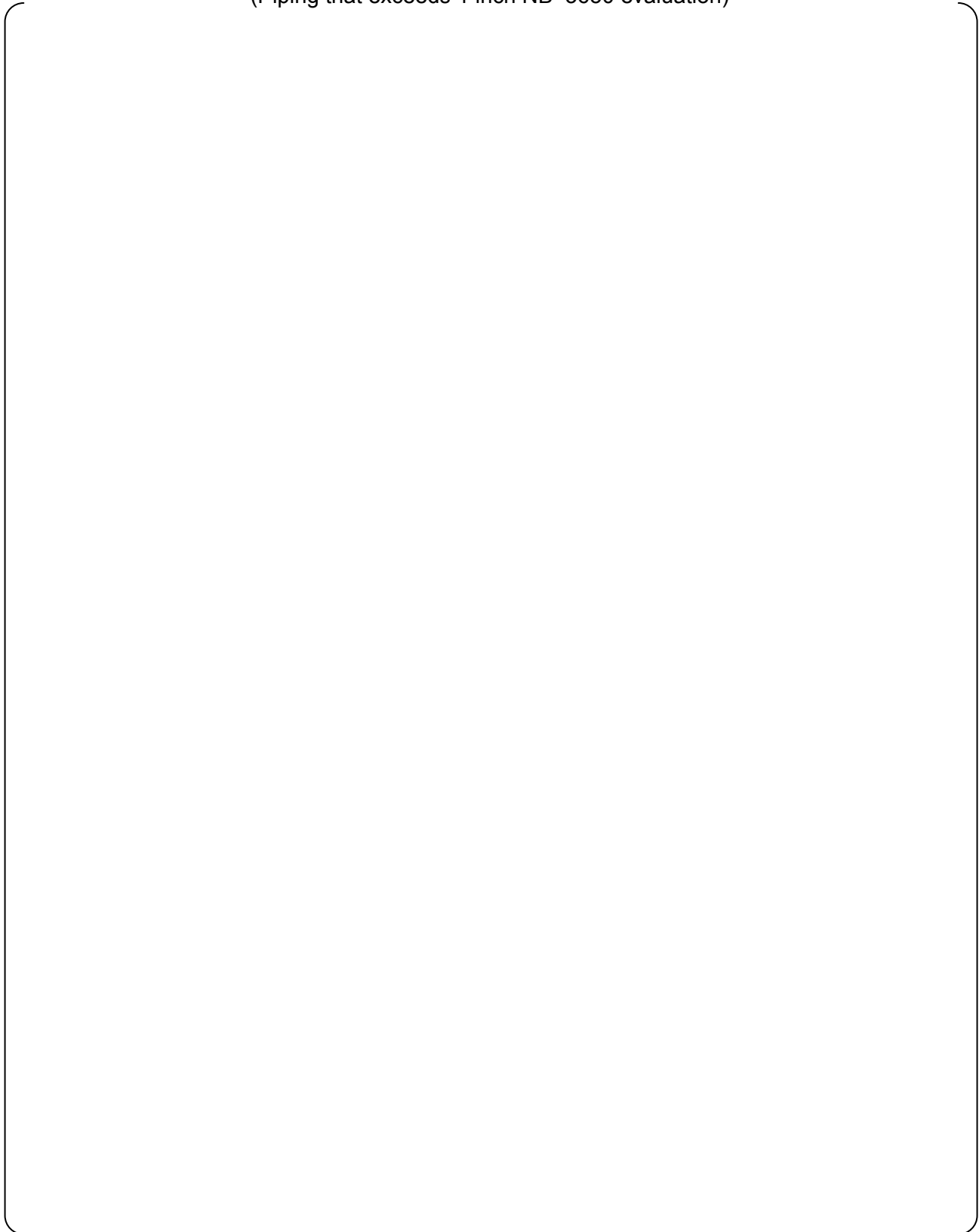


Table A1-4-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (6/6)  
(Section II)





**Table A1-4-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)



**Table A1-4-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

## Appendix 1-5

### RH01 RHRS Suction Loop A Line Piping Analysis Results



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|  |                 |
|--|-----------------|
| 1. INPUT   |                 |
| 1.1 Used for creating the pipe structural model                            |                 |
| 1.1.1 Block division and piping specifications                             | Table A1-5-1-1  |
| 1.1.2 Piping isometrics  | Figure A1-5-1-1 |
| 1.1.3 Concentrated mass  | Table A1-5-1-2  |
| 1.1.4 Support point rigidity   | Table A1-5-1-3  |
| 1.1.5 Valve rigidity   | Table A1-5-1-4  |
| 1.2 Used for creating load conditions                                      |                 |
| 1.2.1 Level A/B design transient   | see main text   |
| 1.2.2 Level A/B thermal displacement input data                            | Table A1-5-1-5  |
| 1.2.3 Level A, B temperature and pressure input data                       | Table A1-5-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data               | Table A1-5-1-7  |
| 1.2.5 Floor response curve   | Figure A1-5-1-2 |
| 1.2.6 Seismic anchor displacement input data                               | Table A1-5-1-8  |
| 1.2.7 DBPB displacement input data   | Table A1-5-1-9  |
| 2. OUTPUT  |                 |
| 2.1 PIPESTRESS analysis model diagram                                      | Figure A1-5-2-1 |
| 2.2 Natural frequency analysis results                                     | Table A1-5-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                           | Figure A1-5-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a - T_b$ ) | Table A1-5-2-2  |
| 2.5 Piping stress and fatigue evaluation results                           | Table A1-5-2-3  |

Table A1-5-1-1 Block division and piping specifications (1/3)

Table A1-5-1-1 Block division and piping specifications (2/3)

Table A1-5-1-1 Block division and piping specifications (3/3)



Figure A1-5-1-1  
Piping isometrics(1/2)



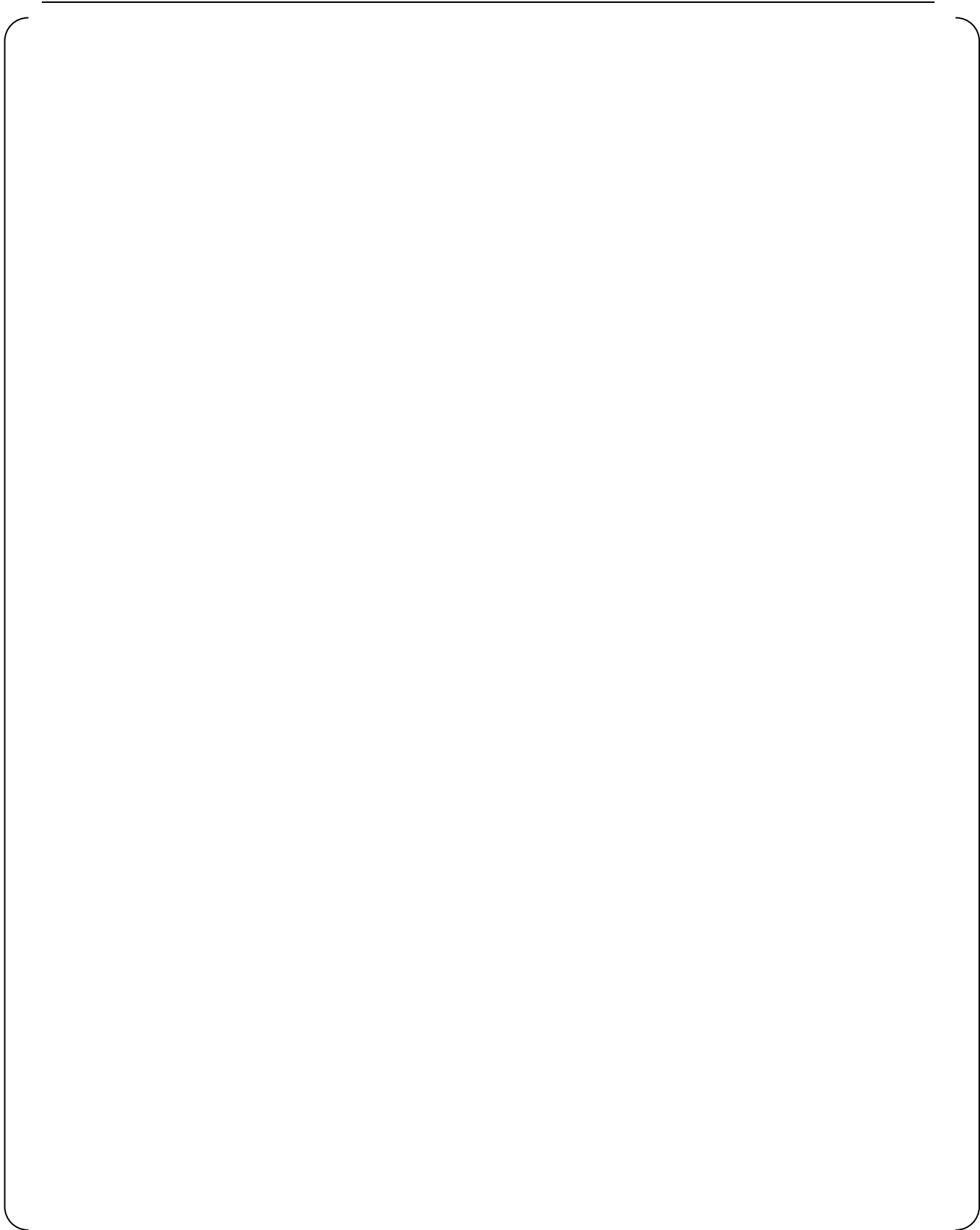
Figure A1-5-1-1  
Piping Isometrics(2/2)

Table A1-5-1-2 Concentrated mass





Table A1-5-1-3 Support point rigidity







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Table A1-5-1-4 Valve rigidity

Table A1-5-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |



Table A1-5-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-5-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |

Table A1-5-1-6 Level A, B temperature and pressure input data (1/33)  
(Section I)

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Table A1-5-1-6 Level A, B temperature and pressure input data (2/33)  
(Section I)

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**Table A1-5-1-6 Level A, B temperature and pressure input data (3/33)**  
(Section I)

Table A1-5-1-6 Level A, B temperature and pressure input data (4/33)  
(Section II)

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Table A1-5-1-6 Level A, B temperature and pressure input data (5/33)  
(Section II)

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**Table A1-5-1-6 Level A, B temperature and pressure input data (6/33)**  
(Section II)



Table A1-5-1-6 Level A, B temperature and pressure input data (7/33)  
(Section III)

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**Table A1-5-1-6 Level A, B temperature and pressure input data (8/33)**  
(Section III)

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**Table A1-5-1-6 Level A, B temperature and pressure input data (9/33)**  
(Section III)

Table A1-5-1-6 Level A, B temperature and pressure input data (10/33)  
(Section IV)

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Table A1-5-1-6 Level A, B temperature and pressure input data (11/33)  
(Section IV)

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Table A1-5-1-6 Level A, B temperature and pressure input data (12/33)  
(Section IV)

Table A1-5-1-6 Level A, B temperature and pressure input data (13/33)  
(Section V)

Table A1-5-1-6 Level A, B temperature and pressure input data (14/33)  
(Section V)



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Table A1-5-1-6 Level A, B temperature and pressure input data (15/33)  
(Section V)

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Table A1-5-1-6 Level A, B temperature and pressure input data (16/33)  
(Section VI)

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Table A1-5-1-6 Level A, B temperature and pressure input data (17/33)  
(Section VI)

Table A1-5-1-6 Level A, B temperature and pressure input data (18/33)  
(Section VI)

Table A1-5-1-6 Level A, B temperature and pressure input data (19/33)  
(Section VII)

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Table A1-5-1-6 Level A, B temperature and pressure input data (20/33)  
(Section VII)

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Table A1-5-1-6 Level A, B temperature and pressure input data (21/33)  
(Section VII)

Table A1-5-1-6 Level A, B temperature and pressure input data (22/33)  
(Section VIII)



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Table A1-5-1-6 Level A, B temperature and pressure input data (23/33)  
(Section VIII)

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Table A1-5-1-6 Level A, B temperature and pressure input data (24/33)  
(Section VIII)

Table A1-5-1-6 Level A, B temperature and pressure input data (25/33)  
(Section IX)

Table A1-5-1-6 Level A, B temperature and pressure input data (26/33)  
(Section IX)

Table A1-5-1-6 Level A, B temperature and pressure input data (27/33)  
(Section IX)

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Table A1-5-1-6 Level A, B temperature and pressure input data (28/33)  
(Section X)

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Table A1-5-1-6 Level A, B temperature and pressure input data (29/33)  
(Section X)

Table A1-5-1-6 Level A, B temperature and pressure input data (30/33)  
(Section X)



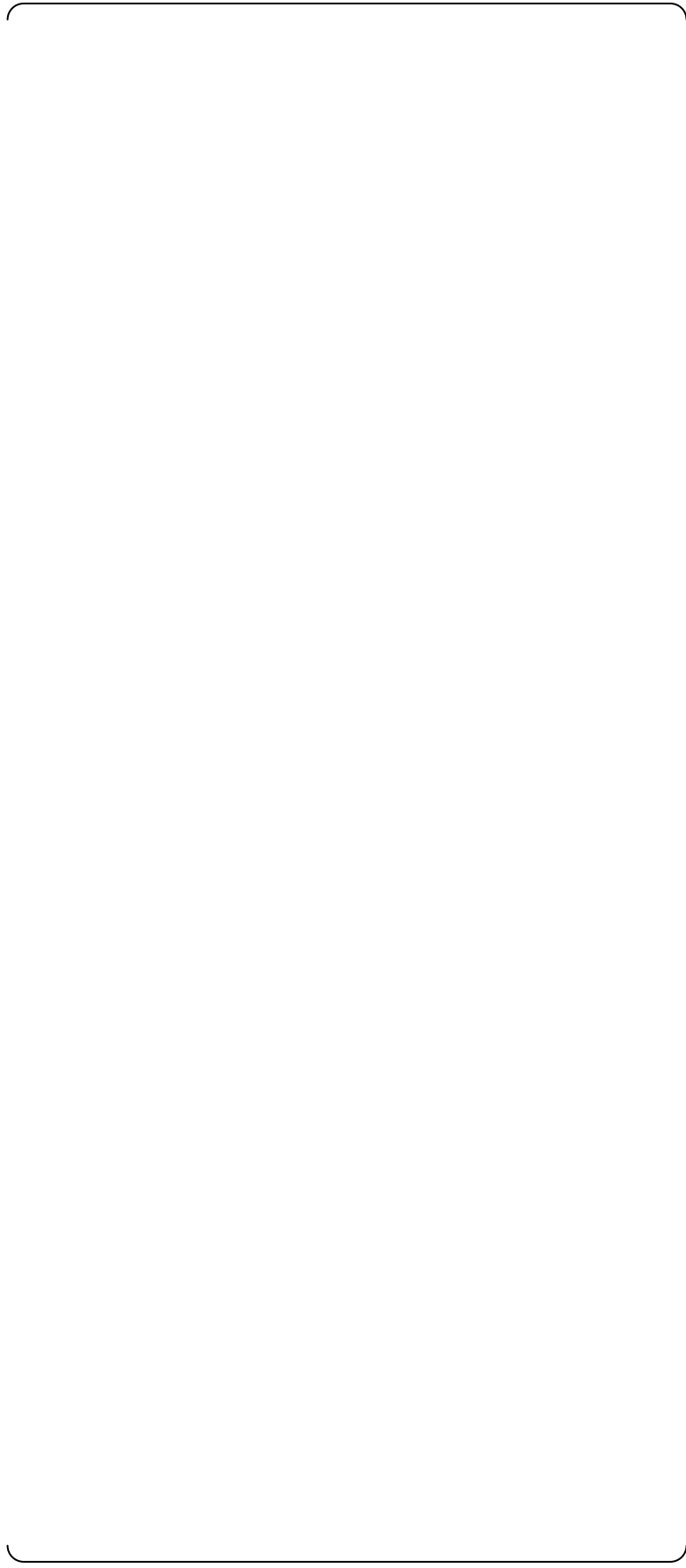
Table A1-5-1-6 Level A, B temperature and pressure input data (31/33)  
(Section XI)

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Table A1-5-1-6 Level A, B temperature and pressure input data (32/33)  
(Section XI)

Table A1-5-1-6 Level A, B temperature and pressure input data (33/33)  
(Section XI)

Table A1-5-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-5-1-2 Floor response curve (1/6)**  
RHRS Suction (RH01-02) FRS for MCP Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-5-1-2 Floor response curve (2/6)**  
RHRS Suction (RH01-02) FRS for MCP Nozzle  
Y (NS) direction (damping 4.0%)



**Figure A1-5-1-2 Floor response curve (3/6)**  
RHRS Suction (RH01-02) FRS for MCP Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-5-1-2 Floor response curve (4/6)**  
RHRS Suction (RH01-02) FRS for Piping  
X (EW) direction (damping 4.0%)



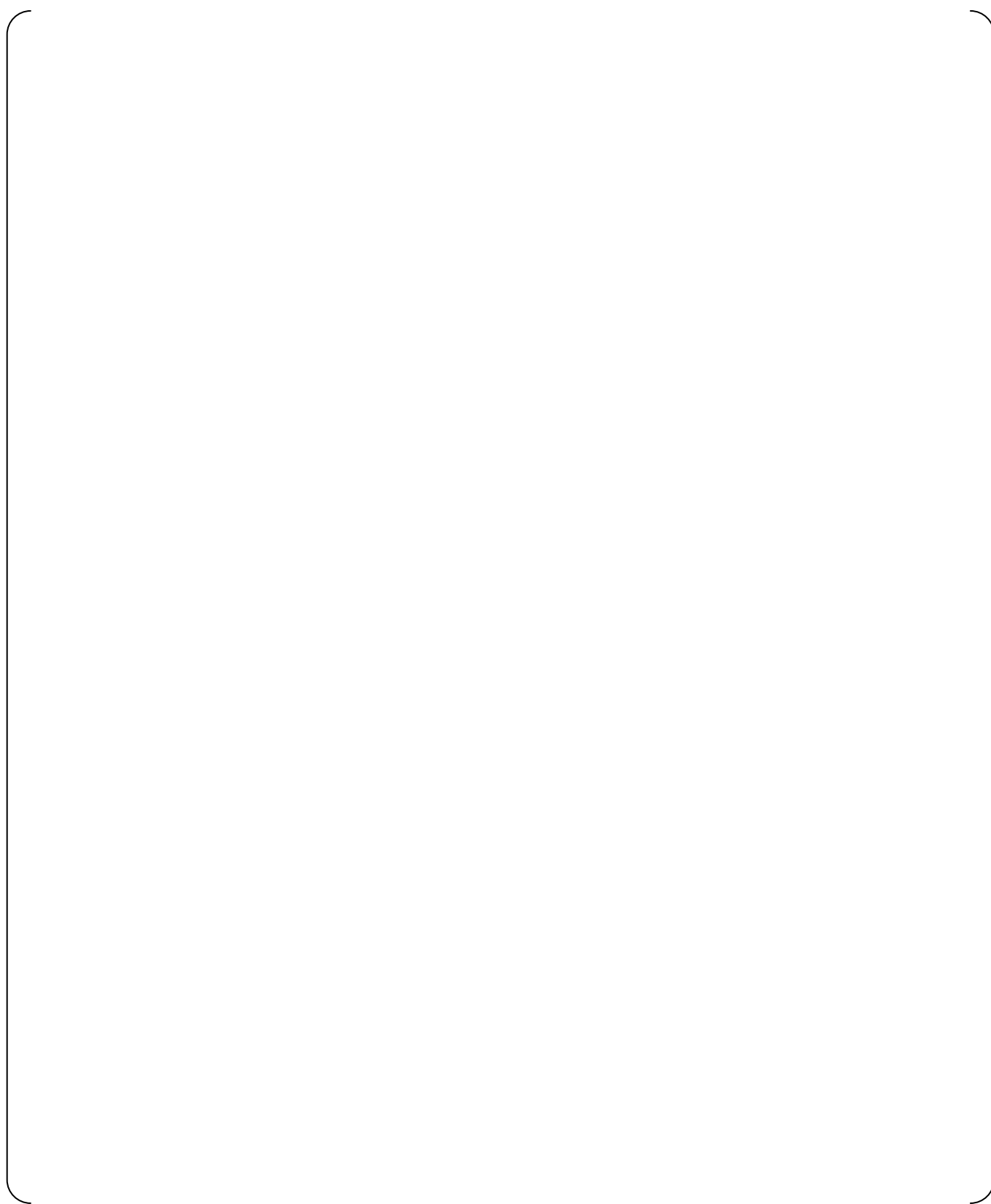


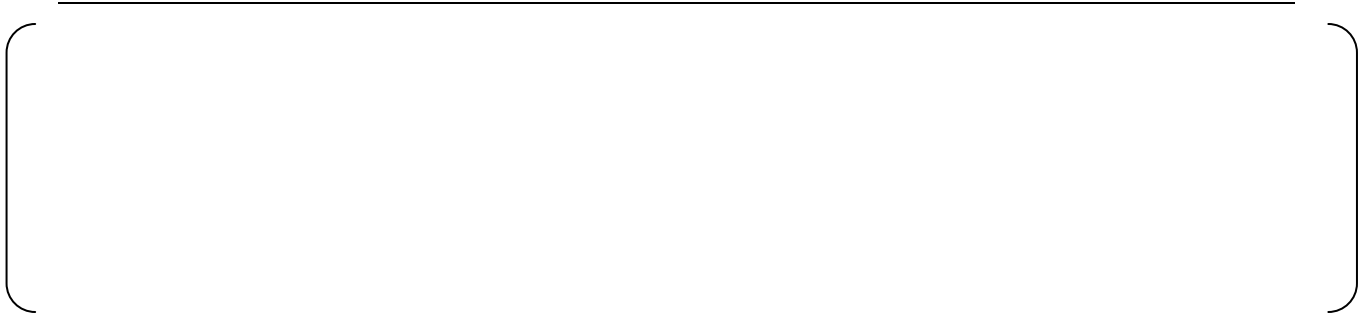
**Figure A1-5-1-2 Floor response curve (5/6)**  
RHRS Suction (RH01-02) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-5-1-2 Floor response curve (6/6)**  
RHRS Suction (RH01-02) FRS for Piping  
Z (Vert.) direction (damping 4.0%)

Table A1-5-1-8 Seismic anchor displacement input data





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Table A1-5-1-9 DBPB displacement input data

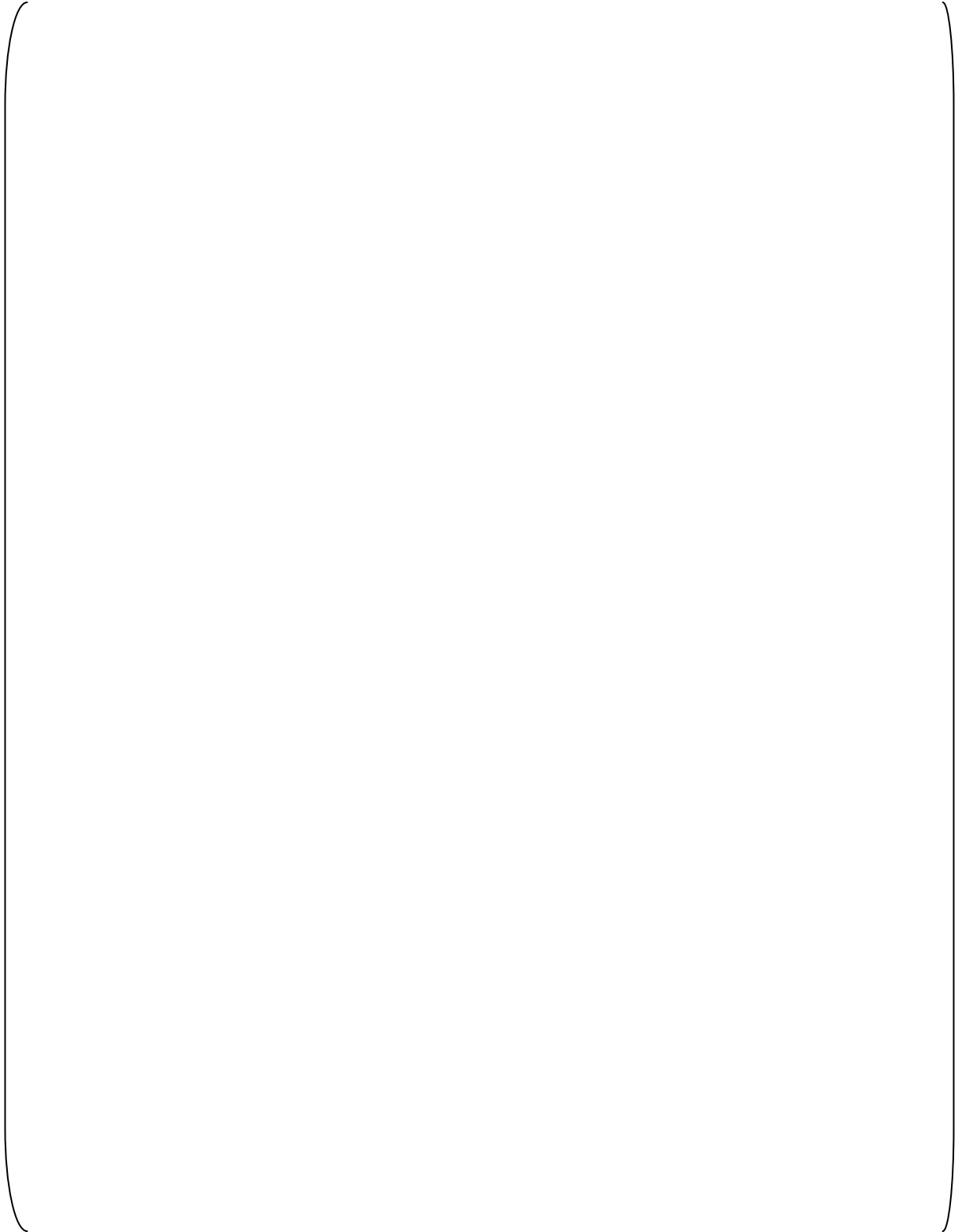


Figure A1-5-2-1 PIPESTRESS analysis model diagram

Table A1-5-2-1 Natural frequency analysis results





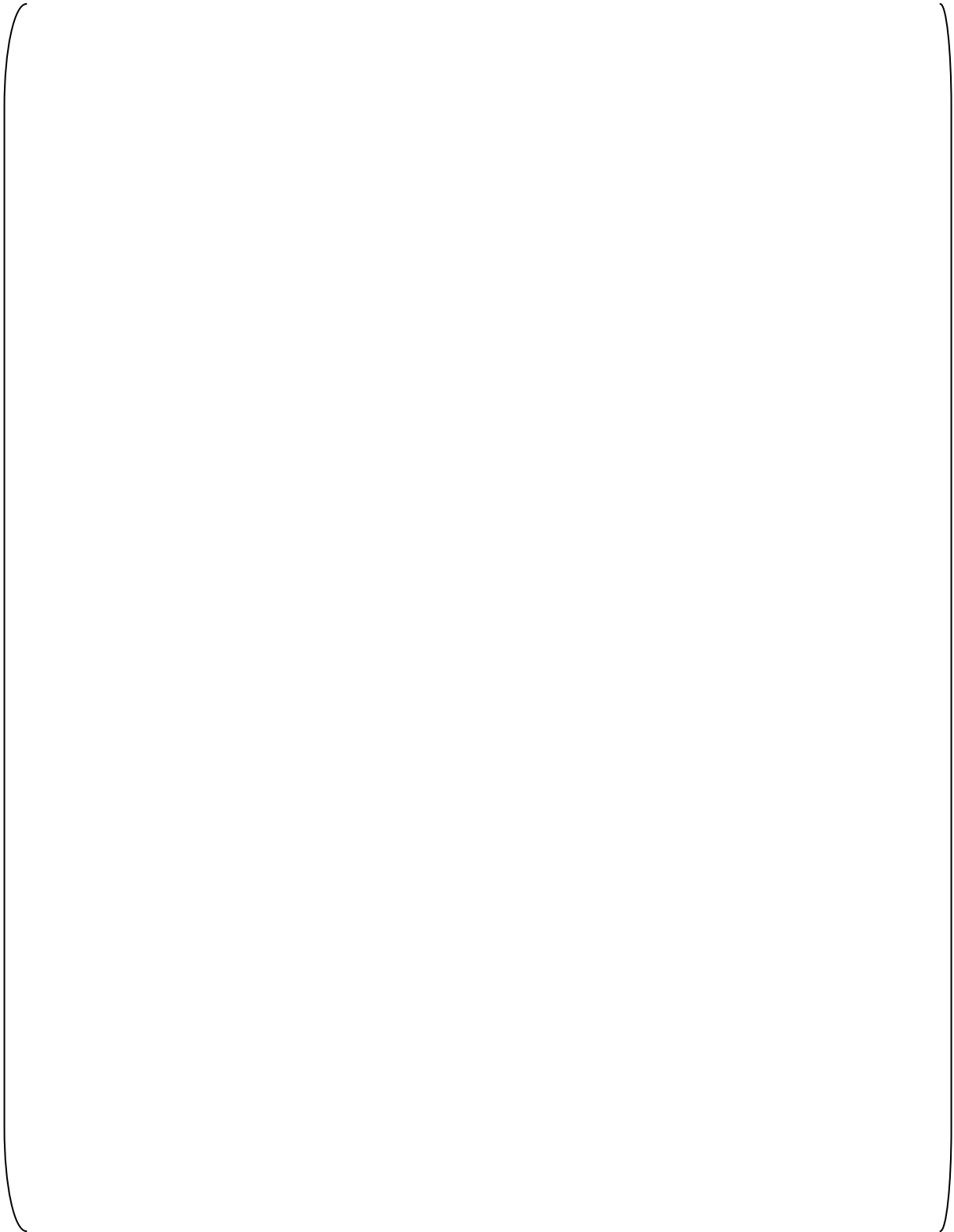


Figure A1-5-2-2 Frequency mode diagram (primary)

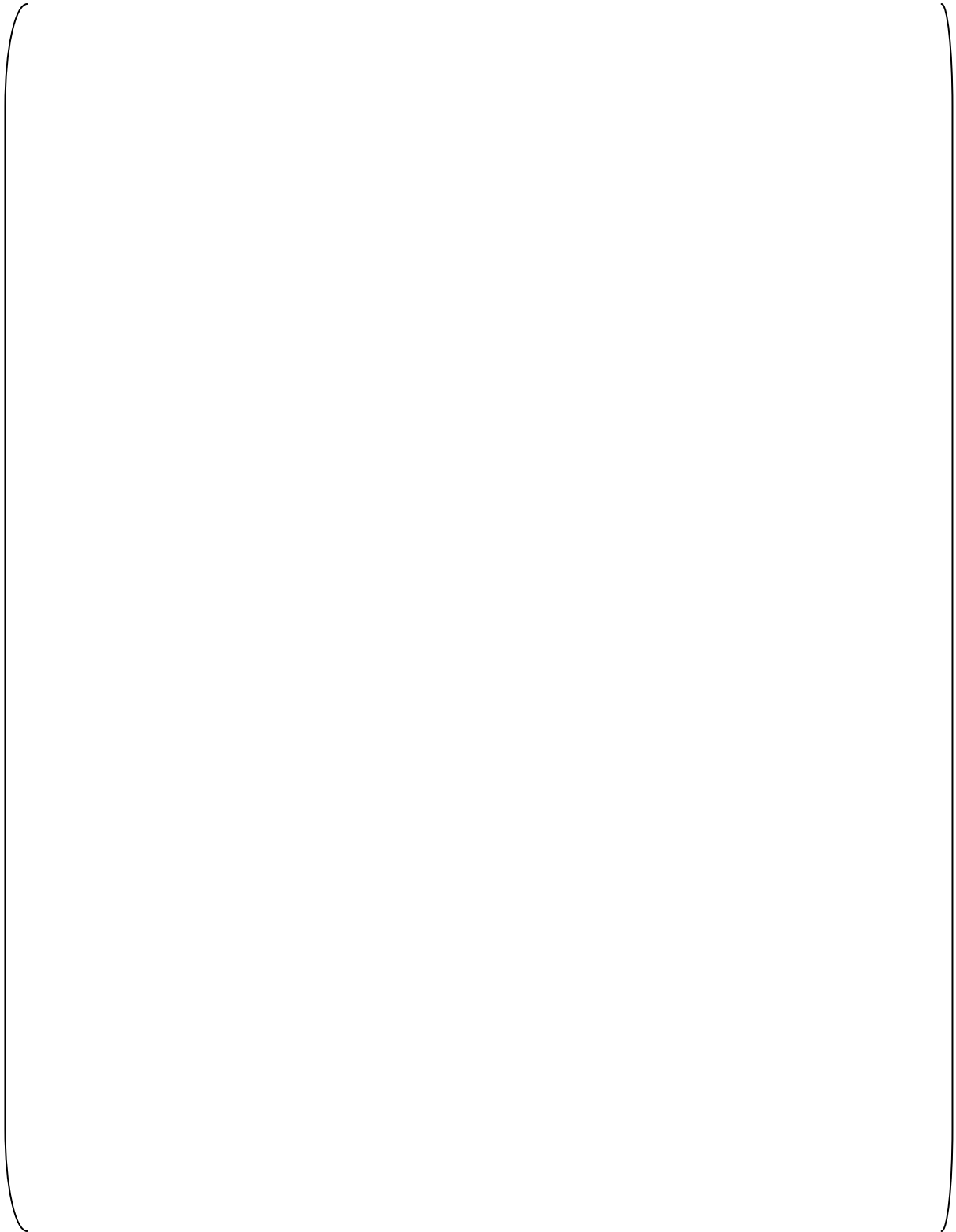


Figure A1-5-2-2 Frequency mode diagram (secondary)

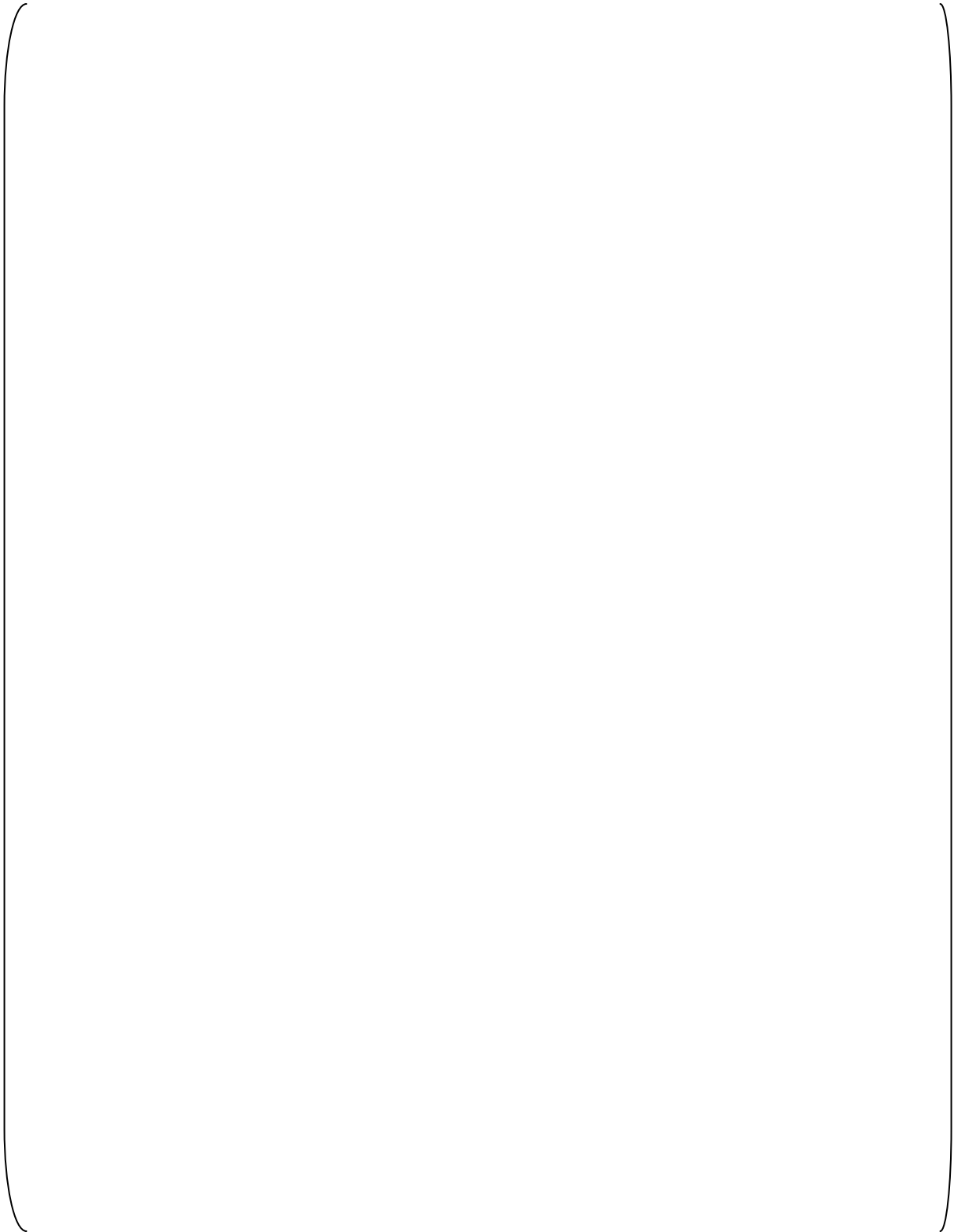


Figure A1-5-2-2 Frequency mode diagram (tertiary)

Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (1/21)  
(Section I)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (2/21)  
(Section I)

| Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ , $\Delta T2$ , Ta-Tb) (2/21)<br>(Section I) |
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**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

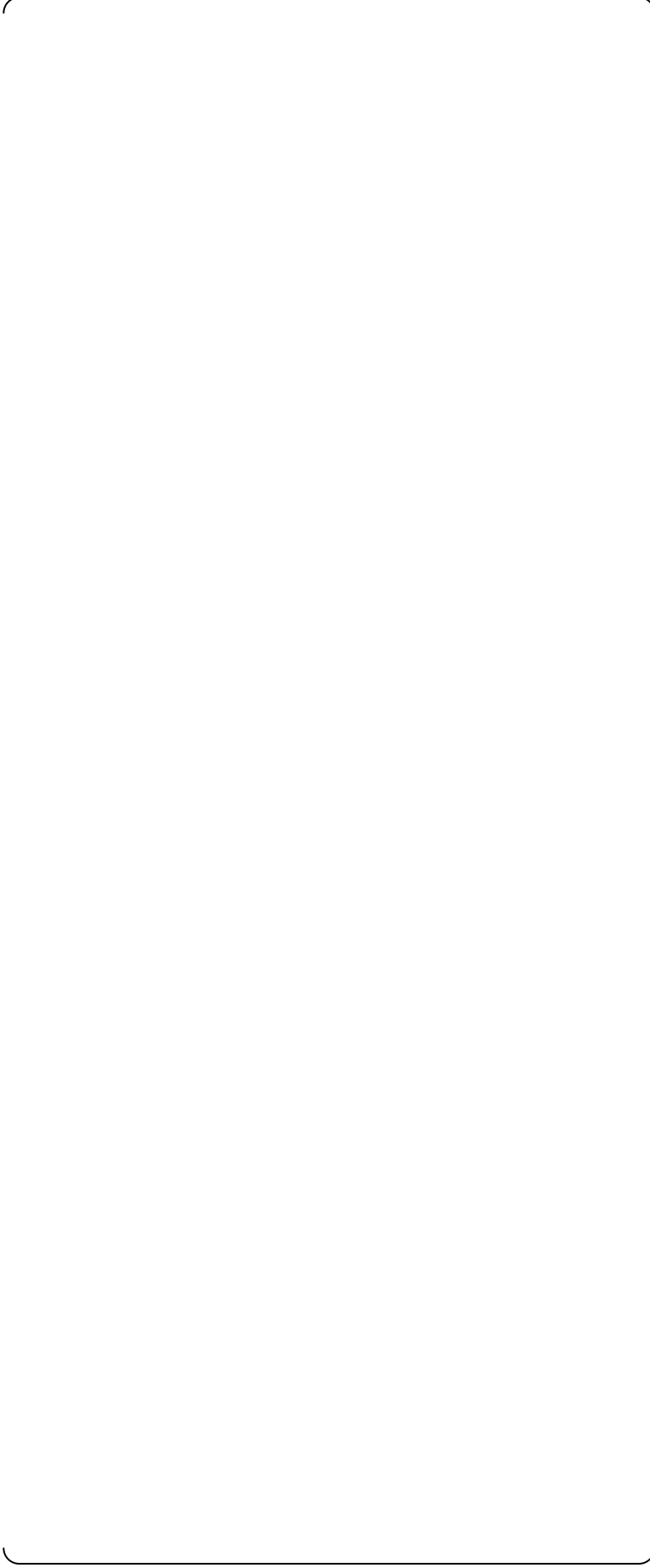
**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**









**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (7/21)  
(Section III)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-5-2-2 Thermal analysis results ( $\Delta T_1$ ,  $\Delta T_2$ , Ta-Tb) (8/21)  
(Section III)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (9/21)  
(Section III)

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**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (13/21)  
(Section V)

| Location | $\Delta T1$ | $\Delta T2$ | Ta-Tb |
|----------|-------------|-------------|-------|
|----------|-------------|-------------|-------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (14/21)  
(Section V)

| Location | $\Delta T1$ | $\Delta T2$ | Ta-Tb |
|----------|-------------|-------------|-------|
|----------|-------------|-------------|-------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (15/21)  
(Section V)

| Location | $\Delta T1$ | $\Delta T2$ | Ta-Tb |
|----------|-------------|-------------|-------|
|----------|-------------|-------------|-------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (16/21)  
(Section VI)

| Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ , $\Delta T2$ , $Ta-Tb$ ) (16/21)<br>(Section VI) |  |
|---|--|
|   |  |



Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (17/21)  
(Section VI)

| Location | $\Delta T1$ | $\Delta T2$ | Ta-Tb |
|----------|-------------|-------------|-------|
|----------|-------------|-------------|-------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (18/21)  
(Section VI)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (19/21)  
(Section VII)





Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (20/21)  
(Section VII)

| Item | Value |
|------|-------|
|------|-------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-5-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (21/21)  
(Section VII)

| Item | Value |
|------|-------|
|------|-------|

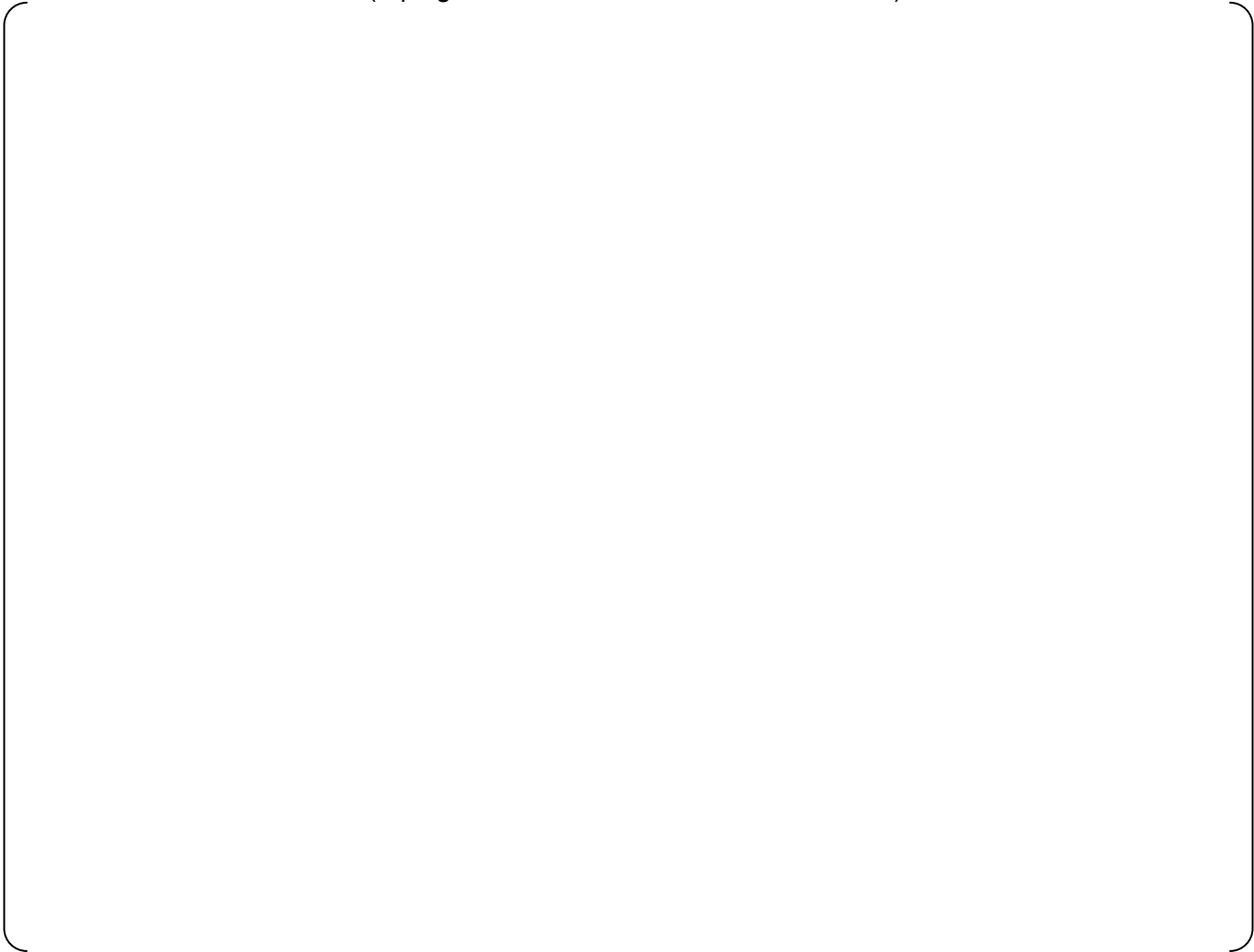
**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



**Table A1-5-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-5-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)





## Appendix 1-6

### RH02 RHRS suction Loop B Line Piping Analysis Results

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|  |                 |
|--|-----------------|
| 1. INPUT   |                 |
| 1.1 Used for creating the pipe structural model                              |                 |
| 1.1.1 Block division and piping specifications                               | Table A1-6-1-1  |
| 1.1.2 Piping isometrics  | Figure A1-6-1-1 |
| 1.1.3 Concentrated mass  | Table A1-6-1-2  |
| 1.1.4 Support point rigidity   | Table A1-6-1-3  |
| 1.1.5 Valve rigidity   | Table A1-6-1-4  |
| 1.2 Used for creating load conditions  |                 |
| 1.2.1 Level A/B design transient   | see main text   |
| 1.2.2 Level A/B thermal displacement input data                              | Table A1-6-1-5  |
| 1.2.3 Level A, B temperature and pressure input data                         | Table A1-6-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data                 | Table A1-6-1-7  |
| 1.2.5 Floor response curve   | Figure A1-6-1-2 |
| 1.2.6 Seismic anchor displacement input data                                 | Table A1-6-1-8  |
| 1.2.7 DBPB displacement input data   | Table A1-6-1-9  |
| 2. OUTPUT  |                 |
| 2.1 PIPESTRESS analysis model diagram  | Figure A1-6-2-1 |
| 2.2 Natural frequency analysis results                                       | Table A1-6-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                             | Figure A1-6-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a$ - $T_b$ ) | Table A1-6-2-2  |
| 2.5 Piping stress and fatigue evaluation results                             | Table A1-6-2-3  |

Table A1-6-1-1 Block division and piping specifications (1/2)

Table A1-6-1-1 Block division and piping specifications (2/2)

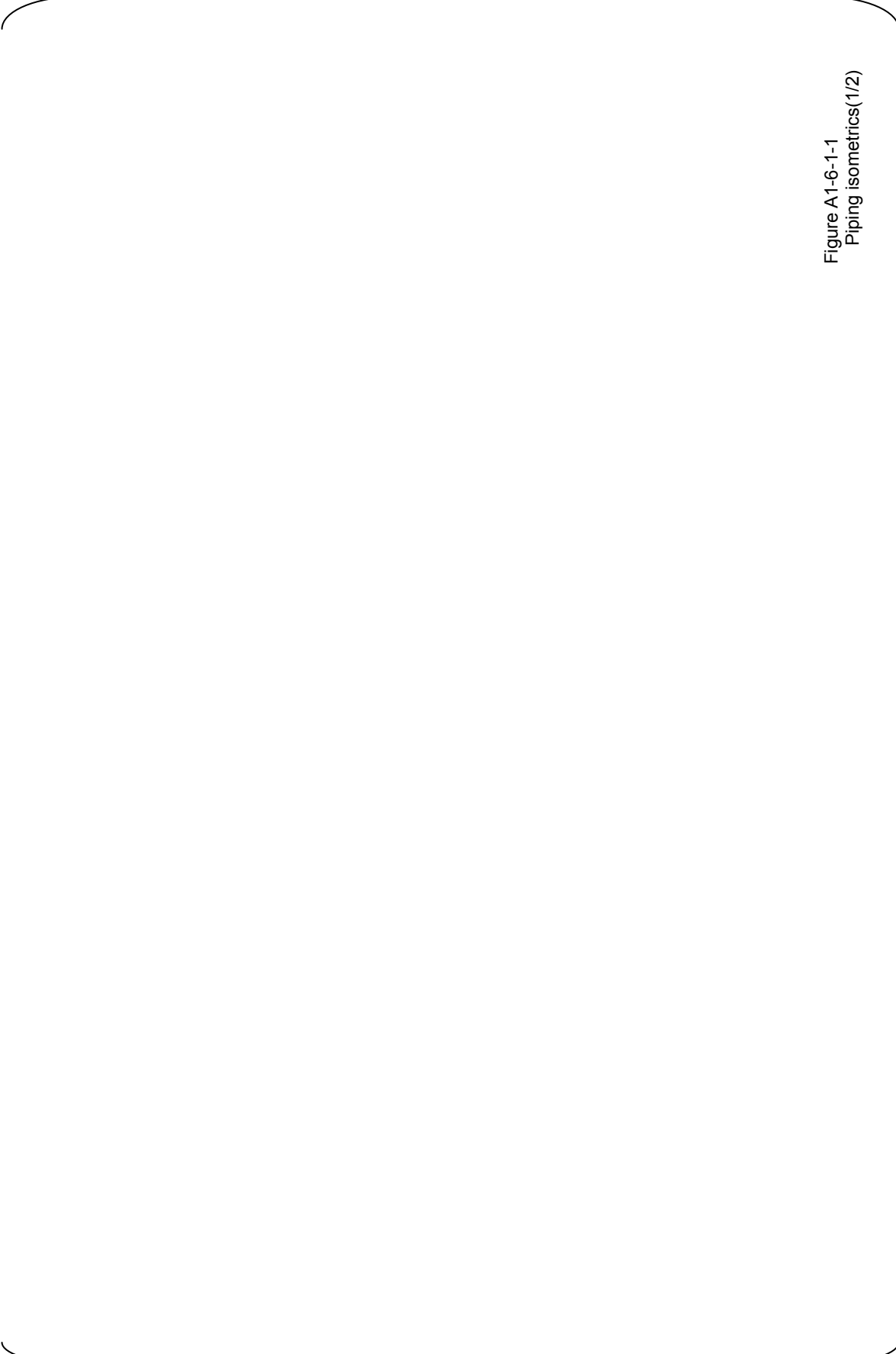


Figure A1-6-1-1  
Piping isometrics(1/2)

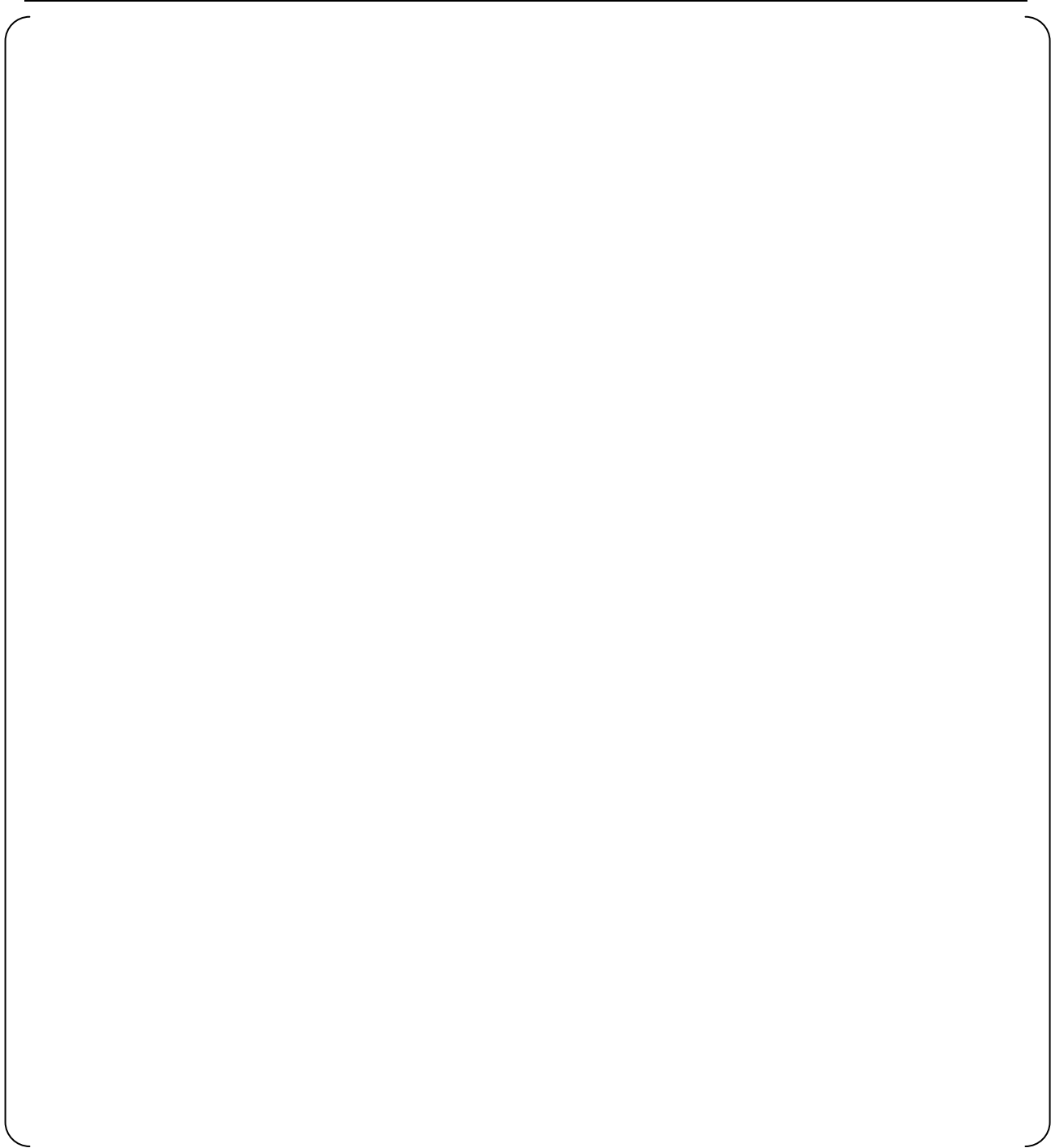


Figure A1-6-1-1  
Piping isometrics(2/2)

Table A1-6-1-2 Concentrated mass

Table A1-6-1-3 Support point rigidity







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Table A1-6-1-4 Valve rigidity

Table A1-6-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



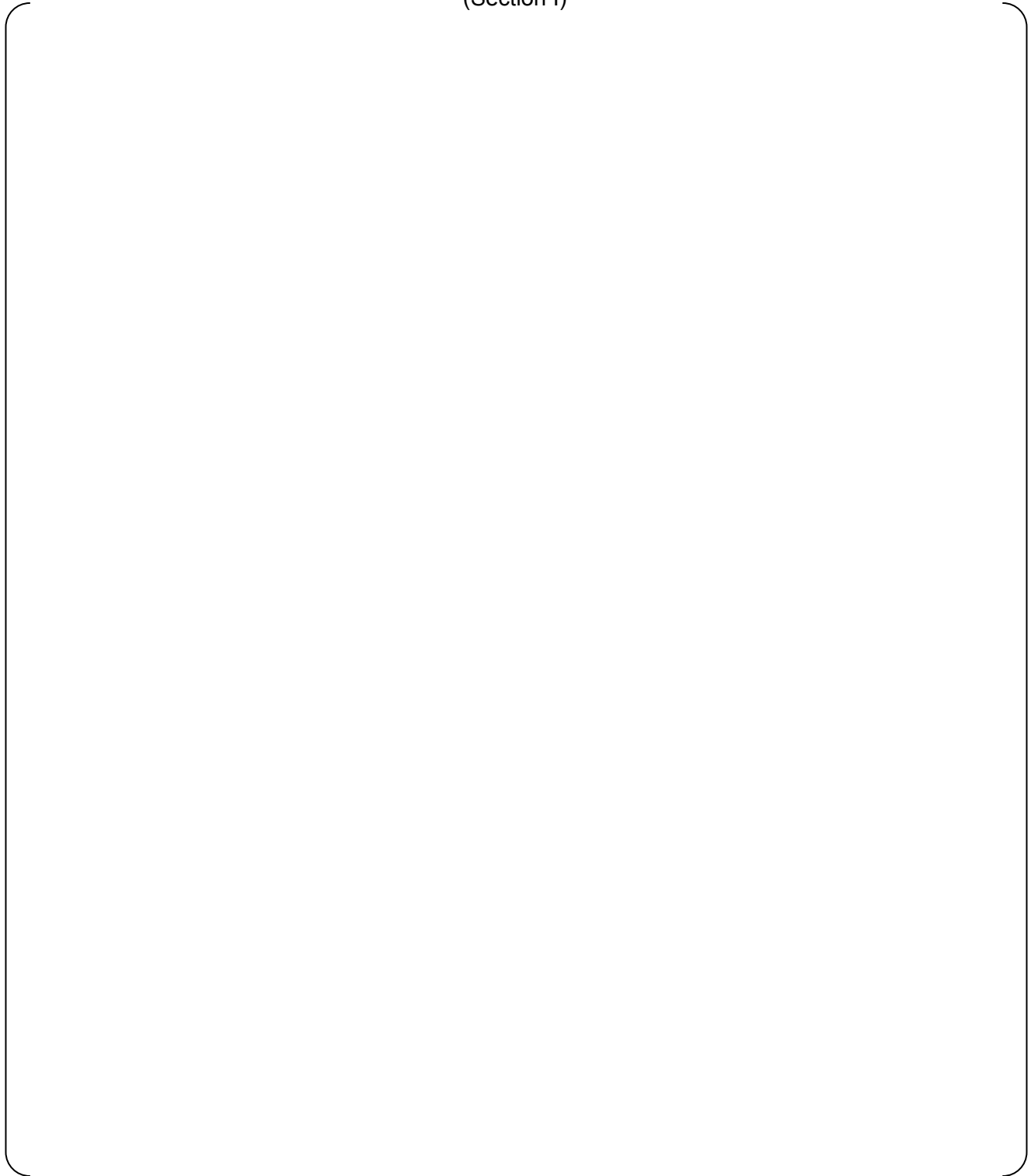
Table A1-6-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |



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**Table A1-6-1-6 Level A, B temperature and pressure input data (1/21)**  
(Section I)

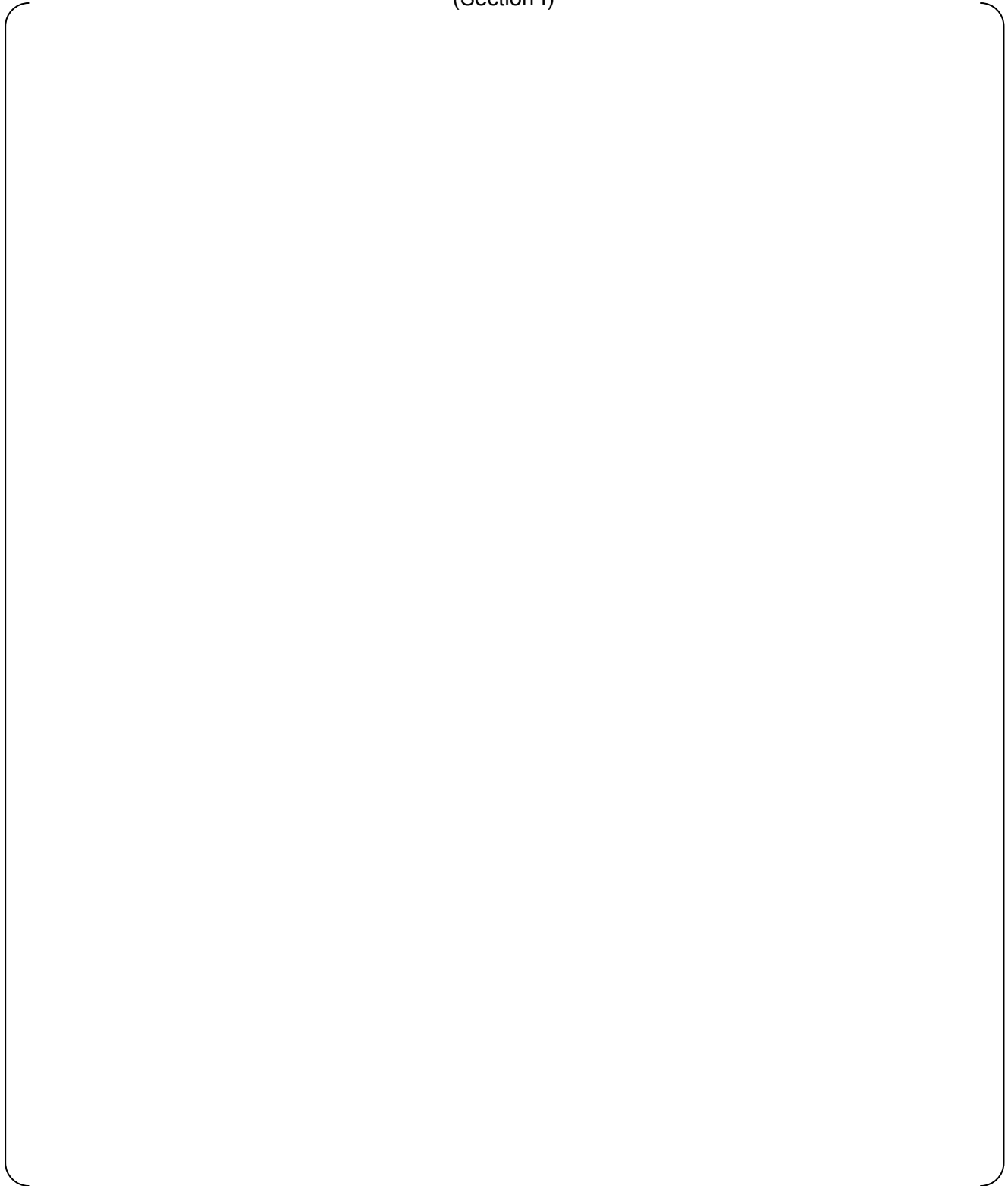


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Table A1-6-1-6 Level A, B temperature and pressure input data (2/21)  
(Section I)

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Table A1-6-1-6 Level A, B temperature and pressure input data (3/21)  
(Section I)



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Table A1-6-1-6 Level A, B temperature and pressure input data (4/21)  
(Section II)

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Table A1-6-1-6 Level A, B temperature and pressure input data (5/21)  
(Section II)

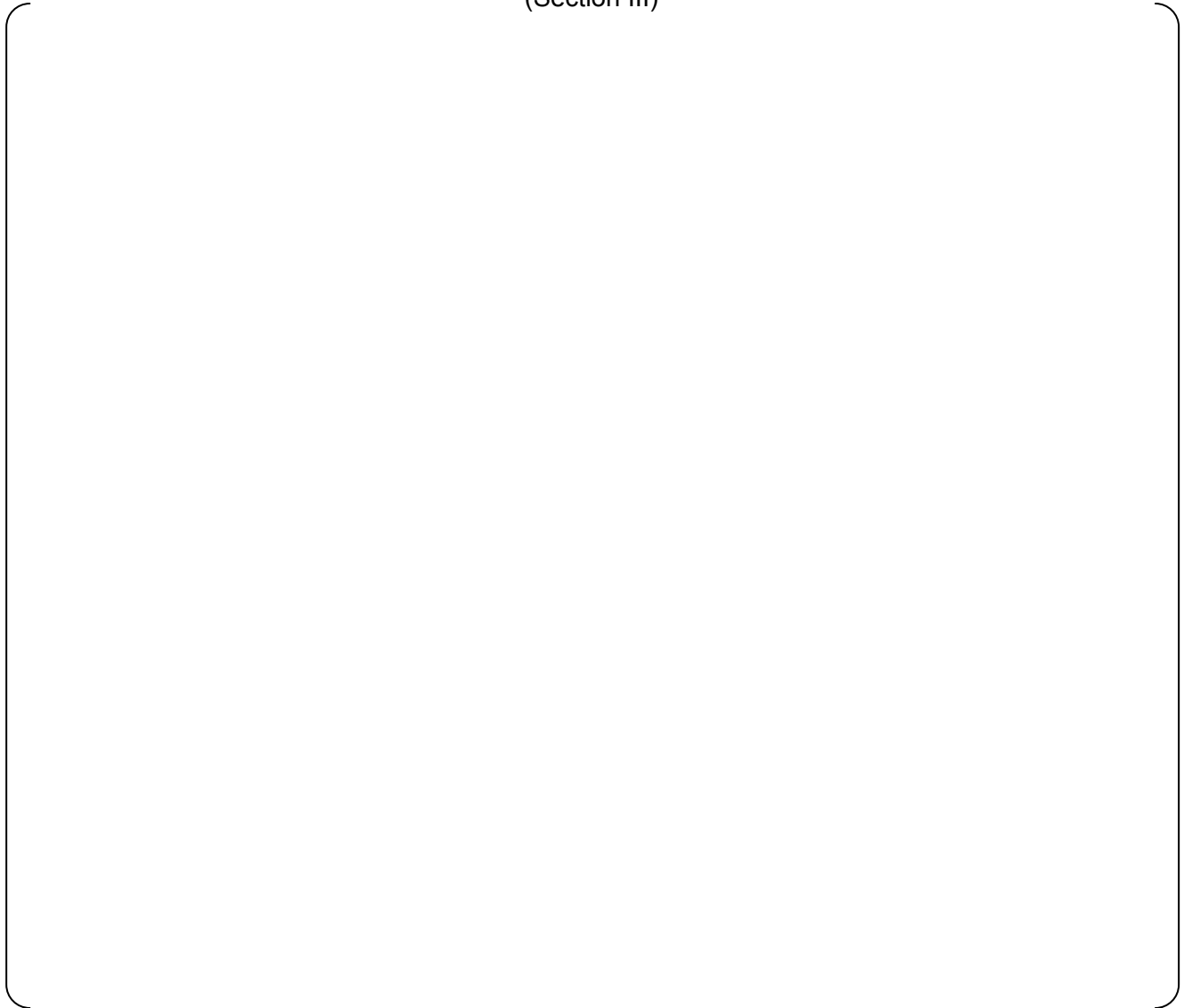
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Table A1-6-1-6 Level A, B temperature and pressure input data (6/21)  
(Section II)

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**Table A1-6-1-6 Level A, B temperature and pressure input data (7/21)**  
(Section III)

Table A1-6-1-6 Level A, B temperature and pressure input data (8/21)  
(Section III)





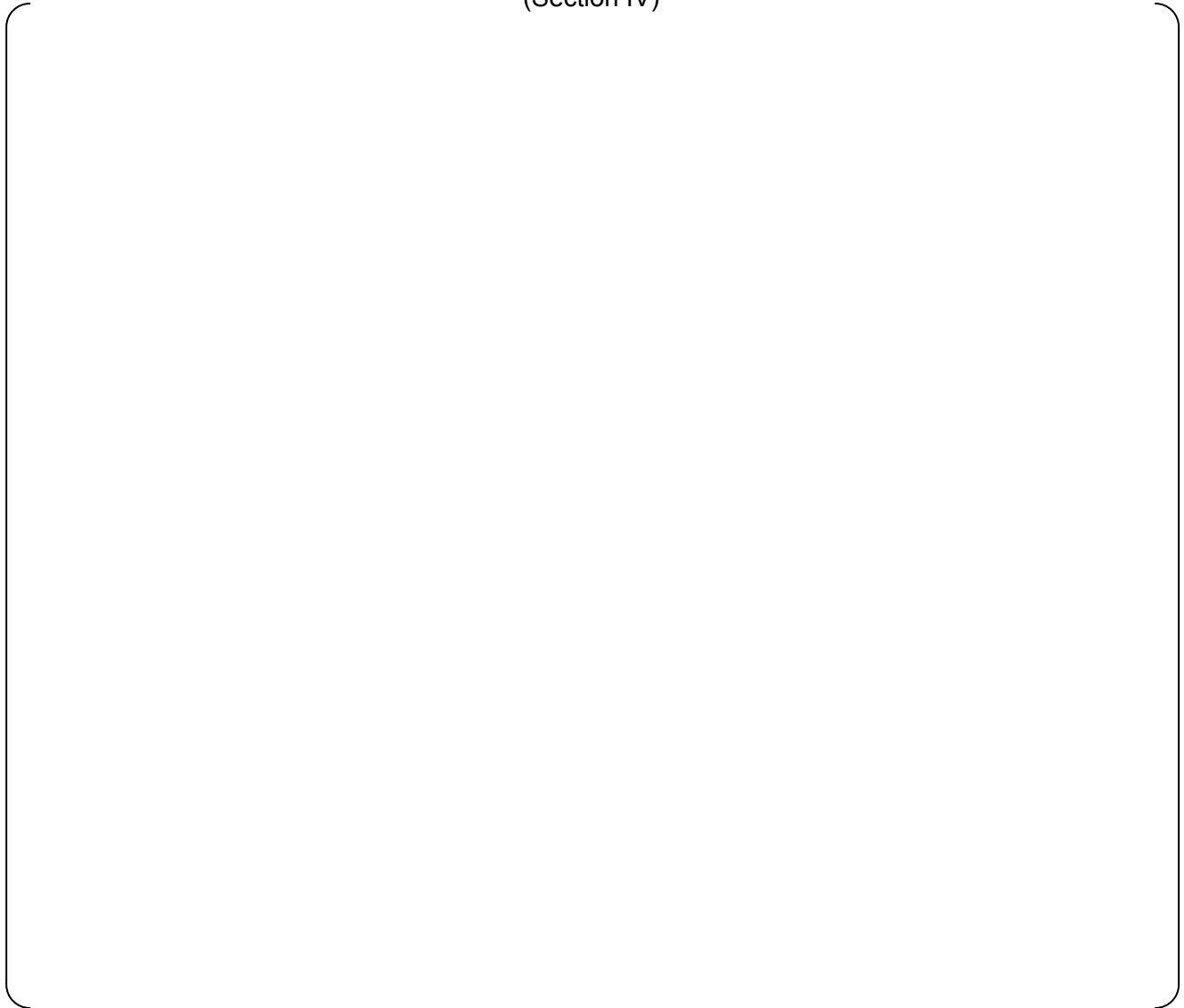
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Table A1-6-1-6 Level A, B temperature and pressure input data (9/21)  
(Section III)

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Table A1-6-1-6 Level A, B temperature and pressure input data (10/21)  
(Section IV)

Table A1-6-1-6 Level A, B temperature and pressure input data (11/21)  
(Section IV)



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Table A1-6-1-6 Level A, B temperature and pressure input data (12/21)  
(Section IV)

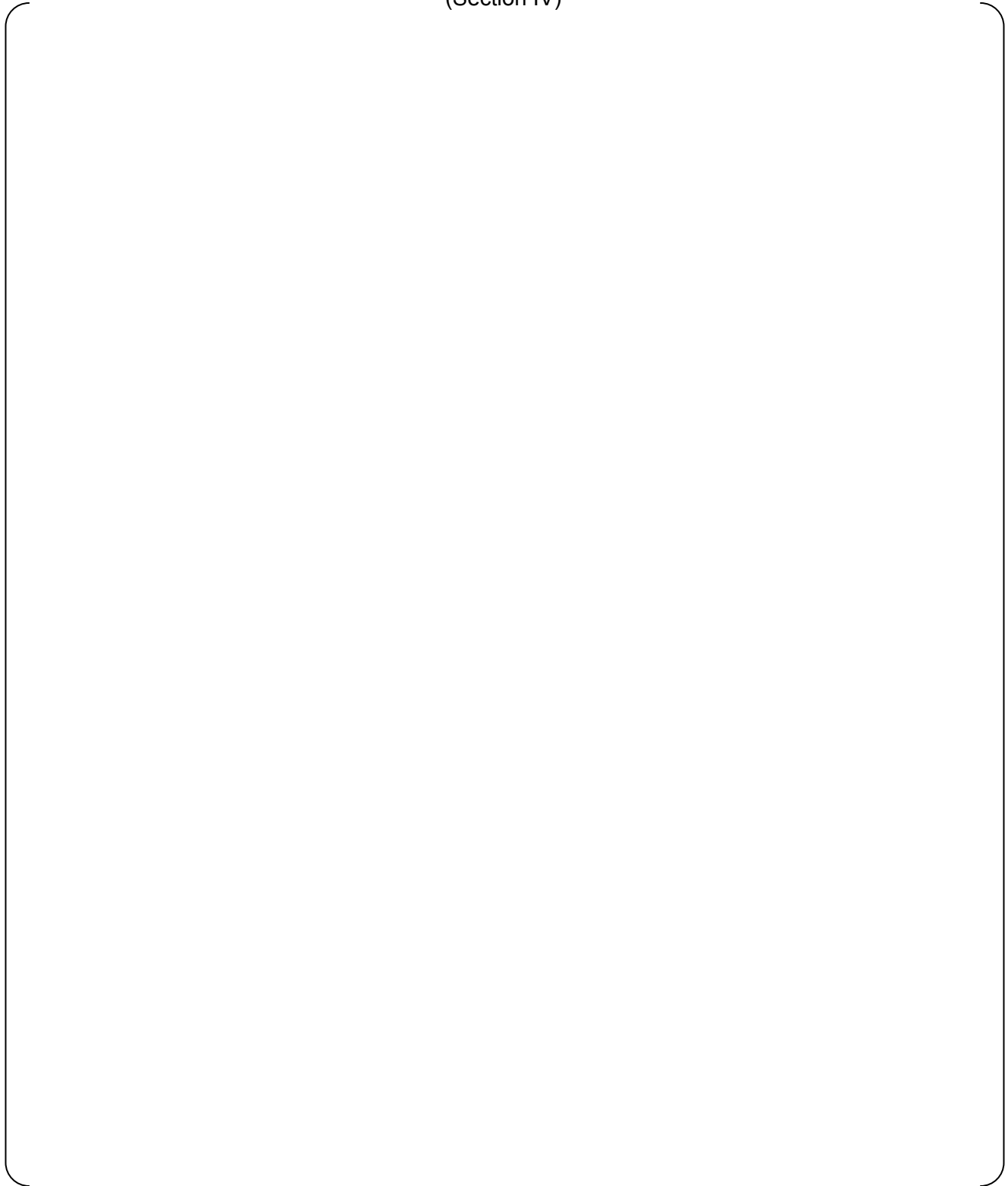


Table A1-6-1-6 Level A, B temperature and pressure input data (13/21)  
(Section V)

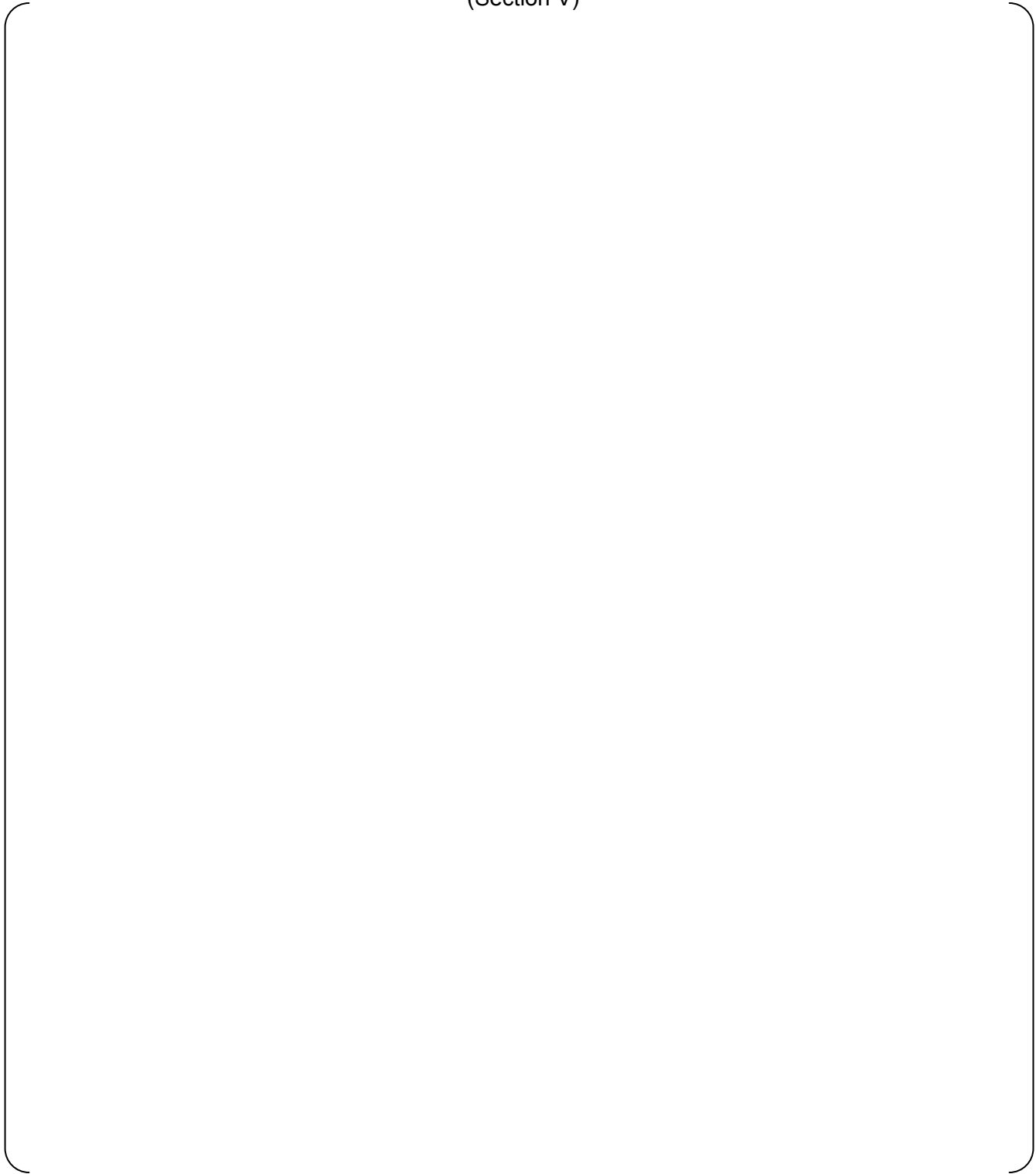
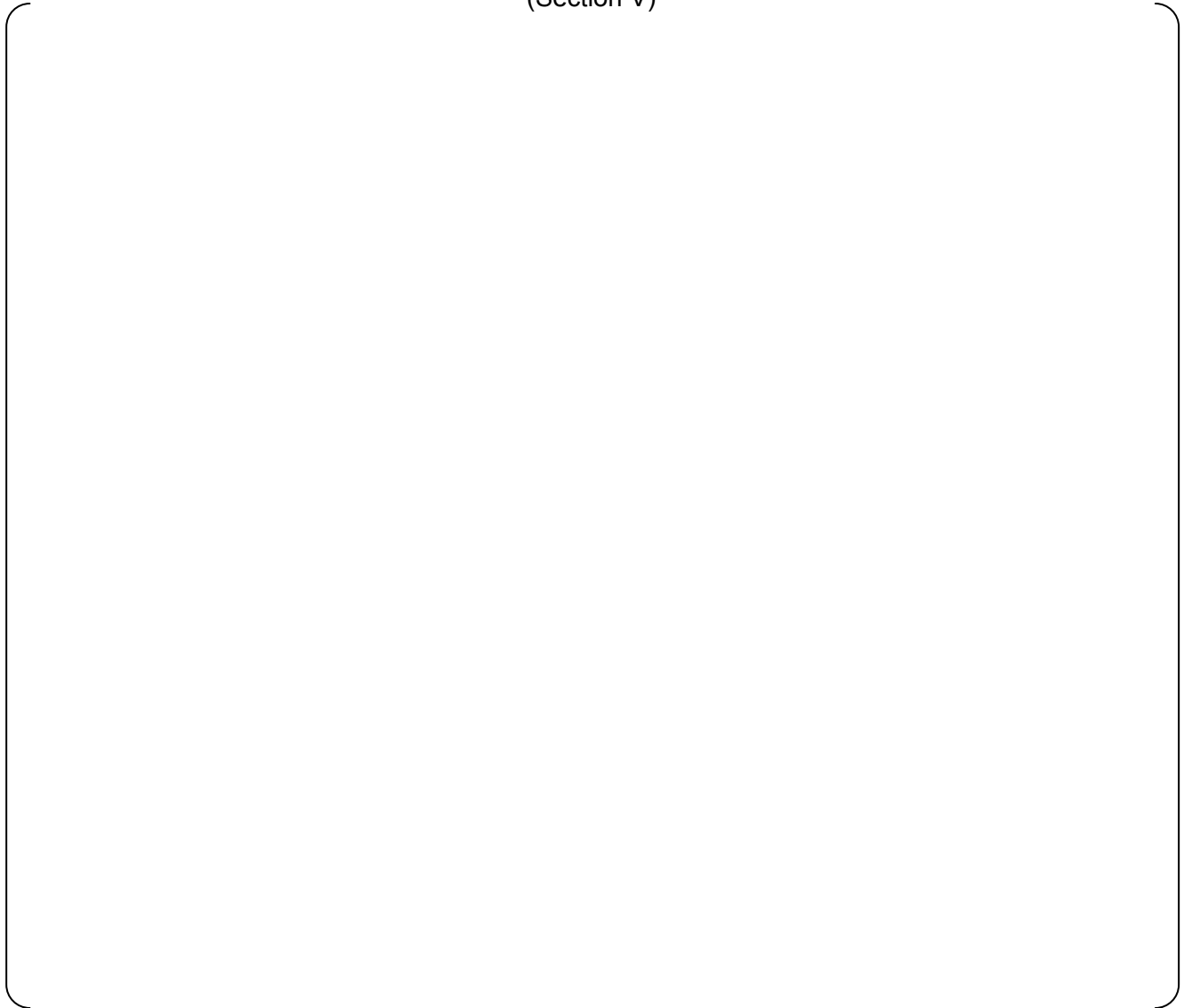


Table A1-6-1-6 Level A, B temperature and pressure input data (14/21)  
(Section V)



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Table A1-6-1-6 Level A, B temperature and pressure input data (15/21)  
(Section V)

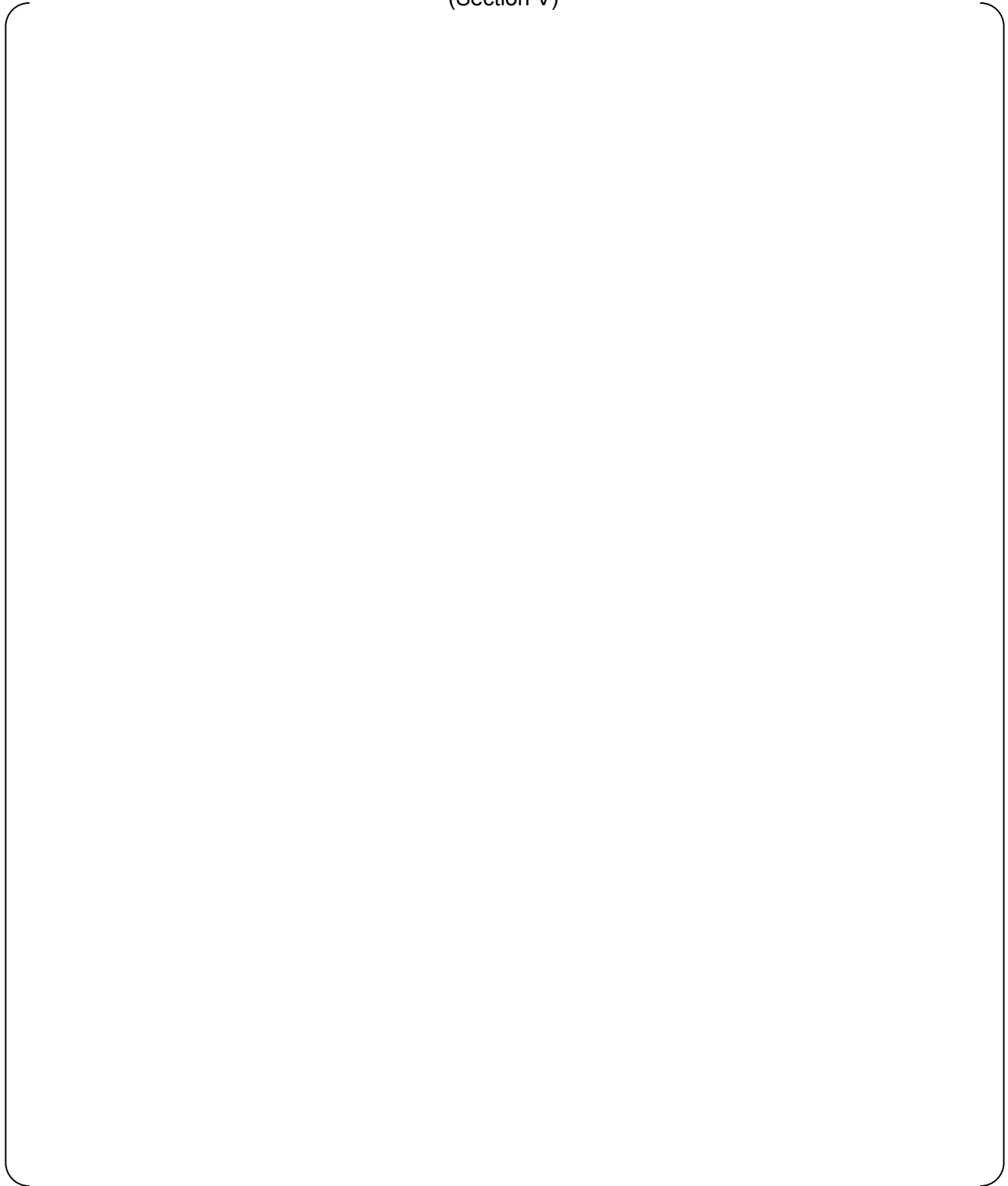
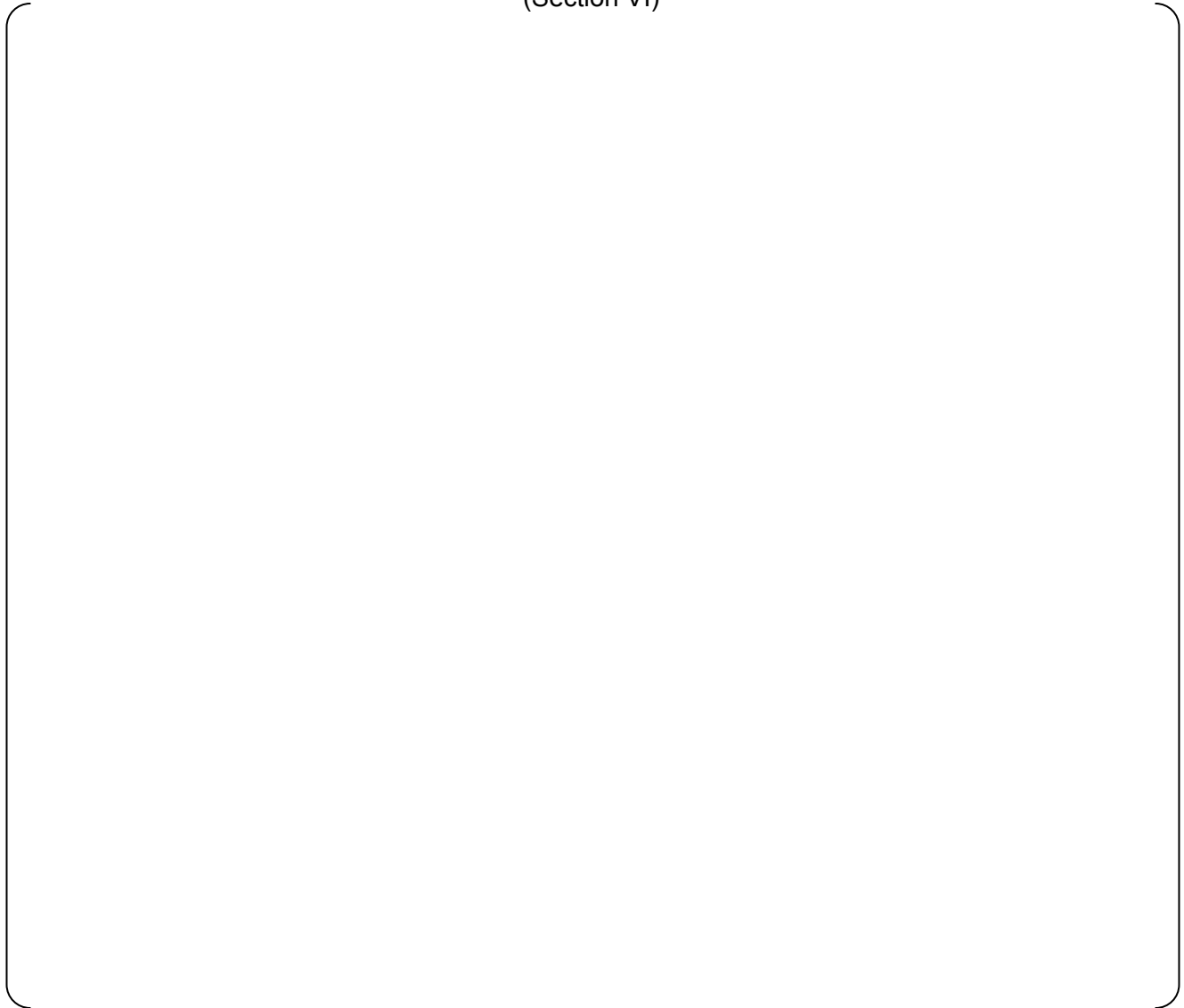


Table A1-6-1-6 Level A, B temperature and pressure input data (16/21)  
(Section VI)



Table A1-6-1-6 Level A, B temperature and pressure input data (17/21)  
(Section VI)



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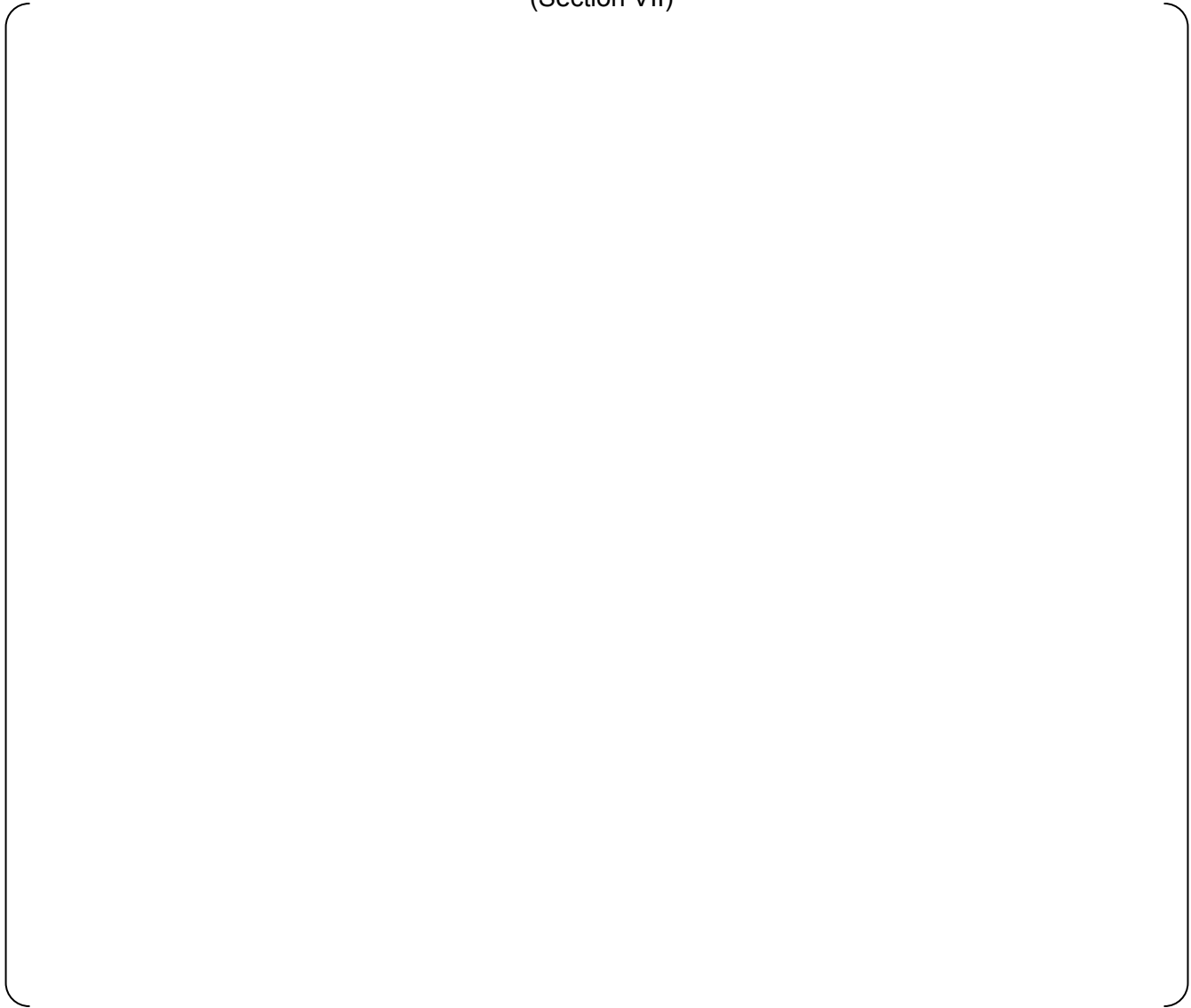
Table A1-6-1-6 Level A, B temperature and pressure input data (18/21)  
(Section VI)

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Table A1-6-1-6 Level A, B temperature and pressure input data (19/21)  
(Section VII)

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Table A1-6-1-6 Level A, B temperature and pressure input data (20/21)  
(Section VII)



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Table A1-6-1-6 Level A, B temperature and pressure input data (21/21)  
(Section VII)

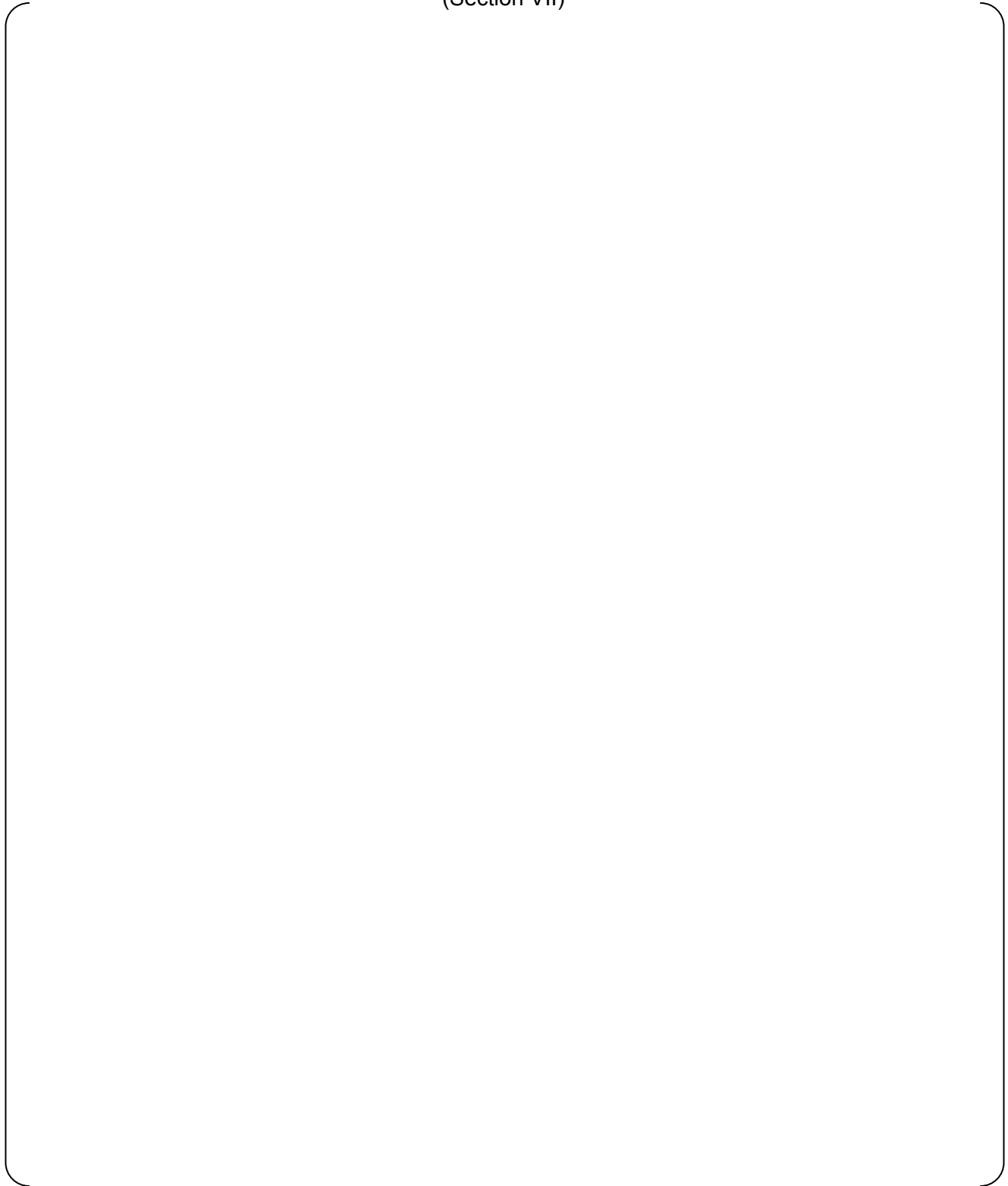


Table A1-6-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-6-1-2 Floor response curve (1/6)**  
RHRS Suction (RH01-02) FRS for MCP Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-6-1-2 Floor response curve (2/6)**  
RHRS Suction (RH01-02) FRS for MCP Nozzle  
Y (NS) direction (damping 4.0%)





**Figure A1-6-1-2 Floor response curve (3/6)**  
RHRS Suction (RH01-02) FRS for MCP Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-6-1-2 Floor response curve (4/6)**  
RHRS Suction (RH01-02) FRS for Piping  
X (EW) direction (damping 4.0%)



**Figure A1-6-1-2 Floor response curve (5/6)**  
RHRS Suction (RH01-02) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-6-1-2 Floor response curve (6/6)**  
RHRS Suction (RH01-02) FRS for Piping  
Z (Vert.) direction (damping 4.0%)

Table A1-6-1-8 Seismic anchor displacement input data



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Table A1-6-1-9 DBPB displacement input data

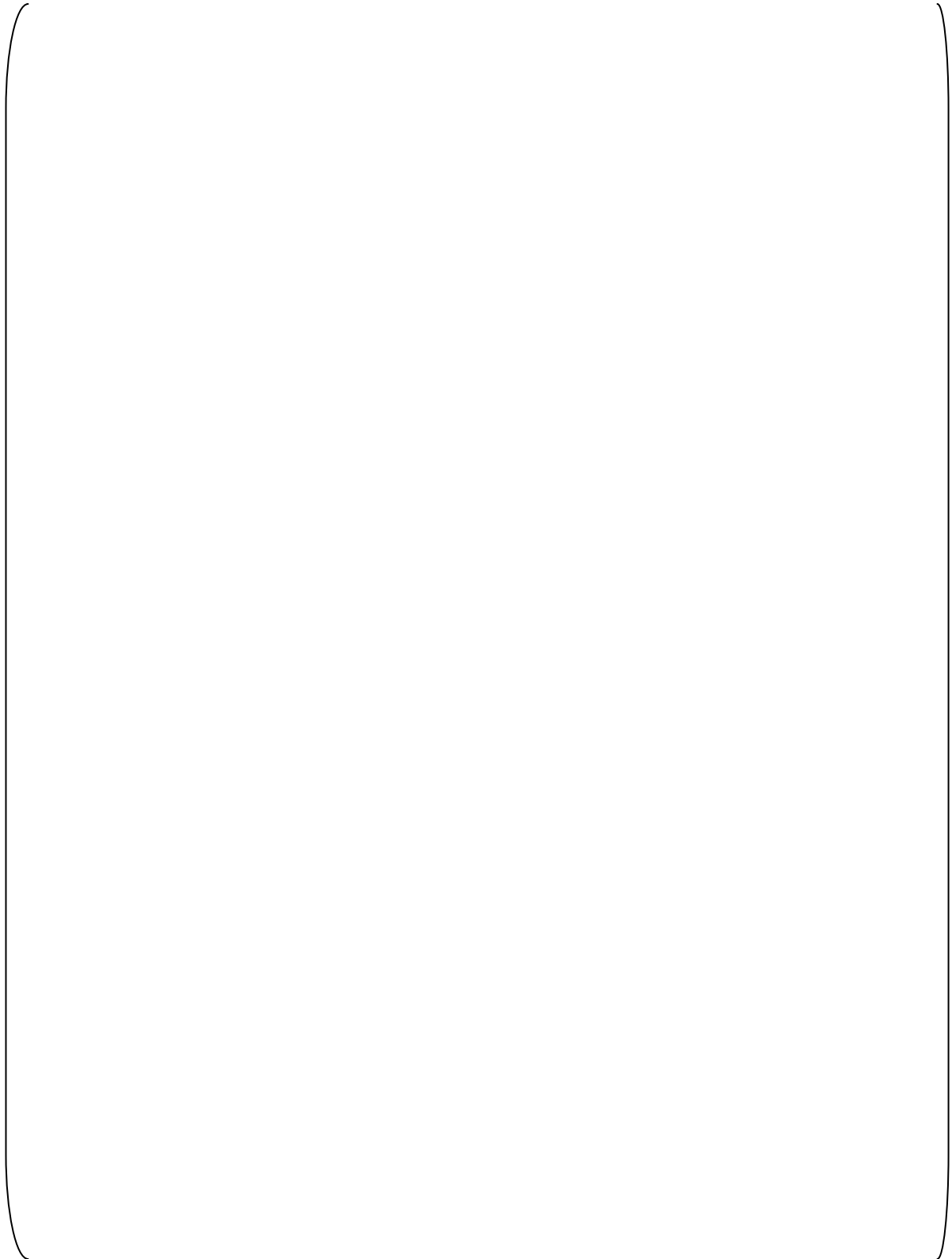


Figure A1-6-2-1 PIPESTRESS analysis model diagram



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Table A1-6-2-1 Natural frequency analysis results



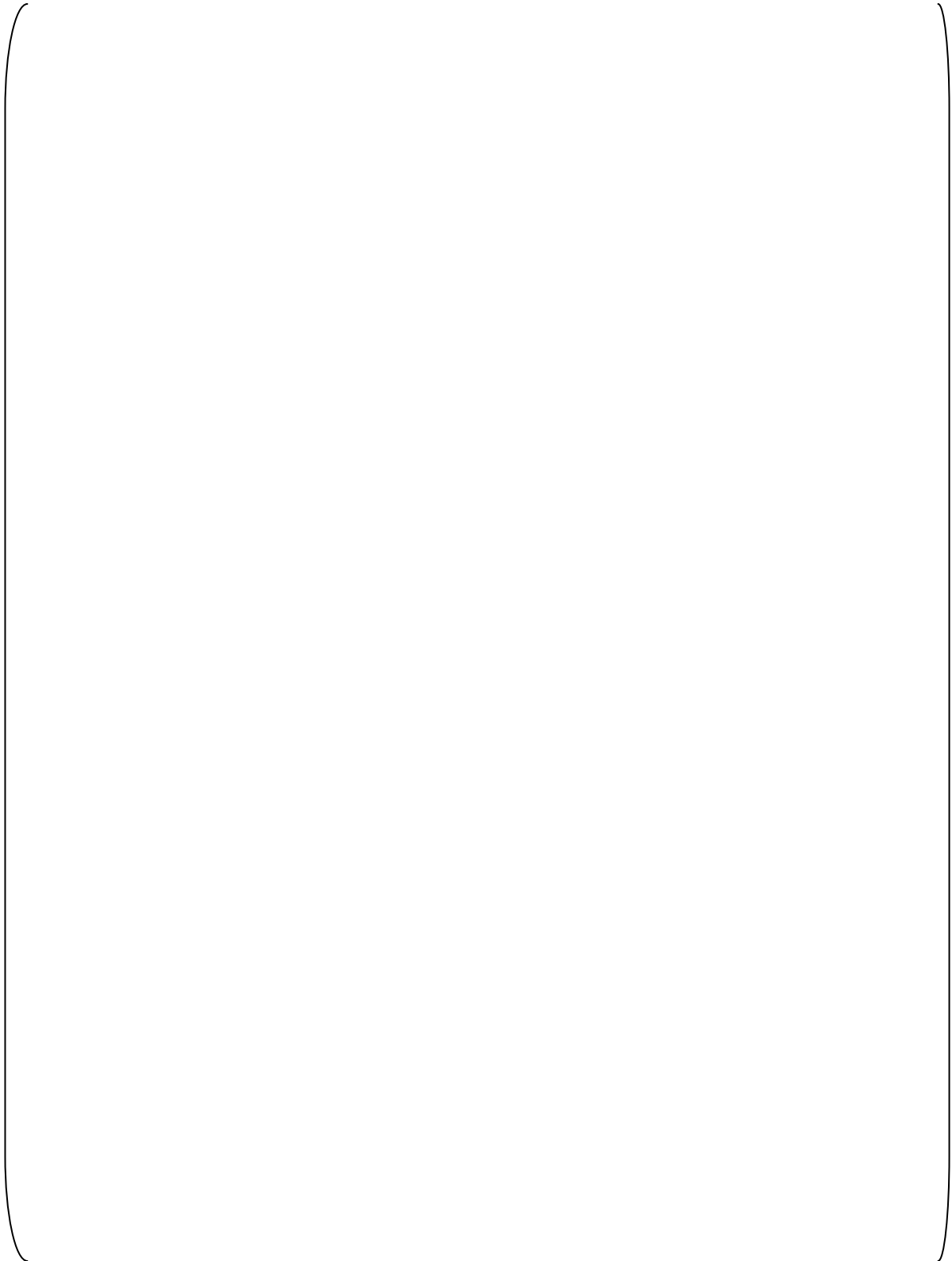


Figure A1-6-2-2 Frequency mode diagram (primary)

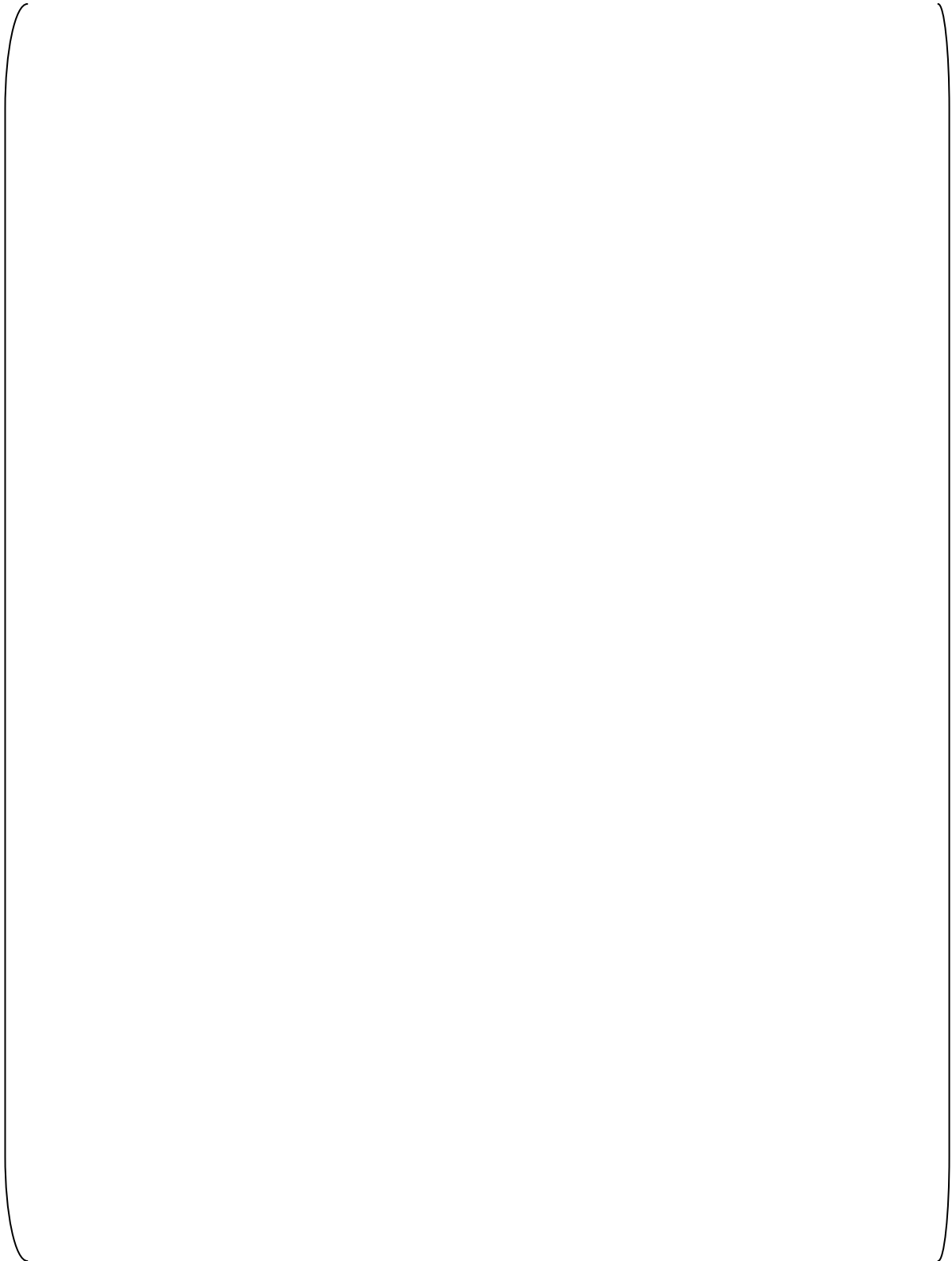


Figure A1-6-2-2 Frequency mode diagram (secondary)

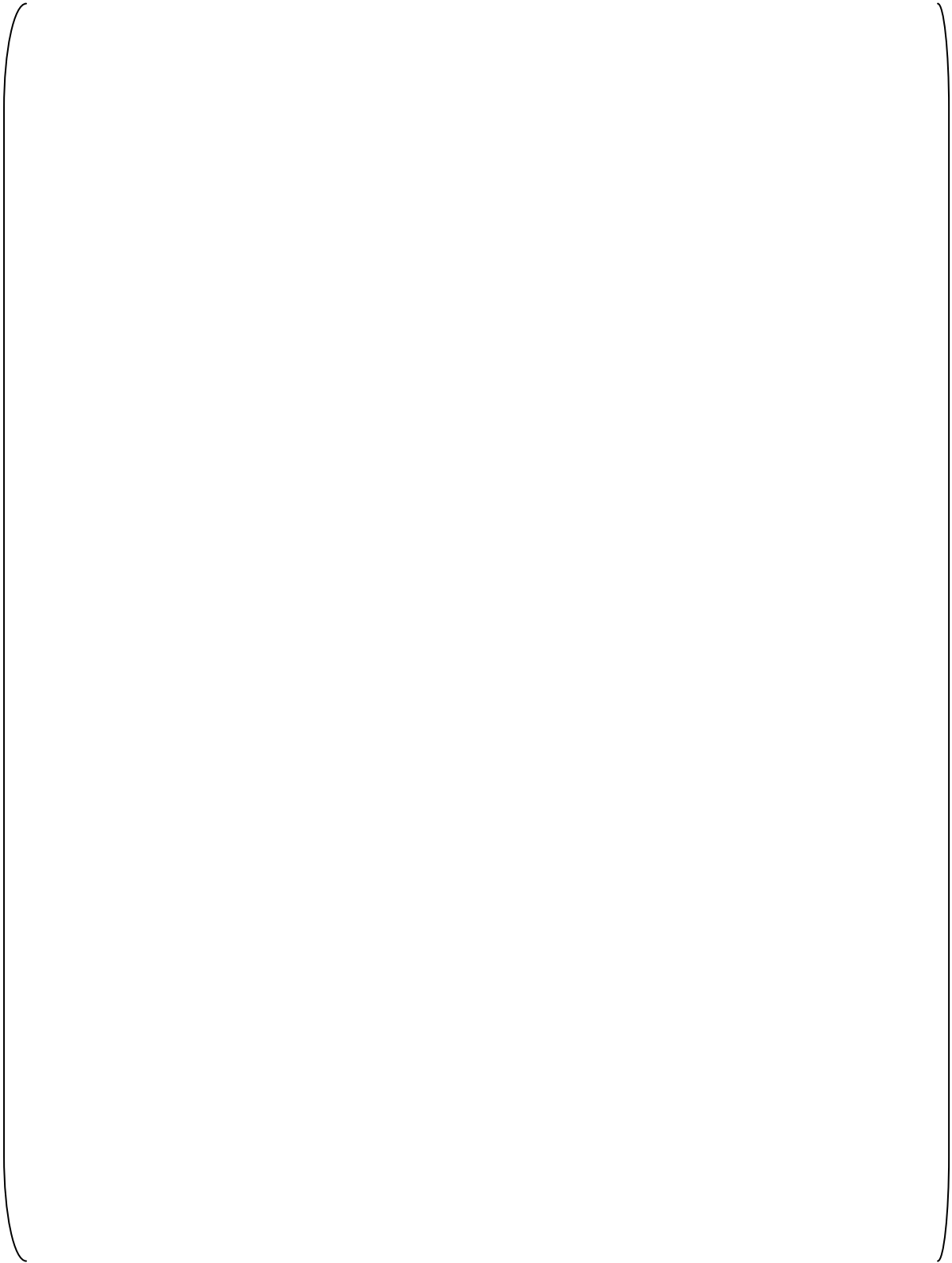


Figure A1-6-2-2 Frequency mode diagram (tertiary)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





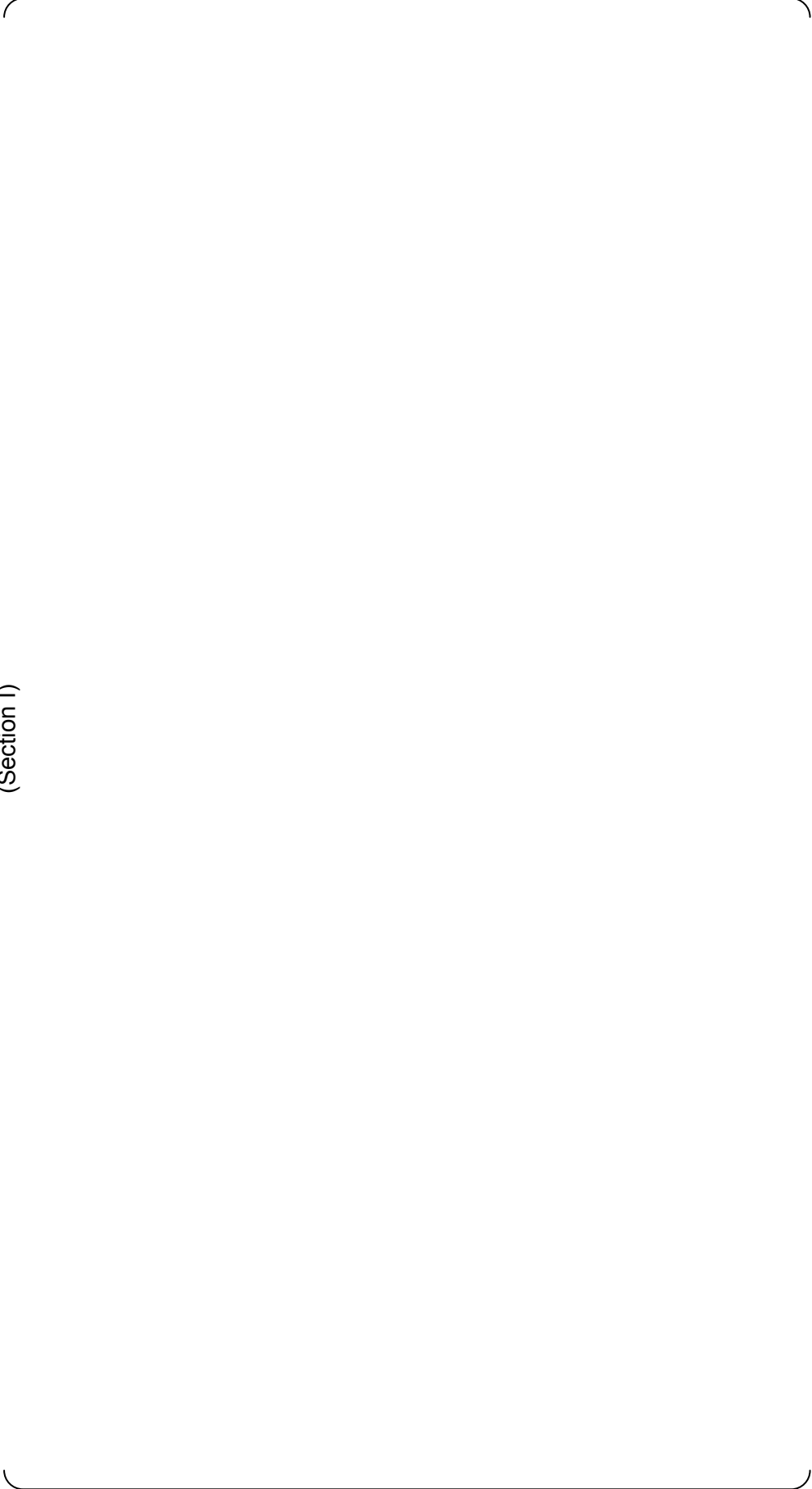


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (3/15)  
(Section I)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (4/15)  
(Section II)

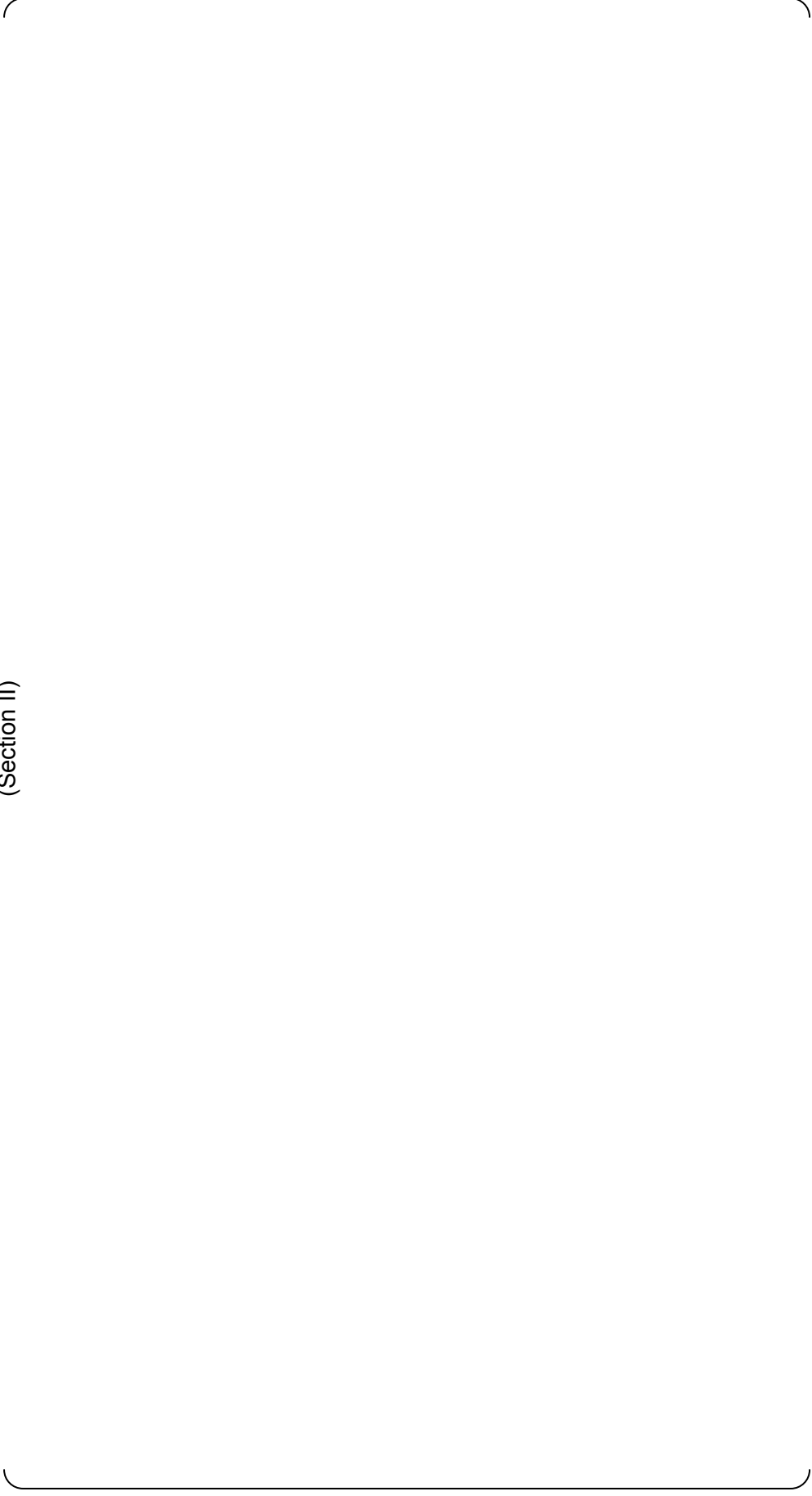
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**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (5/15)  
(Section II)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T_1$ ,  $\Delta T_2$ , Ta-Tb) (6/15)

|  |
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|  |
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**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (7/15)  
(Section III)

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (8/15)  
(Section III)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (9/15)  
(Section III)

| Item | Value |
|------|-------|
|------|-------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (10/15)  
(Section IV)





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



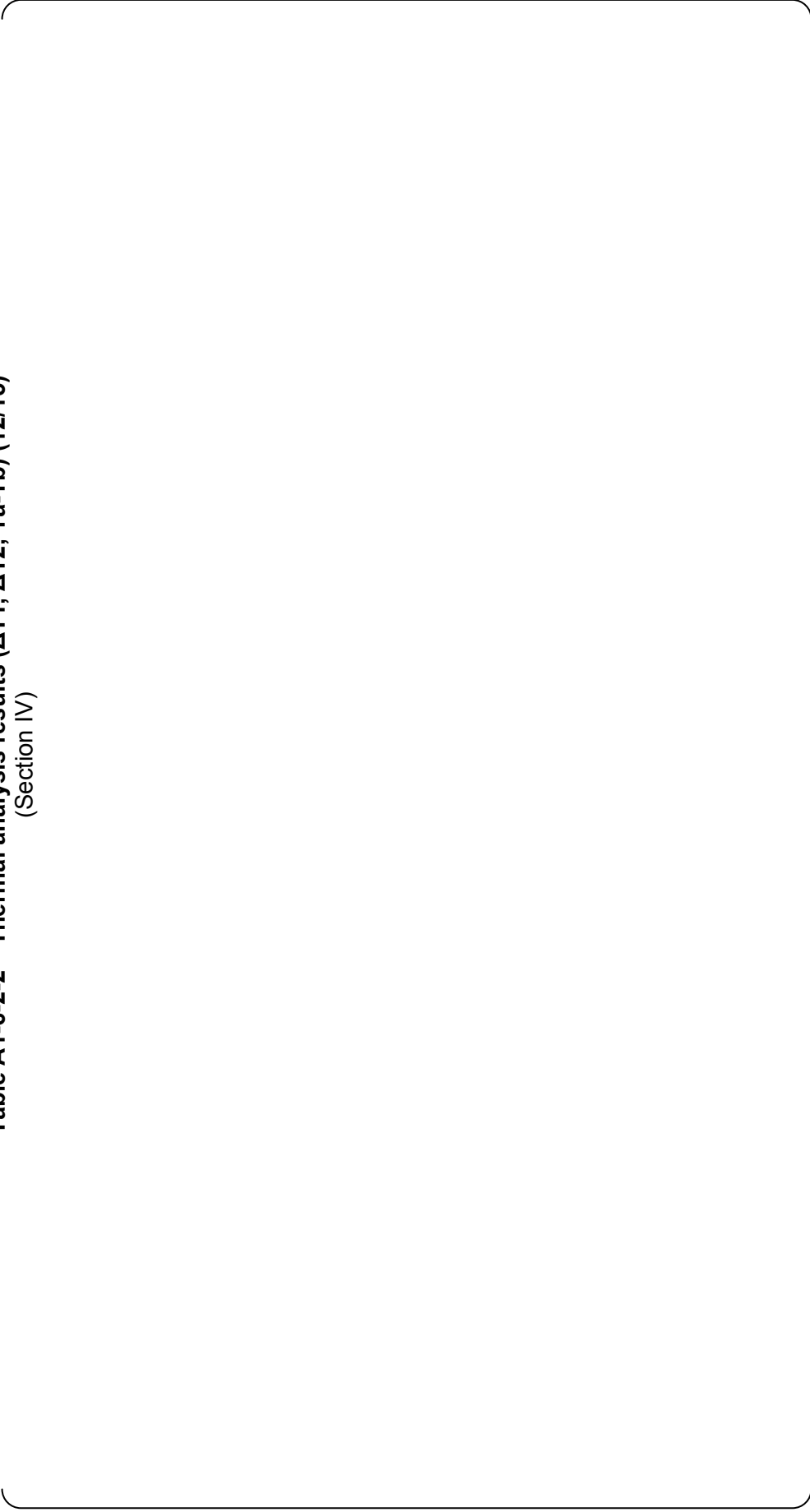


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (12/15)  
(Section IV)

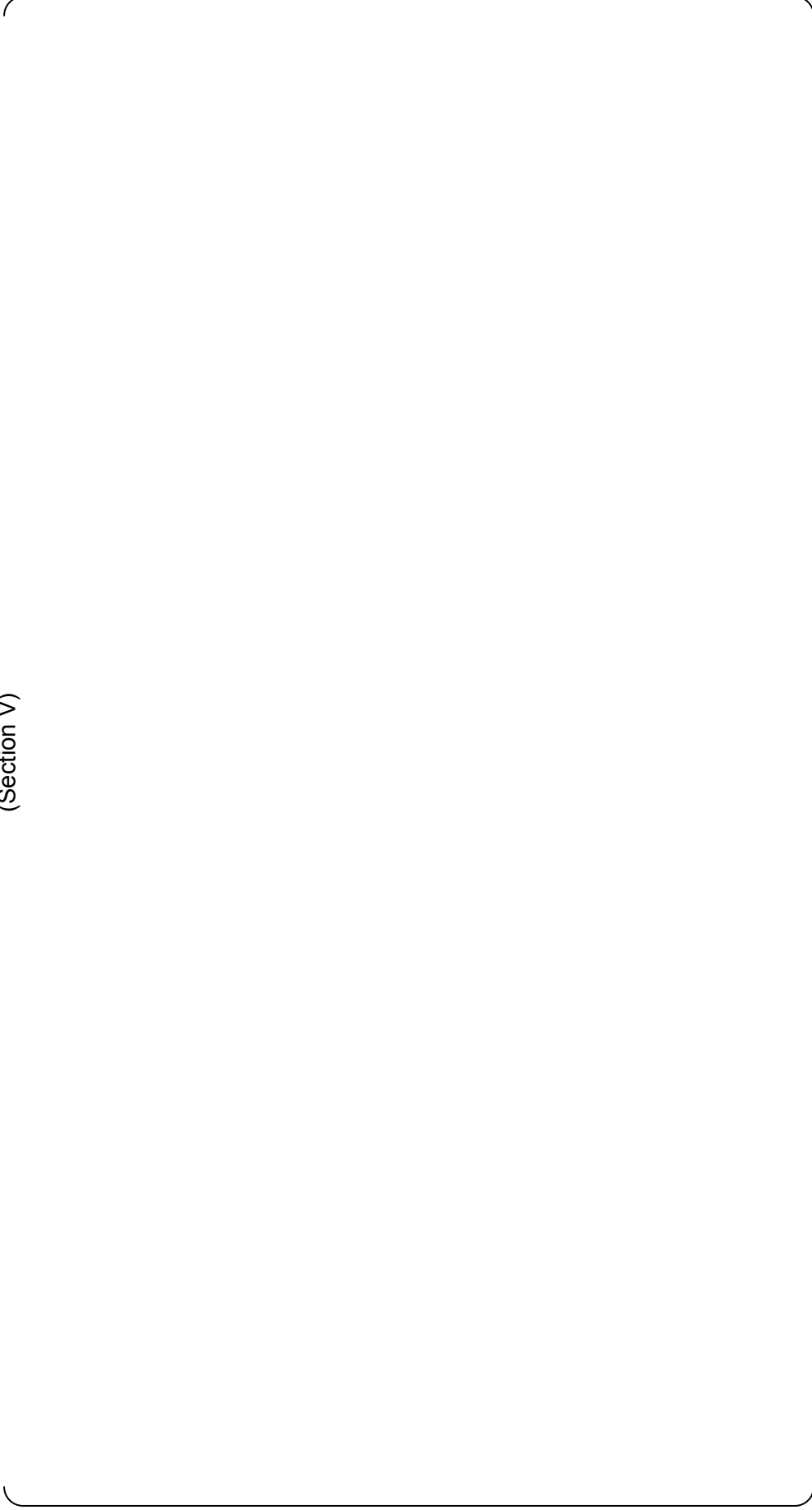


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (13/15)  
(Section V)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (14/15)  
(Section V)

|  |  |
|--|--|
|  |  |
|--|--|



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-6-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (15/15)  
(Section V)

| Location | $\Delta T1$ | $\Delta T2$ | Ta-Tb |
|----------|-------------|-------------|-------|
|----------|-------------|-------------|-------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



**Table A1-6-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-6-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

## Appendix 1-7

### **RH05** **RHR return Loop A Line** **Piping Analysis Results**

1. INPUT

|  |                 |
|--|-----------------|
| 1.1 Used for creating the pipe structural model              |                 |
| 1.1.1 Block division and piping specifications               | Table A1-7-1-1  |
| 1.1.2 Piping isometrics                                      | Figure A1-7-1-1 |
| 1.1.3 Concentrated mass                                      | Table A1-7-1-2  |
| 1.1.4 Support point rigidity                                 | Table A1-7-1-3  |
| 1.1.5 Valve rigidity   | Table A1-7-1-4  |
| 1.2 Used for creating load conditions                        |                 |
| 1.2.1 Level A/B design transient                             | see main text   |
| 1.2.2 Level A/B thermal displacement input data              | Table A1-7-1-5  |
| 1.2.3 Level A, B temperature and pressure input data         | Table A1-7-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data | Table A1-7-1-7  |
| 1.2.5 Floor response curve                                   | Figure A1-7-1-2 |
| 1.2.6 Seismic anchor displacement input data                 | Table A1-7-1-8  |
| 1.2.7 DBPB displacement input data                           | Table A1-7-1-9  |

2. OUTPUT

|  |                 |
|--|-----------------|
| 2.1 PIPESTRESS analysis model diagram                                      | Figure A1-7-2-1 |
| 2.2 Natural frequency analysis results                                     | Table A1-7-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                           | Figure A1-7-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a - T_b$ ) | Table A1-7-2-2  |
| 2.5 Piping stress and fatigue evaluation results                           | Table A1-7-2-3  |

Table A1-7-1-1 Block division and piping specifications (1/2)



Table A1-7-1-1 Block division and piping specifications (2/2)

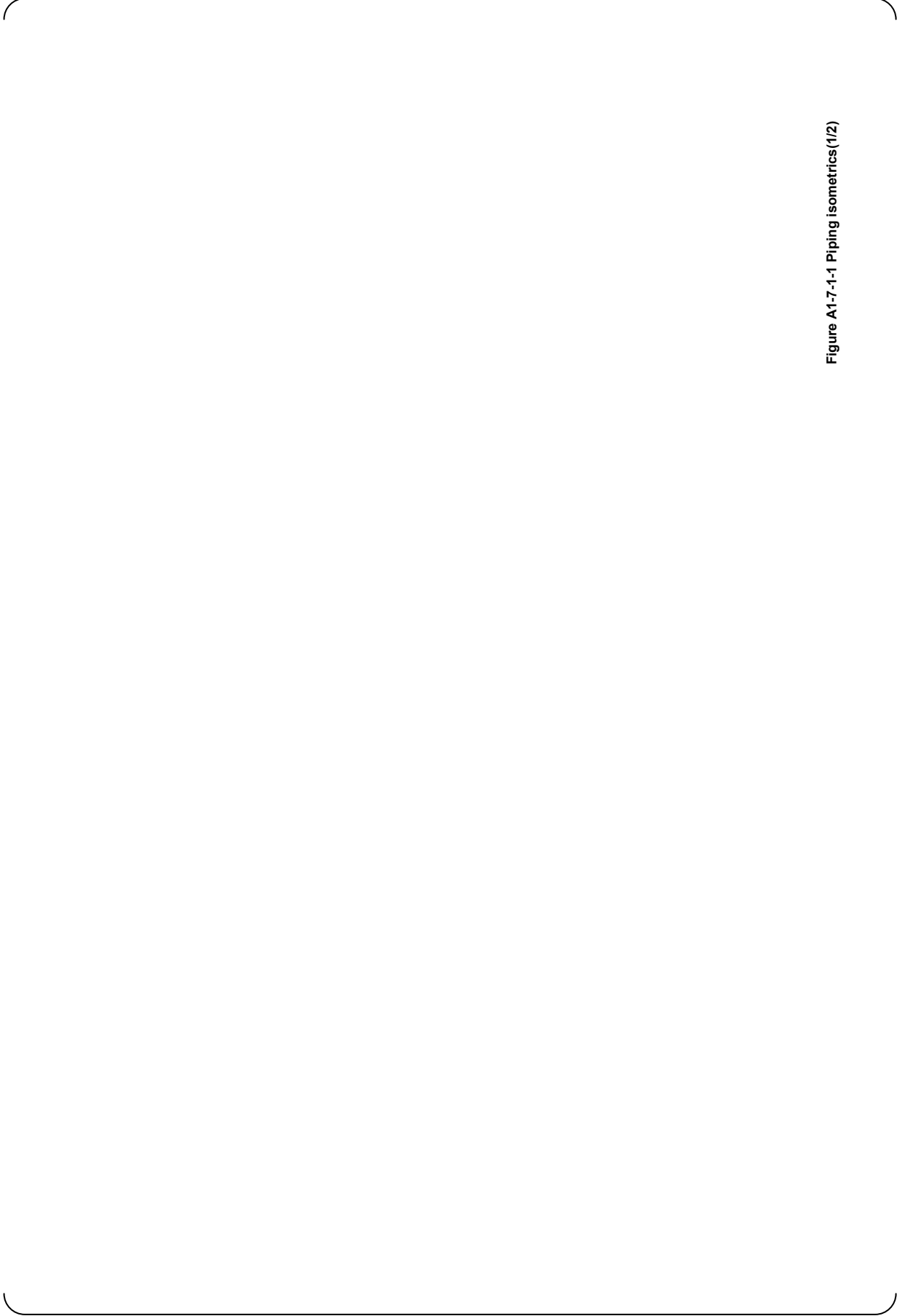


Figure A1-7-1-1 Piping Isometrics(1/2)

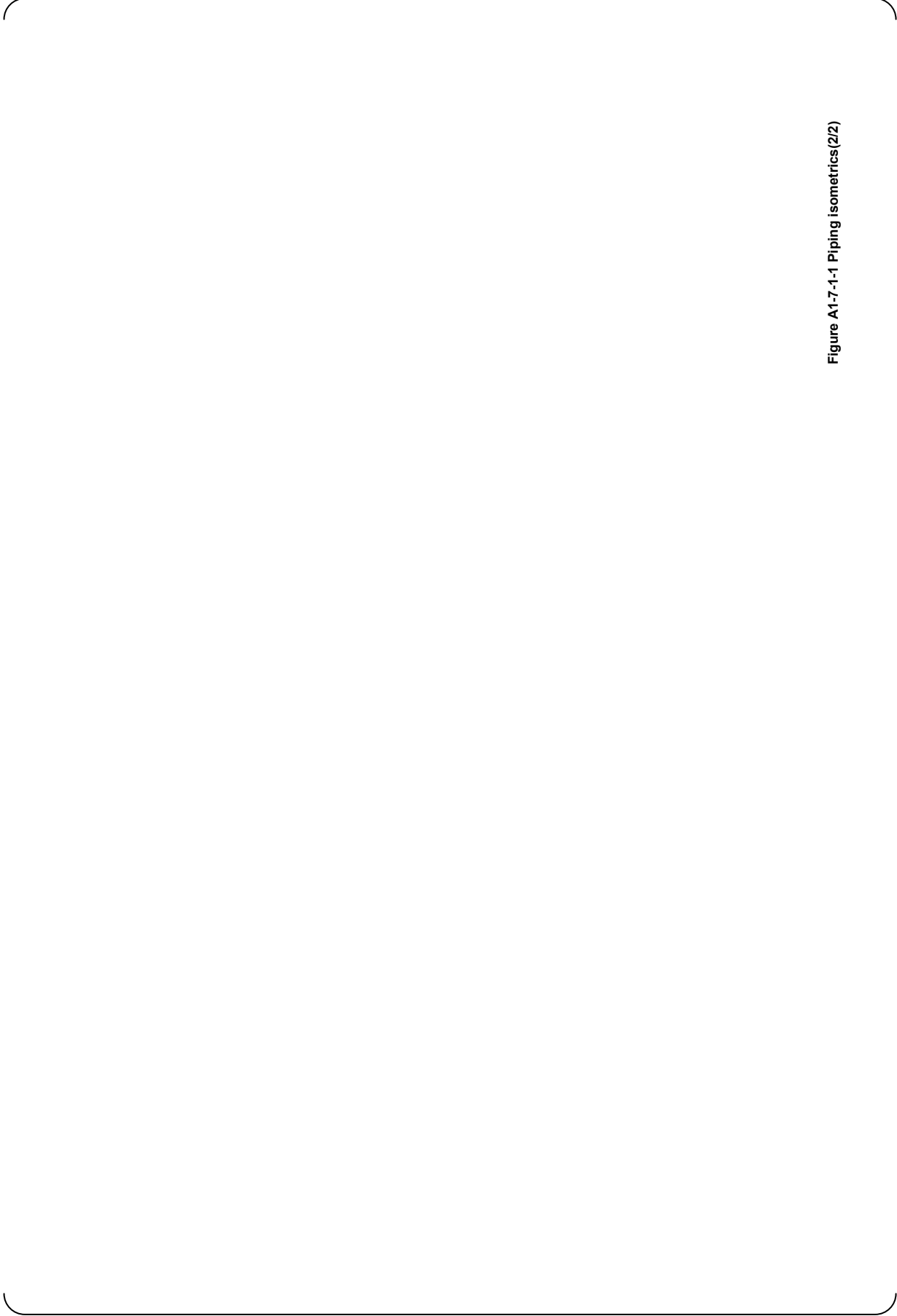


Figure A1-7-1-1 Piping Isometrics (2/2)

Table A1-7-1-2 Concentrated mass

Table A1-7-1-3 Support point rigidity (1/2)

Table A1-7-1-3 Support point rigidity (2/2)



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Table A1-7-1-4 Valve rigidity



Table A1-7-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9010)

| Point | Level A/B | Thermal Displacement Input Data       |
|-------|-----------|---------------------------------------|
| 9010  | Level A/B | Thermal displacement input data (1/3) |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-7-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9010)

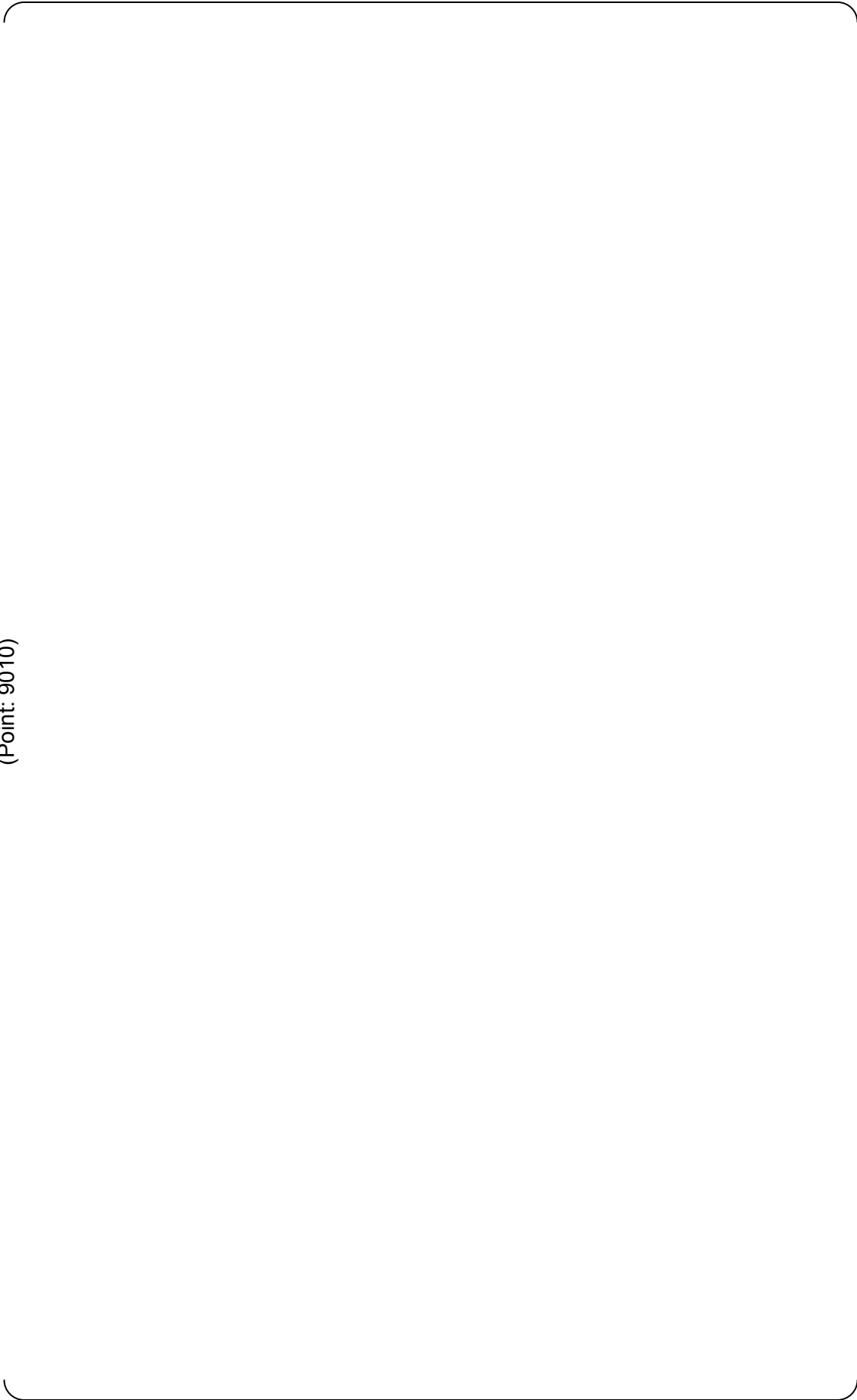
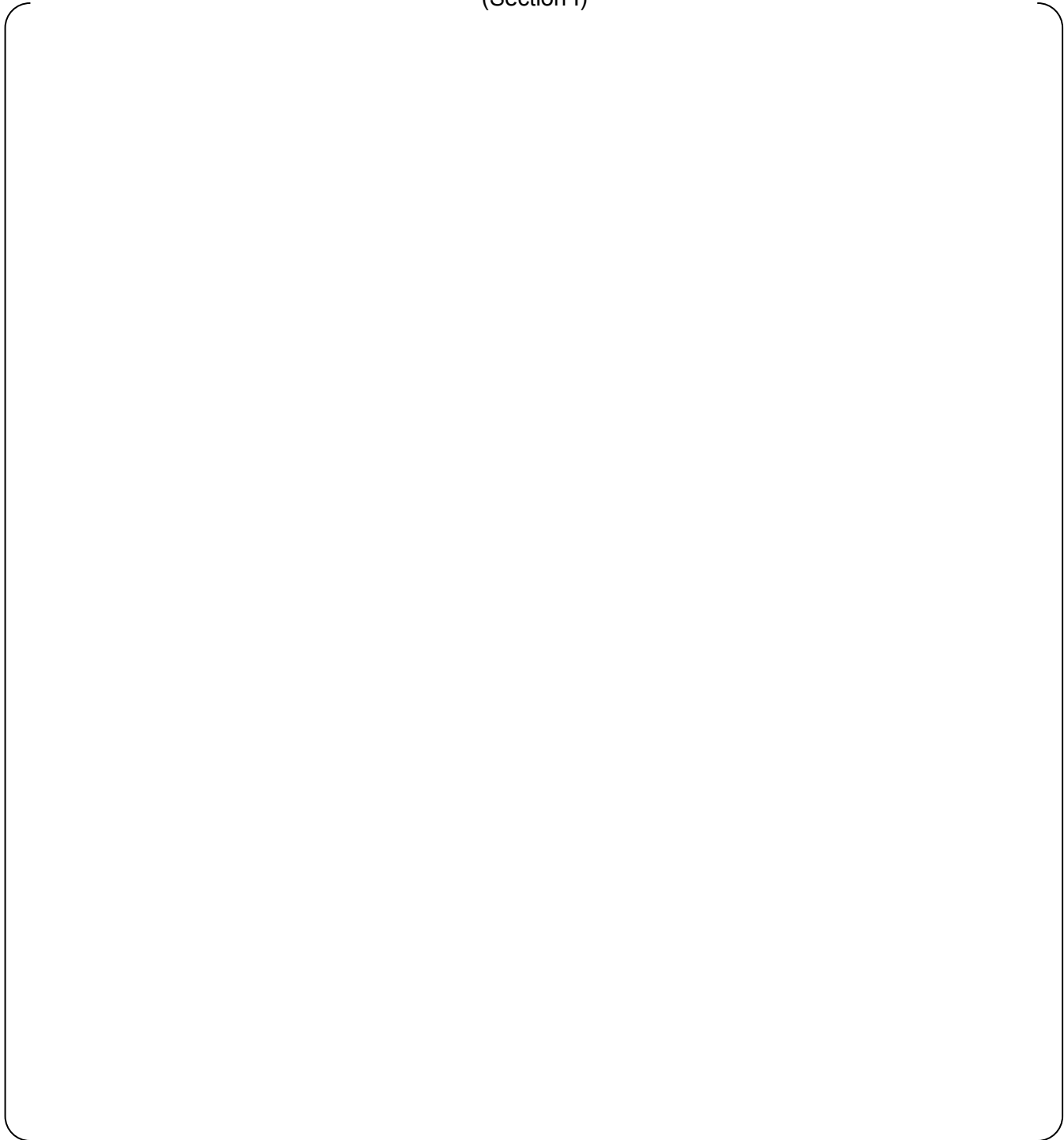


Table A1-7-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |

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**Table A1-7-1-6 Level A, B temperature and pressure input data (1/15)**  
(Section I)

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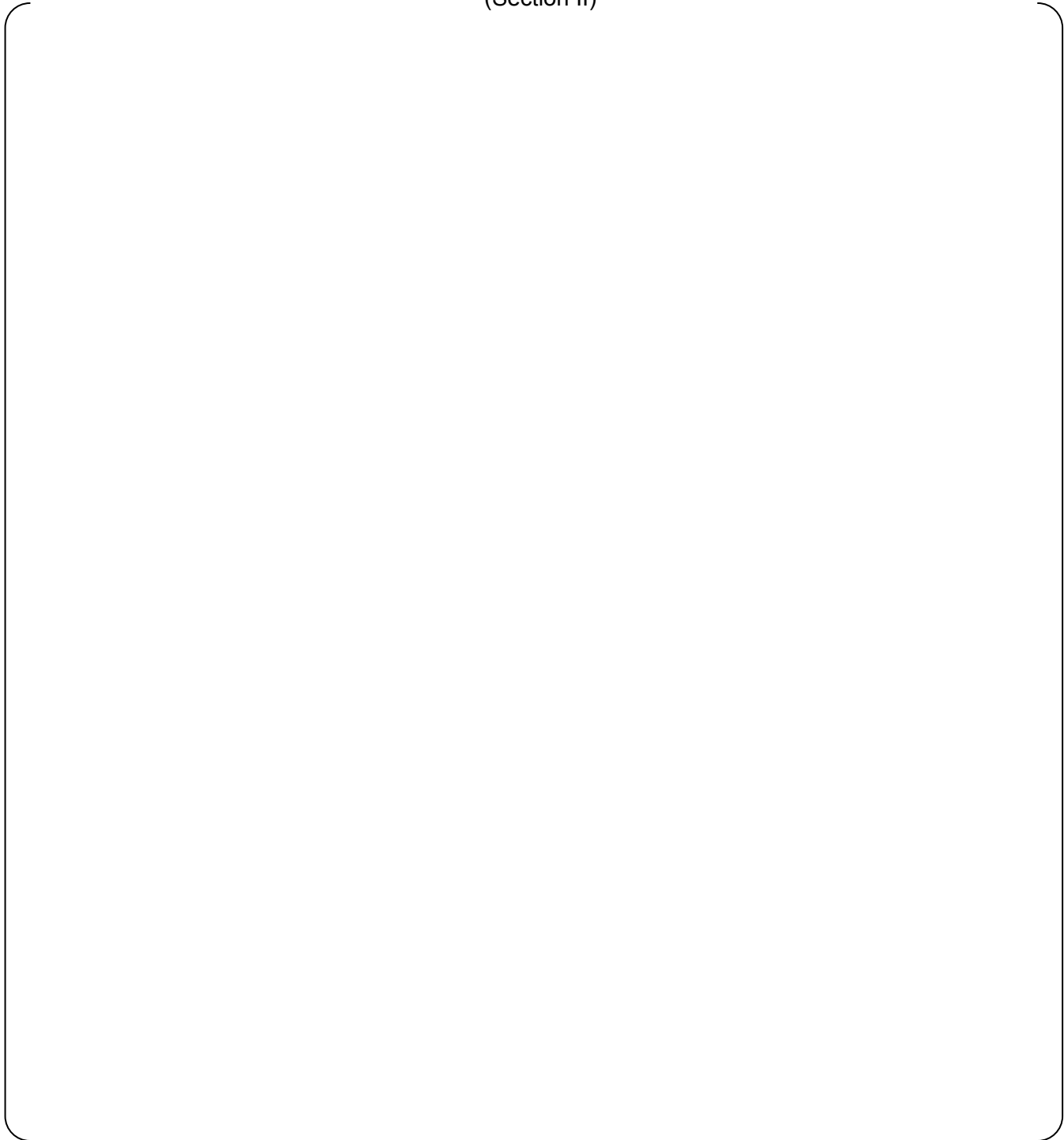
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**Table A1-7-1-6 Level A, B temperature and pressure input data (2/15)**  
(Section I)

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**Table A1-7-1-6 Level A, B temperature and pressure input data (3/15)**  
(Section I)

Table A1-7-1-6 Level A, B temperature and pressure input data (4/15)  
(Section II)





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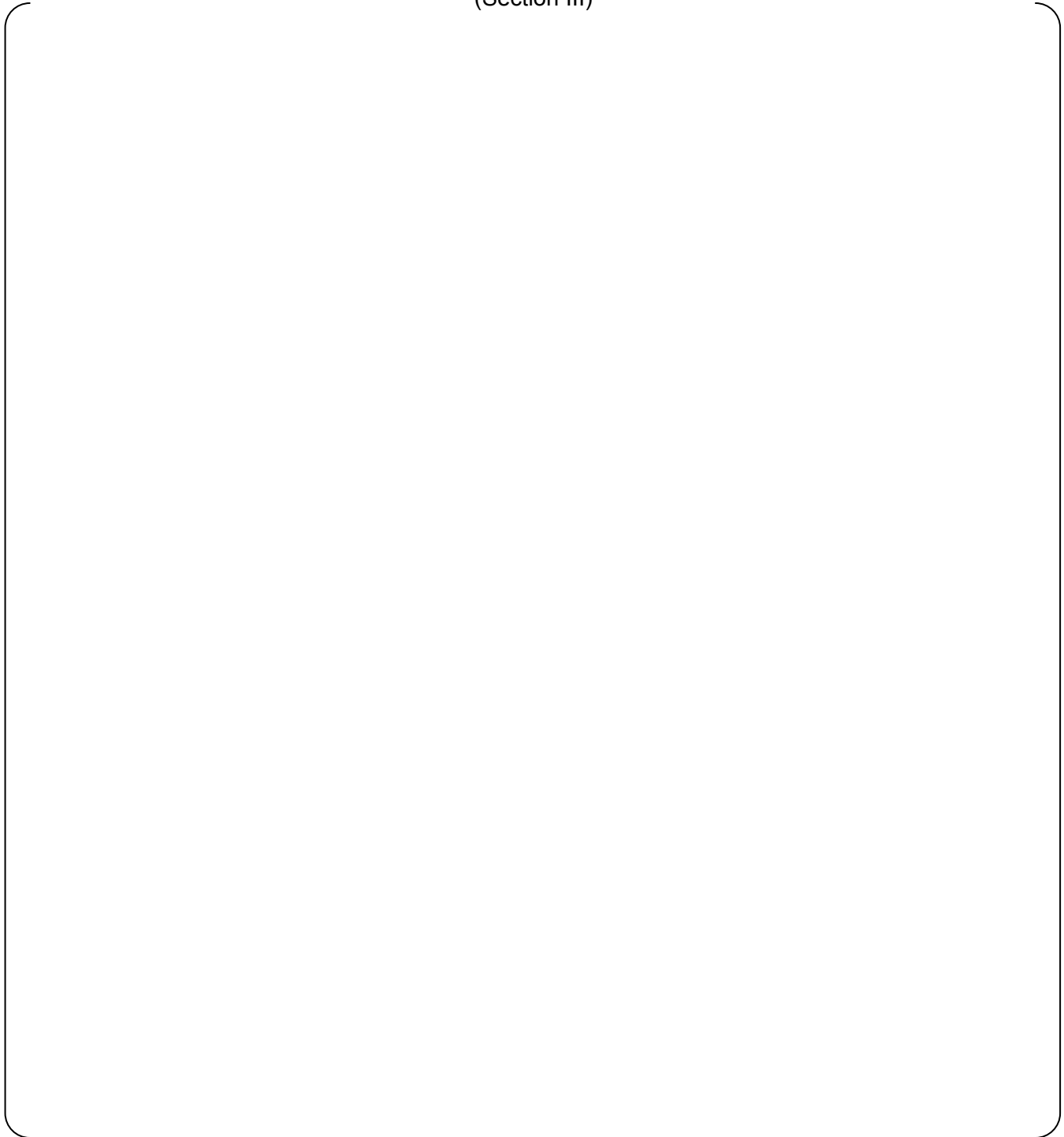
Table A1-7-1-6 Level A, B temperature and pressure input data (5/15)  
(Section II)

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Table A1-7-1-6 Level A, B temperature and pressure input data (6/15)  
(Section II)

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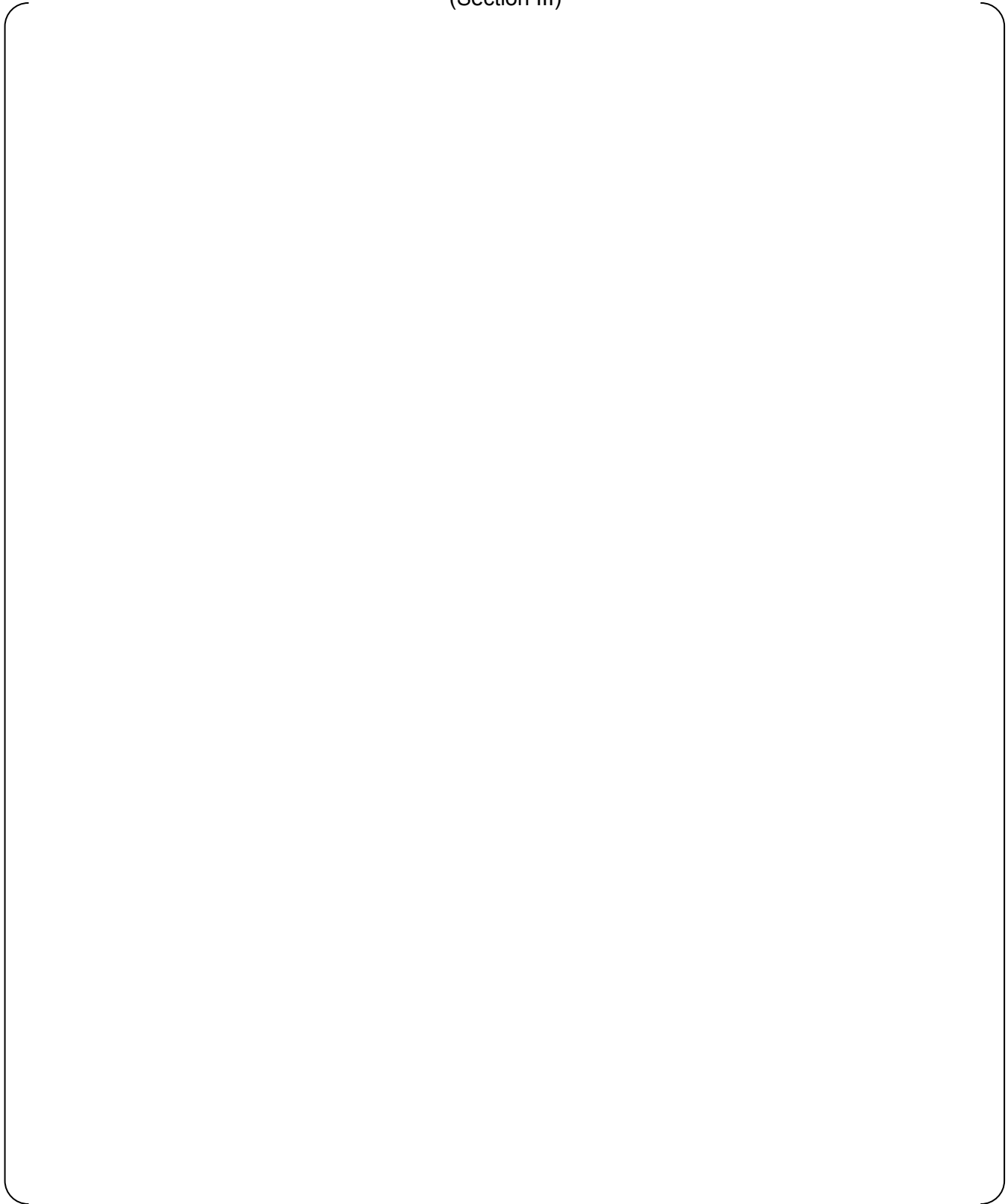
**Table A1-7-1-6 Level A, B temperature and pressure input data (7/15)**  
(Section III)

A large, empty rectangular frame with rounded corners, intended for the data from Table A1-7-1-6. The frame is currently blank.

**Table A1-7-1-6 Level A, B temperature and pressure input data (8/15)  
(Section III)**

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**Table A1-7-1-6 Level A, B temperature and pressure input data (9/15)**  
(Section III)



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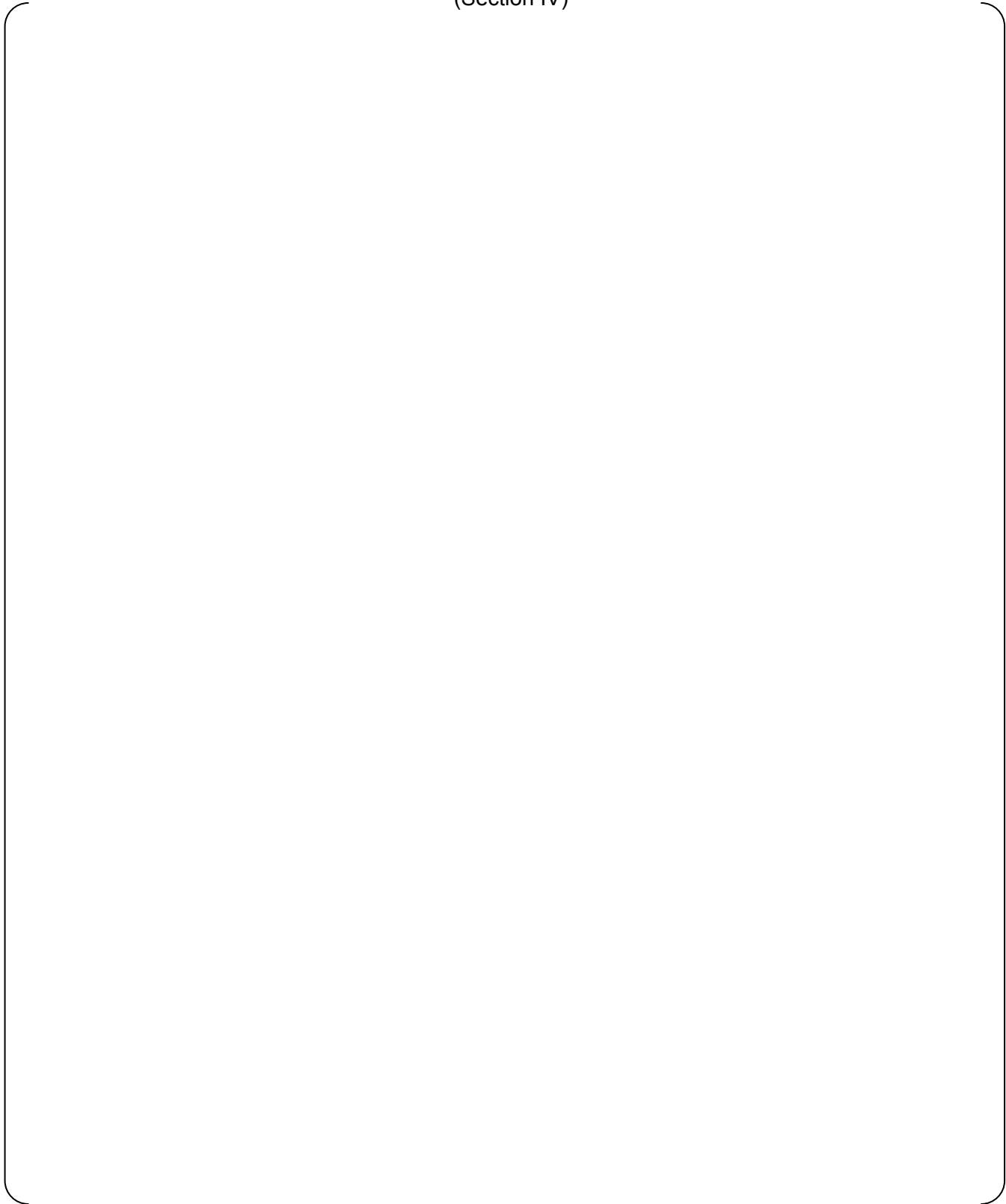
Table A1-7-1-6 Level A, B temperature and pressure input data (10/15)  
(Section IV)

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Table A1-7-1-6 Level A, B temperature and pressure input data (11/15)  
(Section IV)

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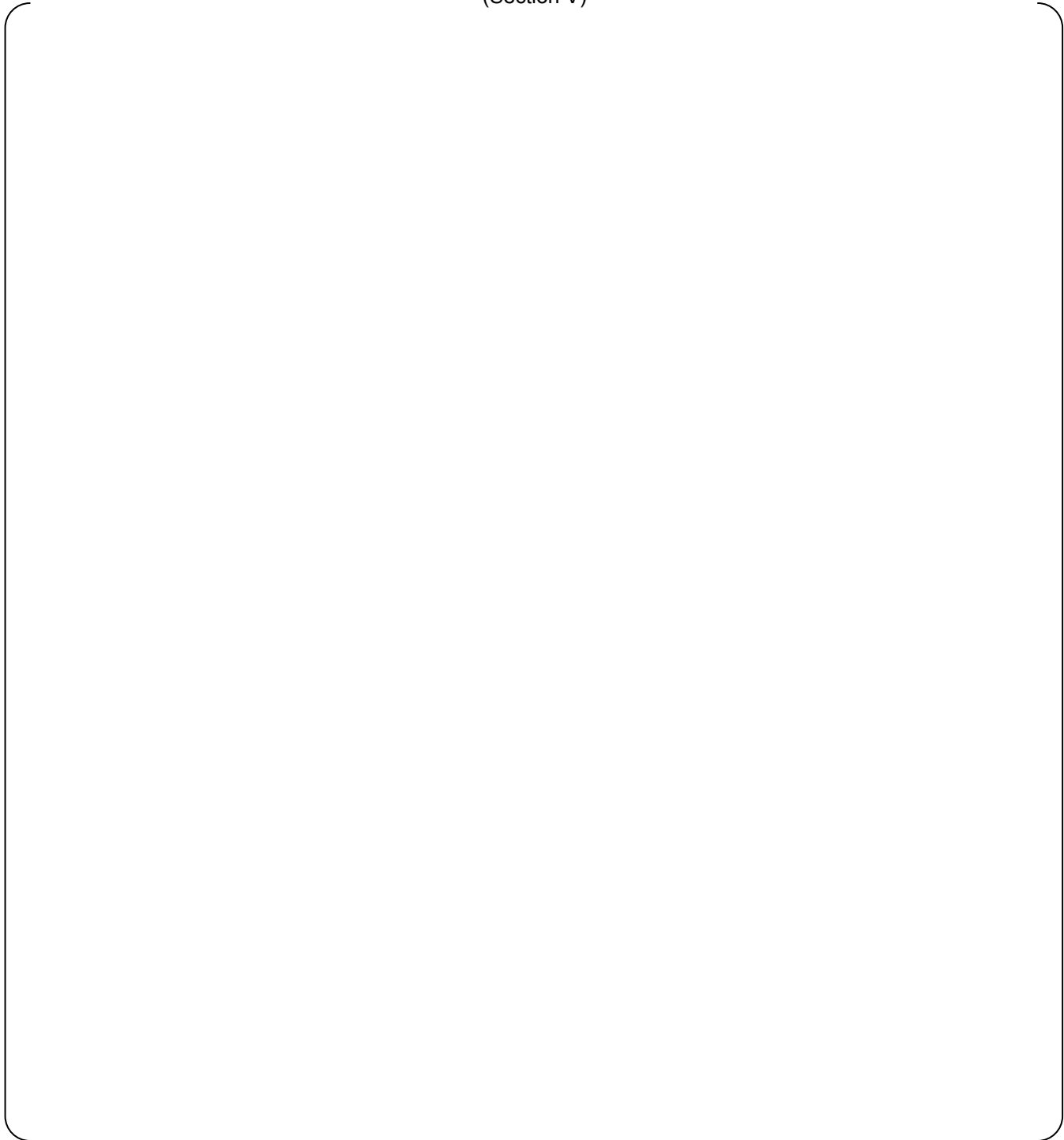
Table A1-7-1-6 Level A, B temperature and pressure input data (12/15)  
(Section IV)





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**Table A1-7-1-6 Level A, B temperature and pressure input data (13/15)**  
(Section V)

A large, empty rectangular frame with rounded corners, intended for the table content. The frame is currently blank.

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Table A1-7-1-6 Level A, B temperature and pressure input data (14/15)  
(Section V)

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**Table A1-7-1-6 Level A, B temperature and pressure input data (15/15)**  
(Section V)

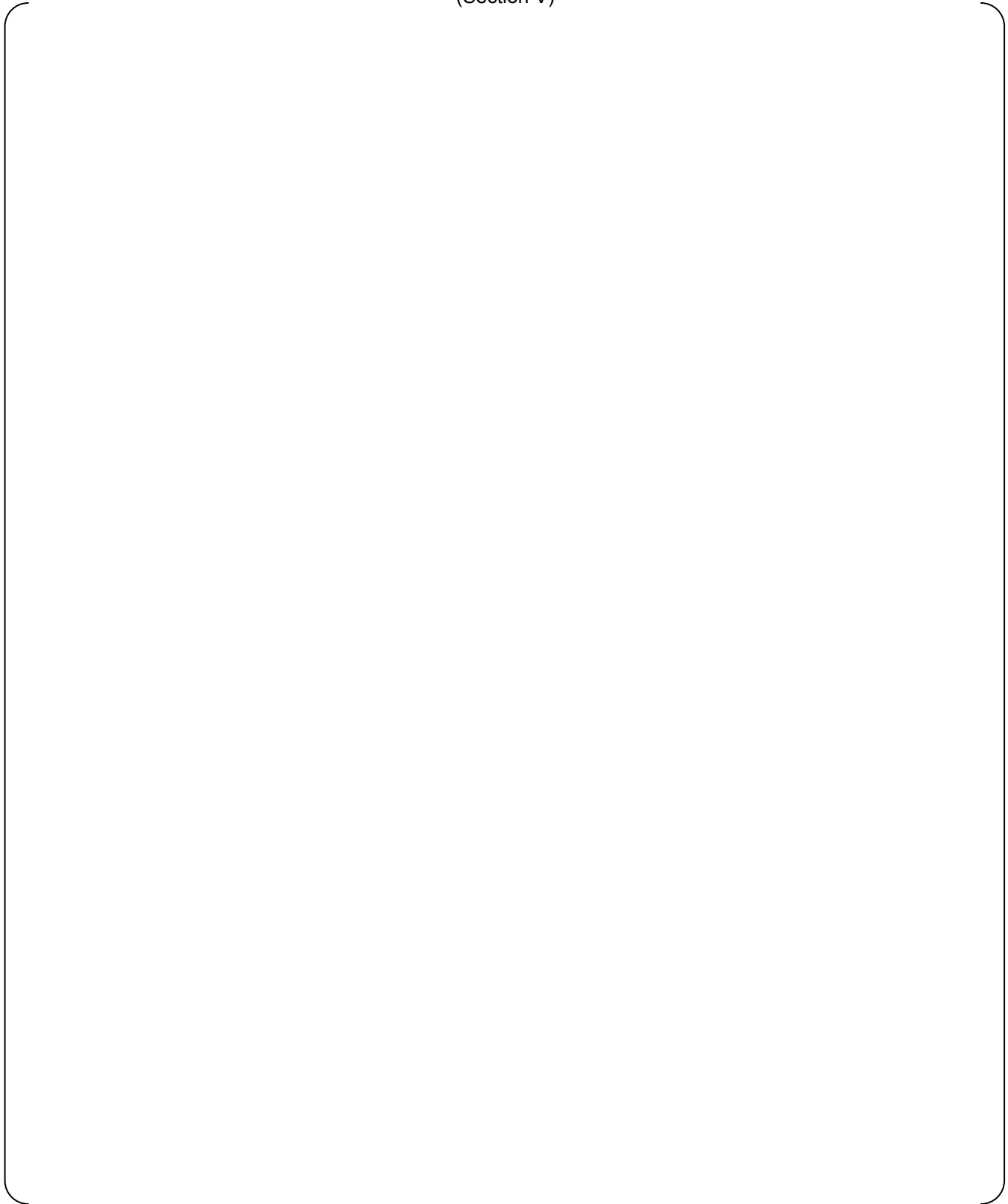


Table A1-7-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-7-1 -2 Floor response curve (1/6)**  
RHRS Return (RH05-06) FRS for MCP Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-7-1-2 Floor response curve (2/6)**  
RHRS Return (RH05-06) FRS for MCP Nozzle  
Y (NS) direction (damping 4.0%)

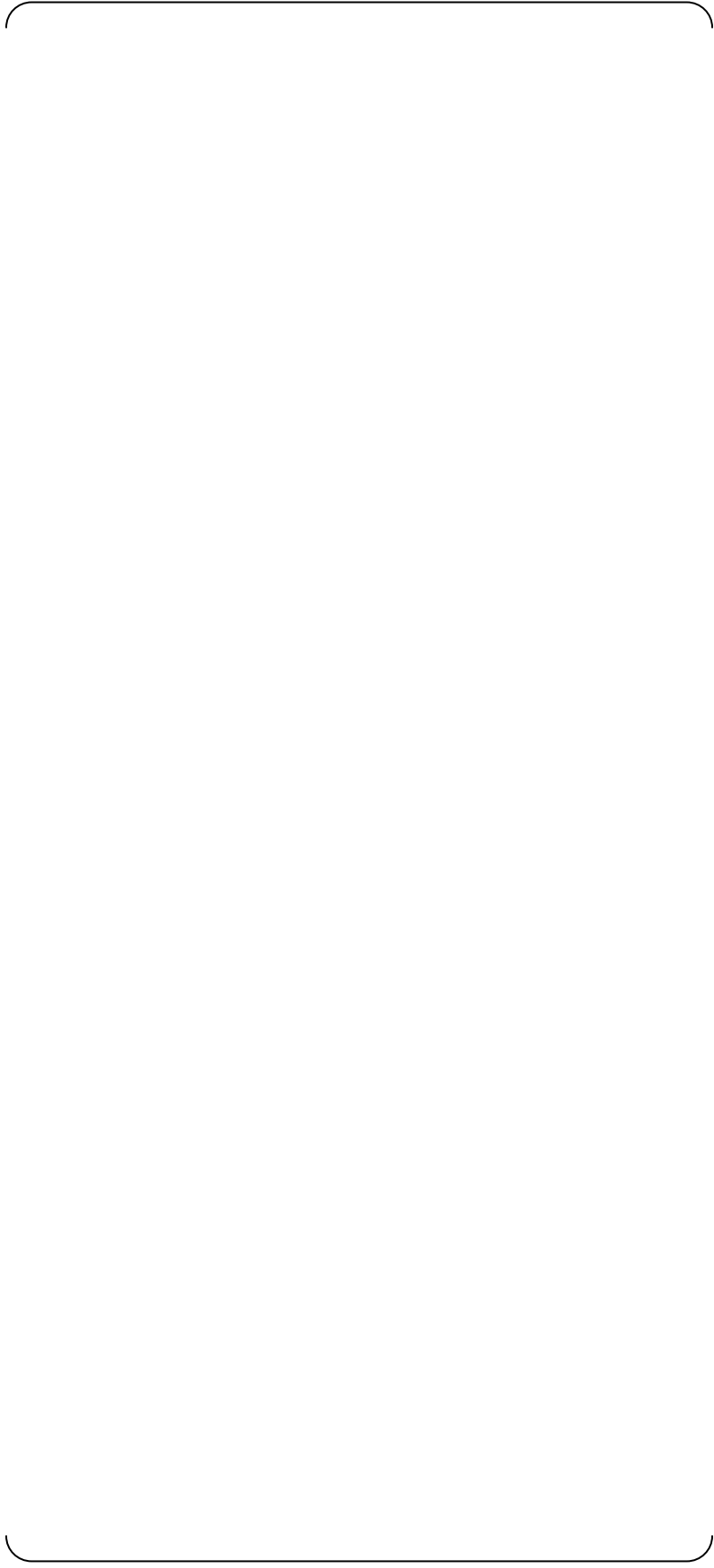


**Figure A1-7-1 -2 Floor response curve (3/6)**  
RHRS Return (RH05-06) FRS for MCP Nozzle  
Z (Vert.) direction (damping 4.0%)

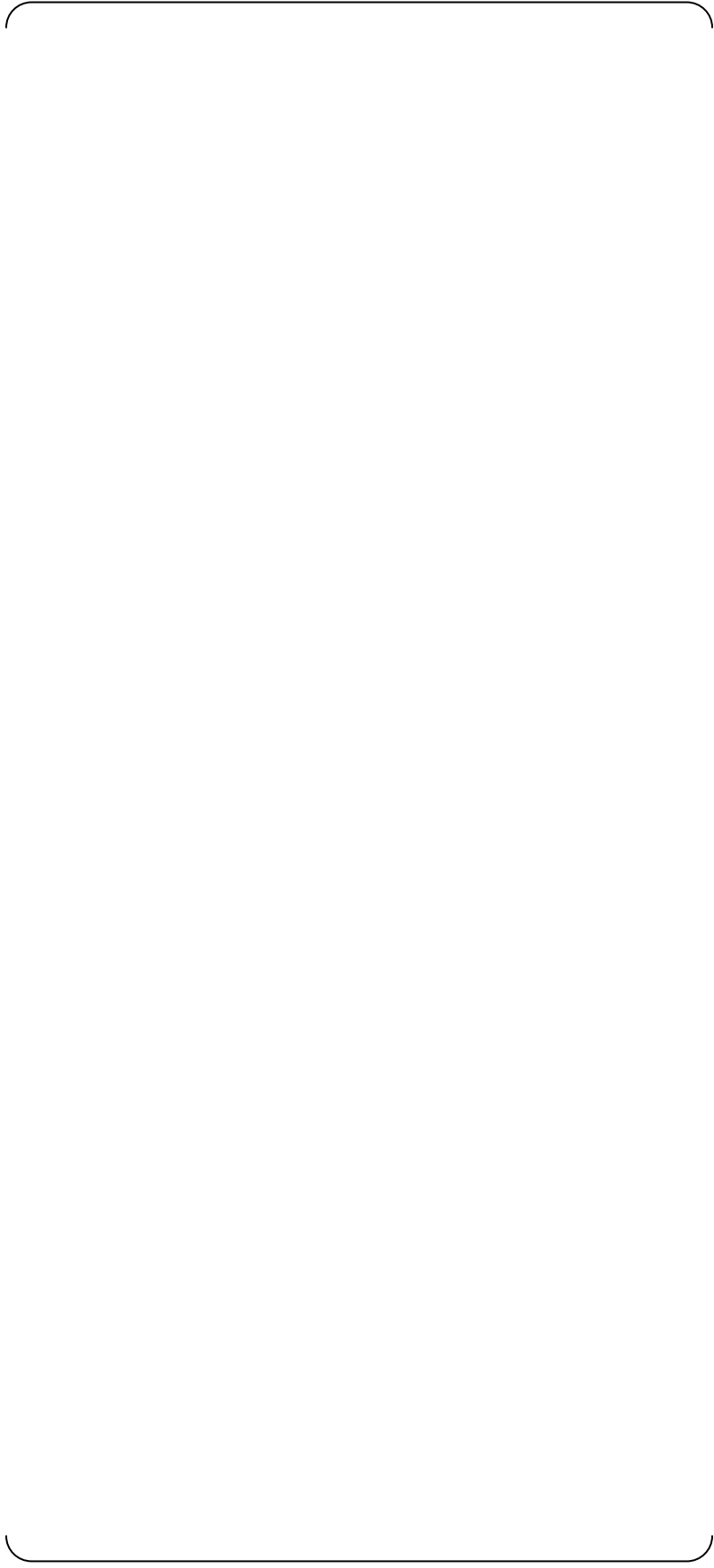


**Figure A1-7-1 -2 Floor response curve (4/6)**  
RHRS Return (RH05-06) FRS for Piping Supports  
X (EW) direction (damping 4.0%)





**Figure A1-7-1-2 Floor response curve (5/6)**  
RHRS Return (RH05-06) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-7-1-2 Floor response curve (6/6)**  
RHRS Return (RH05-06) FRS for Piping  
Z (Vert.) direction (damping 4.0%)

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Table A1-7-1-8 Seismic anchor displacement input data (1/2)

Table A1-7-1-8 Seismic anchor displacement input data (2/2)

Table A1-7-1-9 DBPB displacement input data

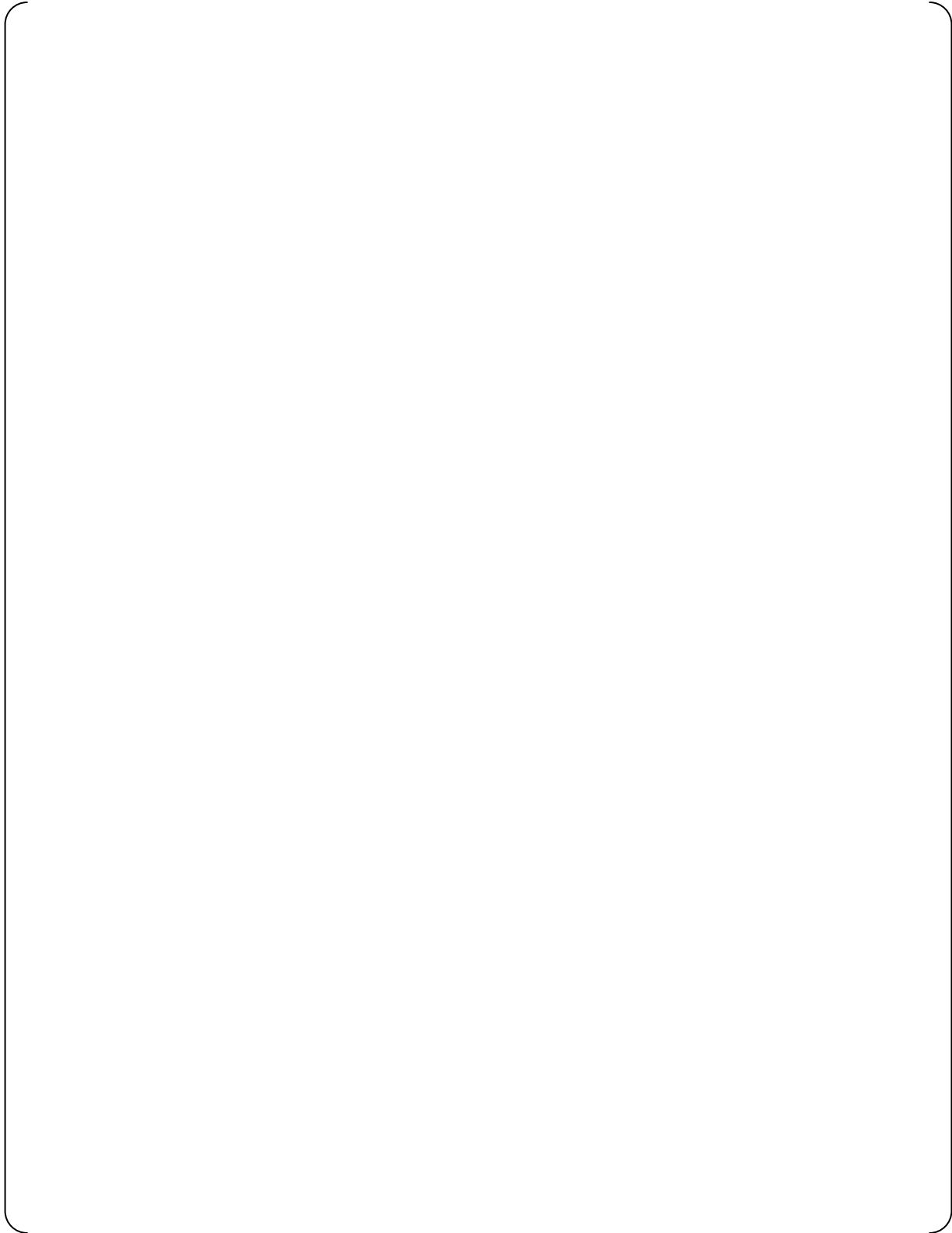


Figure A1-7-2-1 PIPESTRESS analysis model diagram

Table A1-7-2-1 Natural frequency analysis results

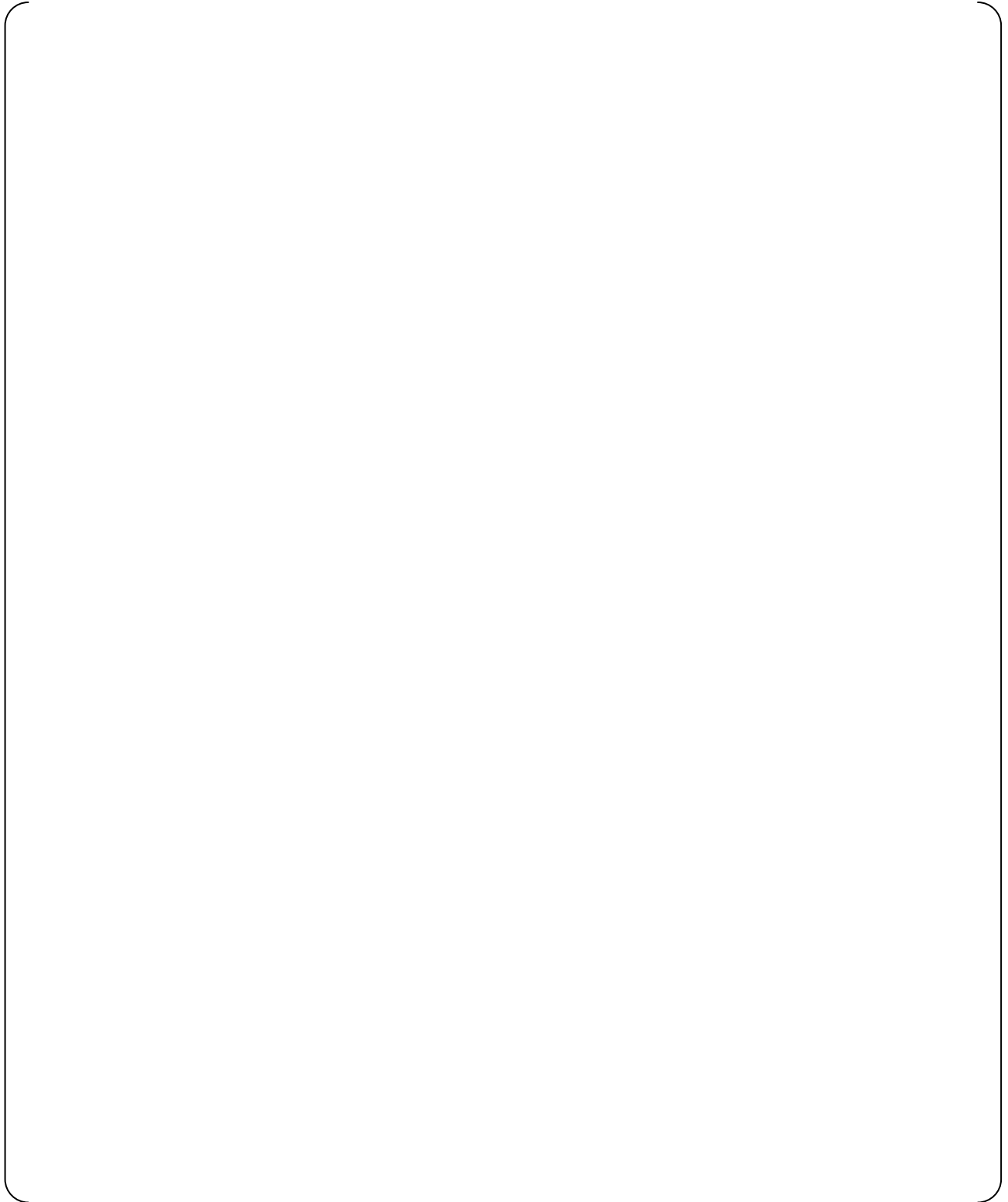


Figure A1-7-2-2 Frequency mode diagram (primary)



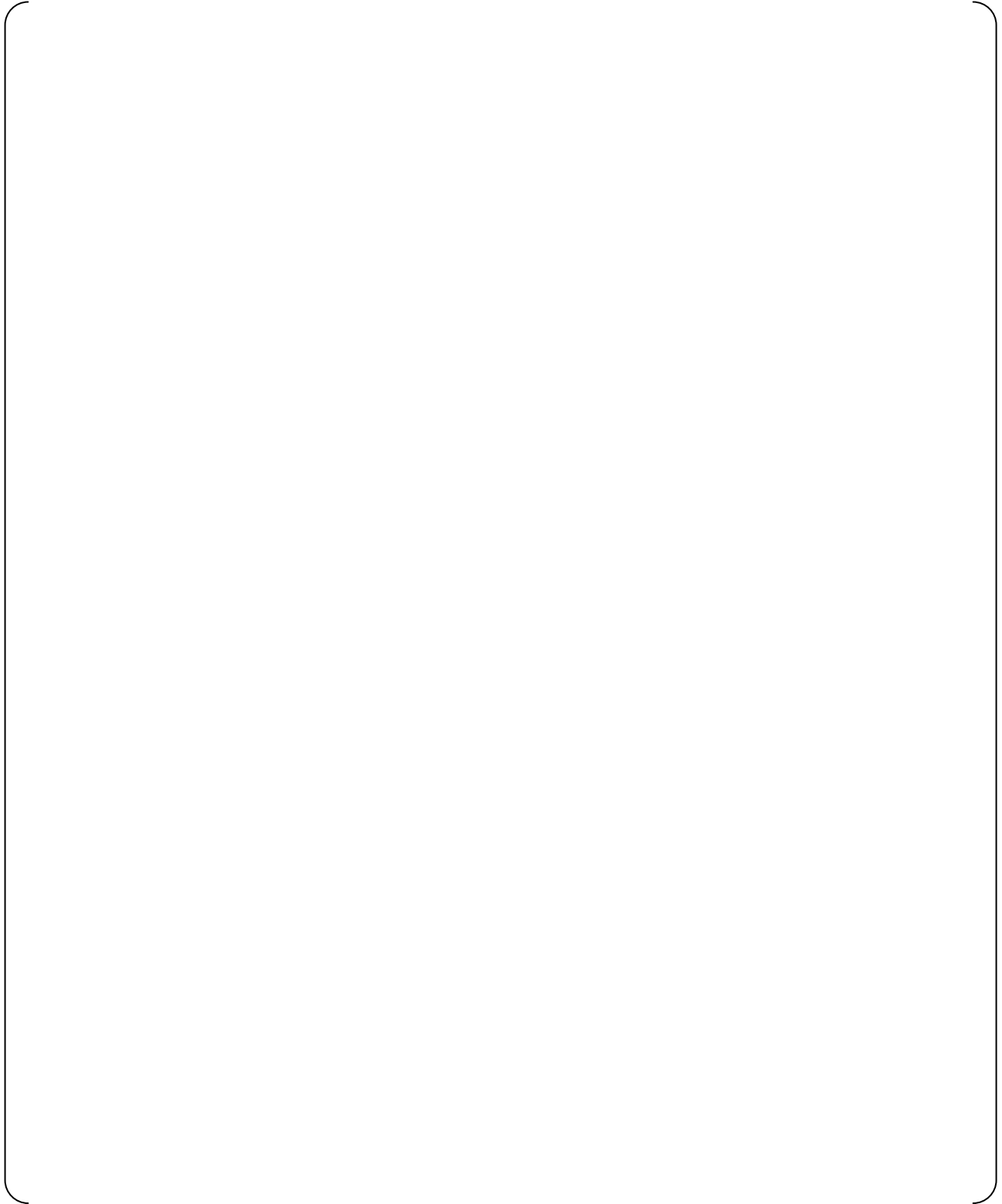


Figure A1-7-2-2 Frequency mode diagram (secondary)

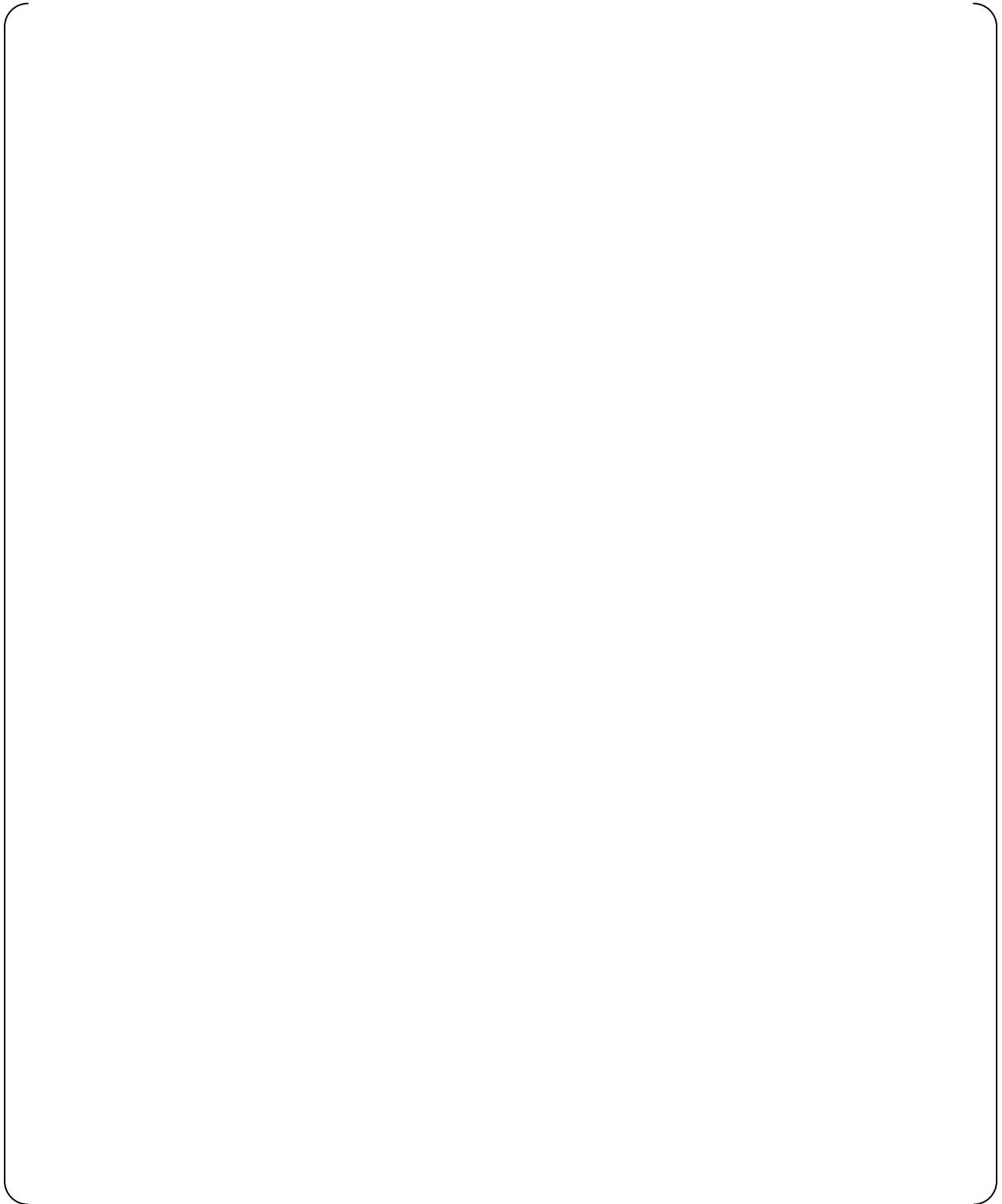


Figure A1-7-2-2 Frequency mode diagram (tertiary)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-7-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (3/6)  
(Section I)

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Table A1-7-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (4/6)  
(Section II)







Table A1-7-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (5/6)  
(Section II)

|  |
|--|
|  |
|--|

Table A1-7-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (6/6)  
(Section II)

| Location | $\Delta T1$ | $\Delta T2$ | $Ta-Tb$ |
|----------|-------------|-------------|---------|
|----------|-------------|-------------|---------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



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**Table A1-7-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-7-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

**Appendix 1-8**

**RH06  
RHR return Loop B Line  
Piping Analysis Results**

1. INPUT

|  |                 |
|--|-----------------|
| 1.1 Used for creating the pipe structural model              |                 |
| 1.1.1 Block division and piping specifications               | Table A1-8-1-1  |
| 1.1.2 Piping isometrics                                      | Figure A1-8-1-1 |
| 1.1.3 Concentrated mass                                      | Table A1-8-1-2  |
| 1.1.4 Support point rigidity                                 | Table A1-8-1-3  |
| 1.1.5 Valve rigidity   | Table A1-8-1-4  |
| 1.2 Used for creating load conditions                        |                 |
| 1.2.1 Level A/B design transient                             | see main text   |
| 1.2.2 Level A/B thermal displacement input data              | Table A1-8-1-5  |
| 1.2.3 Level A, B temperature and pressure input data         | Table A1-8-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data | Table A1-8-1-7  |
| 1.2.5 Floor response curve                                   | Figure A1-8-1-2 |
| 1.2.6 Seismic anchor displacement input data                 | Table A1-8-1-8  |
| 1.2.7 DBPB displacement input data                           | Table A1-8-1-9  |

2. OUTPUT

|  |                 |
|--|-----------------|
| 2.1 PIPESTRESS analysis model diagram                                      | Figure A1-8-2-1 |
| 2.2 Natural frequency analysis results                                     | Table A1-8-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                           | Figure A1-8-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a - T_b$ ) | Table A1-8-2-2  |
| 2.5 Piping stress and fatigue evaluation results                           | Table A1-8-2-3  |



Table A1-8-1-1 Block division and piping specifications (1/2)\*

Table A1-8-1-1 Block division and piping specifications (2/2)



Figure A1-8-1-1 Piping isometrics (1/2)



Figure A1-8-1-1 Piping isometrics(2/2)

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Table A1-8-1-2 Concentrated mass

Table A1-8-1-3 Support point rigidity (1/2)

Table A1-8-1-3 Support point rigidity (2/2)





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Table A1-8-1-4 Valve rigidity

Table A1-8-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9010)

| Point | X | Y | Z | U | V | W |
|-------|---|---|---|---|---|---|
| 9010  |   |   |   |   |   |   |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-8-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9010)

|  |  |
|--|--|
|  |  |
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Table A1-8-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9010)

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|  |
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**Table A1-8-1-6 Level A, B temperature and pressure input data (1/15)**  
(Section I)

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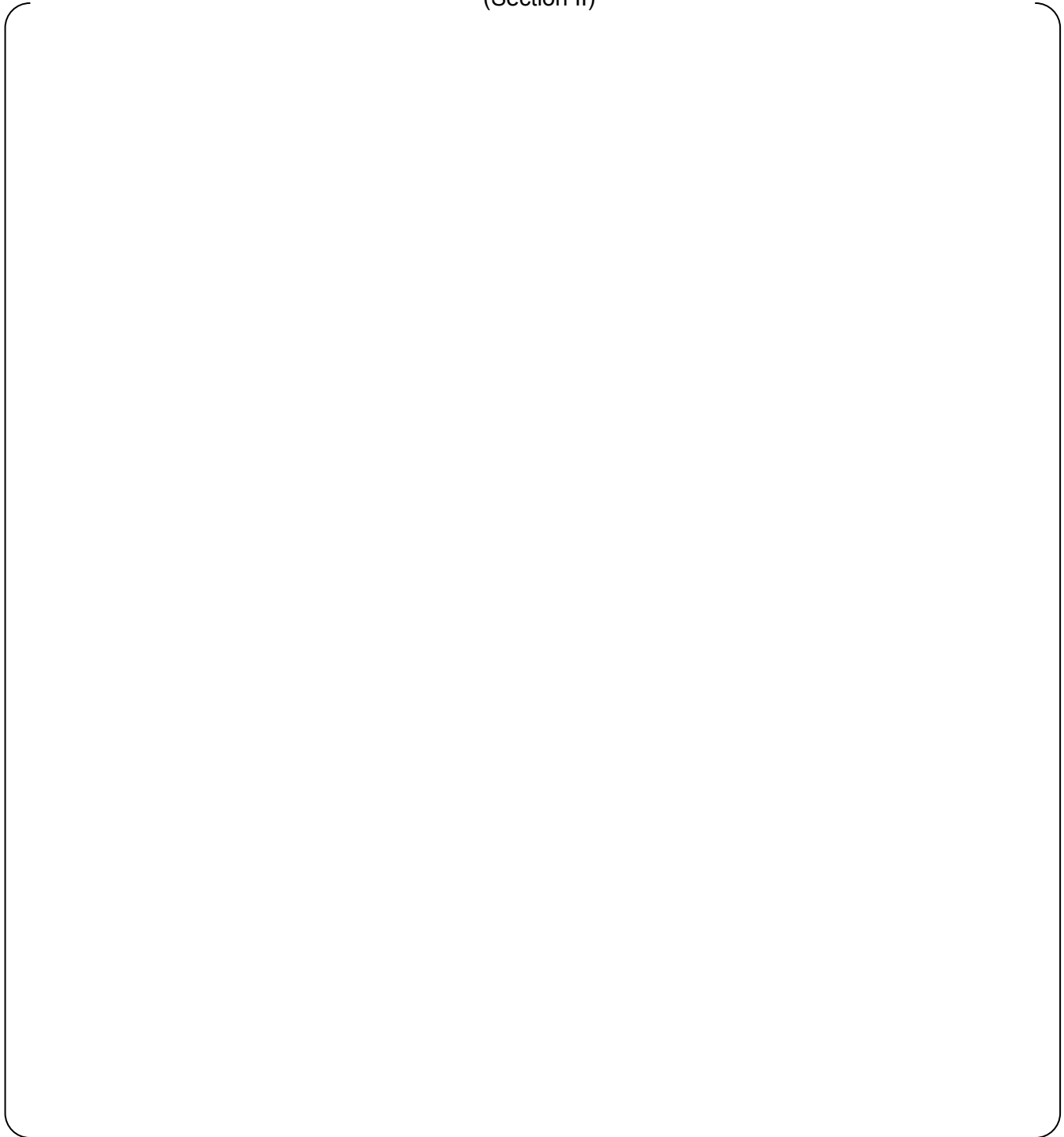
**Table A1-8-1-6 Level A, B temperature and pressure input data (2/15)**  
(Section I)

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**Table A1-8-1-6 Level A, B temperature and pressure input data (3/15)**  
(Section I)



Table A1-8-1-6 Level A, B temperature and pressure input data (4/15)  
(Section II)



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Table A1-8-1-6 Level A, B temperature and pressure input data (5/15)  
(Section II)

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Table A1-8-1-6 Level A, B temperature and pressure input data (6/15)  
(Section II)

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Table A1-8-1-6 Level A, B temperature and pressure input data (7/15)  
(Section III)

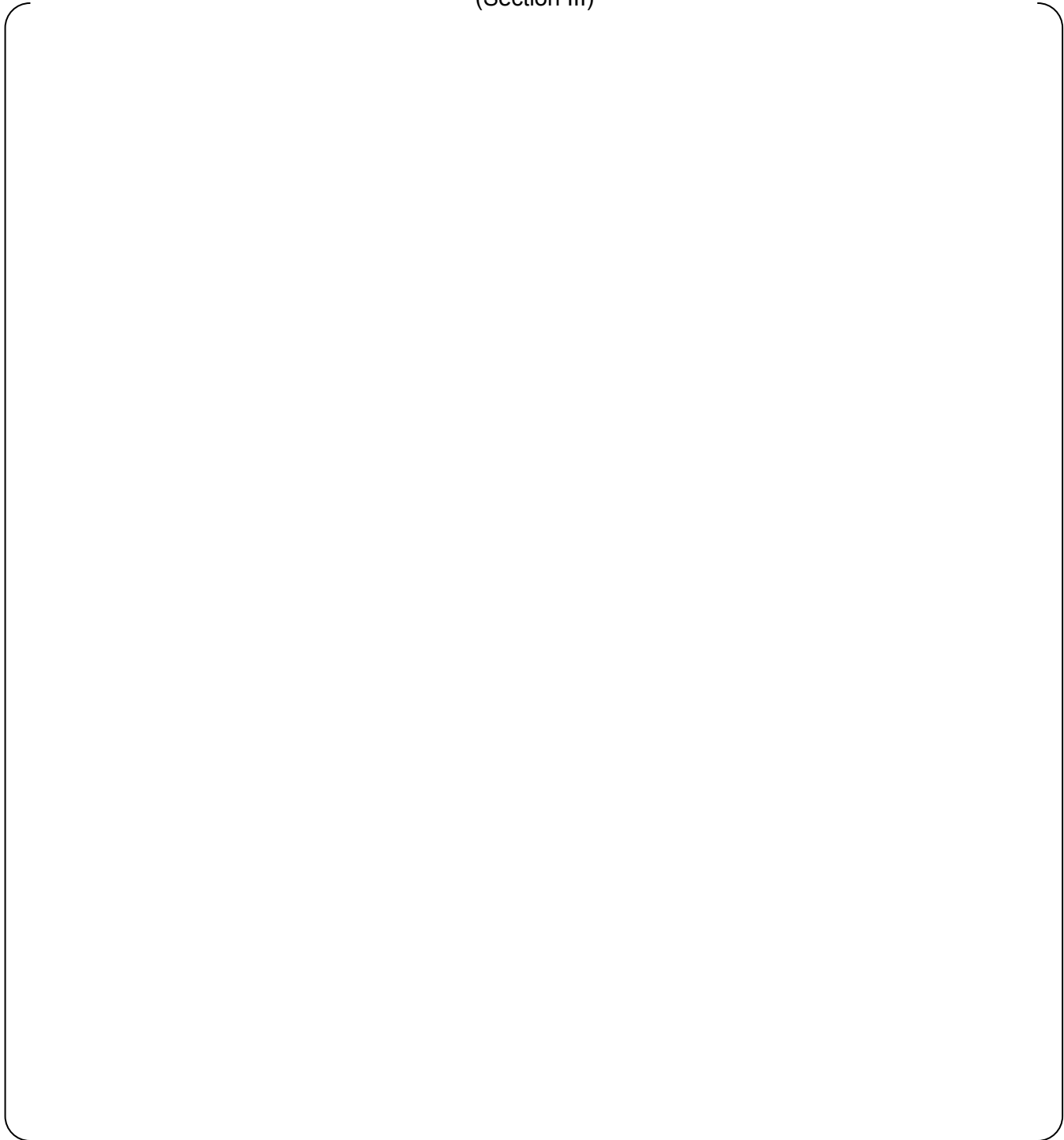


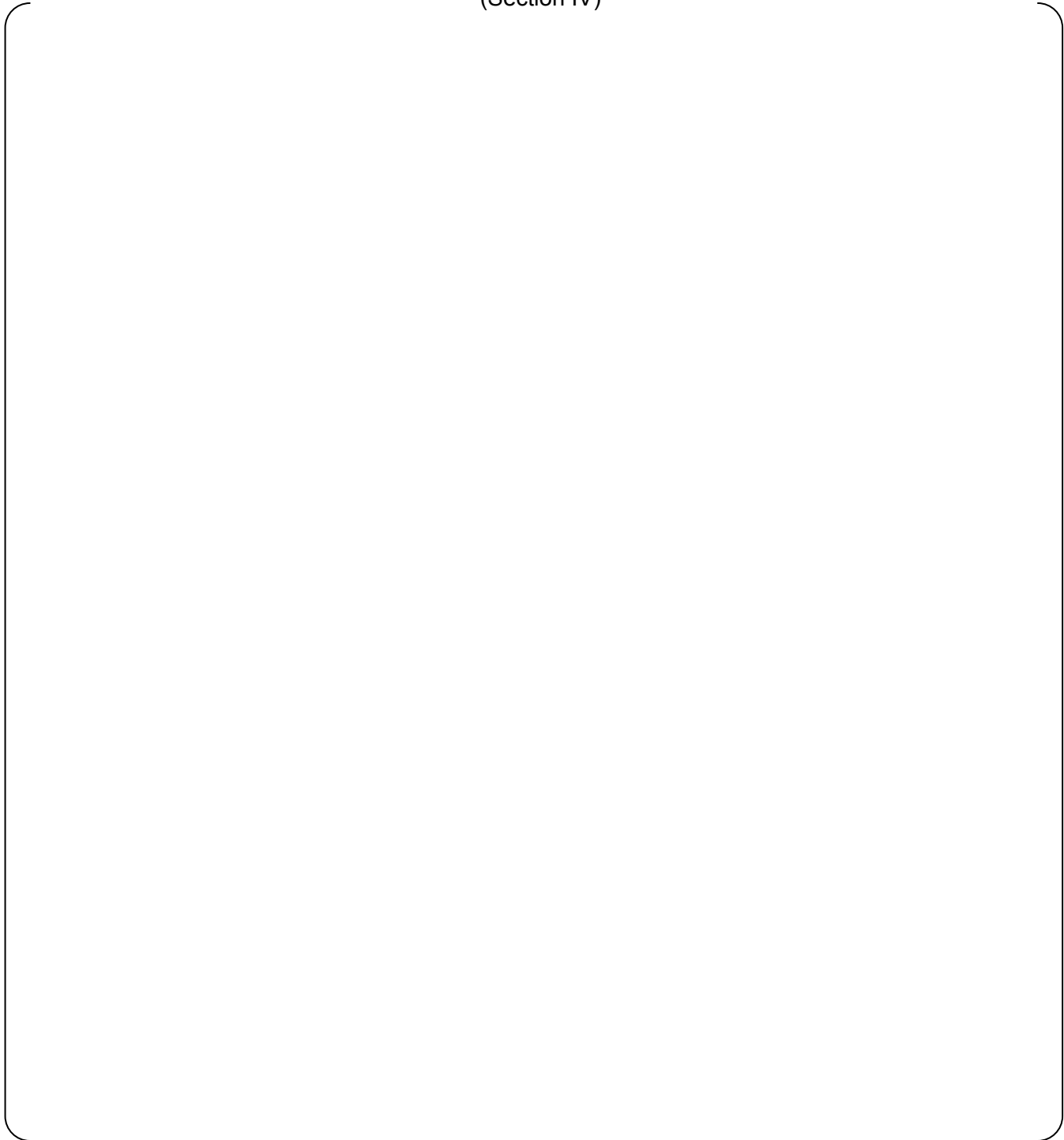
Table A1-8-1-6 Level A, B temperature and pressure input data (8/15)  
(Section III)

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Table A1-8-1-6 Level A, B temperature and pressure input data (9/15)  
(Section III)

---

Table A1-8-1-6 Level A, B temperature and pressure input data (10/15)  
(Section IV)



**Table A1-8-1-6 Level A, B temperature and pressure input data (11/15)  
(Section IV)**



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Table A1-8-1-6 Level A, B temperature and pressure input data (12/15)  
(Section IV)

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**Table A1-8-1-6 Level A, B temperature and pressure input data (13/15)**  
(Section V)

Table A1-8-1-6 Level A, B temperature and pressure input data (14/15)  
(Section V)

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Table A1-8-1-6 Level A, B temperature and pressure input data (15/15)  
(Section V)

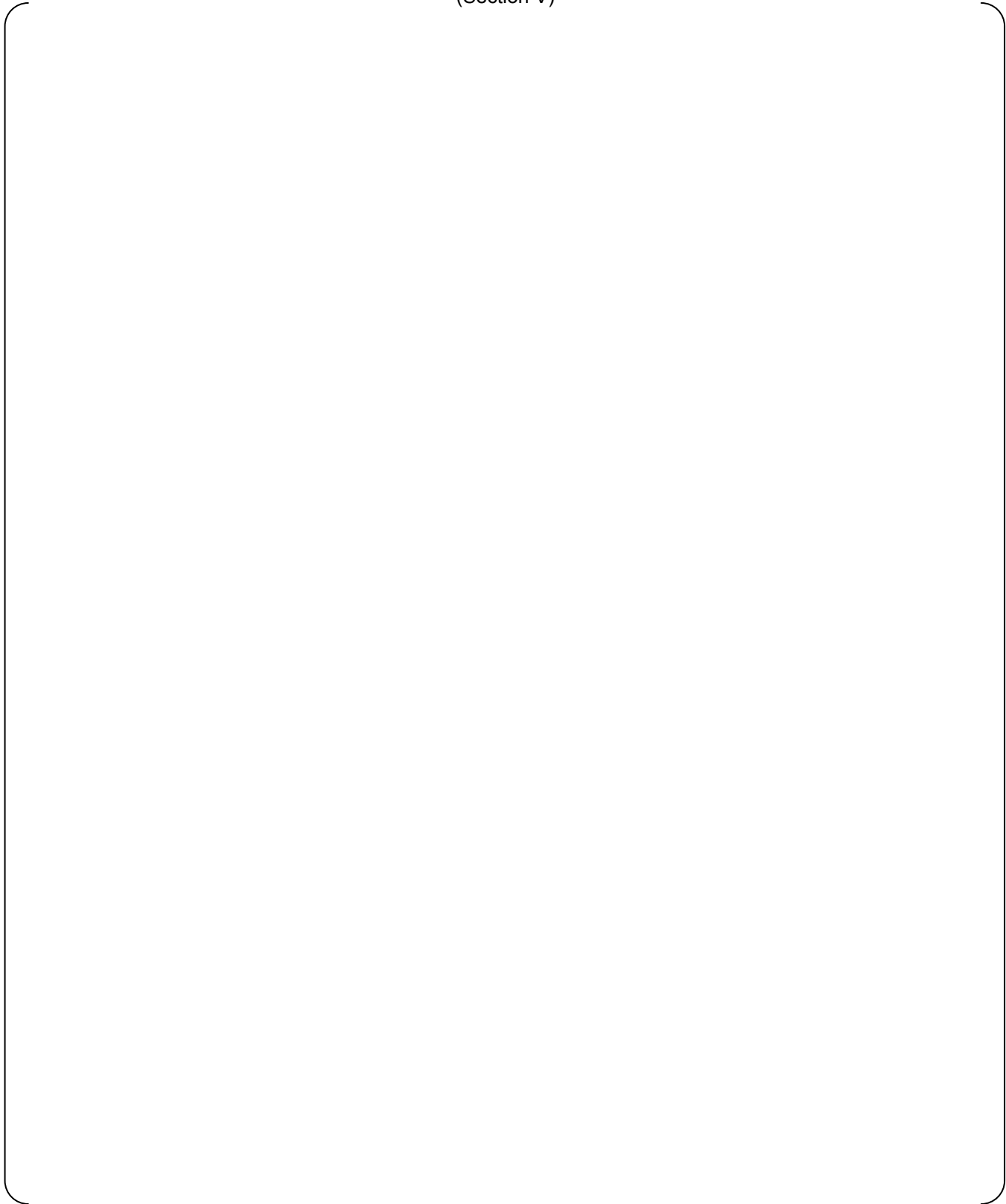


Table A1-8-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-8-1 -2 Floor response curve (1/6)**  
RHRS Return (RH05-06) FRS for MCP Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-8-1-2 Floor response curve (2/6)**  
RHRS Return (RH05-06) FRS for MCP Nozzle  
Y (NS) direction (damping 4.0%)

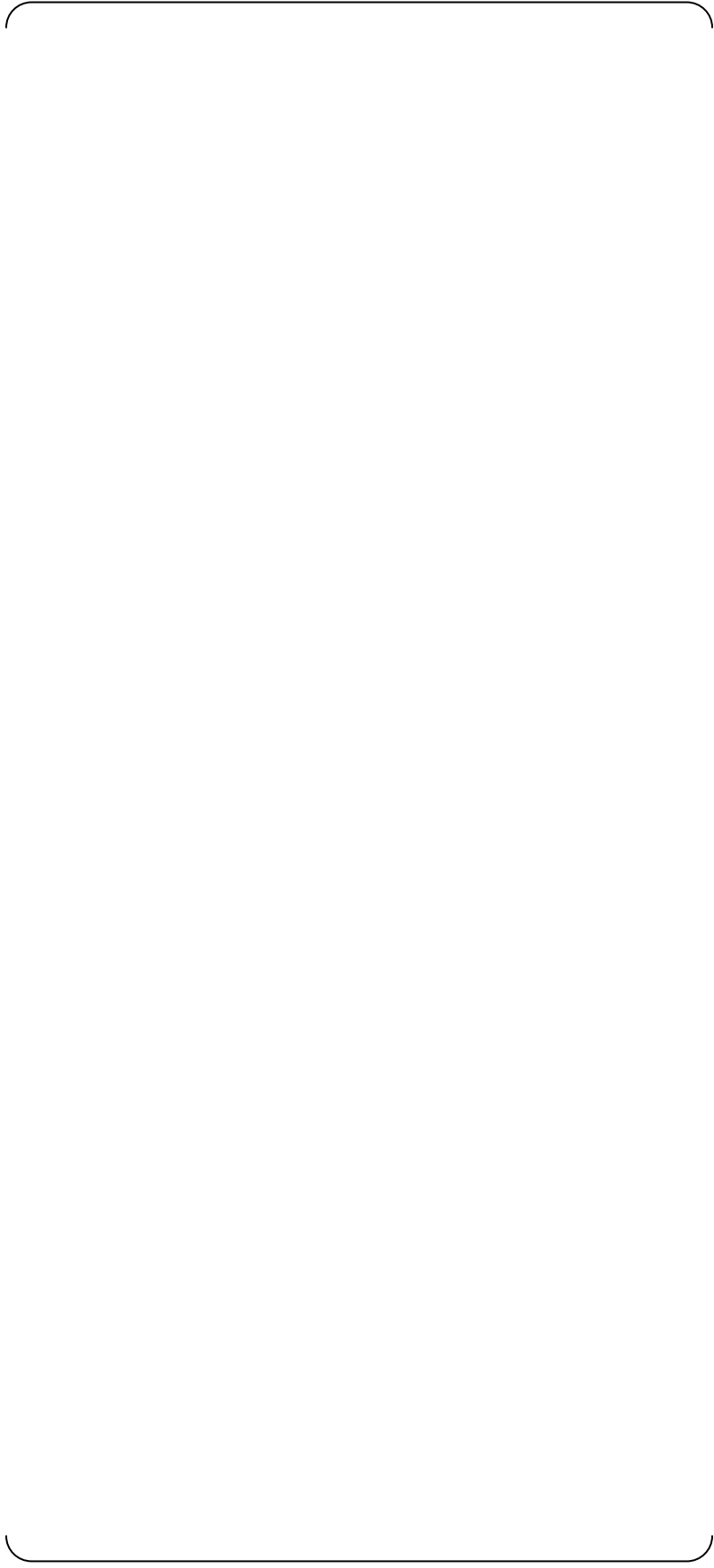


**Figure A1-8-1 -2 Floor response curve (3/6)**  
RHRS Return (RH05-06) FRS for MCP Nozzle  
Z (Vert.) direction (damping 4.0%)





**Figure A1-8-1 -2 Floor response curve (4/6)**  
RHRS Return (RH05-06) FRS for Piping Supports  
X (EW) direction (damping 4.0%)



**Figure A1-8-1-2 Floor response curve (5/6)**  
RHRS Return (RH05-06) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-8-1-2 Floor response curve (6/6)**  
RHRS Return (RH05-06) FRS for Piping  
Z (Vert.) direction (damping 4.0%)

---

Table A1-8-1-8 Seismic anchor displacement input data (1/2)

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**Table A1-8-1-8 Seismic anchor displacement input data (2/2)**

Table A1-8-1-9 DBPB displacement input data

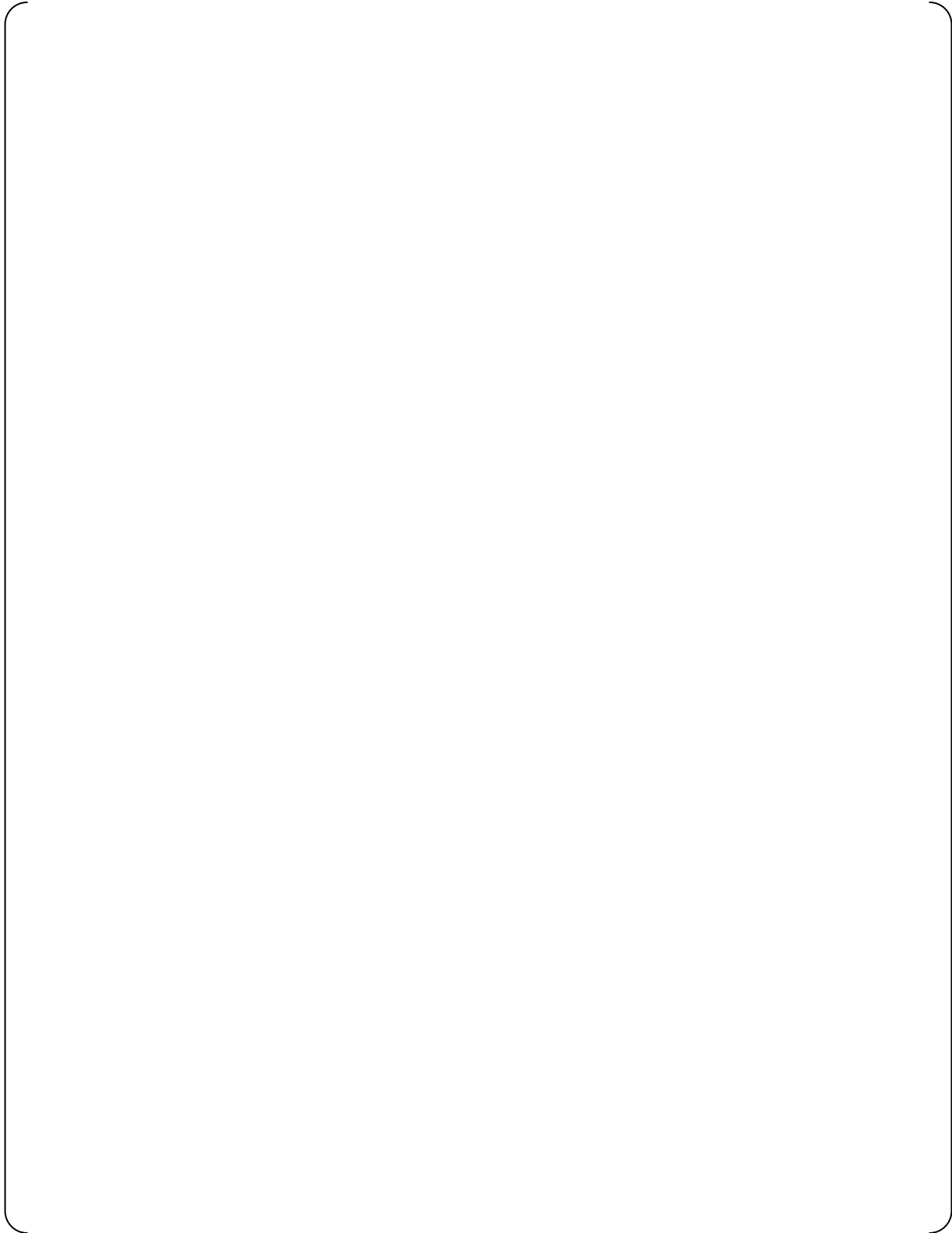


Figure A1-8-2-1 PIPESTRESS analysis model diagram

Table A1-8-2-1 Natural frequency analysis results



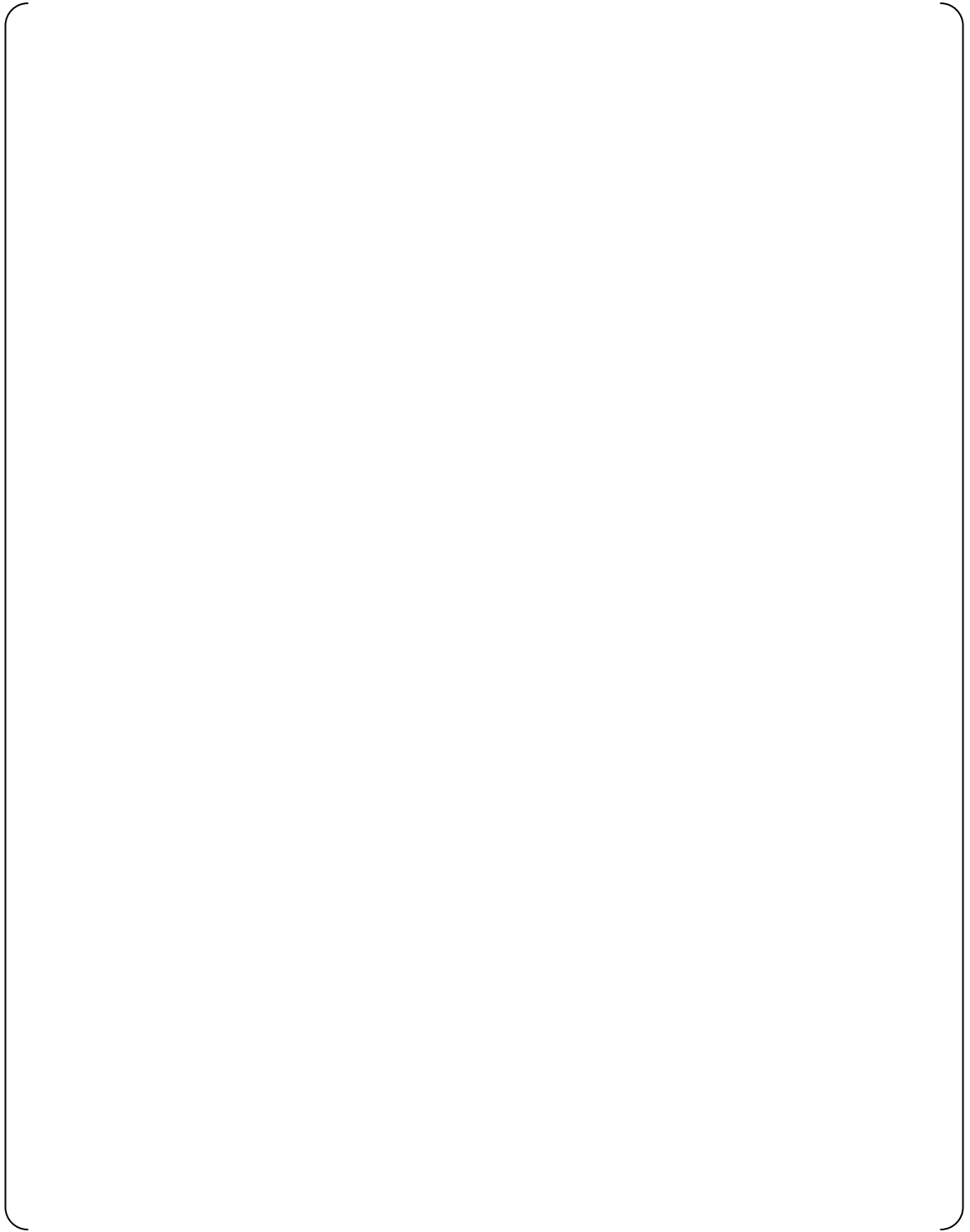


Figure A1-8-2-2 Frequency mode diagram (primary)

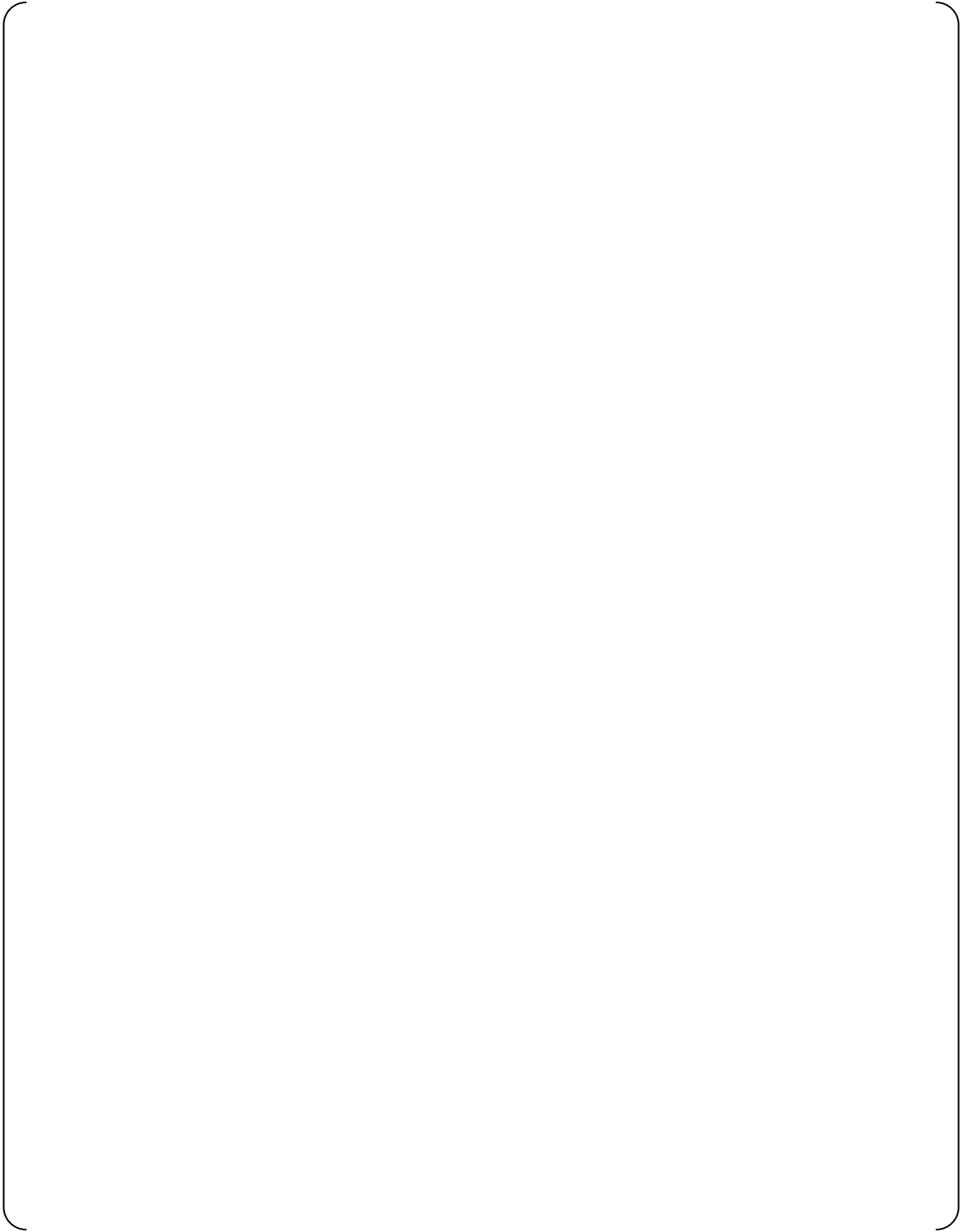


Figure A1-8-2-2 Frequency mode diagram (secondary)

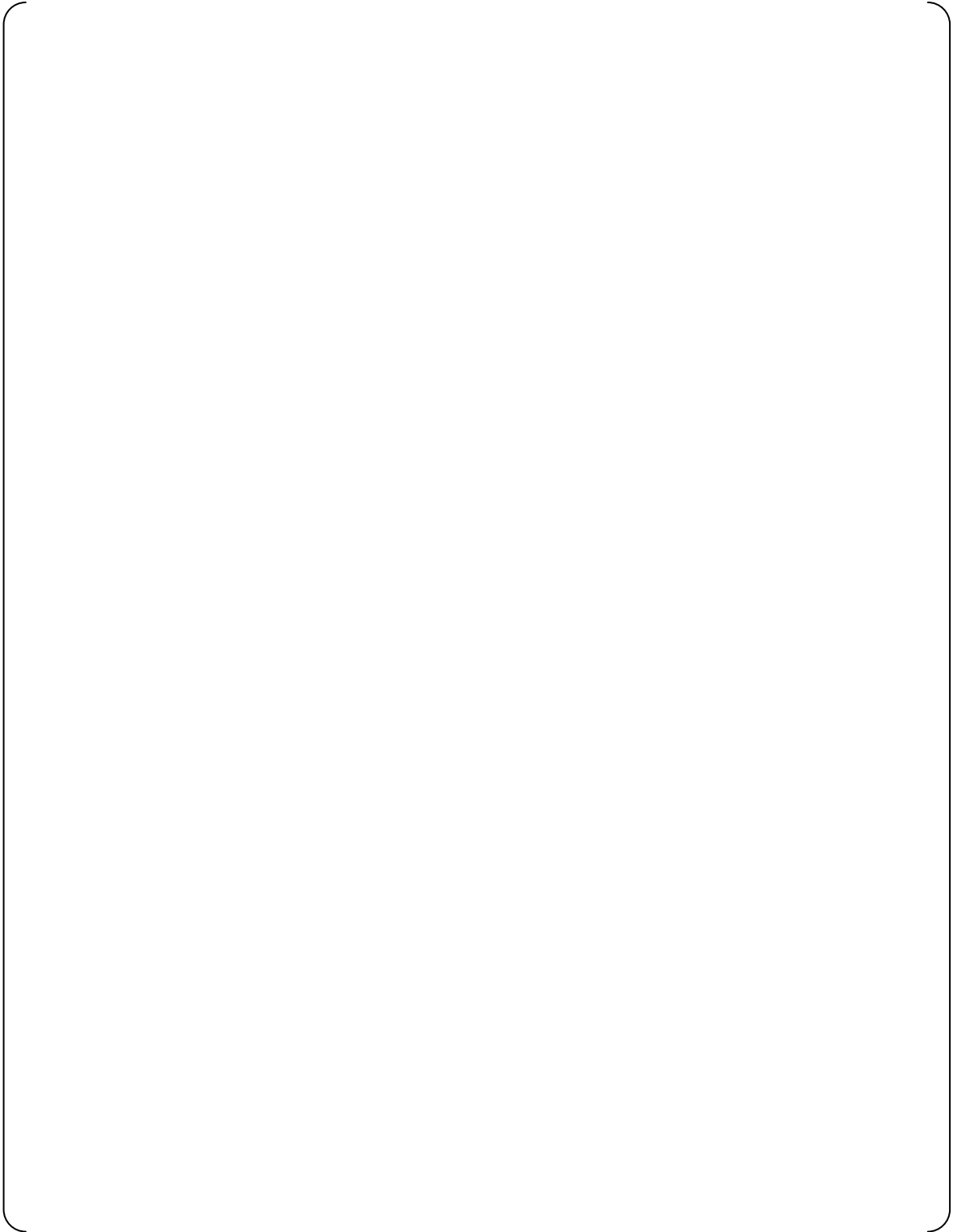


Figure A1-8-2-2 Frequency mode diagram (tertiary)

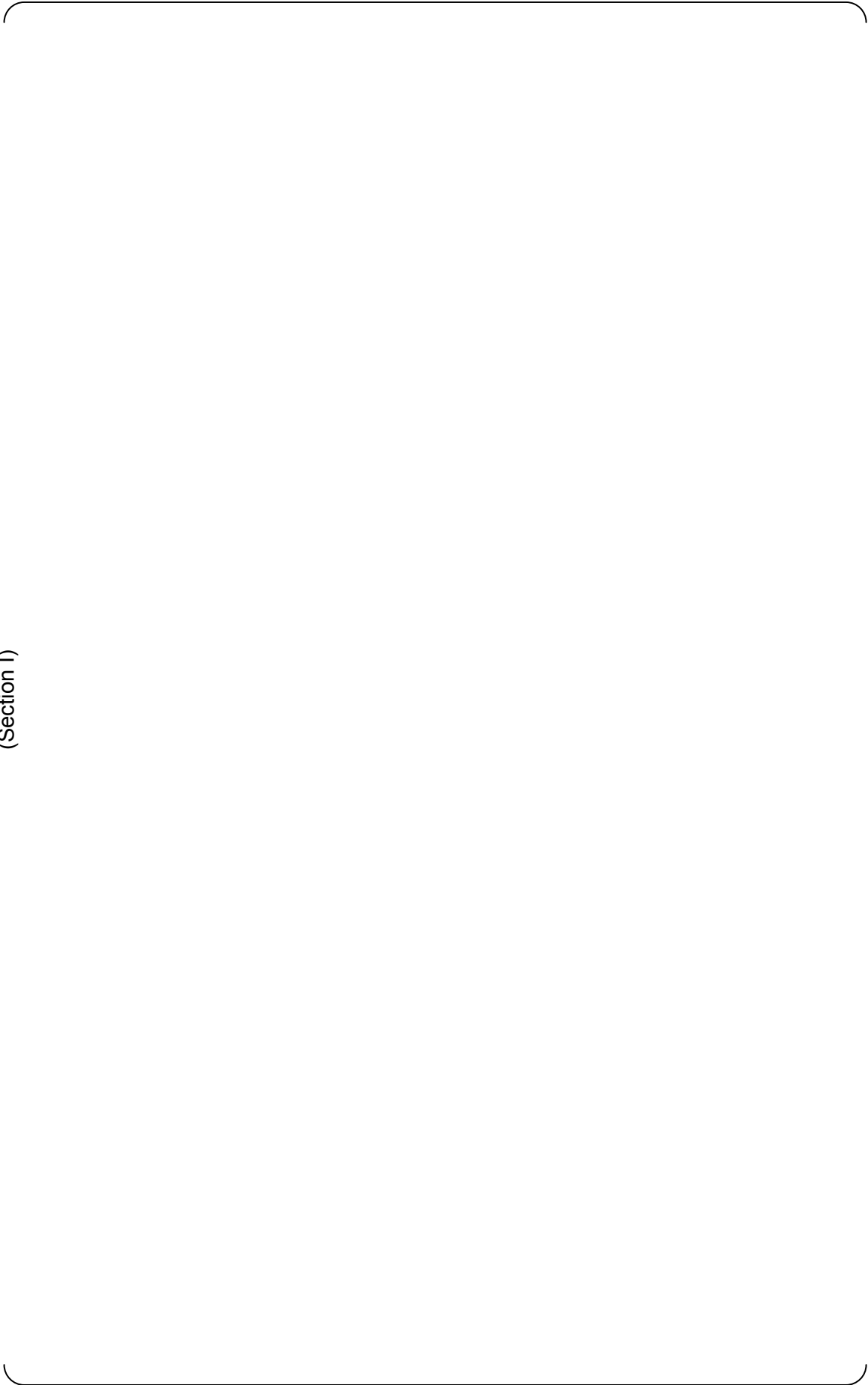


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-8-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (2/6)  
(Section I)





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

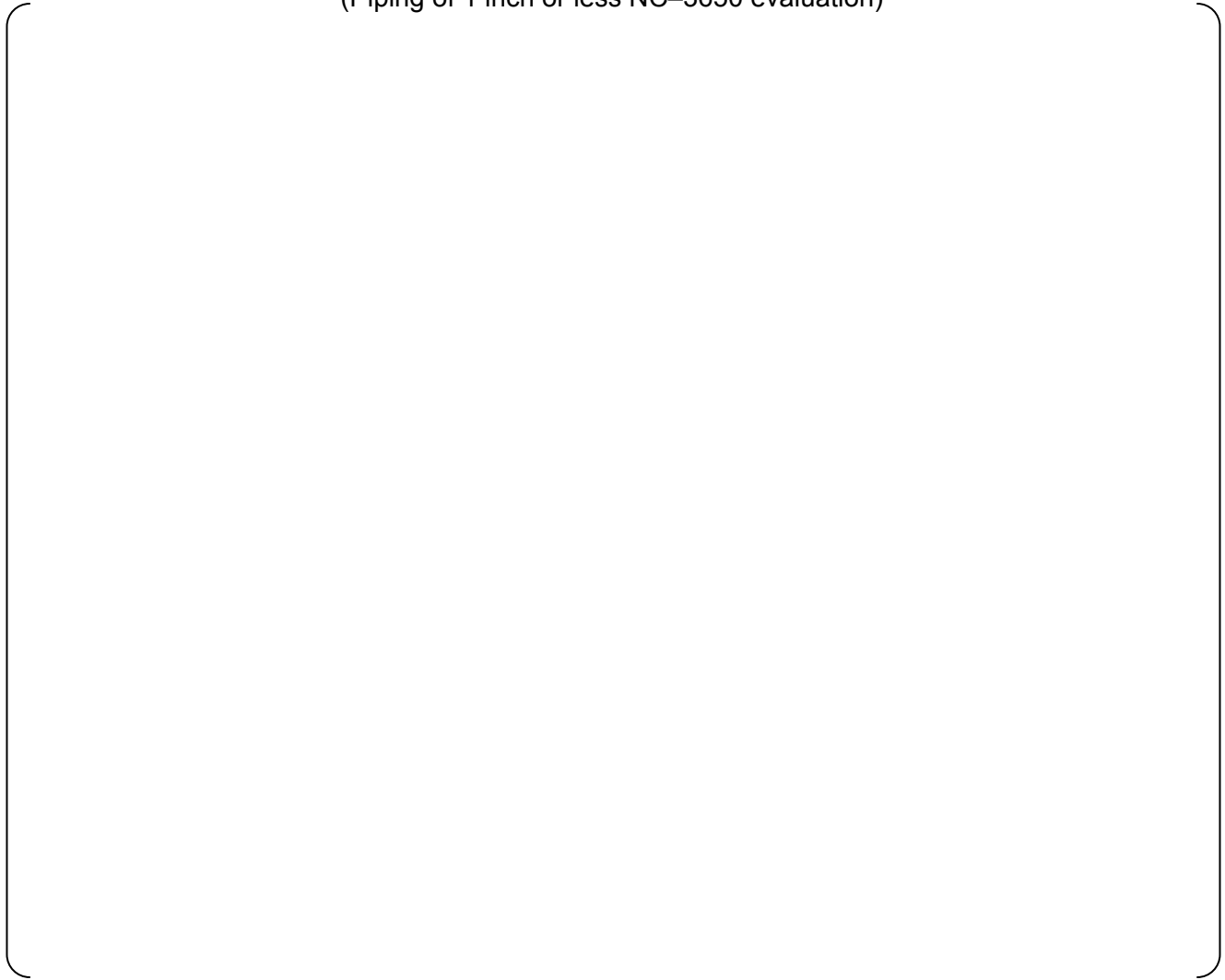
**MUAP-09011-NP (R2)**



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**Table A1-8-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-8-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)



## Appendix 1-9

### SI01 Accumulator Loop A Line Piping Analysis Results



1. INPUT

|  |                 |
|--|-----------------|
| 1.1 Used for creating the pipe structural model              |                 |
| 1.1.1 Block division and piping specifications               | Table A1-9-1-1  |
| 1.1.2 Piping isometrics                                      | Figure A1-9-1-1 |
| 1.1.3 Concentrated mass                                      | Table A1-9-1-2  |
| 1.1.4 Support point rigidity                                 | Table A1-9-1-3  |
| 1.1.5 Valve rigidity   | Table A1-9-1-4  |
| 1.2 Used for creating load conditions                        |                 |
| 1.2.1 Level A/B design transient                             | see main text   |
| 1.2.2 Level A/B thermal displacement input data              | Table A1-9-1-5  |
| 1.2.3 Level A, B temperature and pressure input data         | Table A1-9-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data | Table A1-9-1-7  |
| 1.2.5 Floor response curve                                   | Figure A1-9-1-2 |
| 1.2.6 Seismic anchor displacement input data                 | Table A1-9-1-8  |
| 1.2.7 DBPB displacement input data                           | Table A1-9-1-9  |

2. OUTPUT

|  |                 |
|--|-----------------|
| 2.1 PIPESTRESS analysis model diagram                                      | Figure A1-9-2-1 |
| 2.2 Natural frequency analysis results                                     | Table A1-9-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                           | Figure A1-9-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a - T_b$ ) | Table A1-9-2-2  |
| 2.5 Piping stress and fatigue evaluation results                           | Table A1-9-2-3  |
| 2.6 LBB evaluation results   | Figure A1-9-2-3 |

Table A1-9-1-1 Block division and piping specifications)



Figure A1-9-1-1 Piping isometric

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Table A1-9-1-2 Concentrated mass

Table A1-9-1-3 Support point rigidity



Table A1-9-1-4 Valve rigidity

Table A1-9-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9010)

| Point | X | Y | Z | U | V | W |
|-------|---|---|---|---|---|---|
| 9010  |   |   |   |   |   |   |



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-9-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9010)

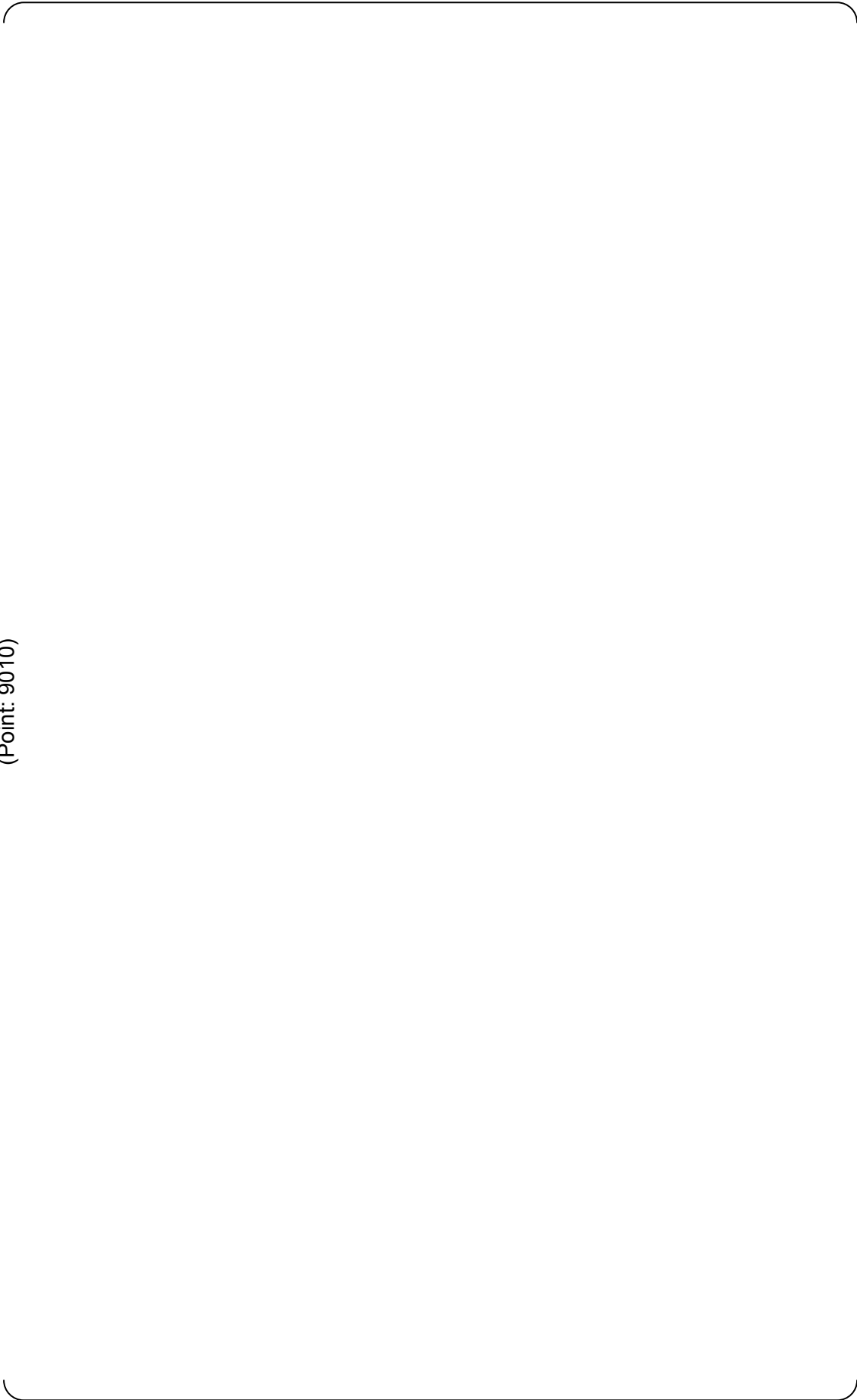


Table A1-9-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |

---

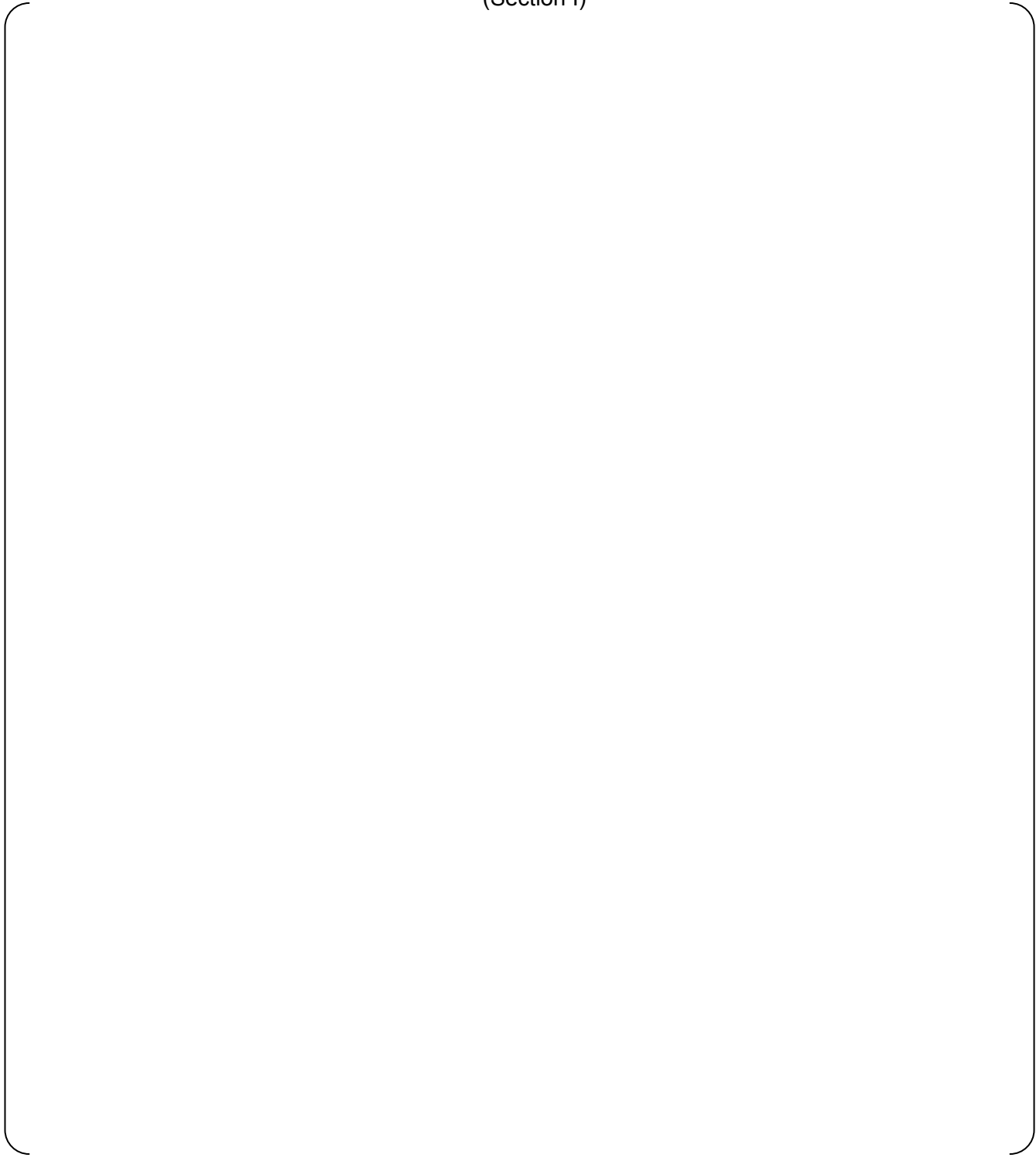
**Table A1-9-1-6 Level A, B temperature and pressure input data (1/12)**  
(Section I)

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**Table A1-9-1-6 Level A, B temperature and pressure input data (2/12)**  
(Section I)

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Table A1-9-1-6 Level A, B temperature and pressure input data (3/12)  
(Section I)



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Table A1-9-1-6 Level A, B temperature and pressure input data (4/12)  
(Section II)

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Table A1-9-1-6 Level A, B temperature and pressure input data (5/12)  
(Section II)



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Table A1-9-1-6 Level A, B temperature and pressure input data (6/12)  
(Section II)

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Table A1-9-1-6 Level A, B temperature and pressure input data (7/12)  
(Section III)

**Table A1-9-1-6 Level A, B temperature and pressure input data (8/12)  
(Section III)**

**Table A1-9-1-6 Level A, B temperature and pressure input data (9/12)  
(Section III)**

---

Table A1-9-1-6 Level A, B temperature and pressure input data (10/12)  
(Section IV)

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Table A1-9-1-6 Level A, B temperature and pressure input data (11/12)  
(Section IV)

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Table A1-9-1-6 Level A, B temperature and pressure input data (12/12)  
(Section IV)

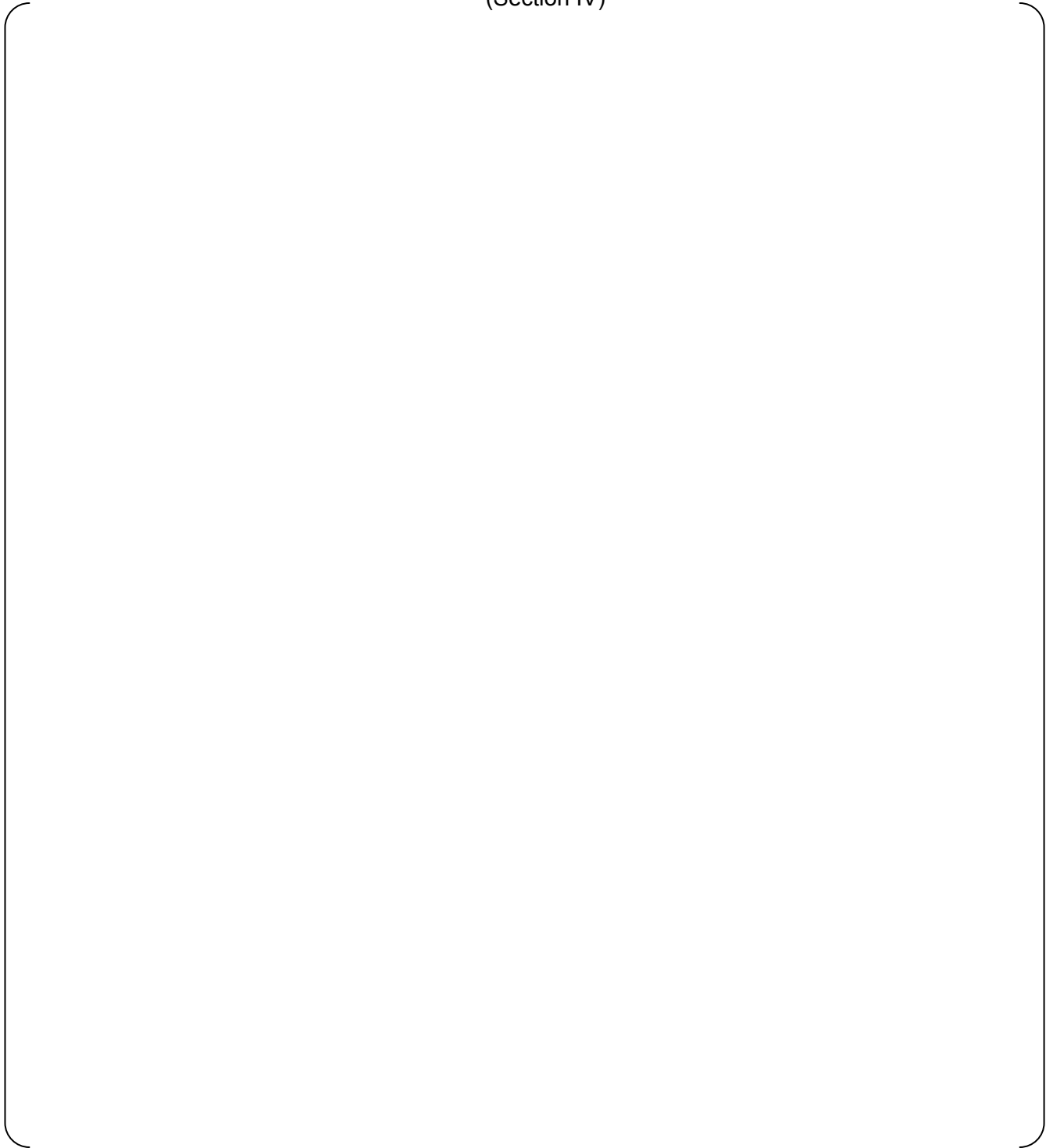
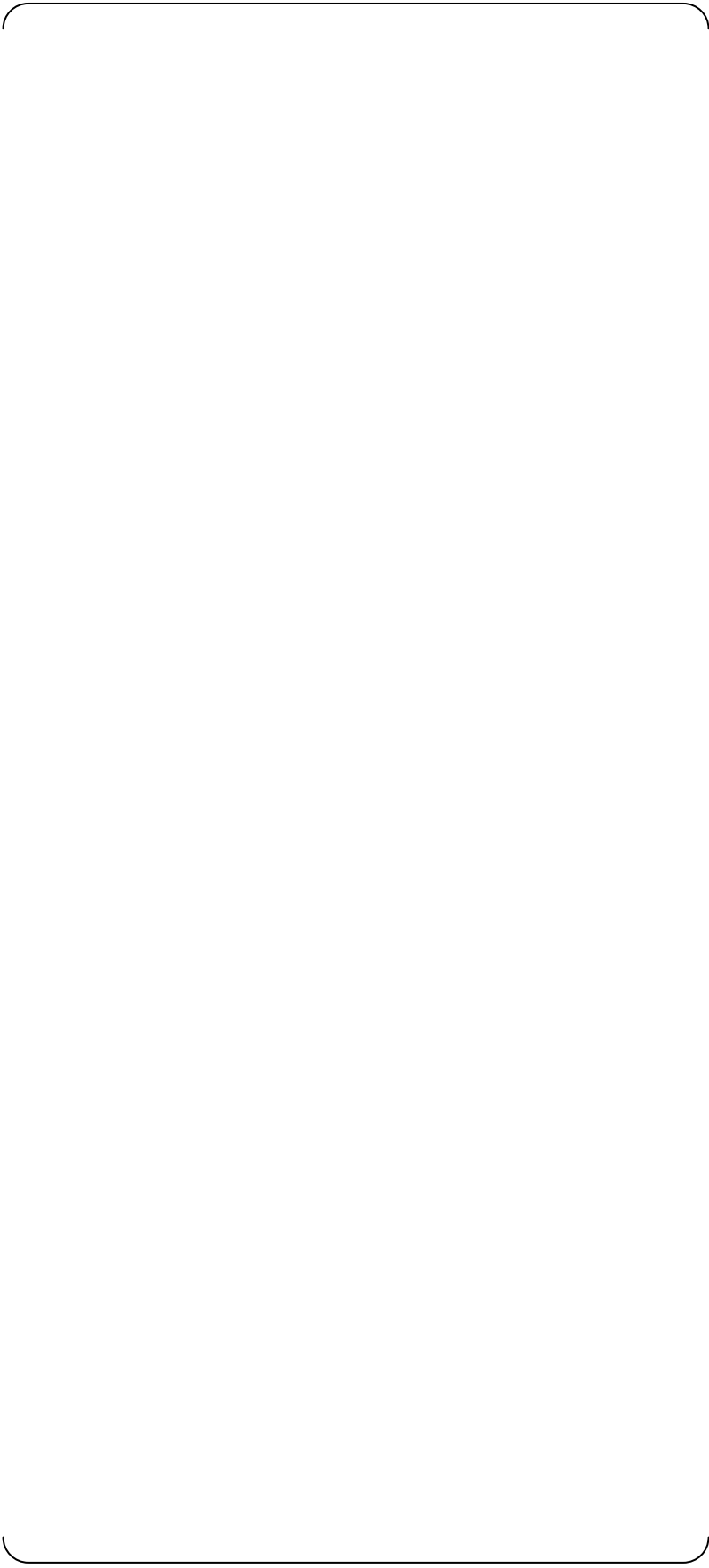


Table A1-9-1-7 Level C, D maximum temperature and pressure input data





**Figure A1-9-1 -2 Floor response curve (1/6)**  
Accumulator (SI01-02) FRS for MCP Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-9-1-2 Floor response curve (2/6)**  
Accumulator (SI01-02) FRS for MCP Nozzle  
Y (NS) direction (damping 4.0%)



**Figure A1-9-1 -2 Floor response curve (3/6)**  
Accumulator (SI01-02) FRS for MCP Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-9-1 -2 Floor response curve (4/6)**  
Accumulator (SI01-02) FRS for Piping  
X (EW) direction (damping 4.0%)



**Figure A1-9-1-2 Floor response curve (5/6)**  
Accumulator (SI01-02) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-9-1-2 Floor response curve (6/6)**  
Accumulator (SI01-02) FRS for Piping  
Z (Vert.) direction (damping 4.0%)

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Table A1-9-1-8 Seismic anchor displacement input data

Table A1-9-1-9 DBPB displacement input data



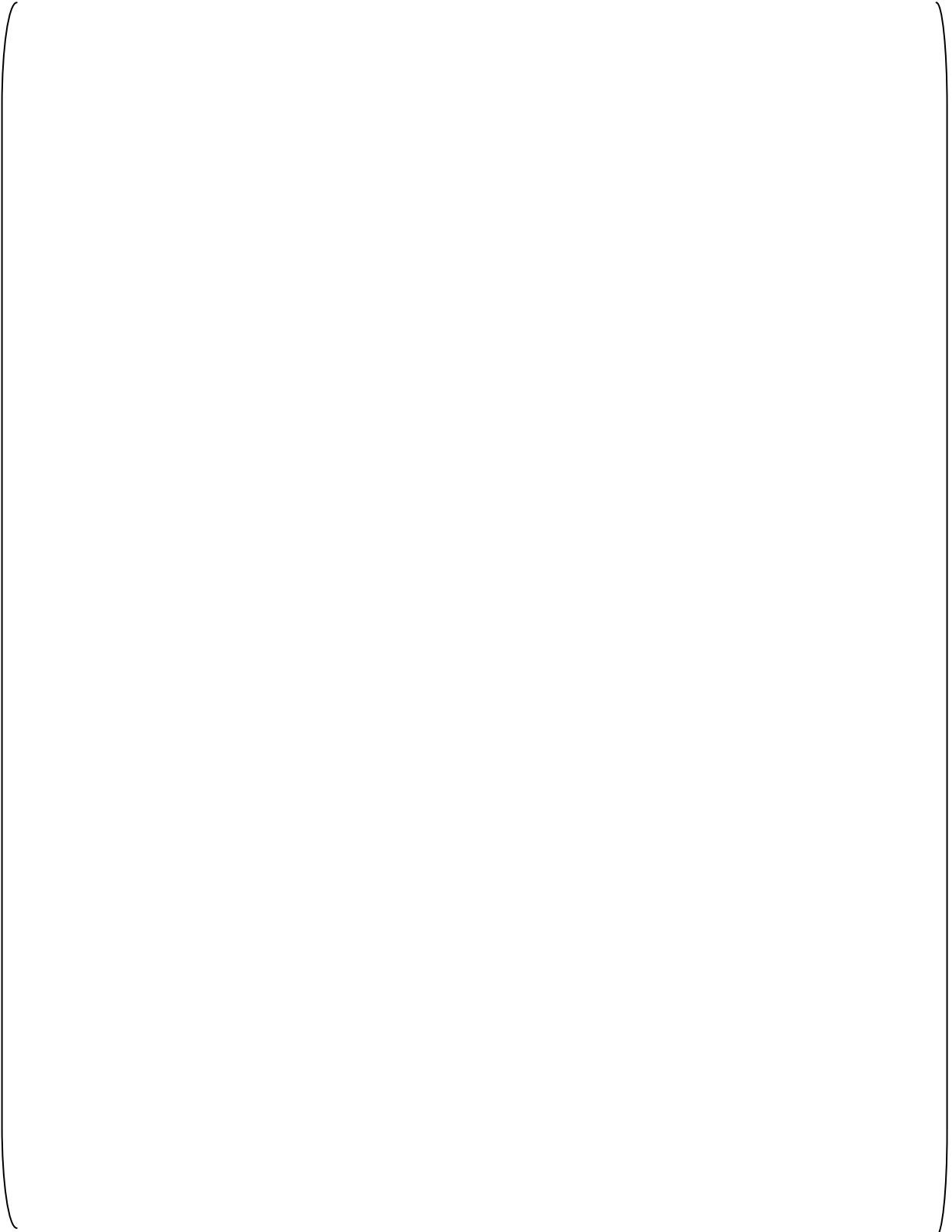
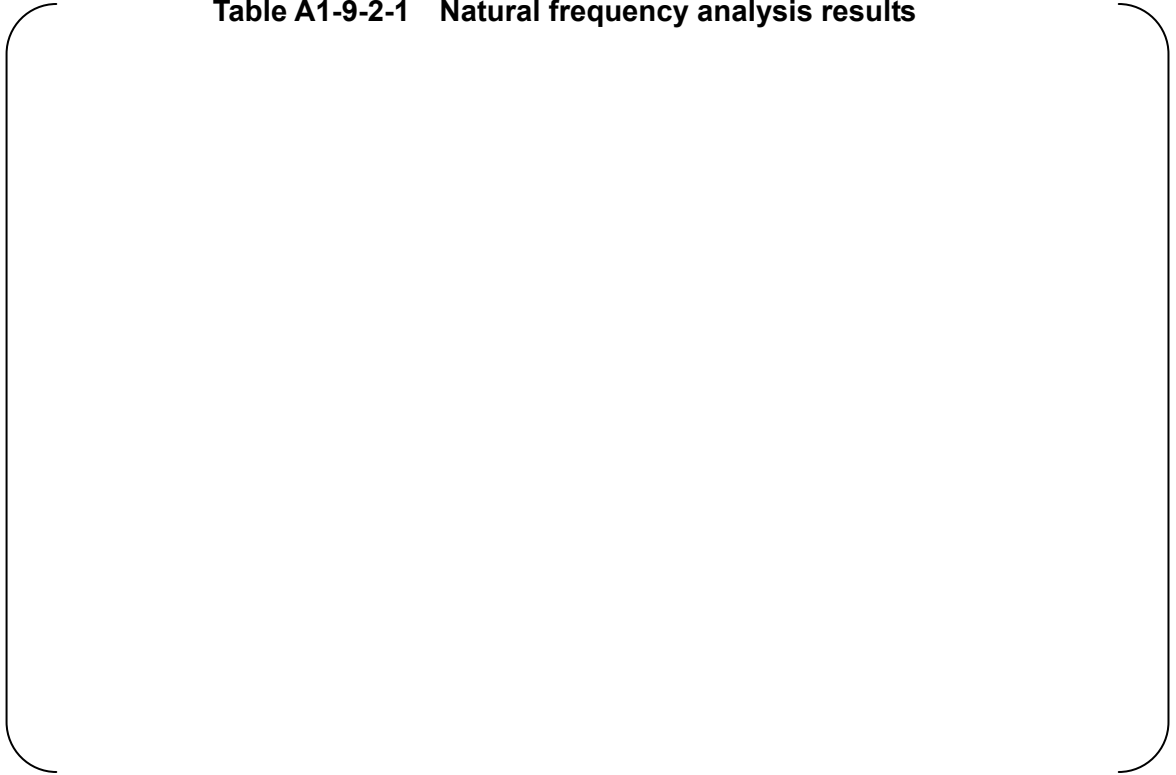


Figure A1-9-2-1 PIPESTRESS analysis model diagram

Table A1-9-2-1 Natural frequency analysis results



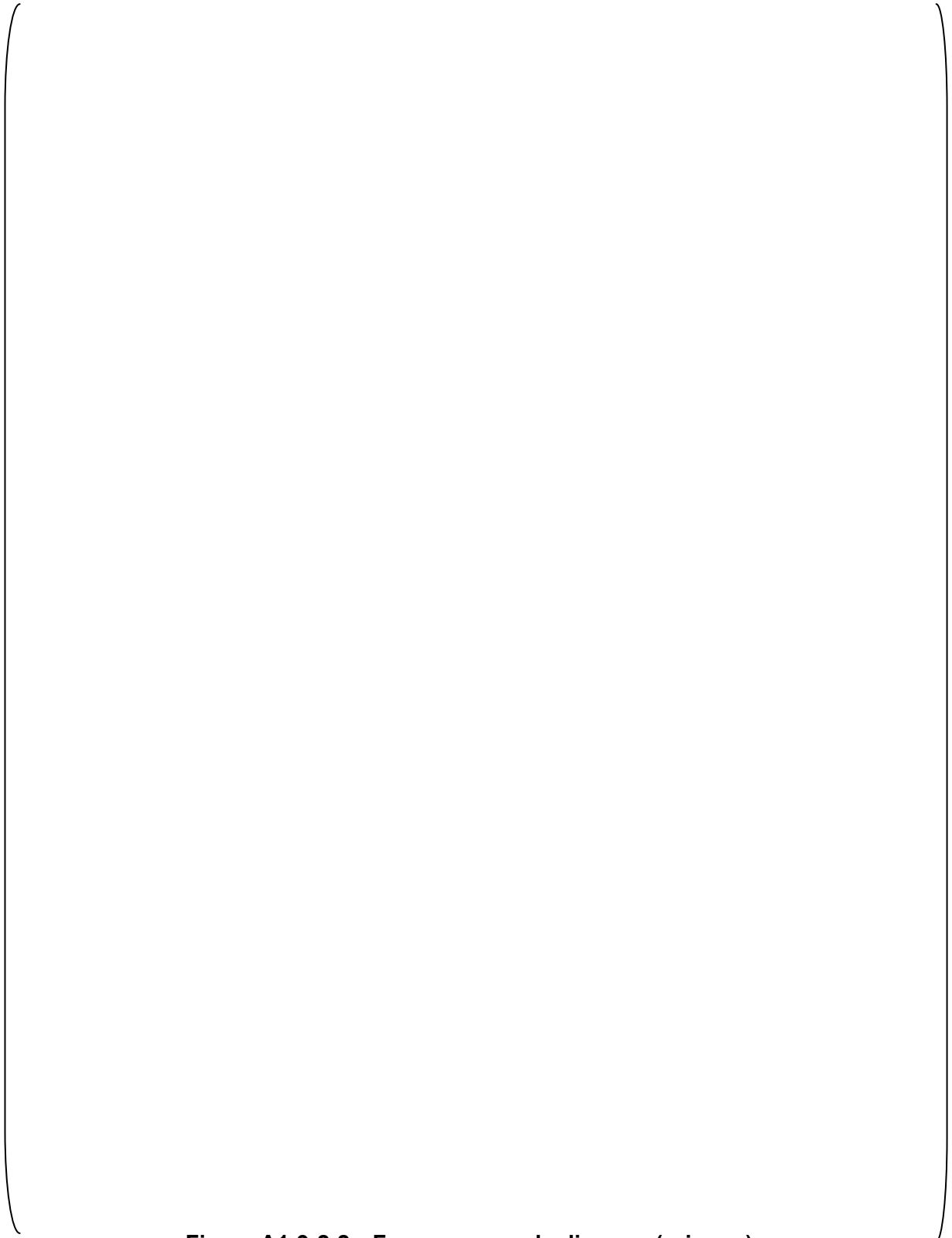


Figure A1-9-2-2 Frequency mode diagram (primary)

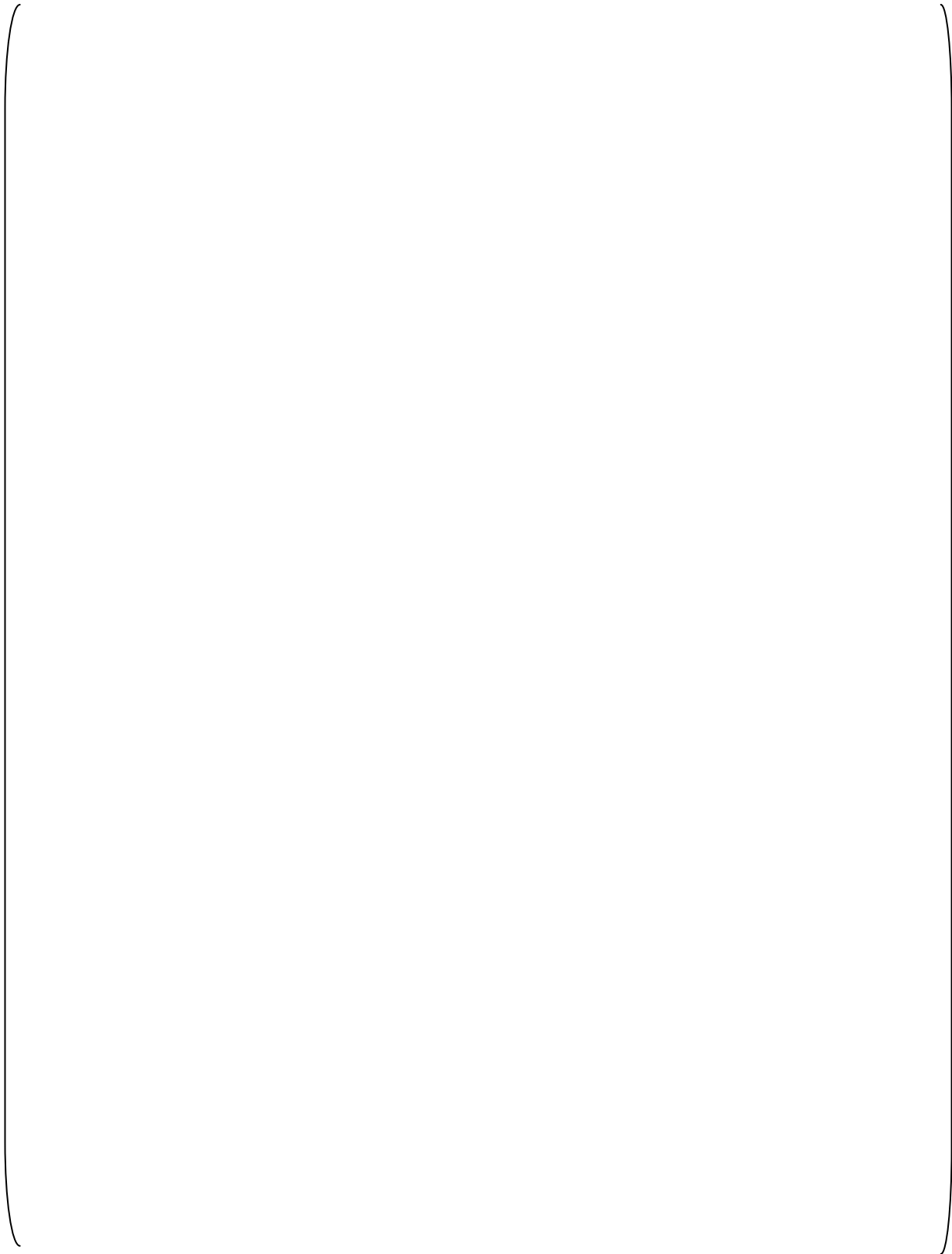


Figure A1-9-2-2 Frequency mode diagram (secondary)

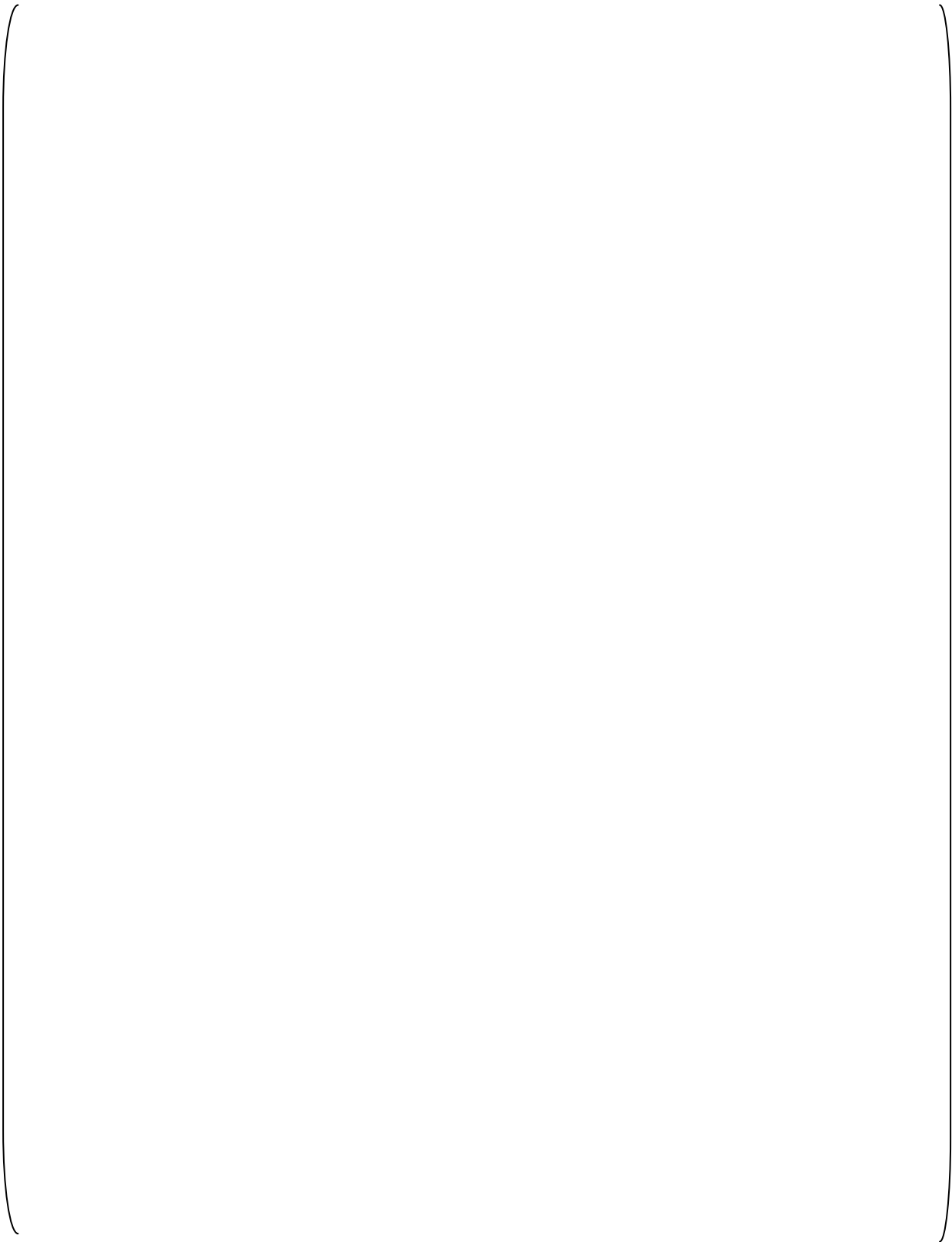


Figure A1-9-2-2 Frequency mode diagram (tertiary)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**

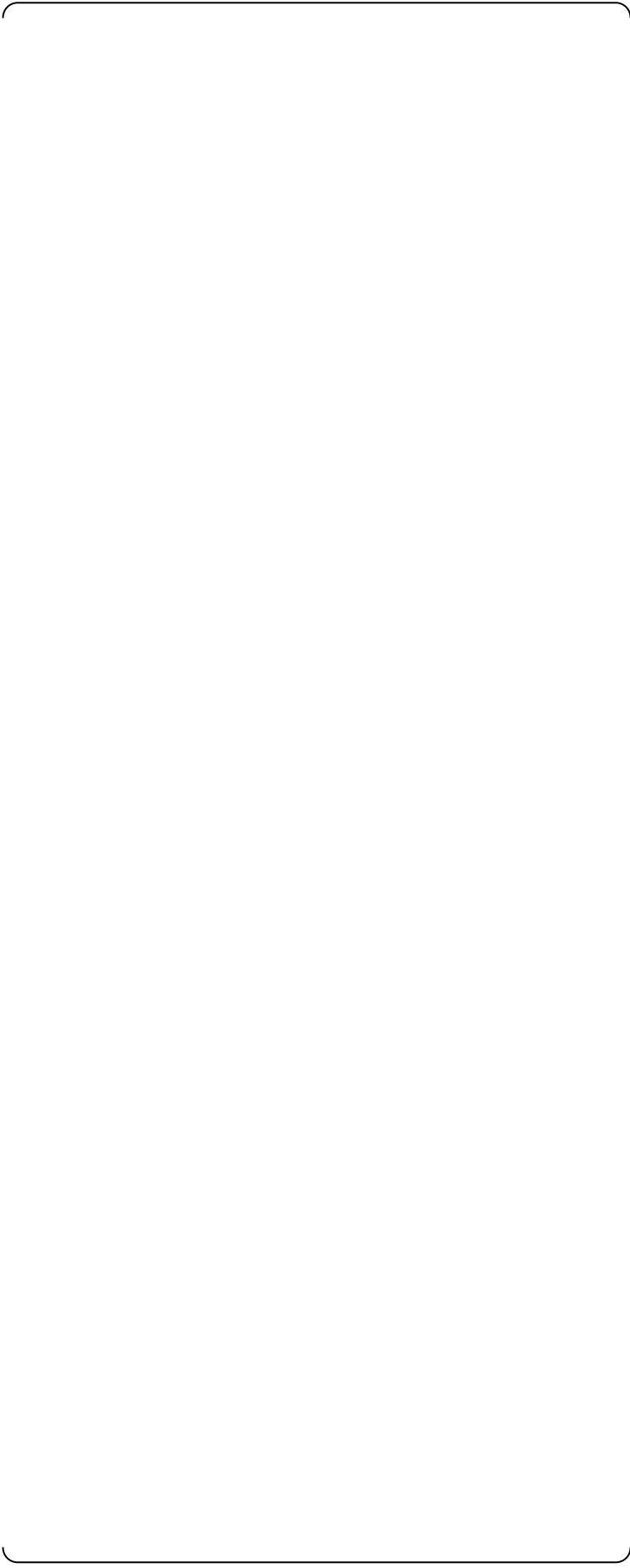
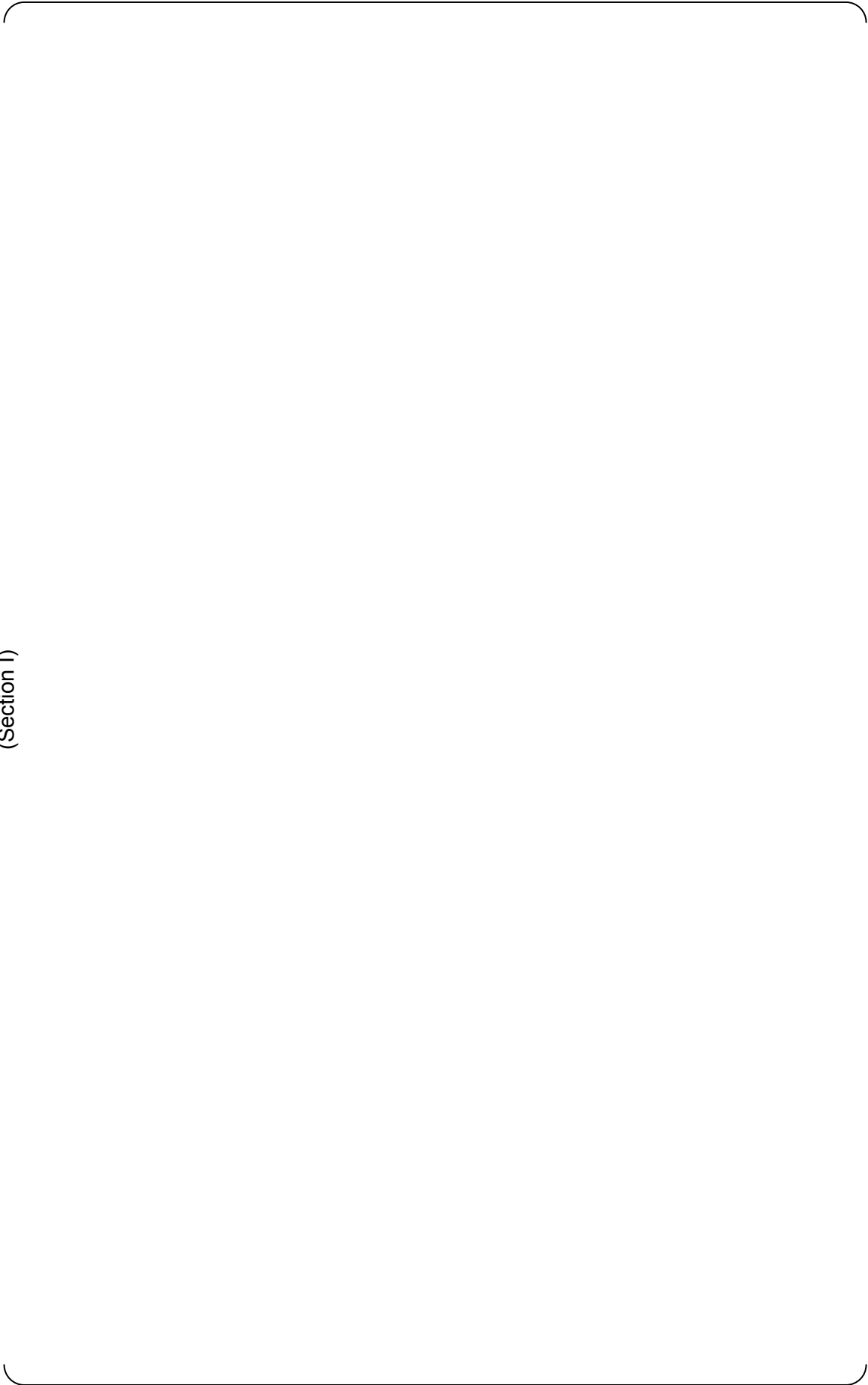


Table A1-9-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (2/6)  
(Section I)







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**

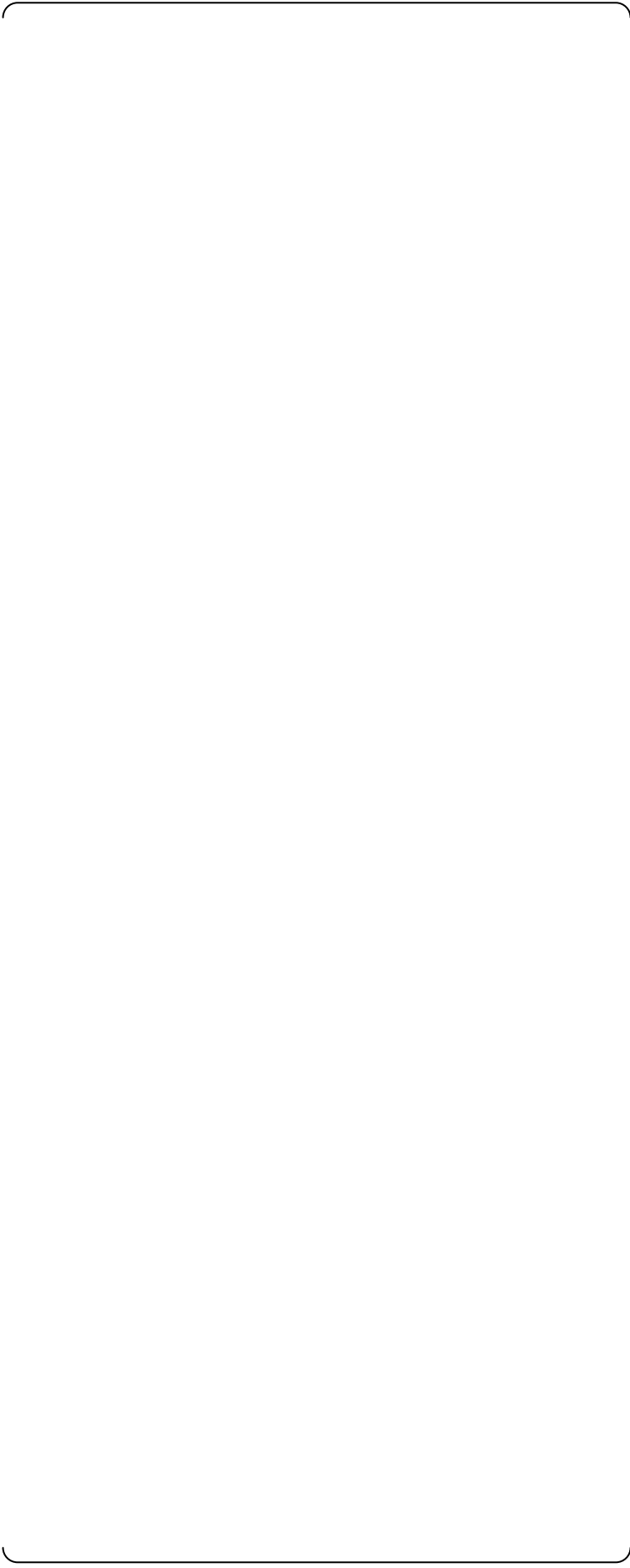


Table A1-9-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (4/6)  
(Section II)

| Location | $\Delta T1$ | $\Delta T2$ | $Ta-Tb$ |
|----------|-------------|-------------|---------|
|----------|-------------|-------------|---------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



**Table A1-9-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

\*1 Evaluation performed for when the primary + secondary stress exceeds  $3S_m$ .



**Table A1-9-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

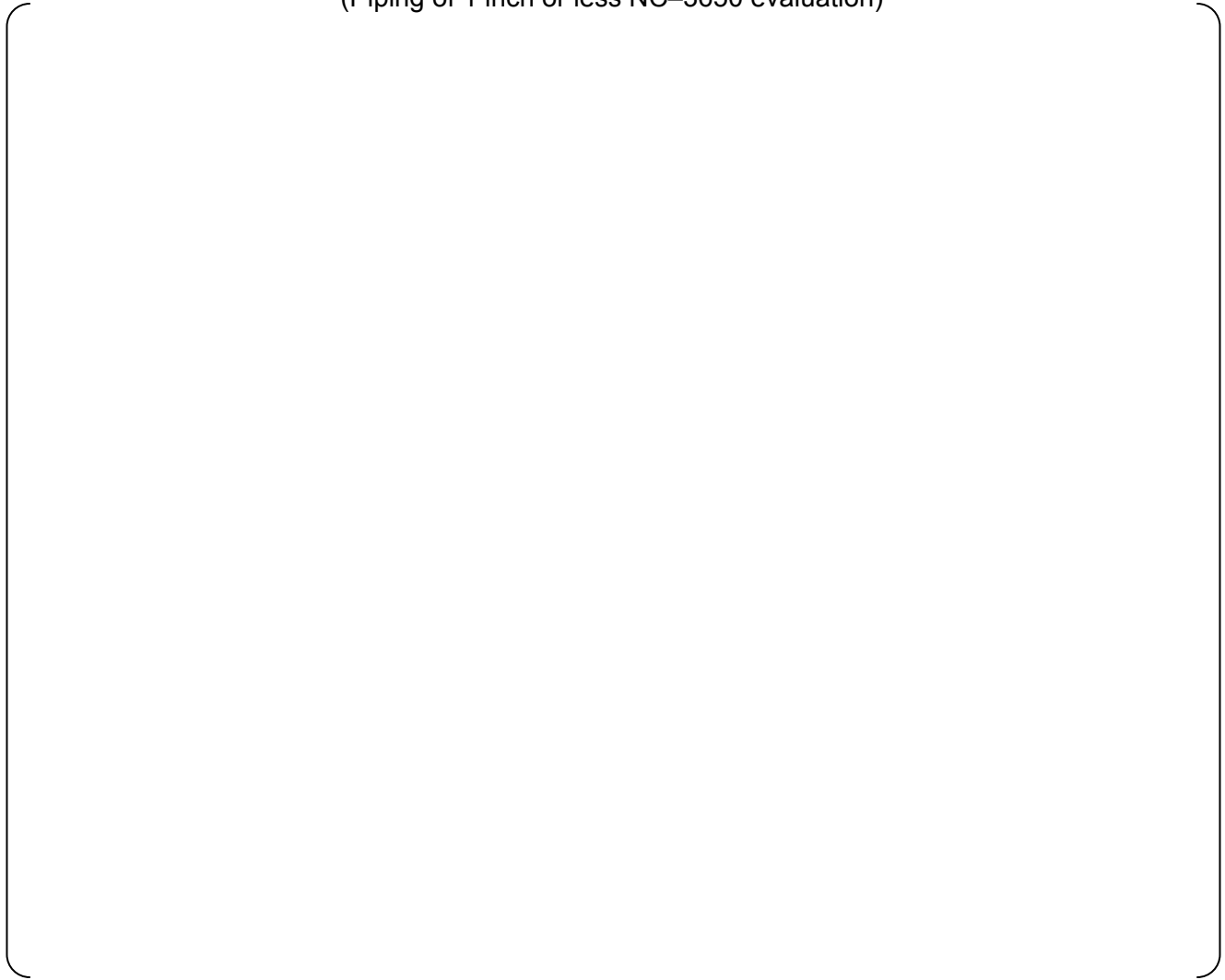




Figure A1-9-2-3 LBB evaluation results

## Appendix 1-10

### SI02 Accumulator Loop B Line Piping Analysis Results

1. INPUT

|  |                  |
|--|------------------|
| 1.1 Used for creating the pipe structural model              |                  |
| 1.1.1 Block division and piping specifications               | Table A1-10-1-1  |
| 1.1.2 Piping isometrics                                      | Figure A1-10-1-1 |
| 1.1.3 Concentrated mass                                      | Table A1-10-1-2  |
| 1.1.4 Support point rigidity                                 | Table A1-10-1-3  |
| 1.1.5 Valve rigidity   | Table A1-10-1-4  |
| 1.2 Used for creating load conditions                        |                  |
| 1.2.1 Level A/B design transient                             | see main text    |
| 1.2.2 Level A/B thermal displacement input data              | Table A1-10-1-5  |
| 1.2.3 Level A, B temperature and pressure input data         | Table A1-10-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data | Table A1-10-1-7  |
| 1.2.5 Floor response curve                                   | Figure A1-10-1-2 |
| 1.2.6 Seismic anchor displacement input data                 | Table A1-10-1-8  |
| 1.2.7 DBPB displacement input data                           | Table A1-10-1-9  |

2. OUTPUT

|  |                  |
|--|------------------|
| 2.1 PIPESTRESS analysis model diagram                                      | Figure A1-10-2-1 |
| 2.2 Natural frequency analysis results                                     | Table A1-10-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                           | Figure A1-10-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a - T_b$ ) | Table A1-10-2-2  |
| 2.5 Piping stress and fatigue evaluation results                           | Table A1-10-2-3  |
| 2.6 LBB evaluation results   | Figure A1-10-2-3 |

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**Table A1-10-1-1 Block division and piping specifications**



Figure A1-10-1-1 Piping isometric

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Table A1-10-1-2 Concentrated mass

Table A1-10-1-3 Support point rigidity



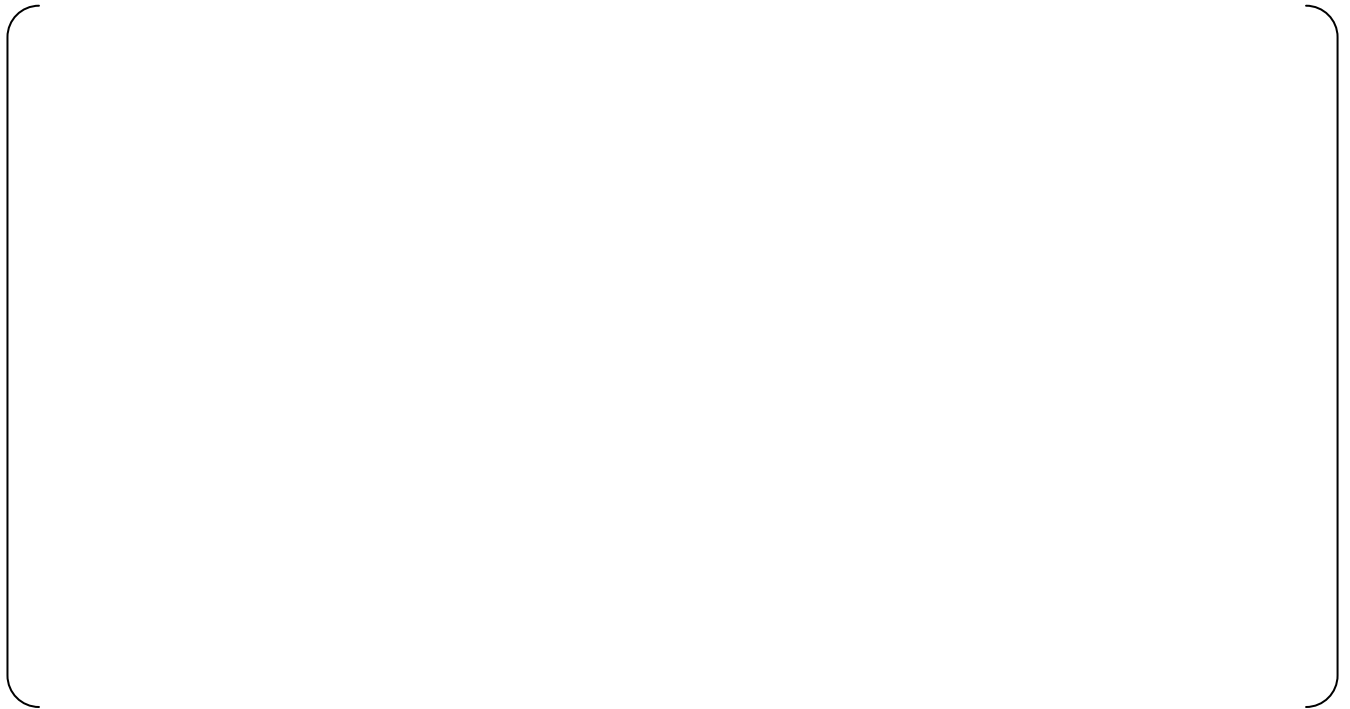


Table A1-10-1-4 Valve rigidity

Table A1-10-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-10-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |

Table A1-10-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |

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Table A1-10-1-6 Level A, B temperature and pressure input data (1/12)  
(Section I)

Table A1-10-1-6 Level A, B temperature and pressure input data (2/12)  
(Section I)

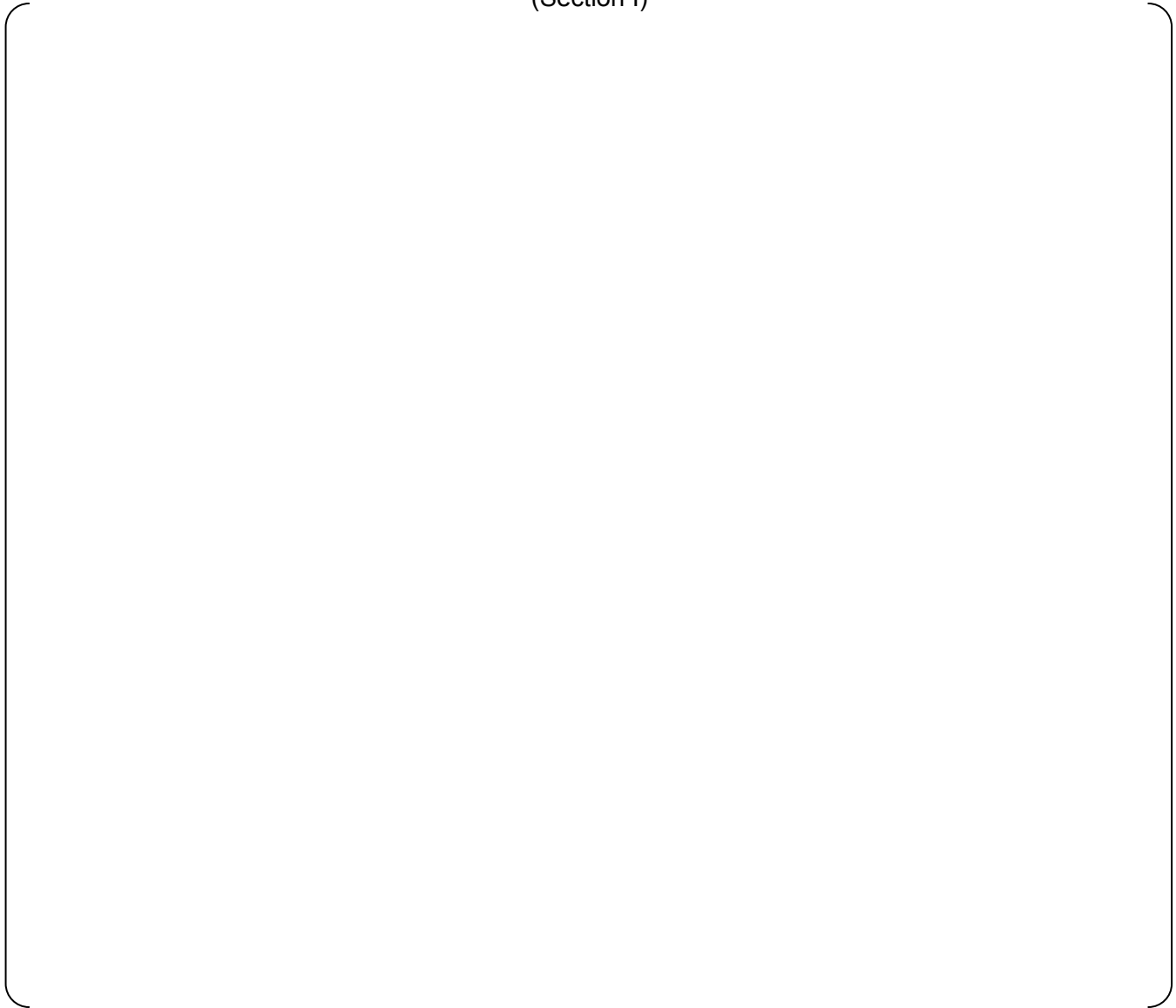




Table A1-10-1-6 Level A, B temperature and pressure input data (3/12)  
(Section I)

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Table A1-10-1-6 Level A, B temperature and pressure input data (4/12)  
(Section II)

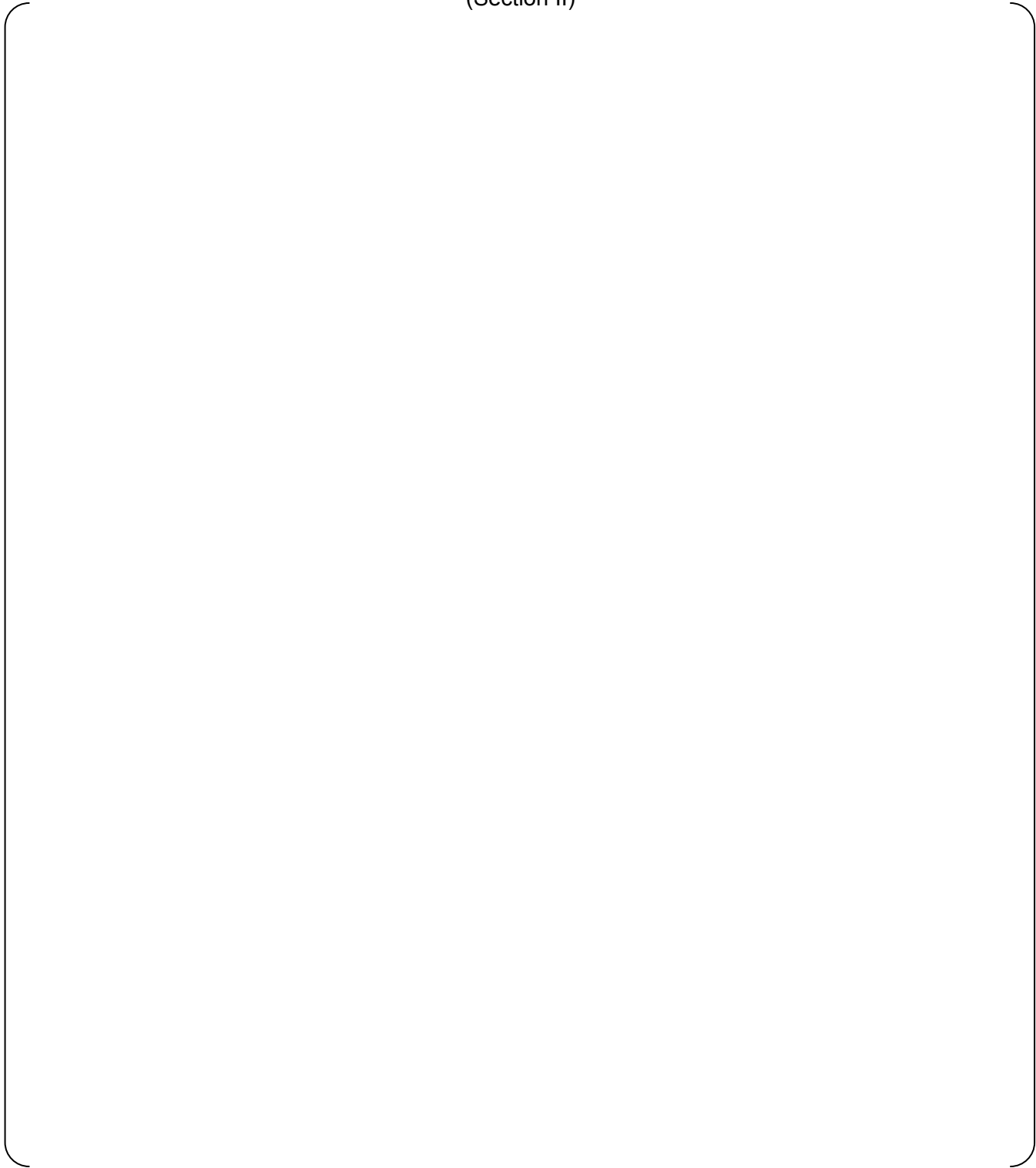
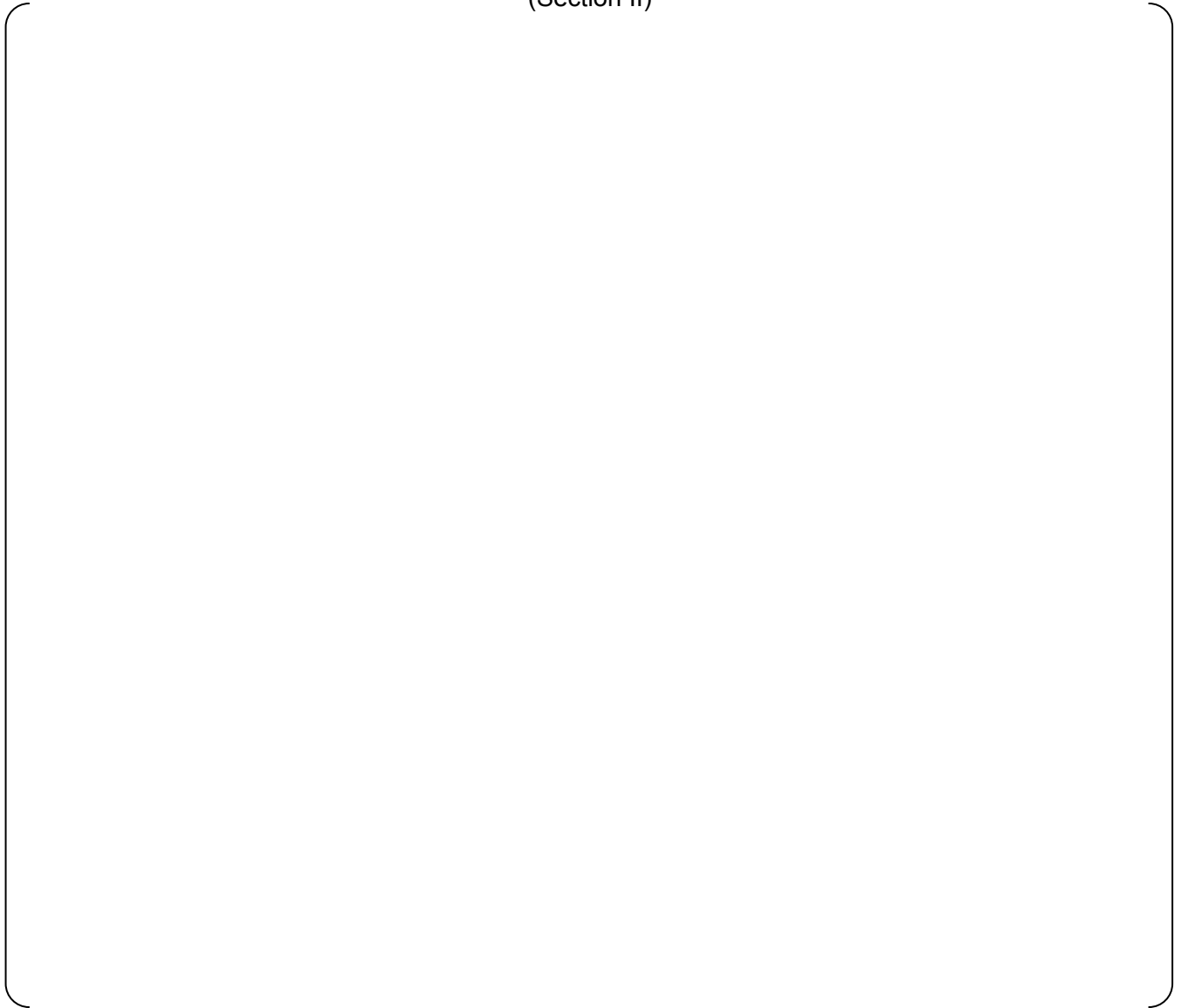


Table A1-10-1-6 Level A, B temperature and pressure input data (5/12)  
(Section II)



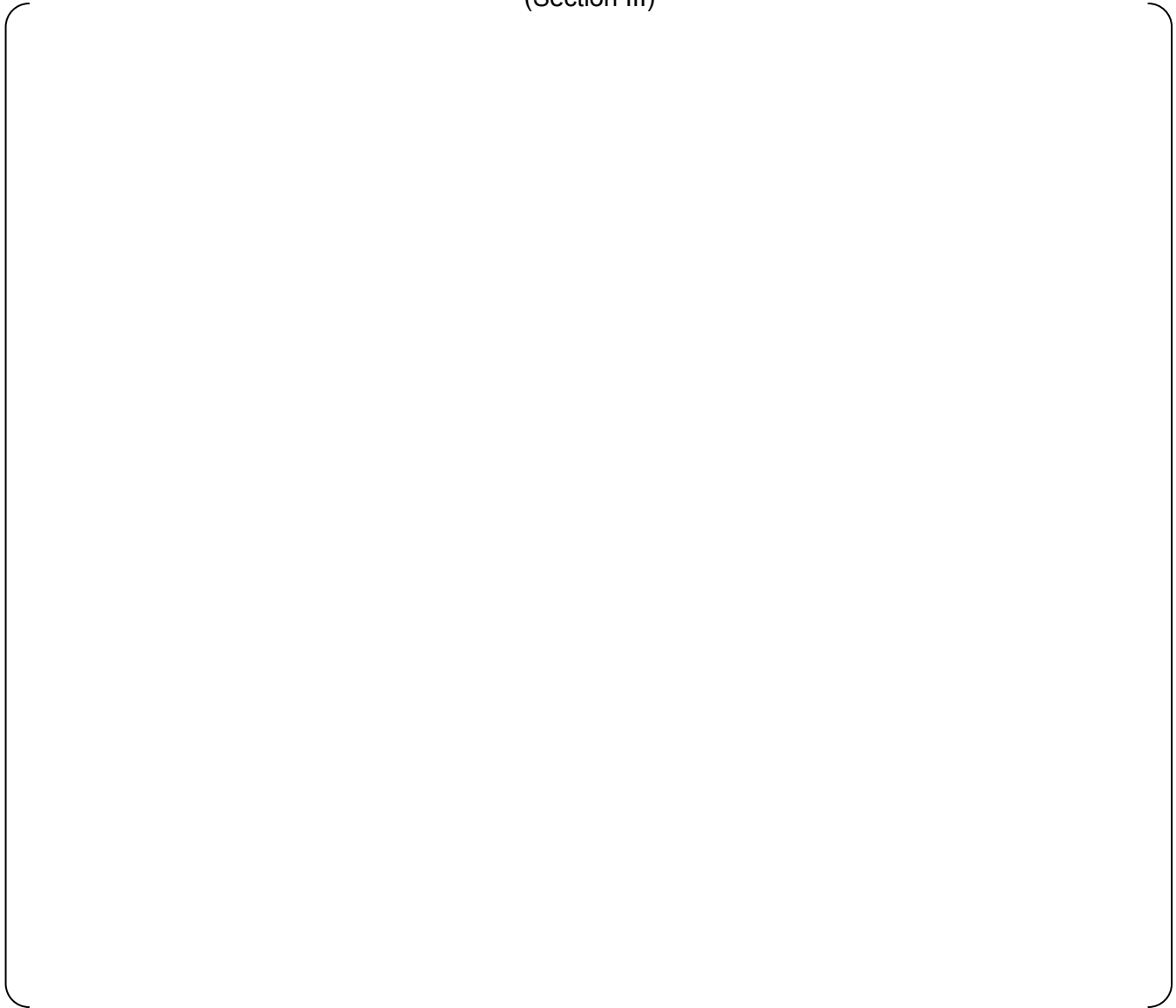
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Table A1-10-1-6 Level A, B temperature and pressure input data (6/12)  
(Section II)

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Table A1-10-1-6 Level A, B temperature and pressure input data (7/12)  
(Section III)

Table A1-10-1-6 Level A, B temperature and pressure input data (8/12)  
(Section III)



**Table A1-10-1-6 Level A, B temperature and pressure input data (9/12)  
(Section III)**

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**Table A1-10-1-6 Level A, B temperature and pressure input data (10/12)**  
(Section IV)

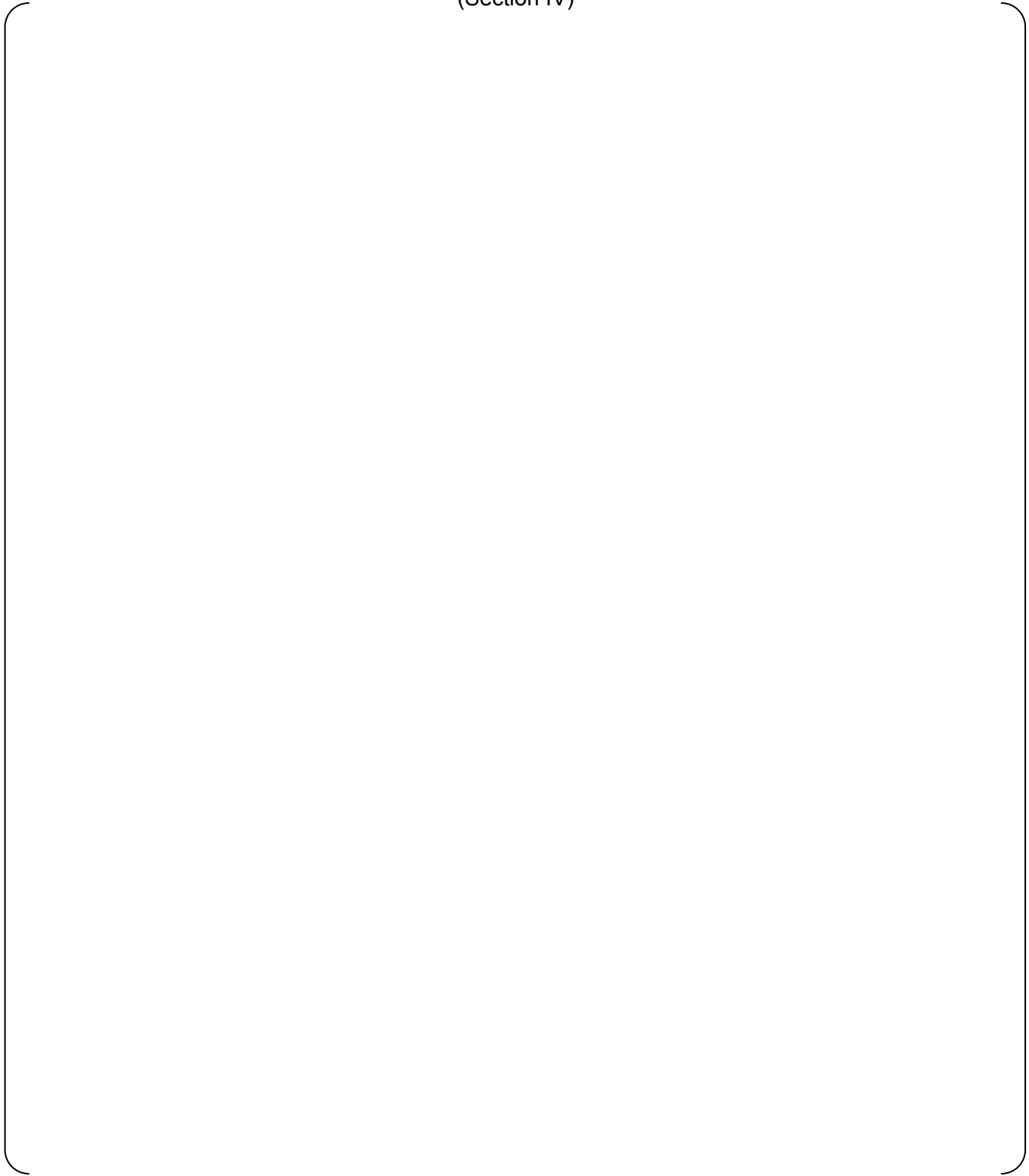
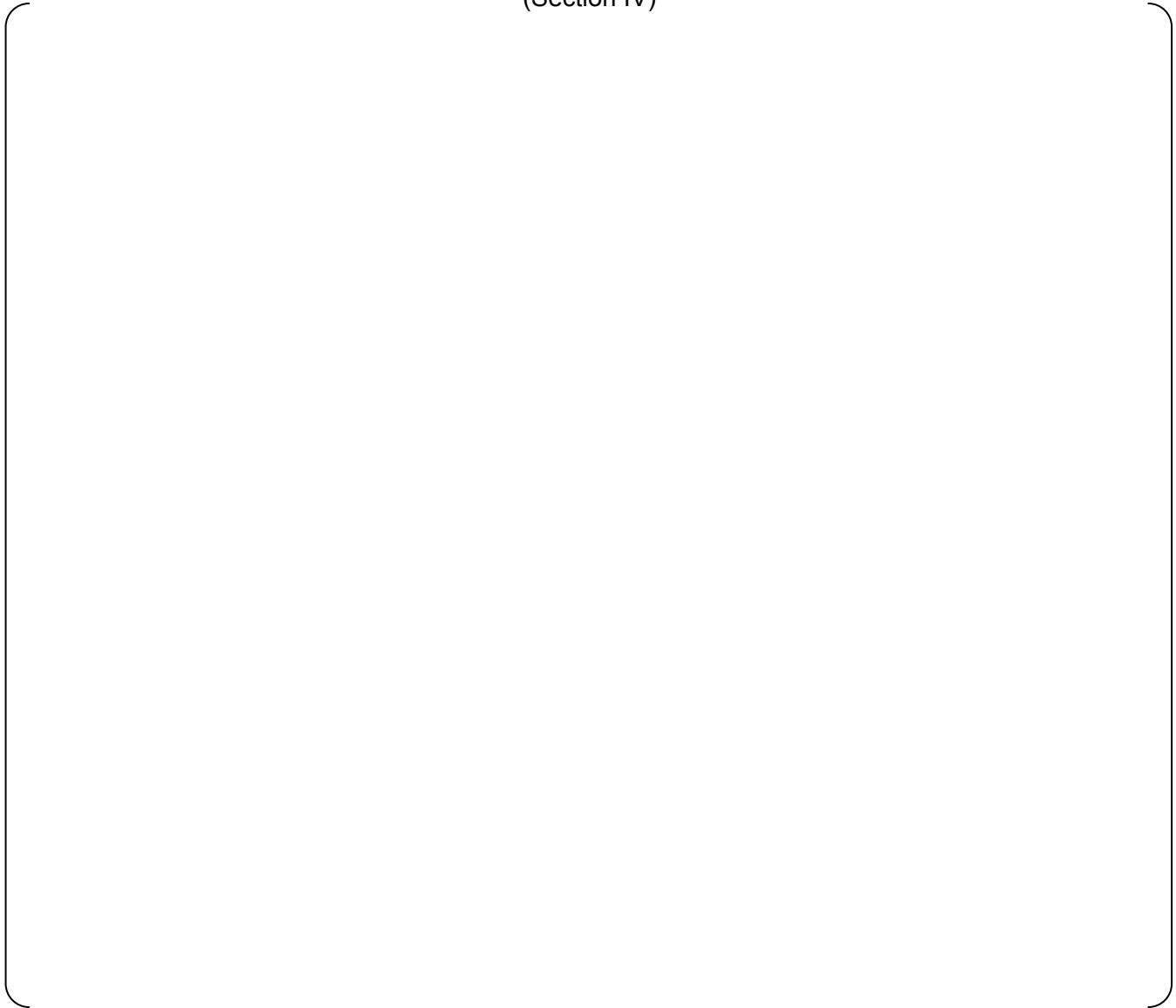




Table A1-10-1-6 Level A, B temperature and pressure input data (11/12)  
(Section IV)



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Table A1-10-1-6 Level A, B temperature and pressure input data (12/12)  
(Section IV)

Table A1-10-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-10-1-2 Floor response curve (1/6)**  
Accumulator (SI01-02) FRS for MCP Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-10-1-2 Floor response curve (2/6)**  
Accumulator (SI01-02) FRS for MCP Nozzle  
Y (NS) direction (damping 4.0%)



**Figure A1-10-1-2 Floor response curve (3/6)**  
Accumulator (SI01-02) FRS for MCP Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-10-1-2 Floor response curve (4/6)**  
Accumulator (SI01-02) FRS for Piping  
X (EW) direction (damping 4.0%)



**Figure A1-10-1-2 Floor response curve (5/6)**  
Accumulator (SI01-02) FRS for Piping  
Y (NS) direction (damping 4.0%)





**Figure A1-10-1-2 Floor response curve (6/6)**  
Accumulator (SI01-02) FRS for Piping  
Z (Vert.) direction (damping 4.0%)

Table A1-10-1-8 Seismic anchor displacement input data

Table A1-10-1-9 DBPB displacement input data

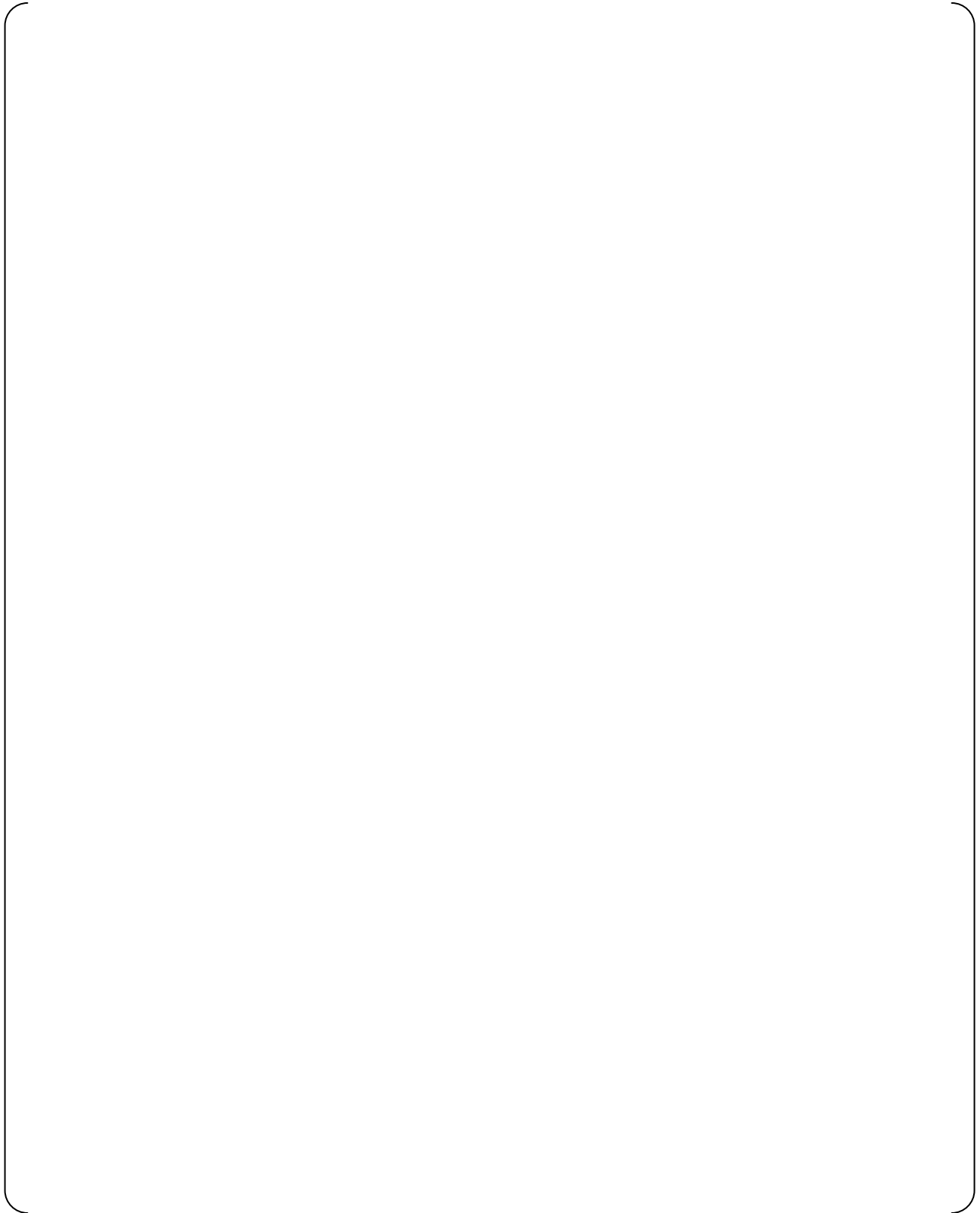


Figure A1-10-2-1 PIPESTRESS analysis model diagram

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Table A1-10-2-1 Natural frequency analysis results

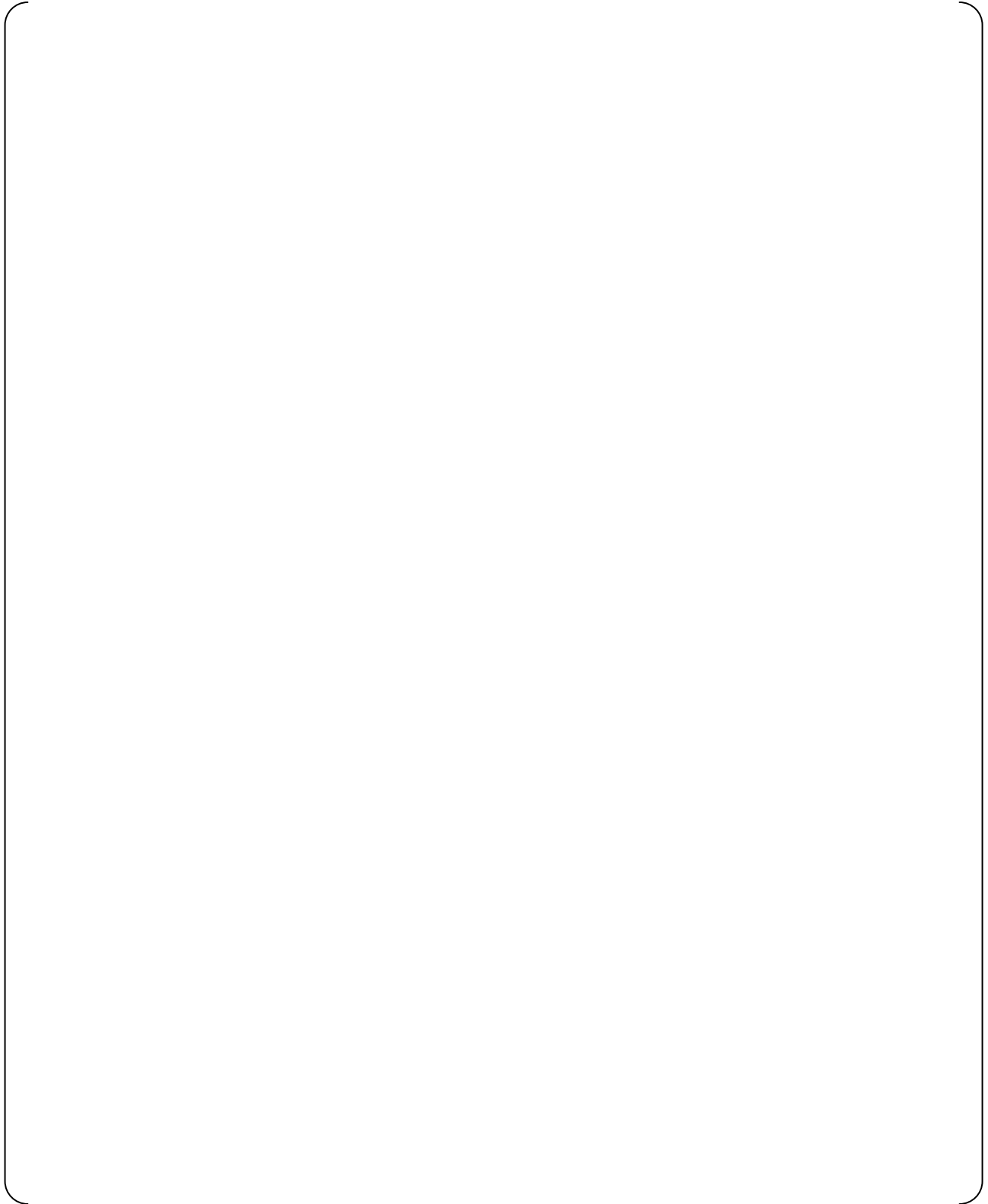


Figure A1-10-2-2 Frequency mode diagram (primary)

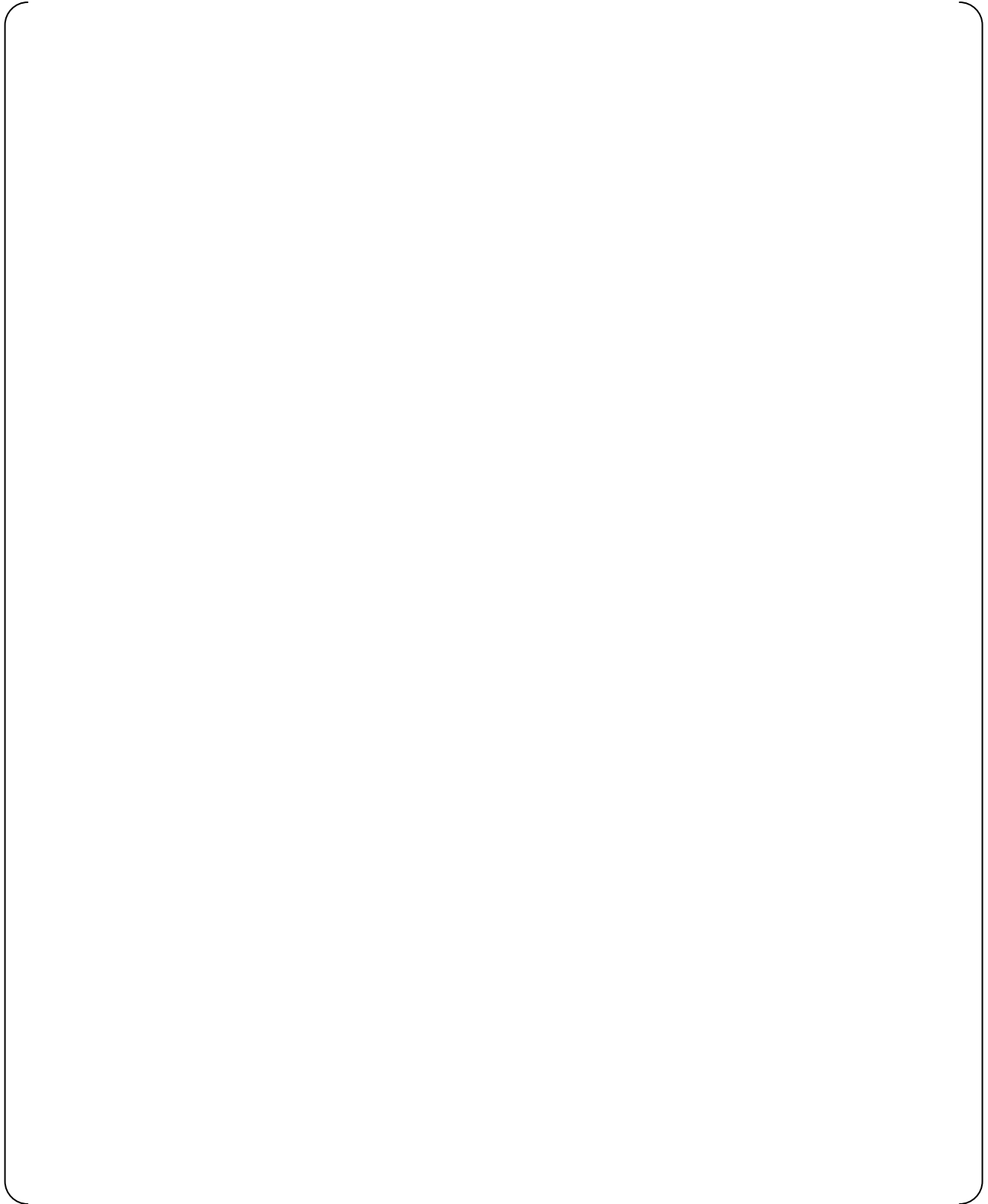


Figure A1-10-2-2 Frequency mode diagram (secondary)

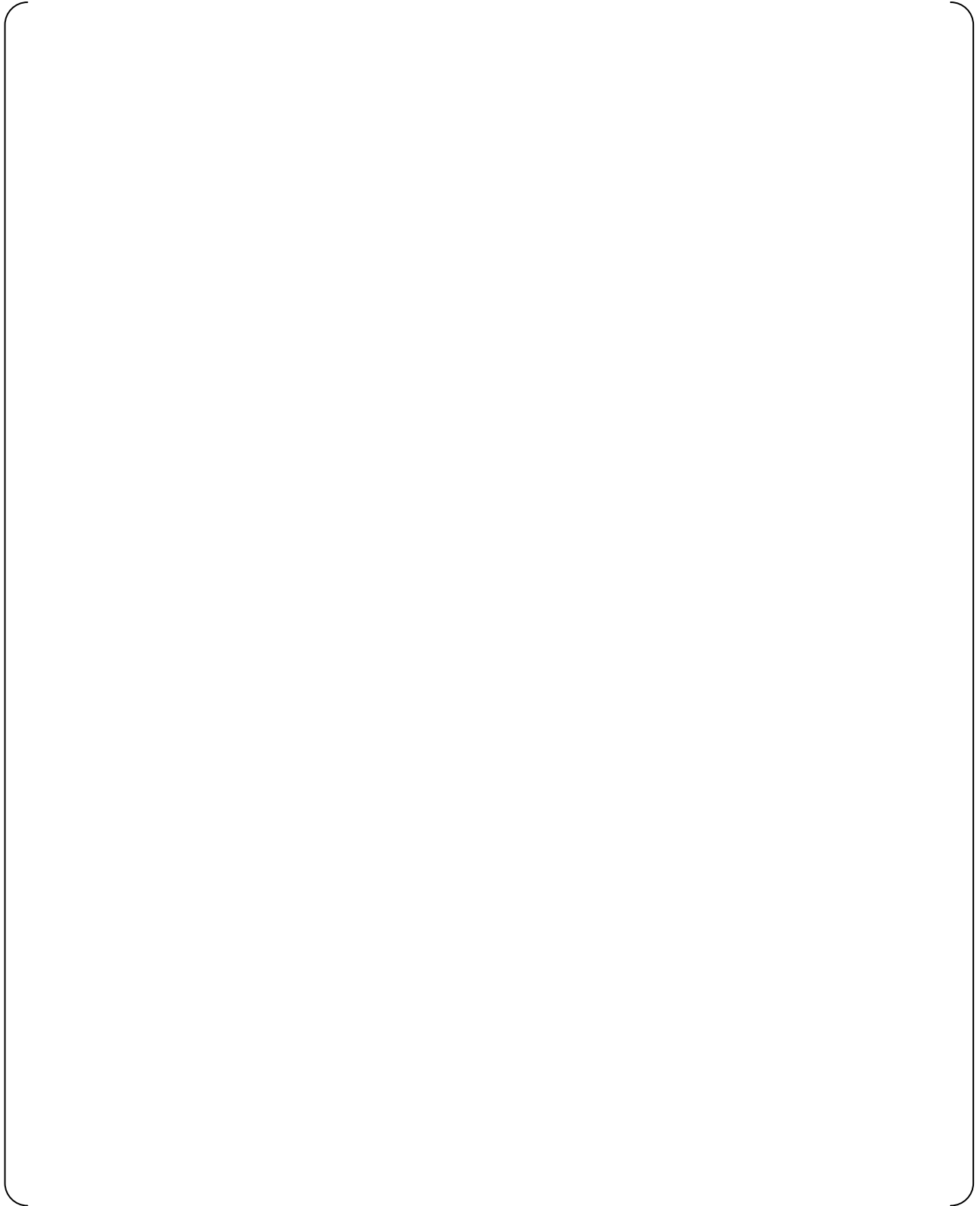
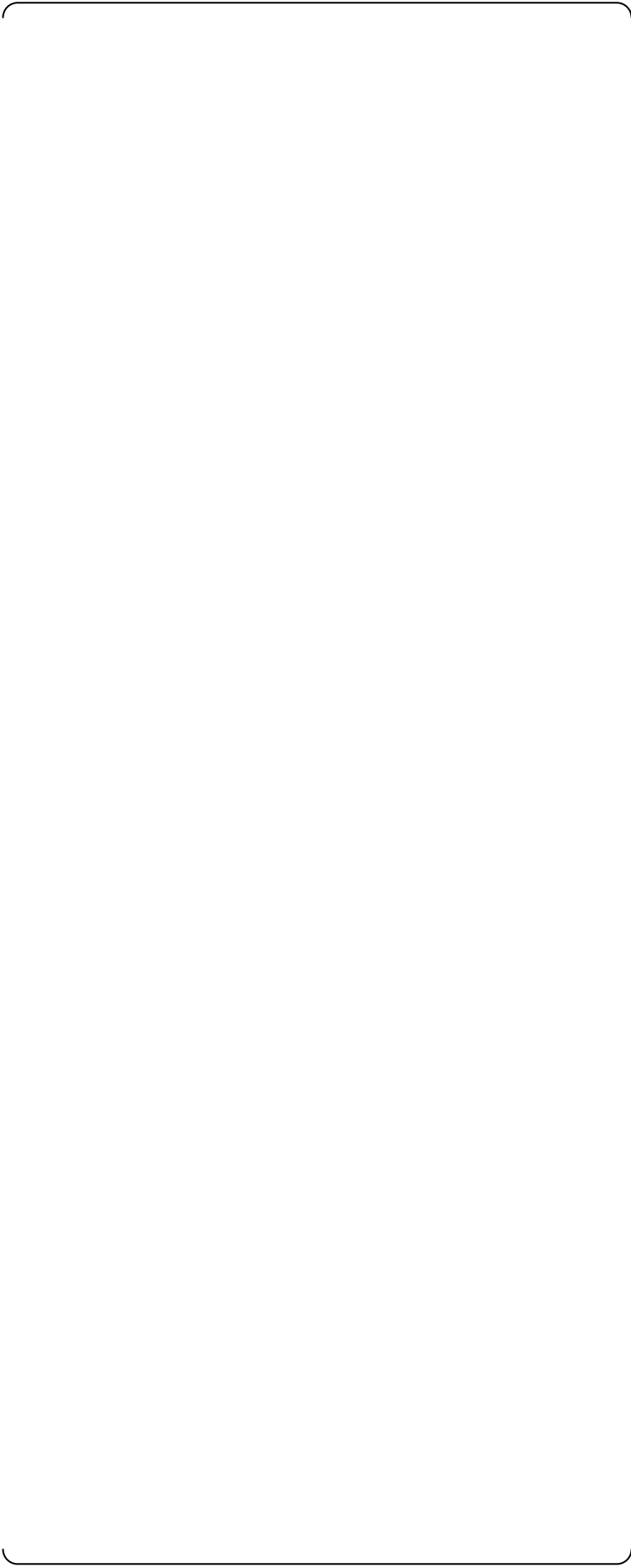


Figure A1-10-2-2 Frequency mode diagram (tertiary)



Table A1-10-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (1/6)  
(Section I)







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**

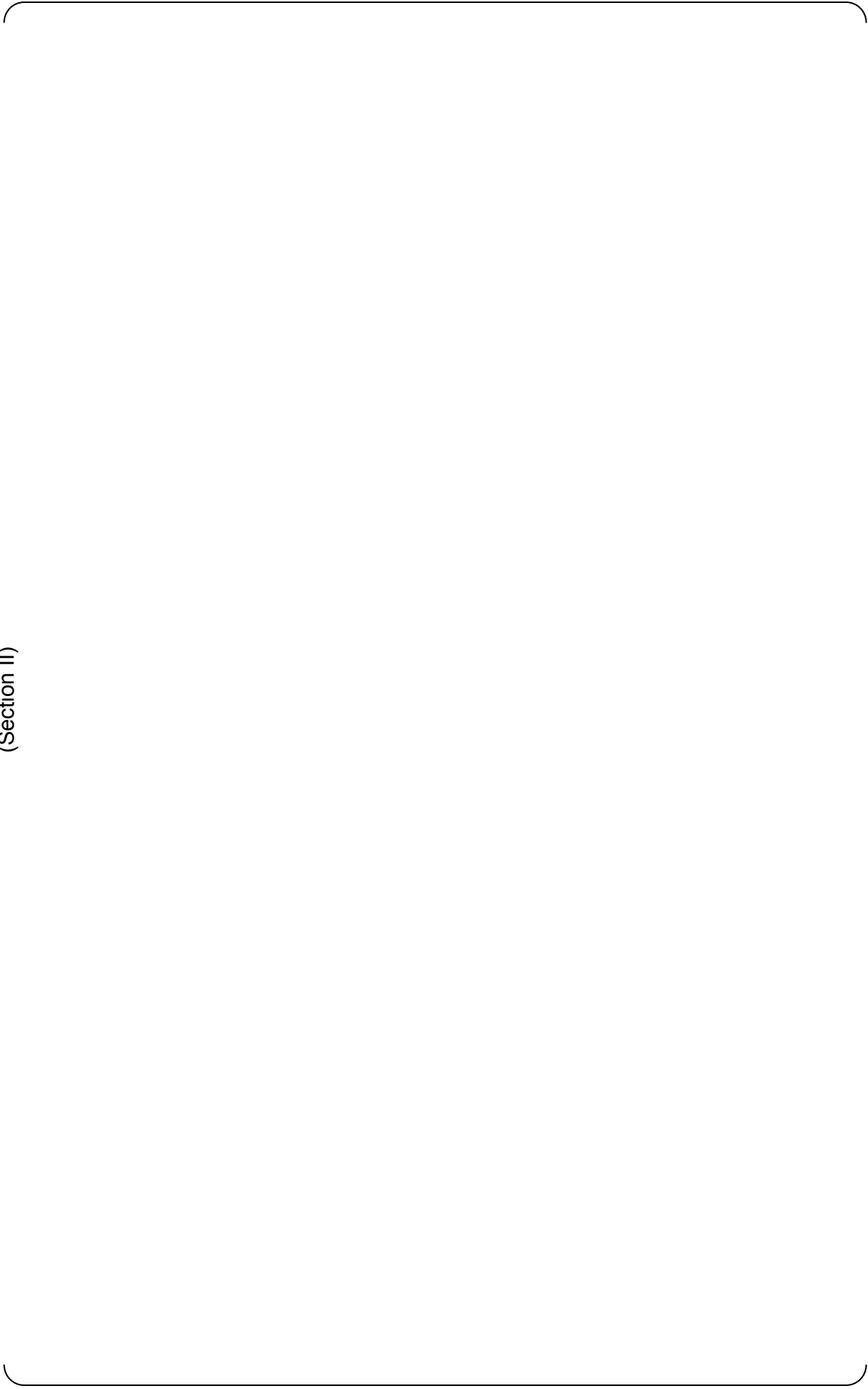


Table A1-10-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (4/6)  
(Section II)





Table A1-10-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (5/6)  
(Section II)







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



**Table A1-10-2-3 Piping stress and Fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-10-2-3 Piping stress and Fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)



Figure A1-10-2-3 LBB evaluation results

## Appendix 1-11

**SI05  
DVI A Line**

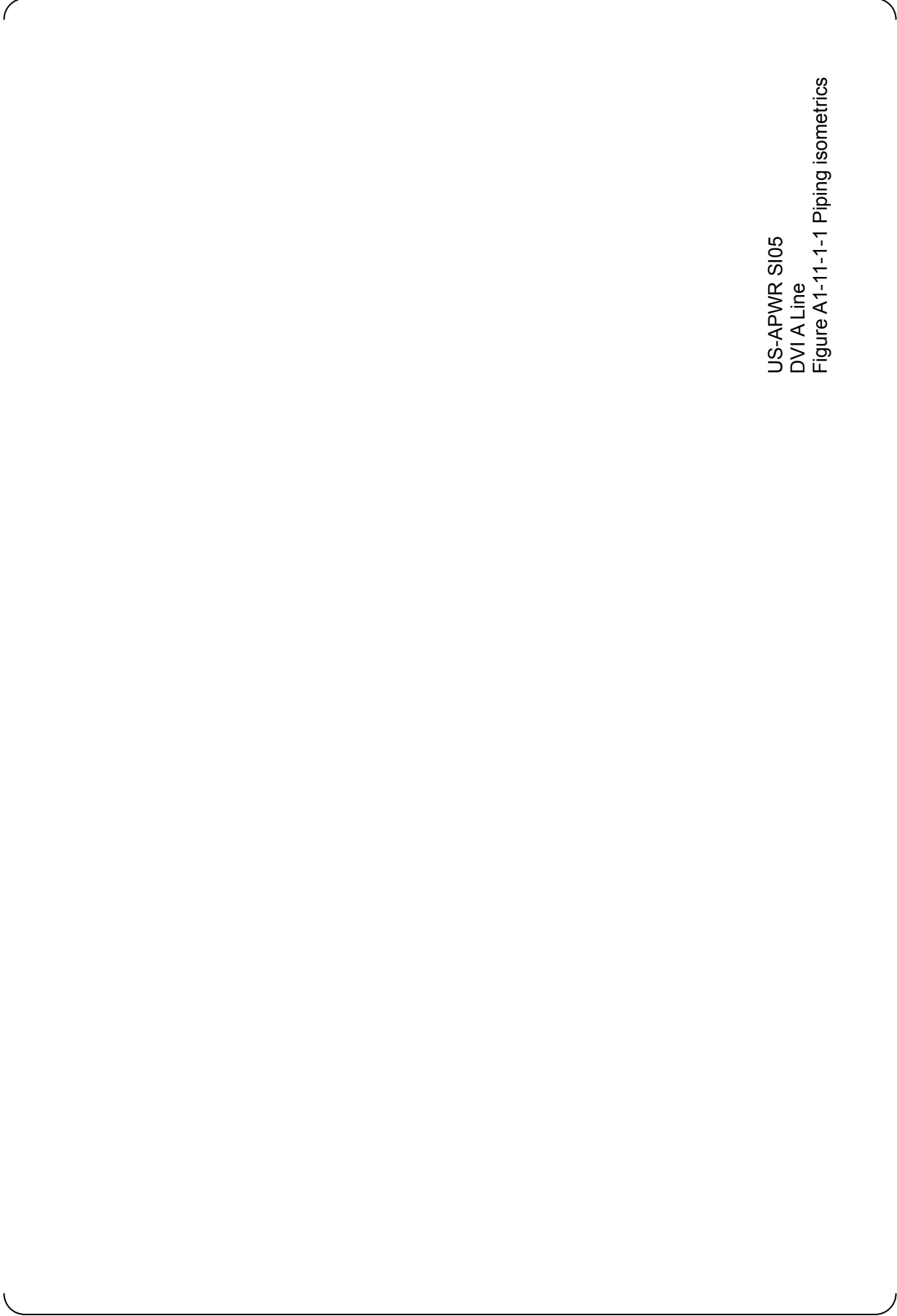
**Piping Analysis Results**

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|  |                  |
|--|------------------|
| 1. INPUT   |                  |
| 1.1 Used for creating the pipe structural model                              |                  |
| 1.1.1 Block division and piping specifications                               | Table A1-11-1-1  |
| 1.1.2 Piping isometrics  | Figure A1-11-1-1 |
| 1.1.3 Concentrated mass  | Table A1-11-1-2  |
| 1.1.4 Support point rigidity   | Table A1-11-1-3  |
| 1.1.5 Valve rigidity   | Table A1-11-1-4  |
| 1.2 Used for creating load conditions  |                  |
| 1.2.1 Level A/B design transient   | see main text    |
| 1.2.2 Level A/B thermal displacement input data                              | Table A1-11-1-5  |
| 1.2.3 Level A, B temperature and pressure input data                         | Table A1-11-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data                 | Table A1-11-1-7  |
| 1.2.5 Floor response curve   | Figure A1-11-1-2 |
| 1.2.6 Seismic anchor displacement input data                                 | Table A1-11-1-8  |
| 1.2.7 DBPB displacement input data   | Table A1-11-1-9  |
| 2. OUTPUT  |                  |
| 2.1 PIPESTRESS analysis model diagram  | Figure A1-11-2-1 |
| 2.2 Natural frequency analysis results                                       | Table A1-11-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                             | Figure A1-11-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a$ - $T_b$ ) | Table A1-11-2-2  |
| 2.5 Piping stress and fatigue evaluation results                             | Table A1-11-2-3  |

**Table A1-11-1-1 Block division and piping specifications**

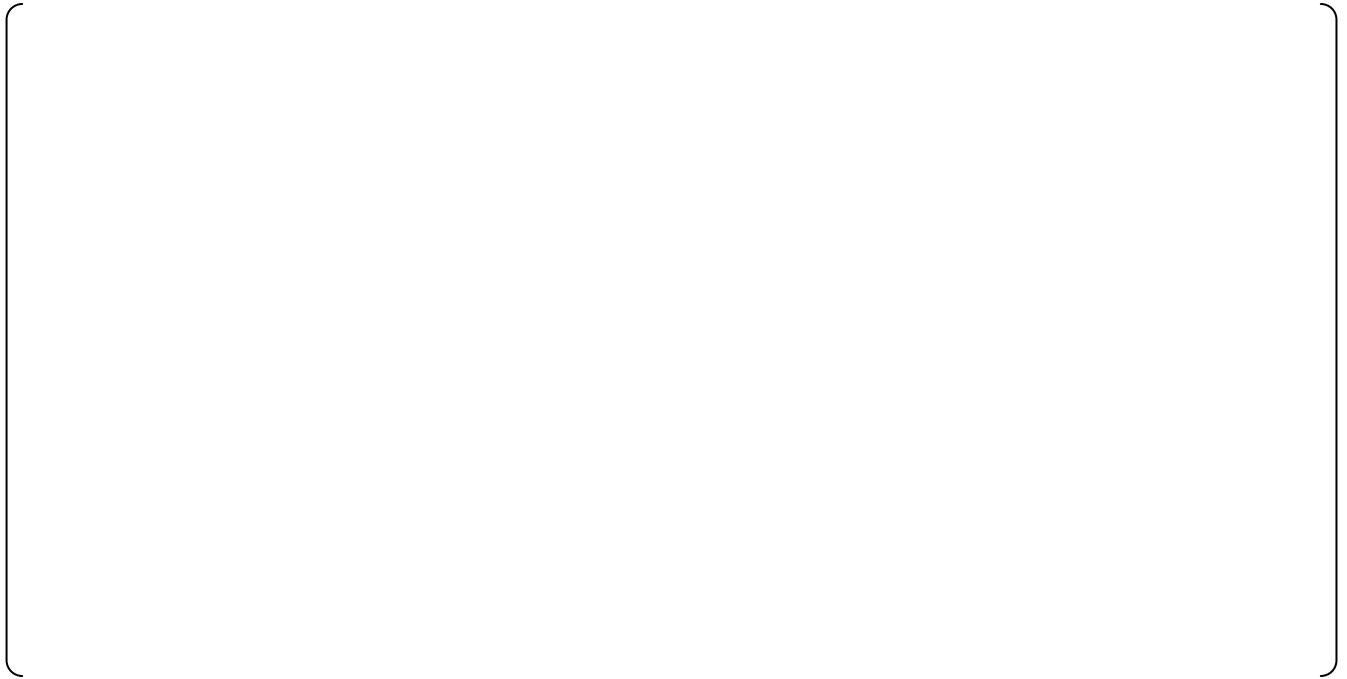




US-APWR SI05  
DVI A Line  
Figure A1-11-1-1 Piping isometrics

Table A1-11-1-2 Concentrated mass

Table A1-11-1-3 Support point rigidity



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Table A1-11-1-4 Valve rigidity

Table A1-11-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9100)

| Point | Level A | Level B |
|-------|---------|---------|
| 9100  |         |         |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-11-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9100)

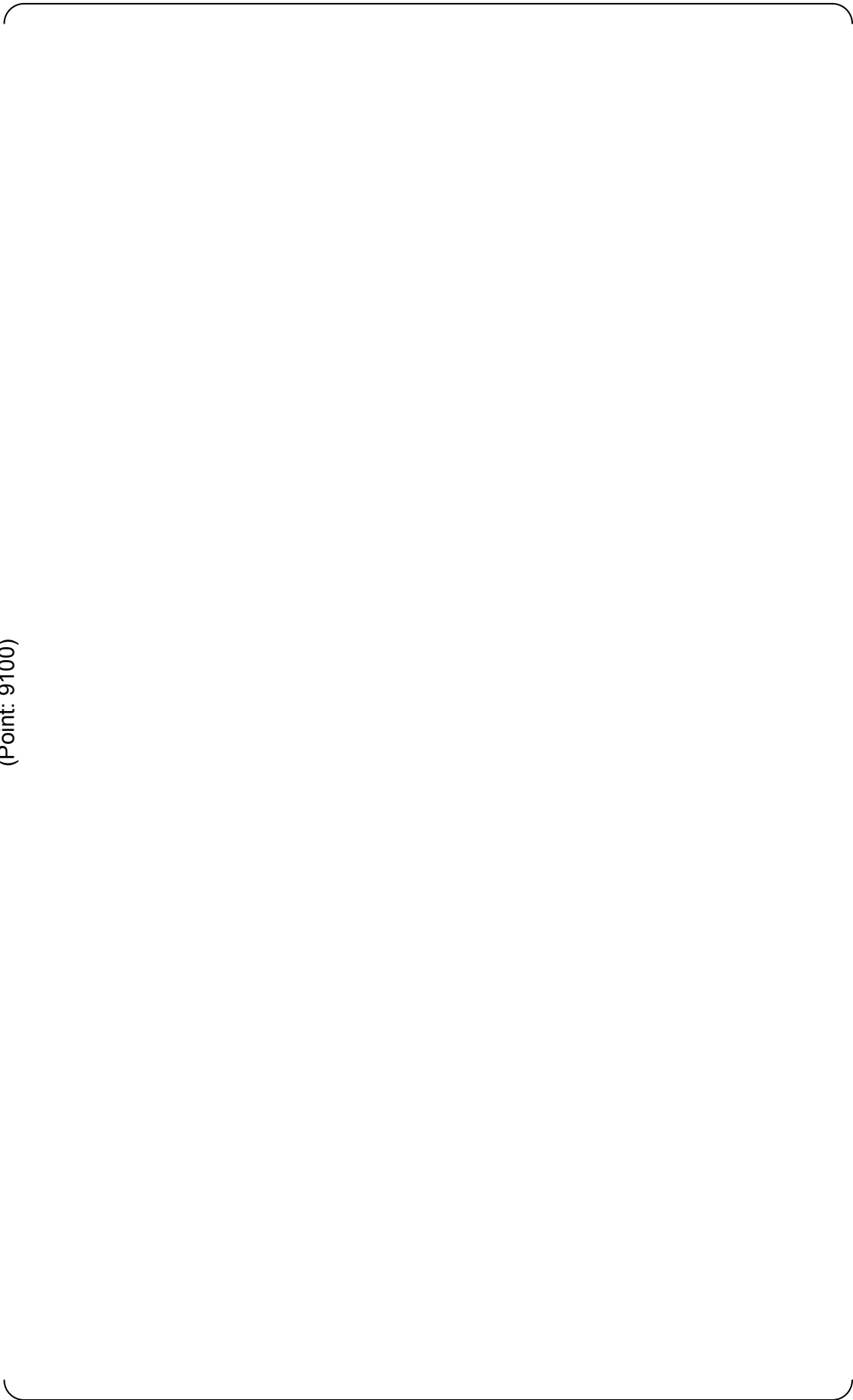




Table A1-11-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9100)

| Point | Level A | Level B |
|-------|---------|---------|
| 9100  |         |         |

Table A1-11-1-6 Level A, B temperature and pressure input data (1/9)  
(Section I)

Table A1-11-1-6 Level A, B temperature and pressure input data (2/9)  
(Section I)

Table A1-11-1-6 Level A, B temperature and pressure input data (3/9)  
(Section I)

Table A1-11-1-6 Level A, B temperature and pressure input data (4/9)  
(Section II)

Table A1-11-1-6 Level A, B temperature and pressure input data (5/9)  
(Section II)

Table A1-11-1-6 Level A, B temperature and pressure input data (6/9)  
(Section II)

Table A1-11-1-6 Level A, B temperature and pressure input data (7/9)  
(Section III)



Table A1-11-1-6 Level A, B temperature and pressure input data (8/9)  
(Section III)

Table A1-11-1-6 Level A, B temperature and pressure input data (9/9)  
(Section III)

Table A1-11-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-11-1-2 Floor response curve (1/6)**  
DVI (SI05-06) FRS for RV Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-11-1-2 Floor response curve (2/6)**  
DVI (SI05-06) FRS for RV Nozzle  
Y (NS) direction (damping 4.0%)



**Figure A1-11-1-2 Floor response curve (3/6)**  
DVI (SI05-06) FRS for RV Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-11-1-2 Floor response curve (4/6)**  
DVI (SI05-06) FRS for Piping  
X (EW) direction (damping 4.0%)



**Figure A1-11-1-2 Floor response curve (5/6)**  
DVI (SI05-06) FRS for Piping  
Y (NS) direction (damping 4.0%)





**Figure A1-11-1-2 Floor response curve (6/6)**  
DVI (SI05-06) FRS for Piping  
Z (Vert.) direction (damping 4.0%)

Table A1-11-1-8 Seismic anchor displacement input data

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Table A1-11-1-9 DBPB displacement input data

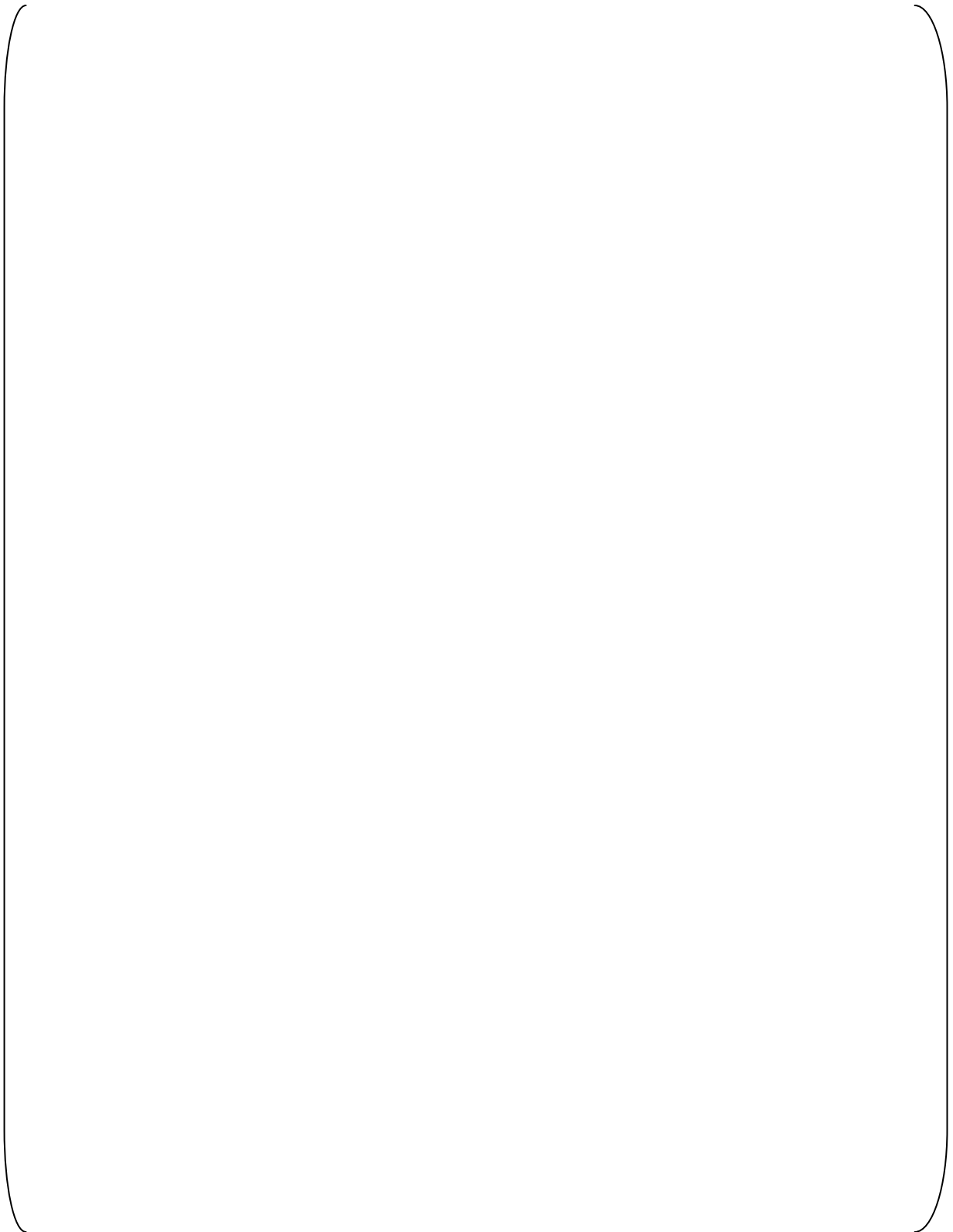


Figure A1-11-2-1 PIPESTRESS analysis model diagram

Table A1-11-2-1 Natural frequency analysis results

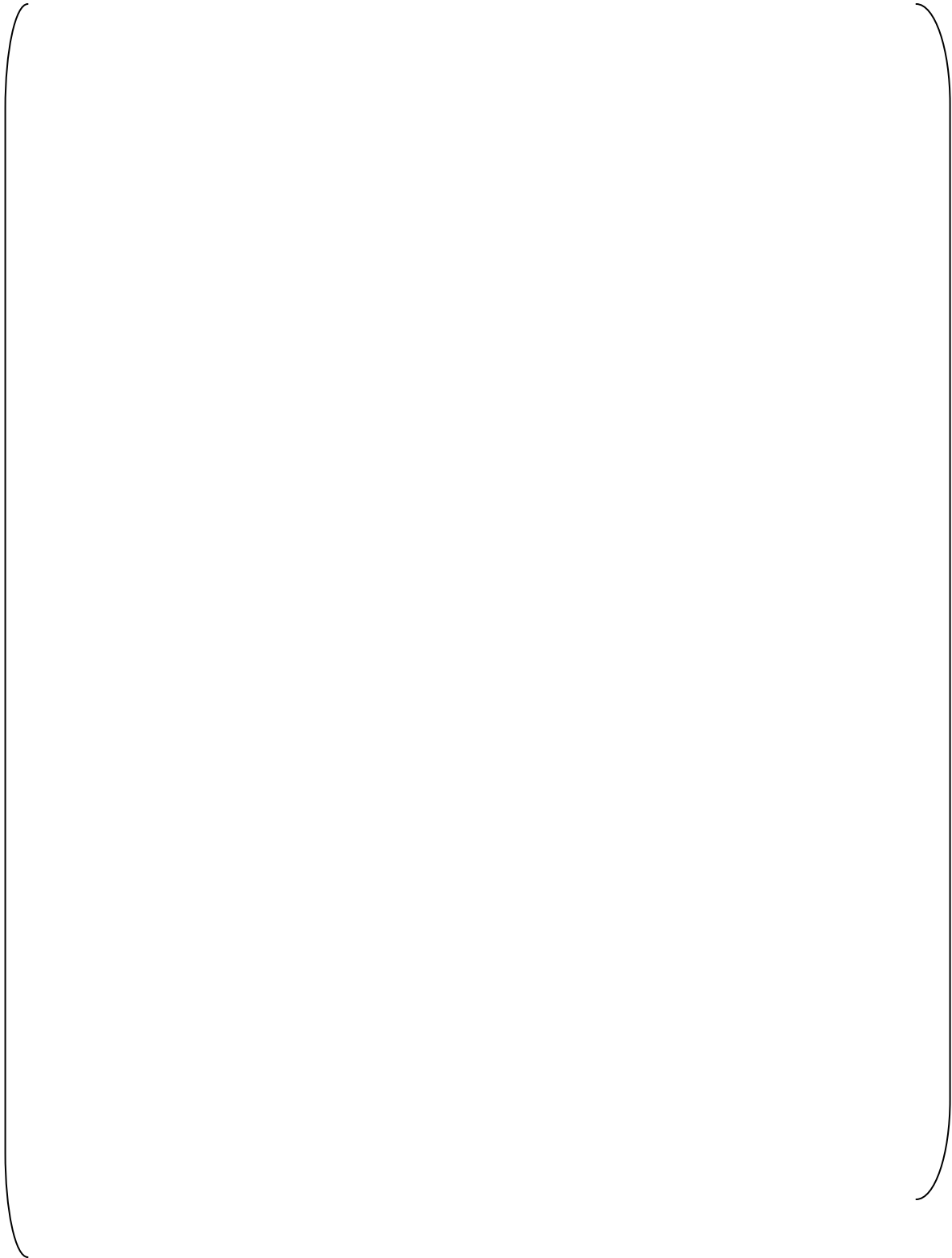


Figure A1-11-2-2 Frequency mode diagram (primary)

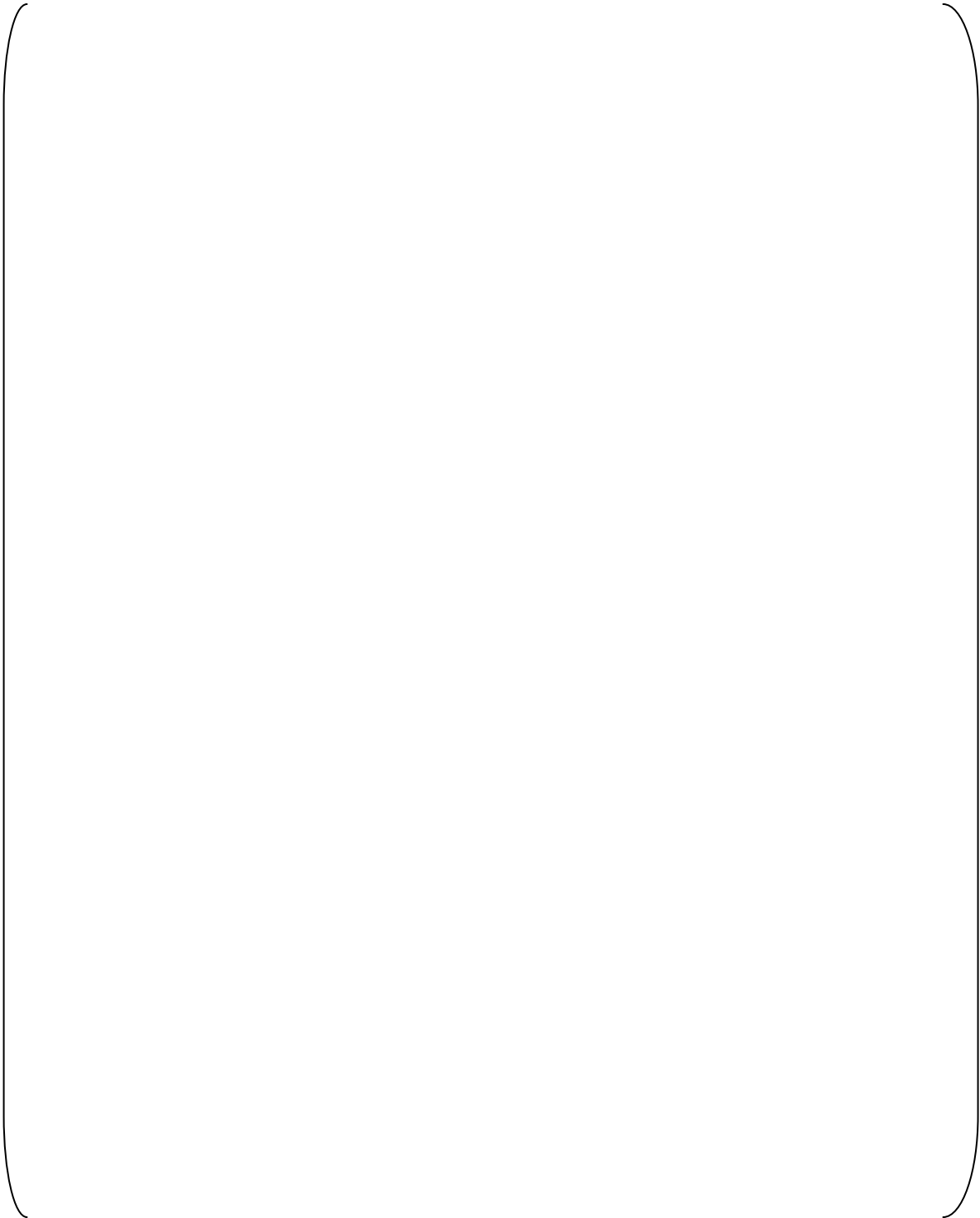


Figure A1-11-2-2 Frequency mode diagram (secondary)

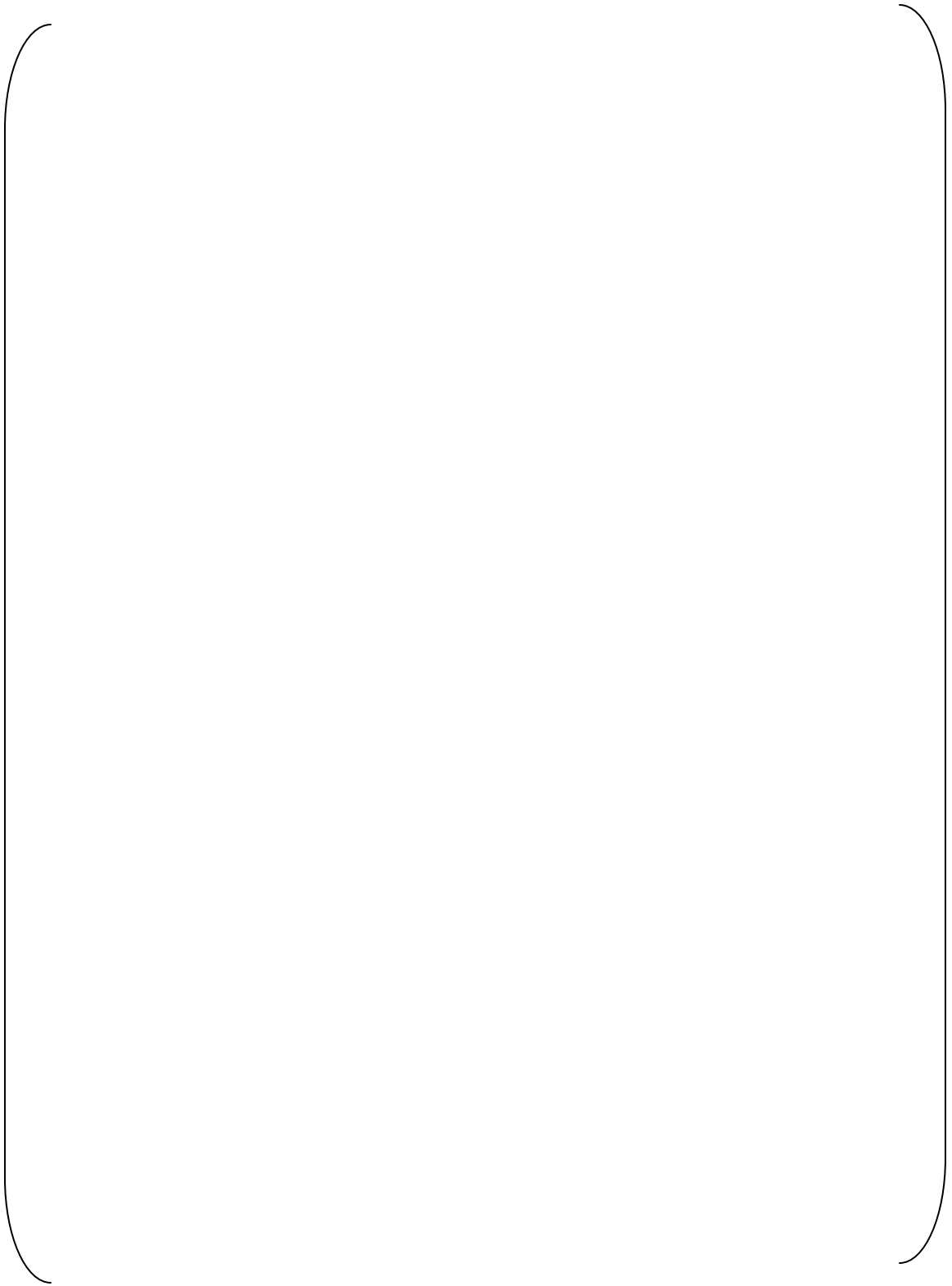


Figure A1-11-2-2 Frequency mode diagram (tertiary)



Table A1-11-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (1/6)  
(Section I)

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**

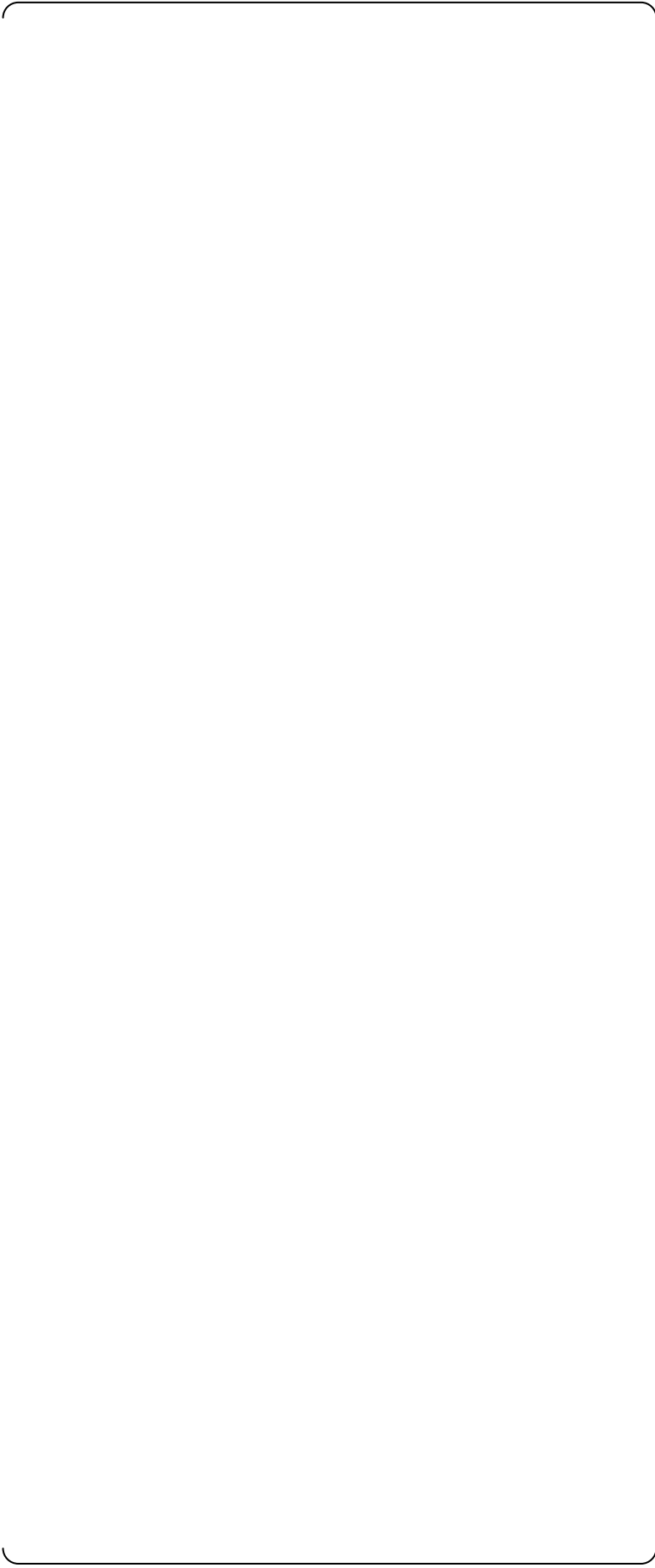


Table A1-11-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (4/6)  
(Section II)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-11-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (5/6)  
(Section II)





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



**Table A1-11-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-11-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

## Appendix 1-12

### SI06 DVI B Line

### Piping Analysis Results

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|  |                  |
|--|------------------|
| 1. INPUT   |                  |
| 1.1 Used for creating the pipe structural model                              |                  |
| 1.1.1 Block division and piping specifications                               | Table A1-12-1-1  |
| 1.1.2 Piping isometrics  | Figure A1-12-1-1 |
| 1.1.3 Concentrated mass  | Table A1-12-1-2  |
| 1.1.4 Support point rigidity   | Table A1-12-1-3  |
| 1.1.5 Valve rigidity   | Table A1-12-1-4  |
| 1.2 Used for creating load conditions  |                  |
| 1.2.1 Level A/B design transient   | see main text    |
| 1.2.2 Level A/B thermal displacement input data                              | Table A1-12-1-5  |
| 1.2.3 Level A, B temperature and pressure input data                         | Table A1-12-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data                 | Table A1-12-1-7  |
| 1.2.5 Floor response curve   | Figure A1-12-1-2 |
| 1.2.6 Seismic anchor displacement input data                                 | Table A1-12-1-8  |
| 1.2.7 DBPB displacement input data   | Table A1-12-1-9  |
| 2. OUTPUT  |                  |
| 2.1 PIPESTRESS analysis model diagram  | Figure A1-12-2-1 |
| 2.2 Natural frequency analysis results                                       | Table A1-12-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                             | Figure A1-12-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a$ - $T_b$ ) | Table A1-12-2-2  |
| 2.5 Piping stress and fatigue evaluation results                             | Table A1-12-2-3  |

Table A1-12-1-1 Block division and piping specifications



US-APWR SI06  
DVI B Line  
Figure A1-12-1-1 Piping Isometrics



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Table A1-12-1-2 Concentrated mass

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Table A1-12-1-3 Support point rigidity



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Table A1-12-1-4 Valve rigidity

Table A1-12-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9100)

| Point | Level A/B | Thermal Displacement Input Data       |
|-------|-----------|---------------------------------------|
| 9100  | Level A/B | Thermal displacement input data (1/3) |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-12-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9100)

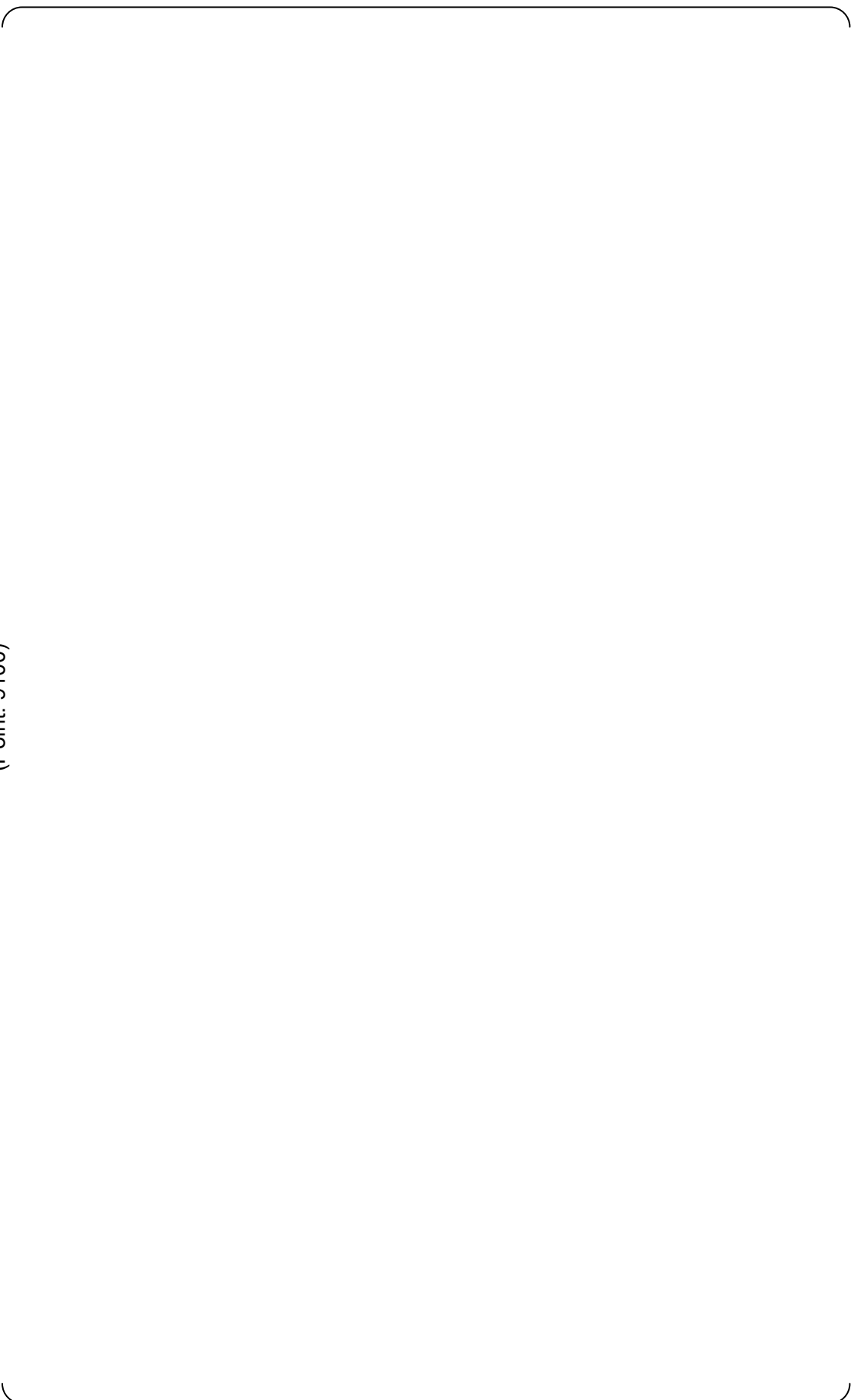


Table A1-12-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9100)

| Point | Level A | Level B |
|-------|---------|---------|
| 9100  |         |         |



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**Table A1-12-1-6 Level A, B temperature and pressure input data (1/9)**  
(Section I)

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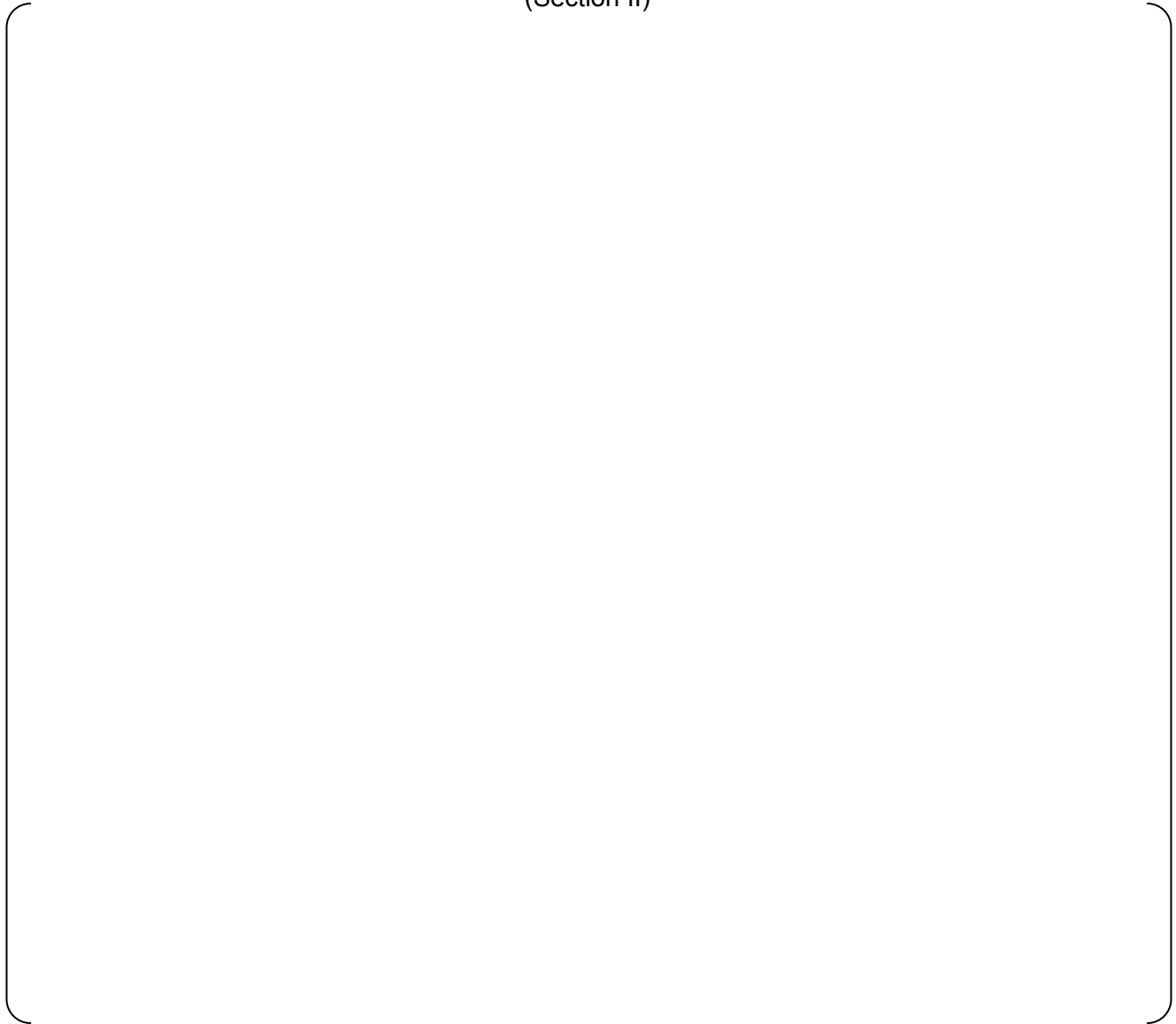
Table A1-12-1-6 Level A, B temperature and pressure input data (2/9)  
(Section I)

Table A1-12-1-6 Level A, B temperature and pressure input data (3/9)  
(Section I)

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Table A1-12-1-6 Level A, B temperature and pressure input data (4/9)  
(Section II)

Table A1-12-1-6 Level A, B temperature and pressure input data (5/9)  
(Section II)



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**Table A1-12-1-6 Level A, B temperature and pressure input data (6/9)**  
(Section II)

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Table A1-12-1-6 Level A, B temperature and pressure input data (7/9)  
(Section III)

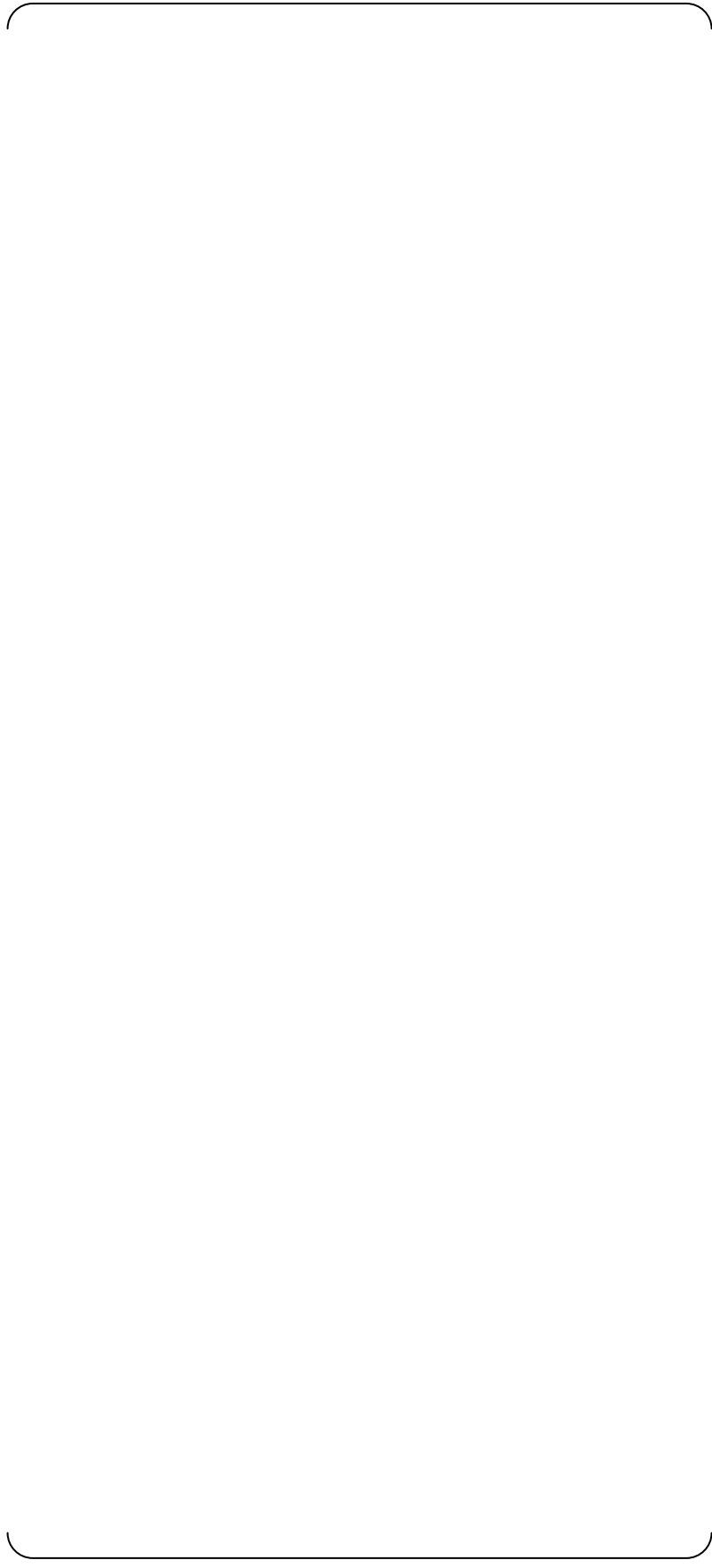
Table A1-12-1-6 Level A, B temperature and pressure input data (8/9)  
(Section III)



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**Table A1-12-1-6 Level A, B temperature and pressure input data (9/9)**  
(Section III)

Table A1-12-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-12-1-2 Floor response curve (1/6)**  
DVI (SI05-06) FRS for RV Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-12-1-2 Floor response curve (2/6)**  
DVI (SI05-06) FRS for RV Nozzle  
Y (NS) direction (damping 4.0%)



**Figure A1-12-1-2 Floor response curve (3/6)**  
DVI (SI05-06) FRS for RV Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-12-1-2 Floor response curve (4/6)**  
DVI (SI05-06) FRS for Piping  
X (EW) direction (damping 4.0%)



**Figure A1-12-1-2 Floor response curve (5/6)**  
DVI (SI05-06) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-12-1-2 Floor response curve (6/6)**  
DVI (SI05-06) FRS for Piping  
Z (Vert.) direction (damping 4.0%)



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Table A1-12-1-8 Seismic anchor displacement input data

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Table A1-12-1-9 DBPB displacement input data

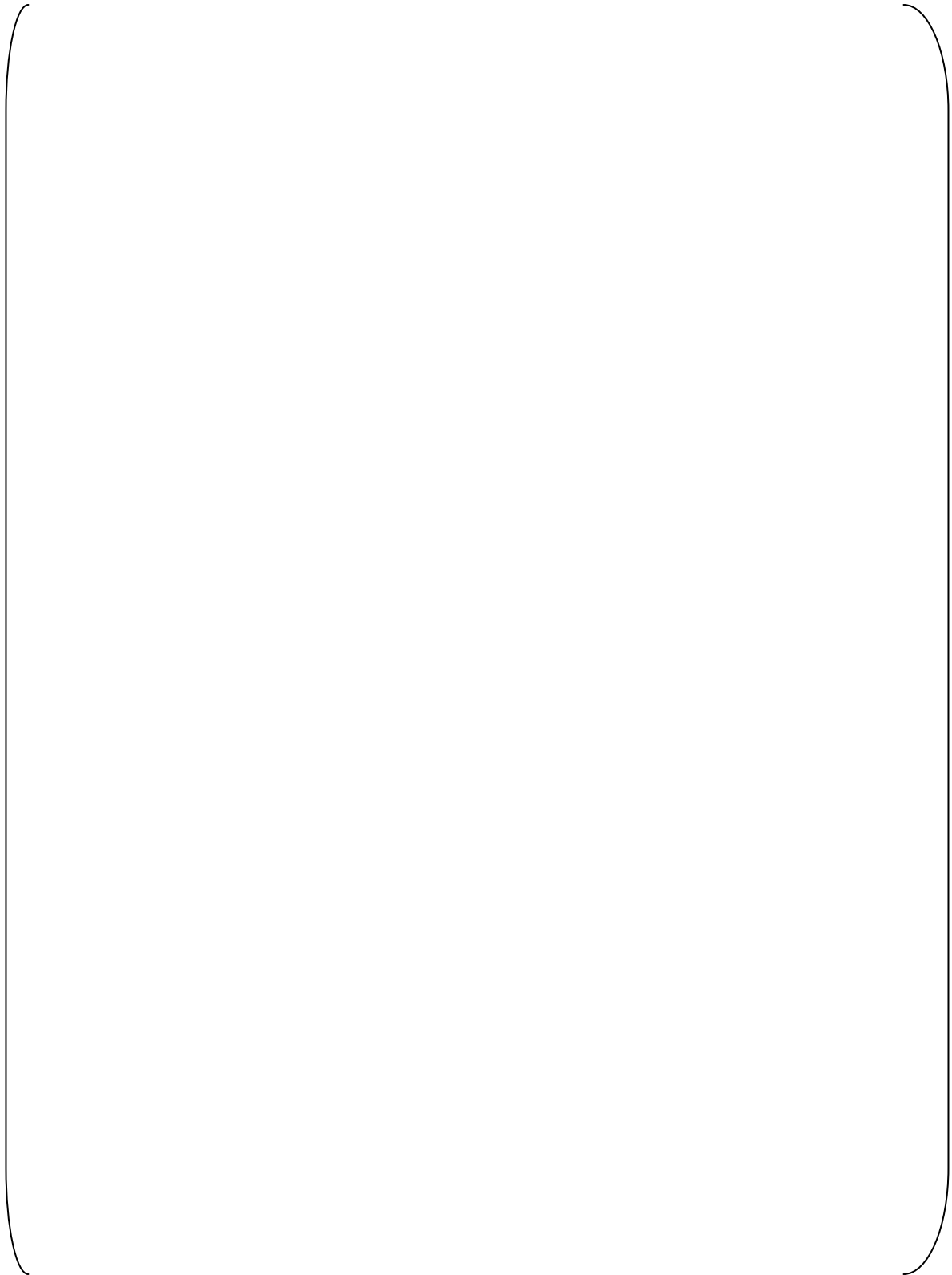
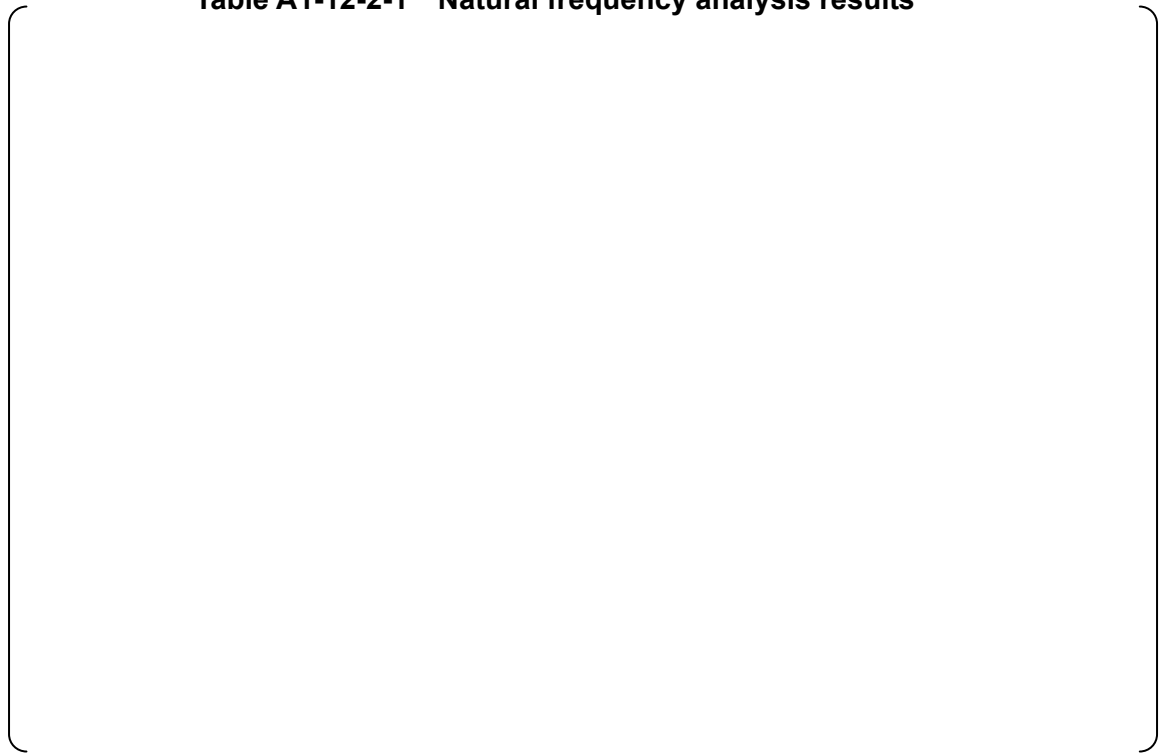


Figure A1-12-2-1 PIPESTRESS analysis model diagram

Table A1-12-2-1 Natural frequency analysis results



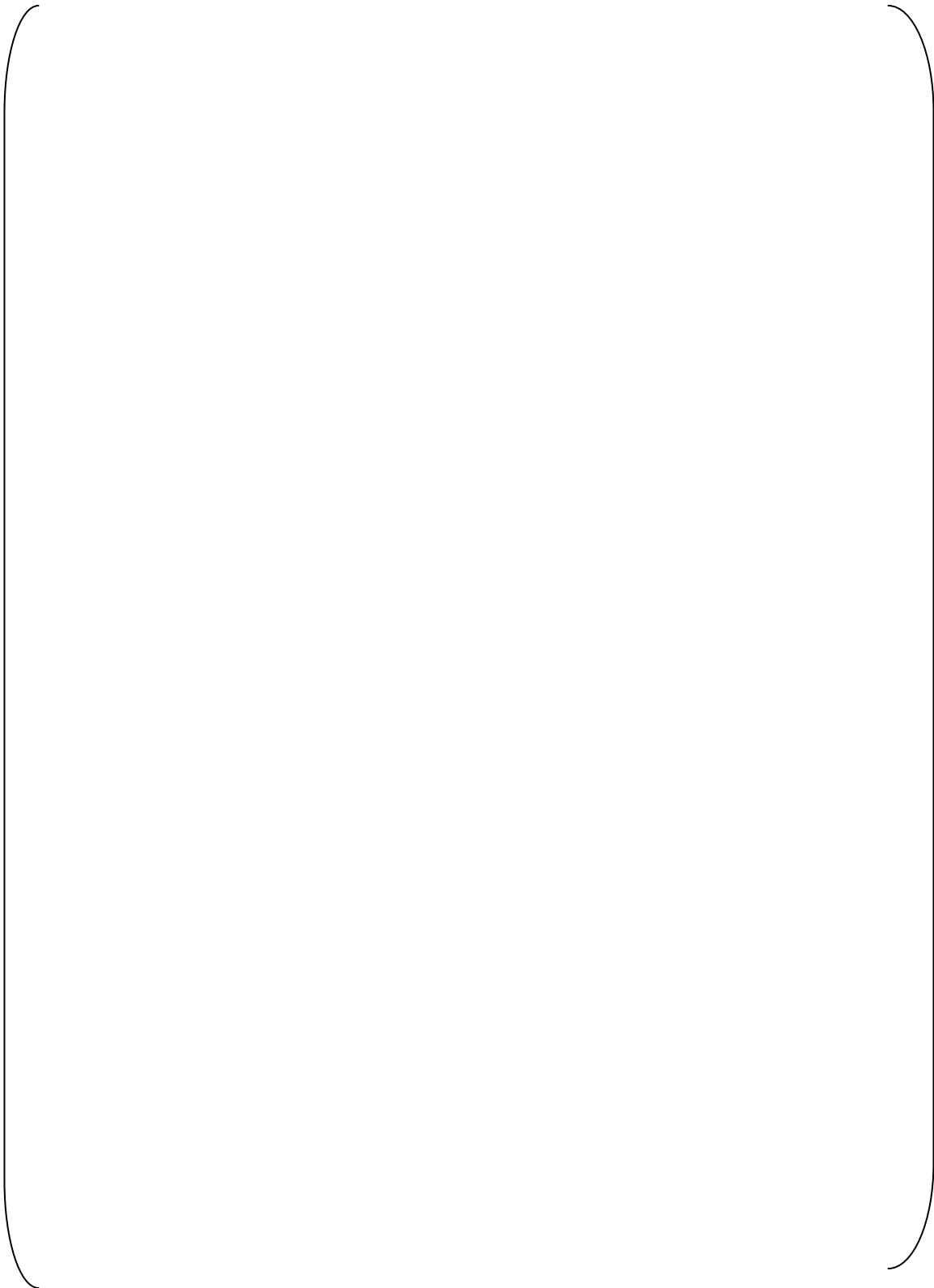


Figure A1-12-2-2 Frequency mode diagram (primary)

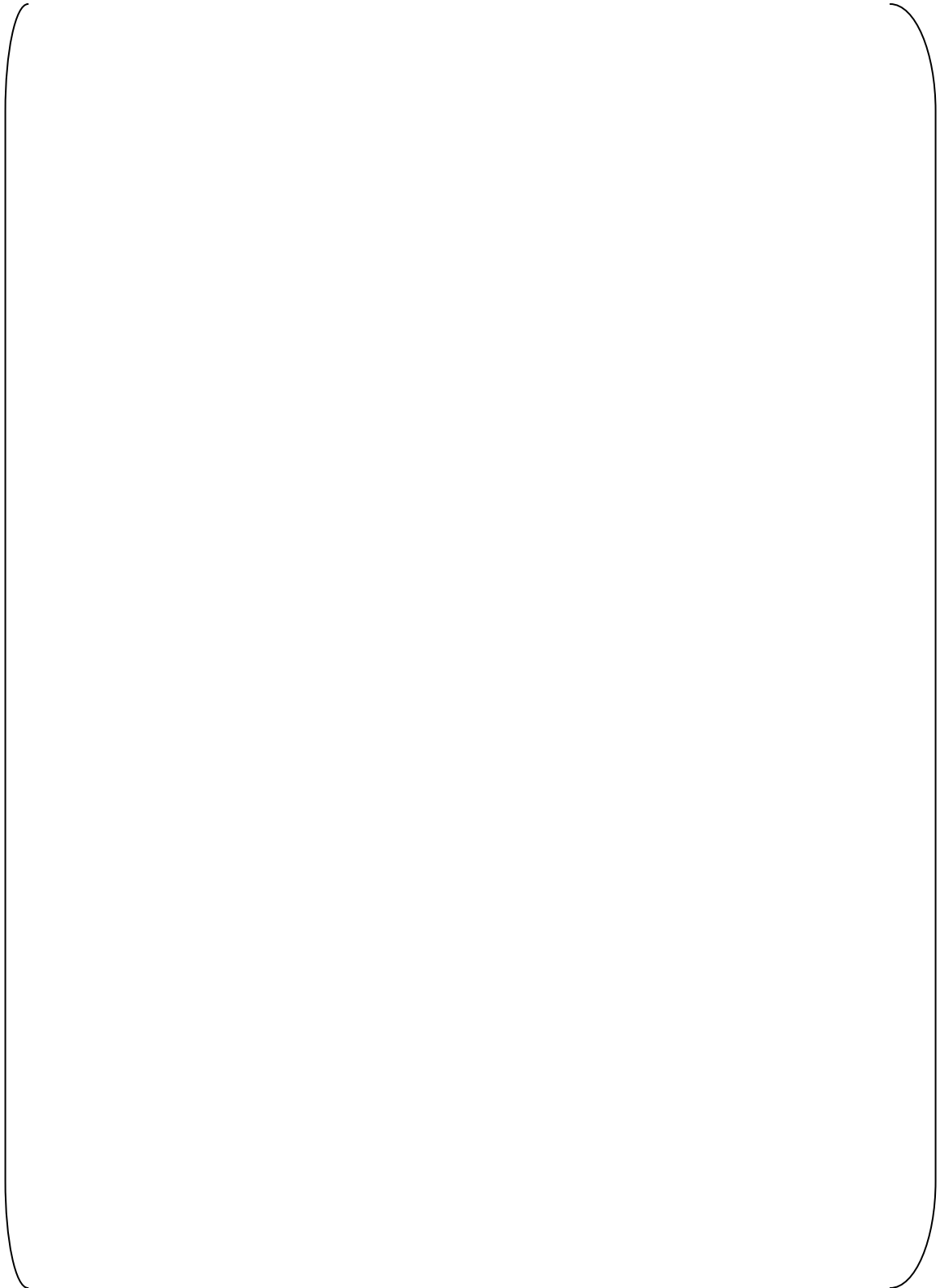


Figure A1-12-2-2 Frequency mode diagram (secondary)

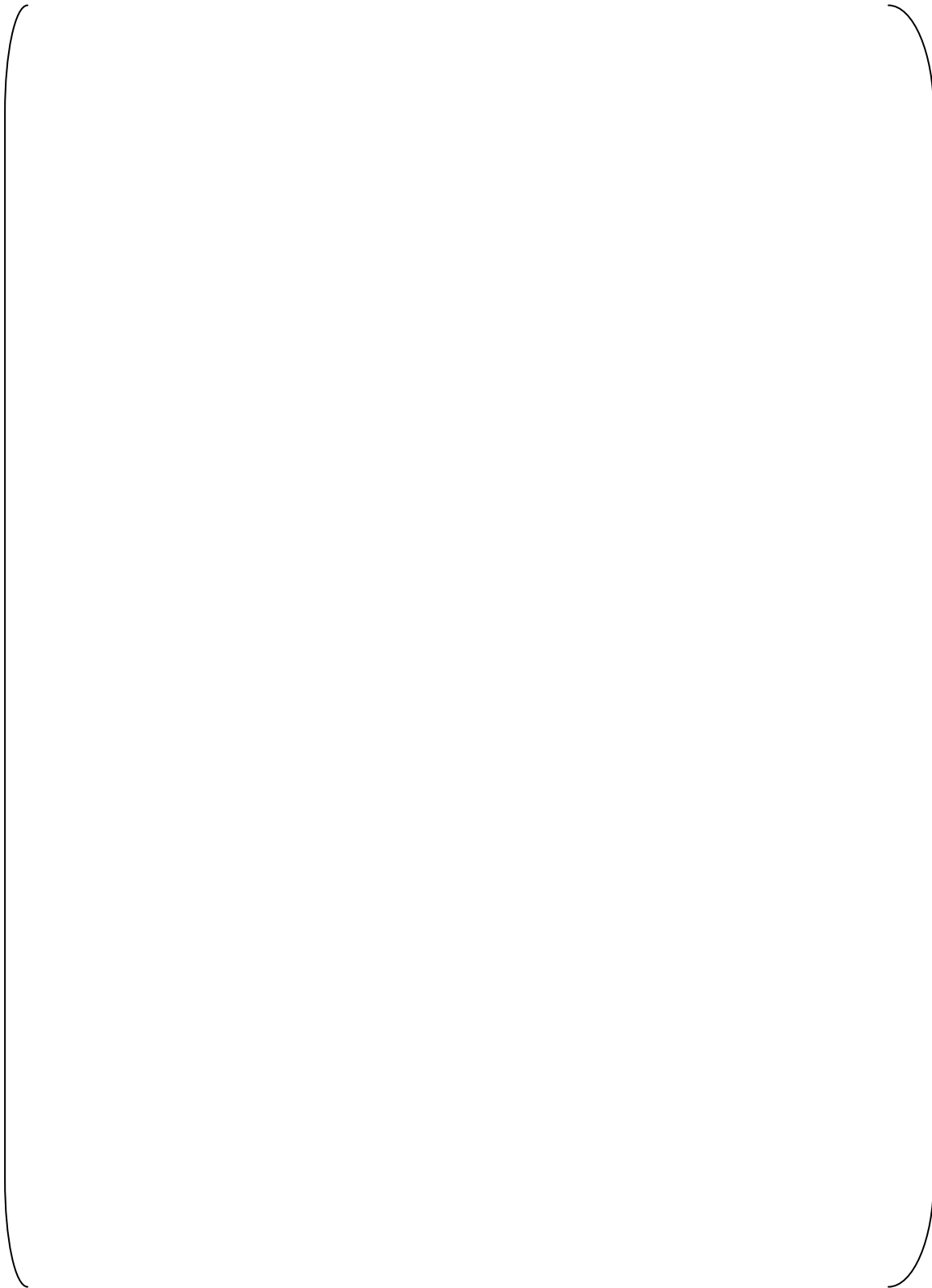


Figure A1-12-2-2 Frequency mode diagram (tertiary)





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-12-2-2 Thermal analysis results ( $\Delta T_1$ ,  $\Delta T_2$ ,  $T_a$ - $T_b$ ) (3/6)  
(Section I)





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**











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**Table A1-12-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-12-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

## Appendix 1-13

### CS01 CVCS Charging Line Piping Analysis Results

1. INPUT

|  |                  |
|--|------------------|
| 1.1 Used for creating the pipe structural model              |                  |
| 1.1.1 Block division and piping specifications               | Table A1-13-1-1  |
| 1.1.2 Piping isometrics                                      | Figure A1-13-1-1 |
| 1.1.3 Concentrated mass                                      | Table A1-13-1-2  |
| 1.1.4 Support point rigidity                                 | Table A1-13-1-3  |
| 1.1.5 Valve rigidity   | Table A1-13-1-4  |
| 1.2 Used for creating load conditions                        |                  |
| 1.2.1 Level A/B design transient                             | see main text    |
| 1.2.2 Level A/B thermal displacement input data              | Table A1-13-1-5  |
| 1.2.3 Level A, B temperature and pressure input data         | Table A1-13-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data | Table A1-13-1-7  |
| 1.2.5 Floor response curve                                   | Figure A1-13-1-2 |
| 1.2.6 Seismic anchor displacement input data                 | Table A1-13-1-8  |
| 1.2.7 DBPB displacement input data                           | Table A1-13-1-9  |

2. OUTPUT

|  |                  |
|--|------------------|
| 2.1 PIPESTRESS analysis model diagram  | Figure A1-13-2-1 |
| 2.2 Natural frequency analysis results                                       | Table A1-13-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                             | Figure A1-13-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a$ - $T_b$ ) | Table A1-13-2-2  |
| 2.5 Piping stress and fatigue evaluation results                             | Table A1-13-2-3  |

Table A1-13-1-1 Block division and piping specifications

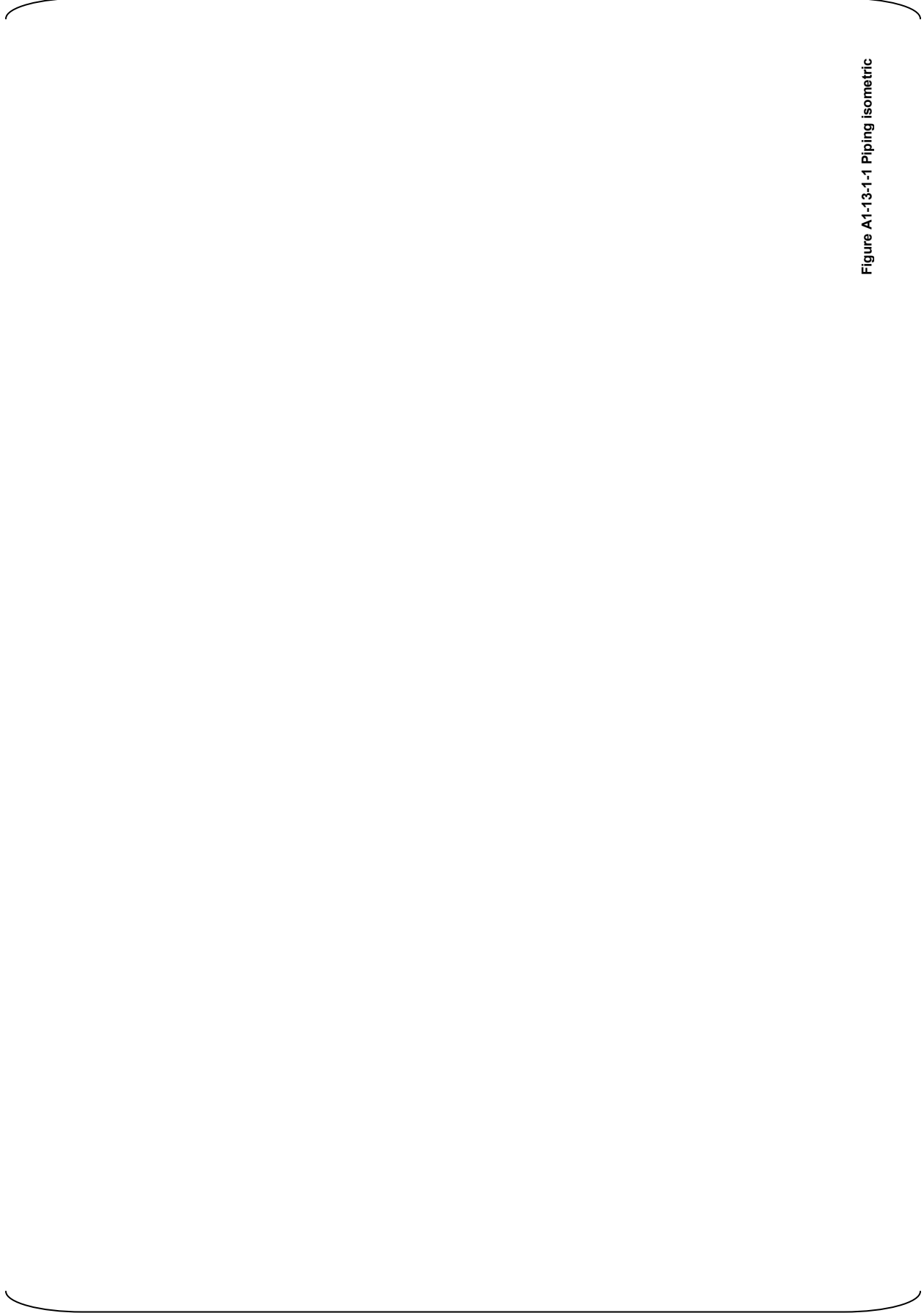


Figure A1-13-1-1 Piping isometric

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Table A1-13-1-2 Concentrated mass



Table A1-13-1-3 Support point rigidity





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Table A1-13-1-4 Valve rigidity

Table A1-13-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-13-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9010)

|  |  |
|--|--|
|  |  |
|--|--|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-13-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9010)

|  |
|--|
|  |
|--|

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Table A1-13-1-6 Level A, B temperature and pressure input data (1/12)  
(Section I)

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Table A1-13-1-6 Level A, B temperature and pressure input data (2/12)  
(Section I)



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Table A1-13-1-6 Level A, B temperature and pressure input data (3/12)  
(Section I)

Table A1-13-1-6 Level A, B temperature and pressure input data (4/12)  
(Section II)

Table A1-13-1-6 Level A, B temperature and pressure input data (5/12)  
(Section II)





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Table A1-13-1-6 Level A, B temperature and pressure input data (6/12)  
(Section II)

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Table A1-13-1-6 Level A, B temperature and pressure input data (7/12)  
(Section III)

---

Table A1-13-1-6 Level A, B temperature and pressure input data (8/12)  
(Section III)



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Table A1-13-1-6 Level A, B temperature and pressure input data (9/12)  
(Section III)

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Table A1-13-1-6 Level A, B temperature and pressure input data (10/12)  
(Section IV)

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Table A1-13-1-6 Level A, B temperature and pressure input data (11/12)  
(Section IV)





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**Table A1-13-1-6 Level A, B temperature and pressure input data (12/12)**  
(Section IV)

Table A1-13-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-13-1-2 Floor response curve (1/6)**  
CVCS Charging (CS01) FRS for MCP Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-13-1-2 Floor response curve (2/6)**  
CVCS Charging (CS01) FRS for MCP Nozzle  
Y (NS) direction (damping 4.0%)



**Figure A1-13-1-2 Floor response curve (3/6)**  
CVCS Charging (CS01) FRS for MCP Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-13-1-2 Floor response curve (4/6)**  
CVCS Charging (CS01) FRS for Piping  
X (EW) direction (damping 4.0%)



**Figure A1-13-1-2 Floor response curve (5/6)**  
CVCS Charging (CS01) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-13-1-2 Floor response curve (6/6)**  
CVCS Charging (CS01) FRS for Piping  
Z (Vert.) direction (damping 4.0%)



Table A1-13-1-8 Seismic anchor displacement input data



Table A1-13-1-9 DBPB displacement input data

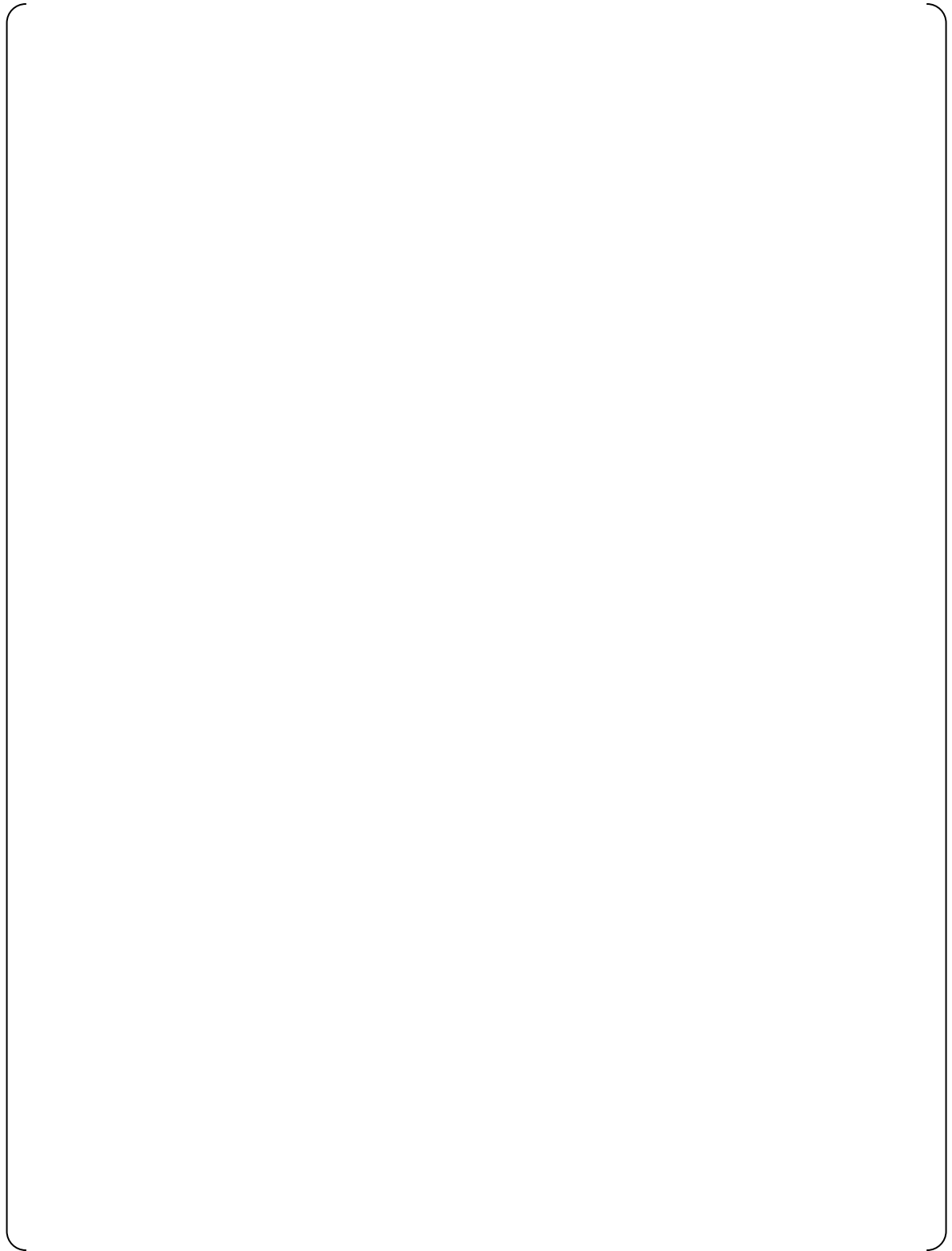


Figure A1-13-2-1 PIPESTRESS analysis model diagram

Table A1-13-2-1 Natural frequency analysis results

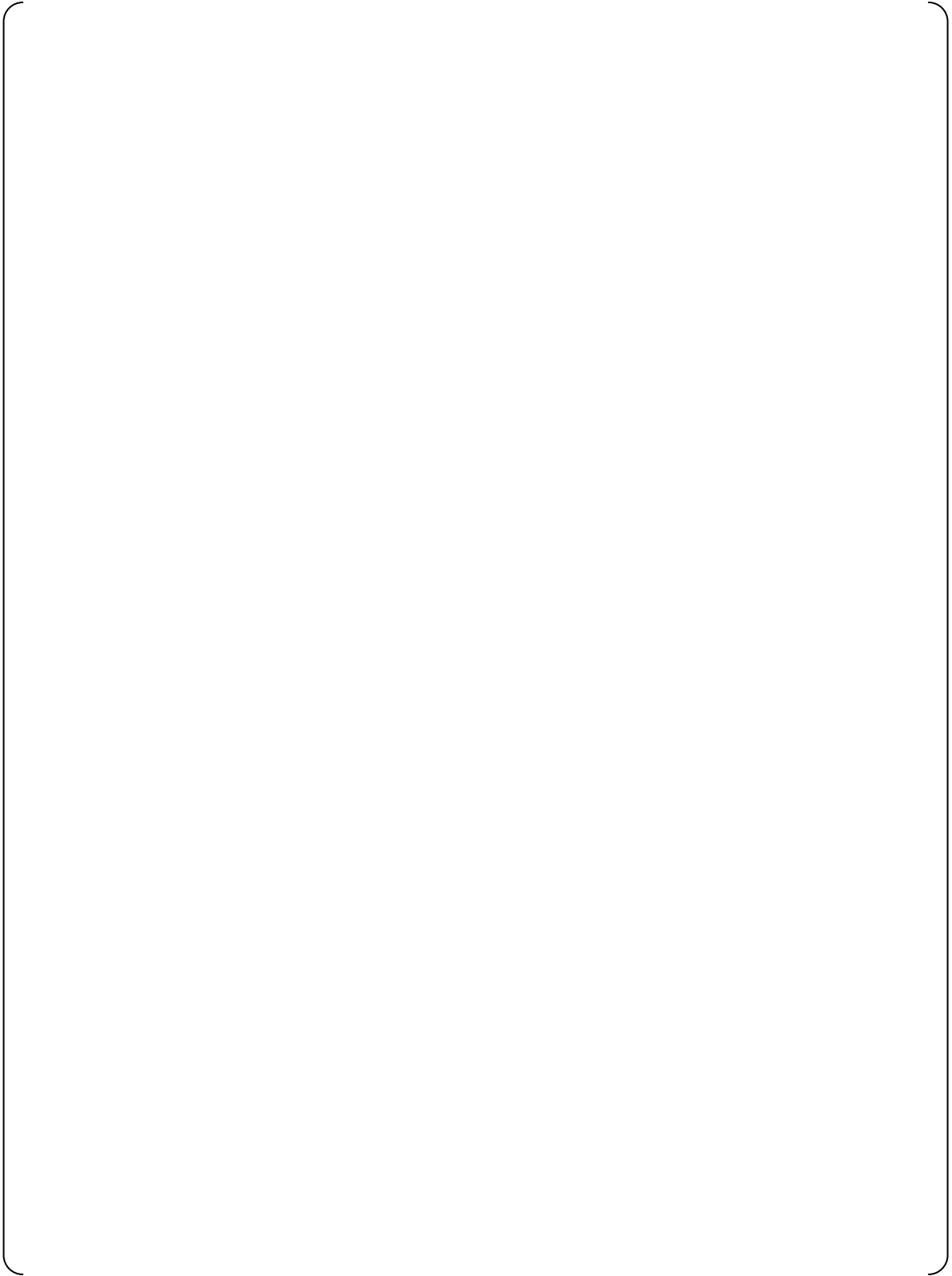


Figure A1-13-2-2 Frequency mode diagram (primary)

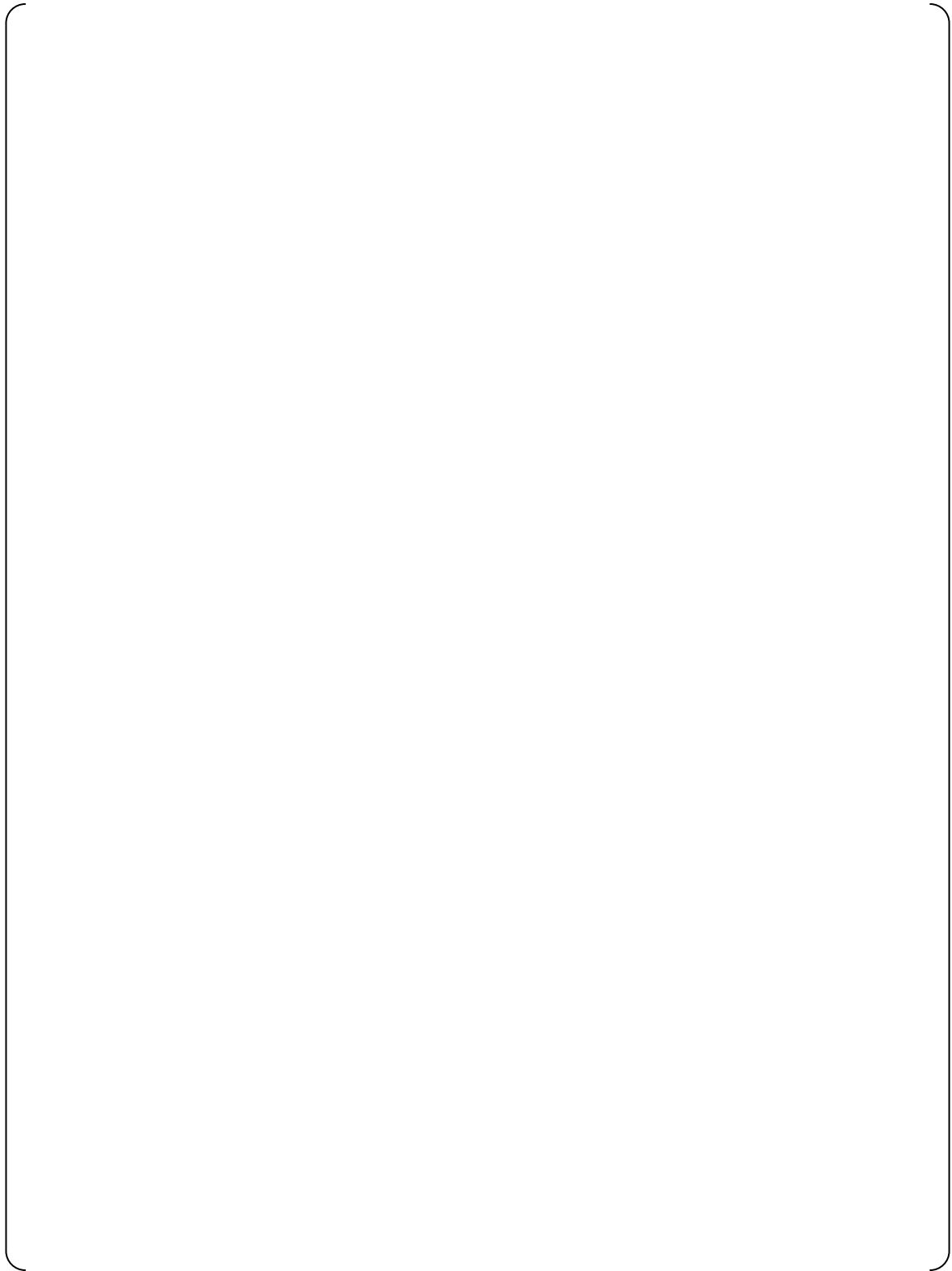


Figure A1-13-2-2 Frequency mode diagram (secondary)

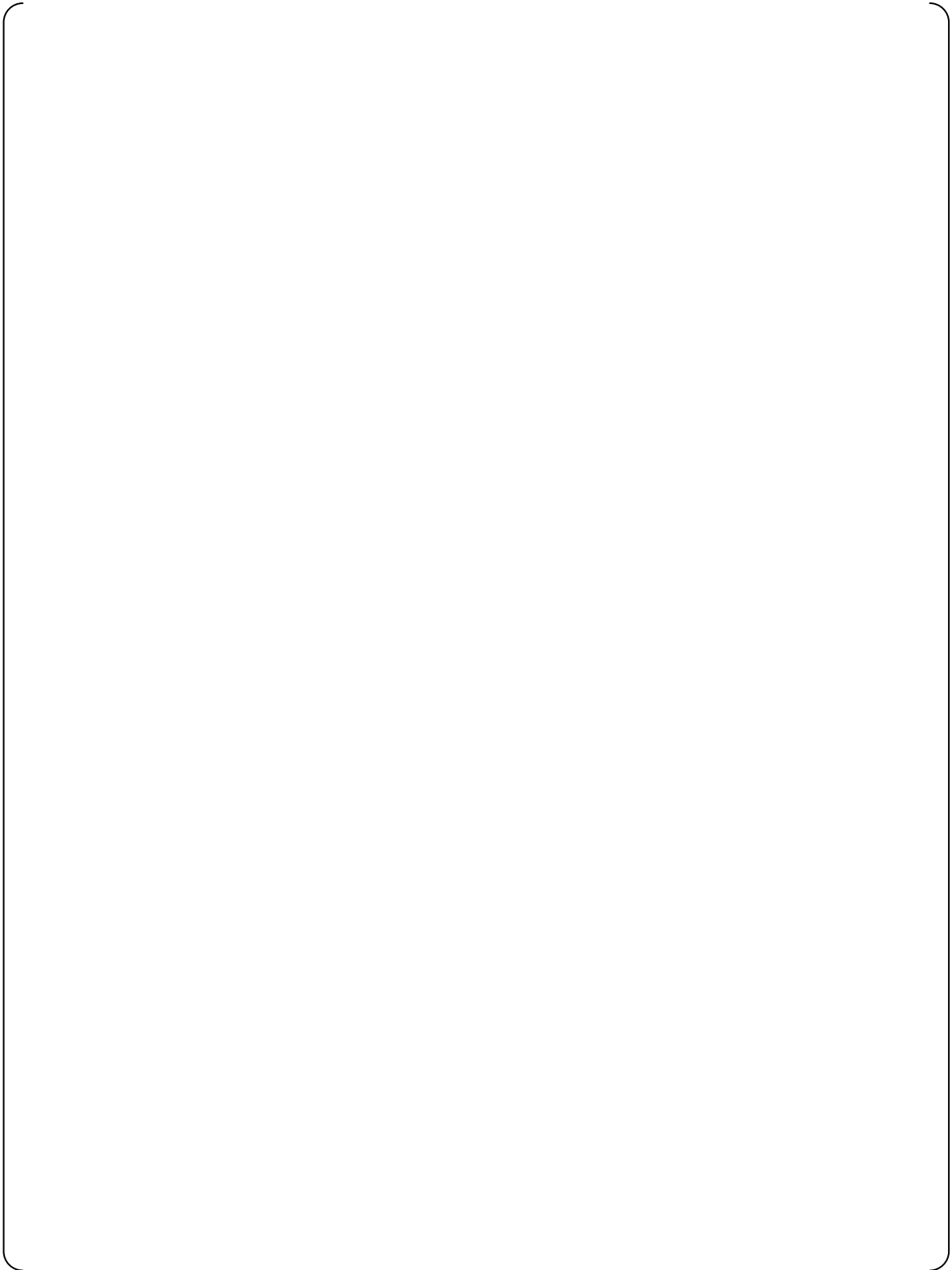


Figure A1-13-2-2 Frequency mode diagram (tertiary)





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**









**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







Table A1-13-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (7/9)  
(Section III)

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**









**Table A1-13-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-13-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

## Appendix 1-14

### CS02 CVCS Letdown Line Piping Analysis Results

1. INPUT

|  |                  |
|--|------------------|
| 1.1 Used for creating the pipe structural model              |                  |
| 1.1.1 Block division and piping specifications               | Table A1-14-1-1  |
| 1.1.2 Piping isometrics                                      | Figure A1-14-1-1 |
| 1.1.3 Concentrated mass                                      | Table A1-14-1-2  |
| 1.1.4 Support point rigidity                                 | Table A1-14-1-3  |
| 1.1.5 Valve rigidity   | Table A1-14-1-4  |
| 1.2 Used for creating load conditions                        |                  |
| 1.2.1 Level A/B design transient                             | see main text    |
| 1.2.2 Level A/B thermal displacement input data              | Table A1-14-1-5  |
| 1.2.3 Level A, B temperature and pressure input data         | Table A1-14-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data | Table A1-14-1-7  |
| 1.2.5 Floor response curve                                   | Figure A1-14-1-2 |
| 1.2.6 Seismic anchor displacement input data                 | Table A1-14-1-8  |
| 1.2.7 DBPB displacement input data                           | Table A1-14-1-9  |

2. OUTPUT

|  |                  |
|--|------------------|
| 2.1 PIPESTRESS analysis model diagram                                      | Figure A1-14-2-1 |
| 2.2 Natural frequency analysis results                                     | Table A1-14-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                           | Figure A1-14-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a - T_b$ ) | Table A1-14-2-2  |
| 2.5 Piping stress and fatigue evaluation results                           | Table A1-14-2-3  |

Table A1-14-1-1 Block division and piping specifications (1/2)

Table A1-14-1-1 Block division and piping specifications (2/2)



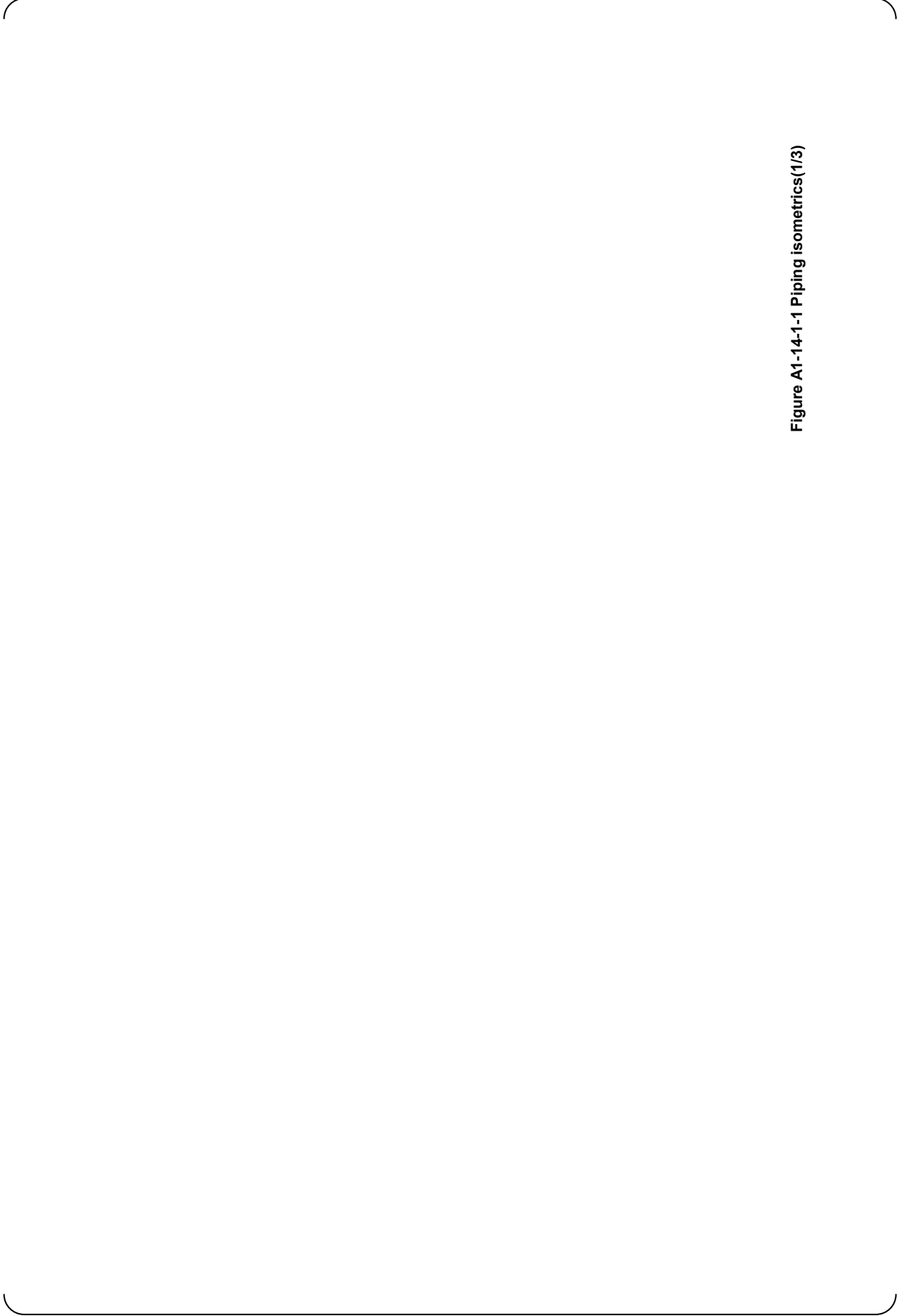


Figure A1-14-1-1 Piping isometrics(1/3)

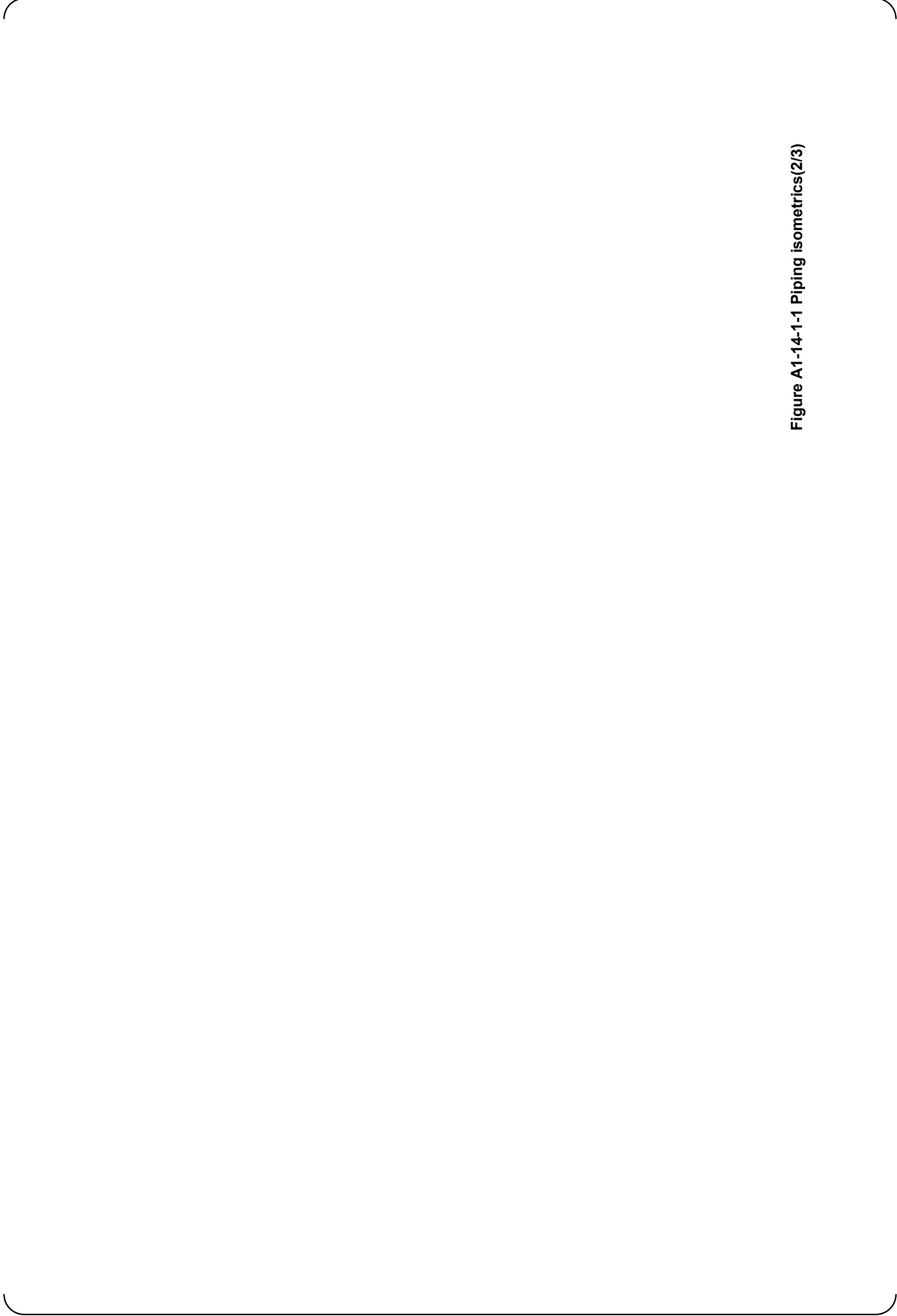


Figure A1-14-1-1 Piping isometrics(2/3)



Figure A1-14-1-1 Piping isometrics(3/3)

Table A1-14-1-2 Concentrated mass

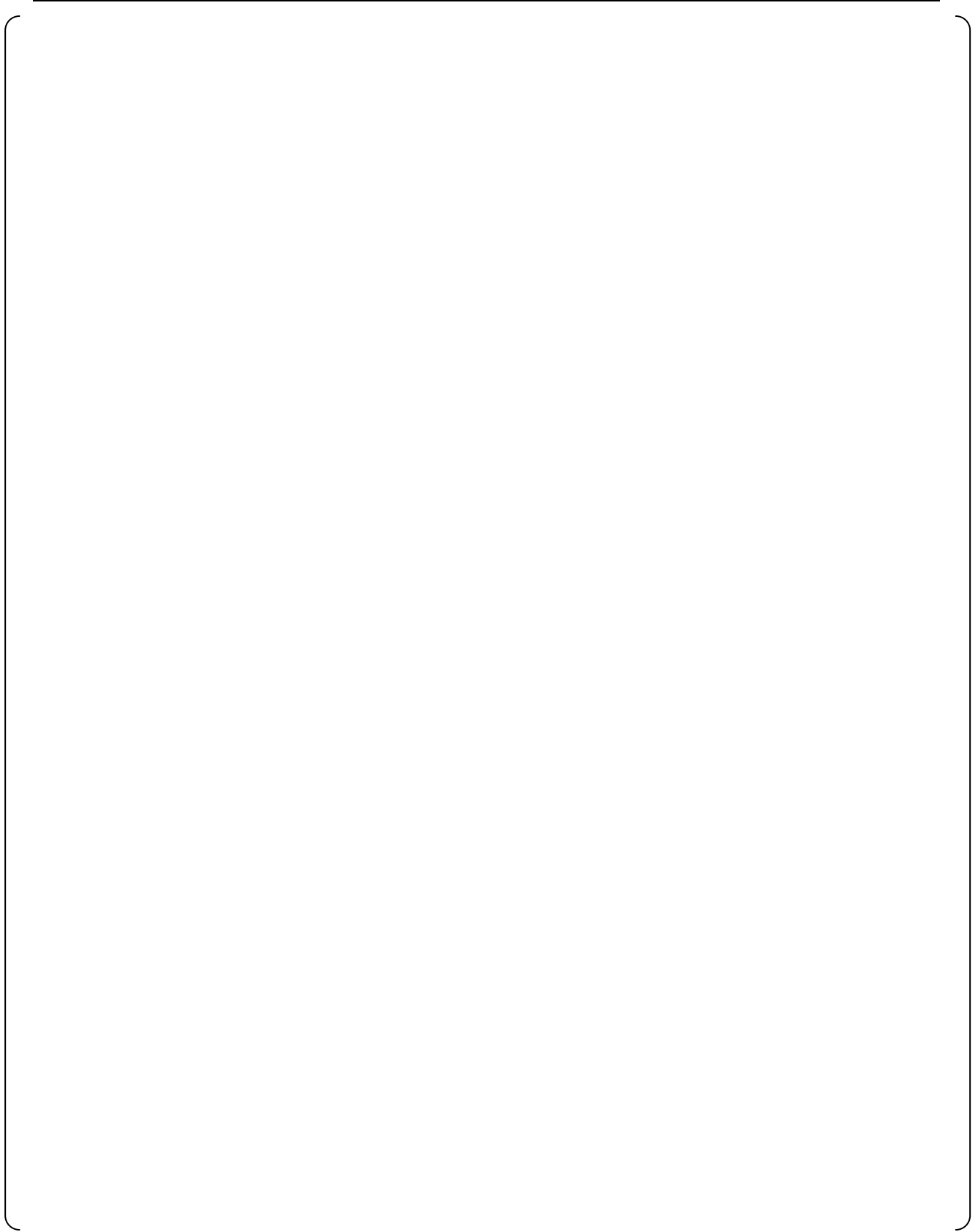
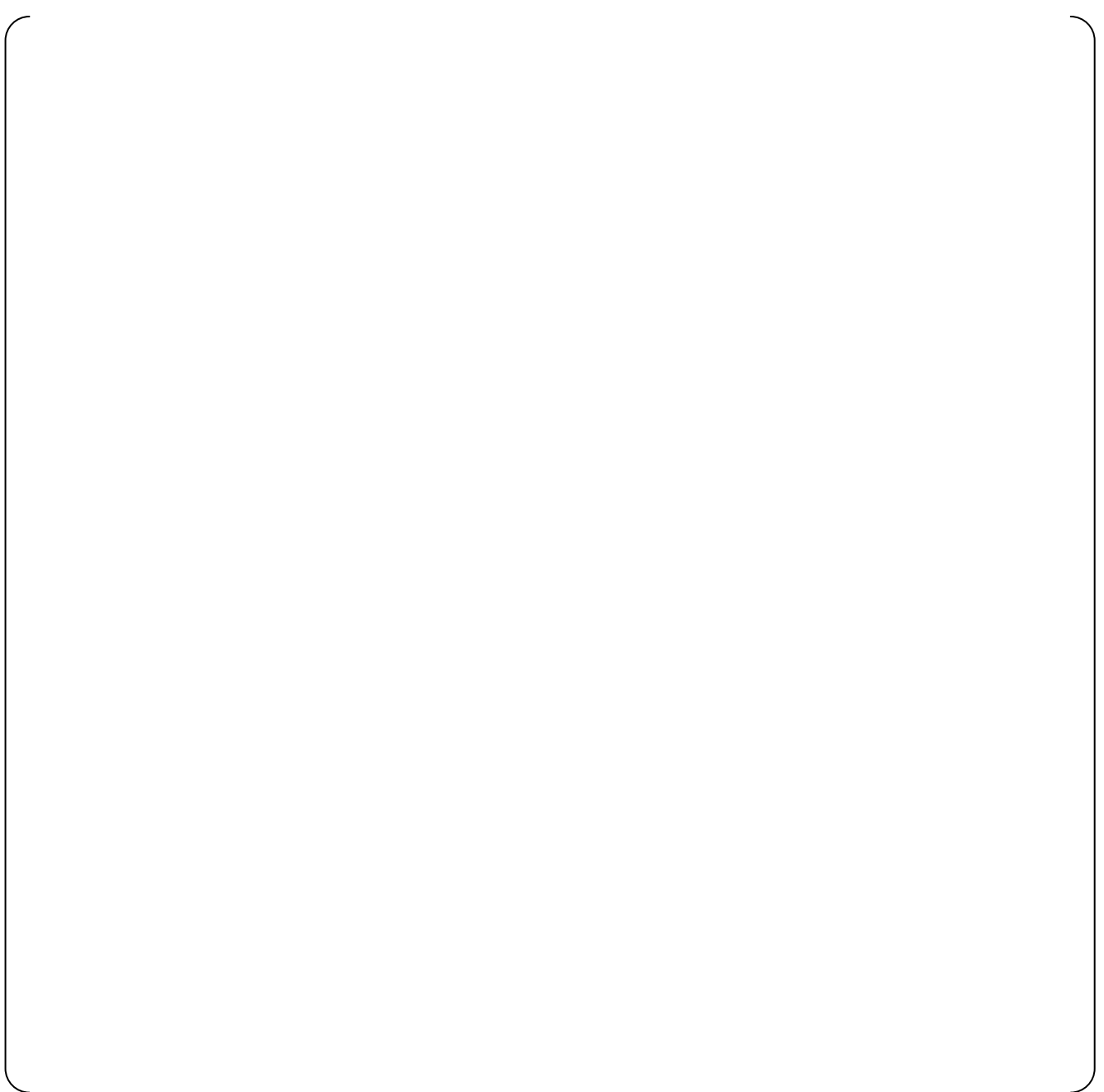


Table A1-14-1-3 Support point rigidity







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Table A1-14-1-4 Valve rigidity

Table A1-14-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9010)

|  |
|--|
|  |
|--|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-14-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9010)

| Point | Level A | Level B |
|-------|---------|---------|
| 9010  |         |         |

Table A1-14-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9010)

|  |
|--|
|  |
|--|

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Table A1-14-1-6 Level A, B temperature and pressure input data (1/27)  
(Section I)

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Table A1-14-1-6 Level A, B temperature and pressure input data (2/27)  
(Section I)

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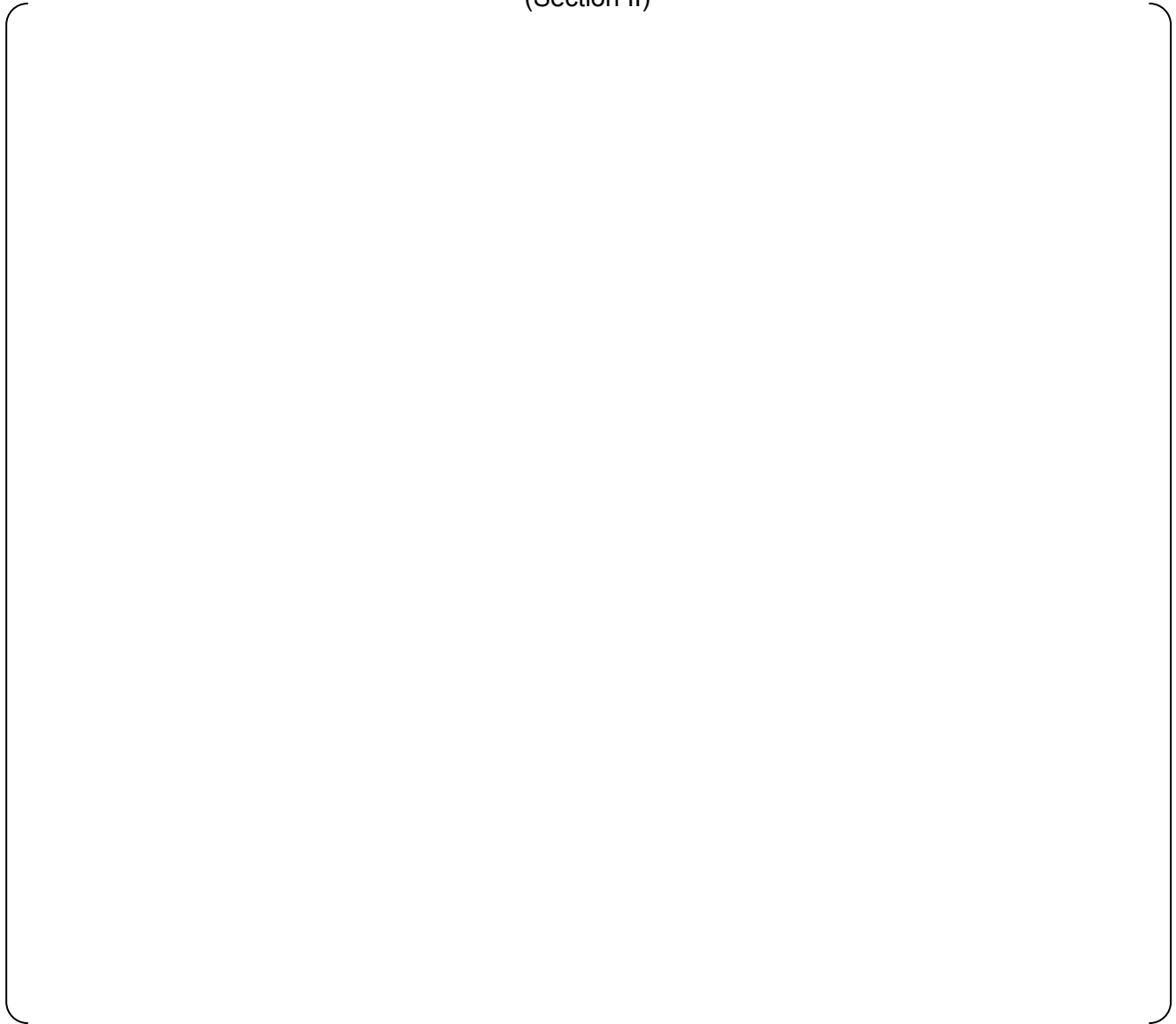
Table A1-14-1-6 Level A, B temperature and pressure input data (3/27)  
(Section I)



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Table A1-14-1-6 Level A, B temperature and pressure input data (4/27)  
(Section II)

Table A1-14-1-6 Level A, B temperature and pressure input data (5/27)  
(Section II)



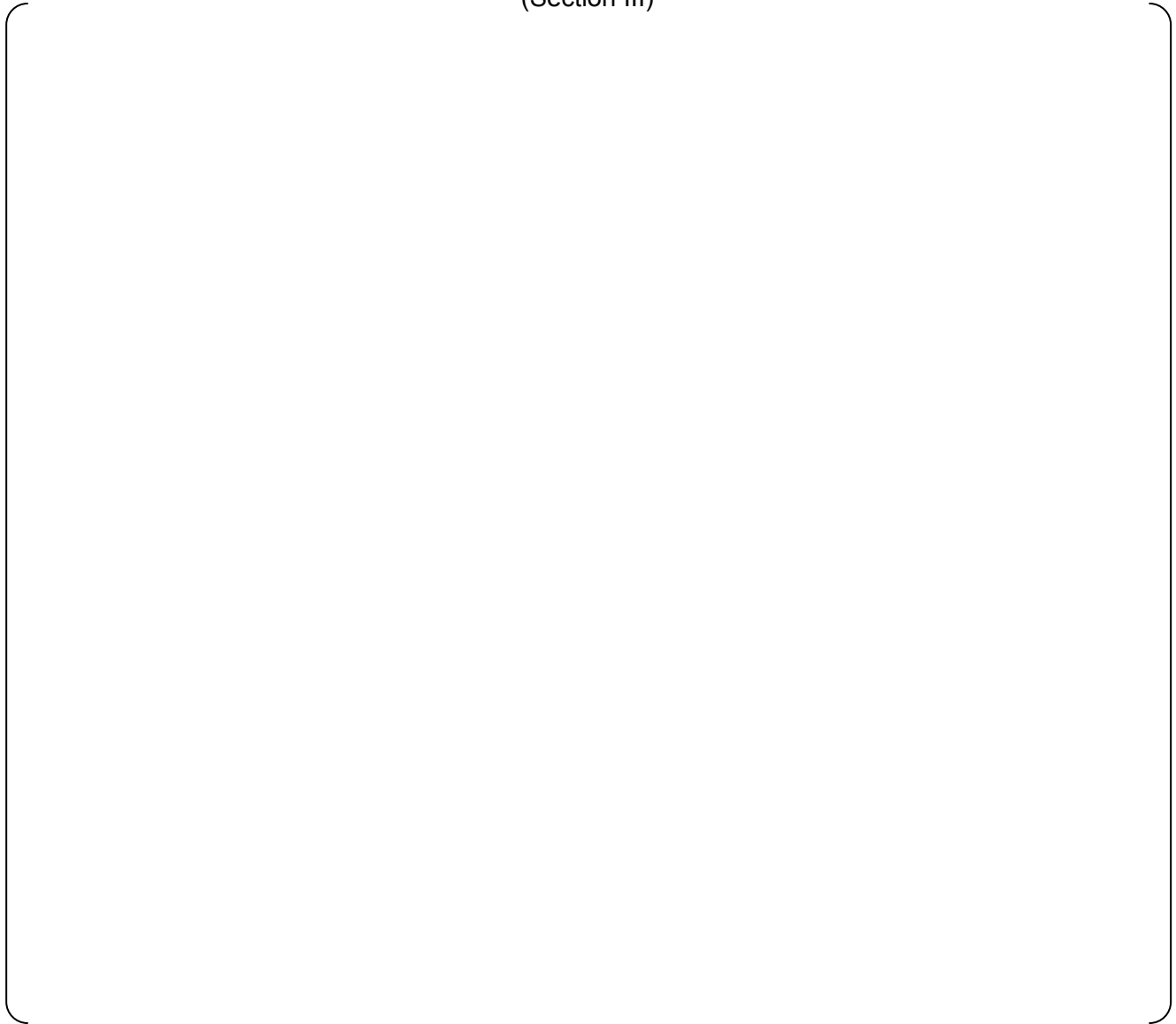
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Table A1-14-1-6 Level A, B temperature and pressure input data (6/27)  
(Section II)

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Table A1-14-1-6 Level A, B temperature and pressure input data (7/27)  
(Section III)

Table A1-14-1-6 Level A, B temperature and pressure input data (8/27)  
(Section III)



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Table A1-14-1-6 Level A, B temperature and pressure input data (9/27)  
(Section III)

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Table A1-14-1-6 Level A, B temperature and pressure input data (10/27)  
(Section IV)

Table A1-14-1-6 Level A, B temperature and pressure input data (11/27)  
(Section IV)

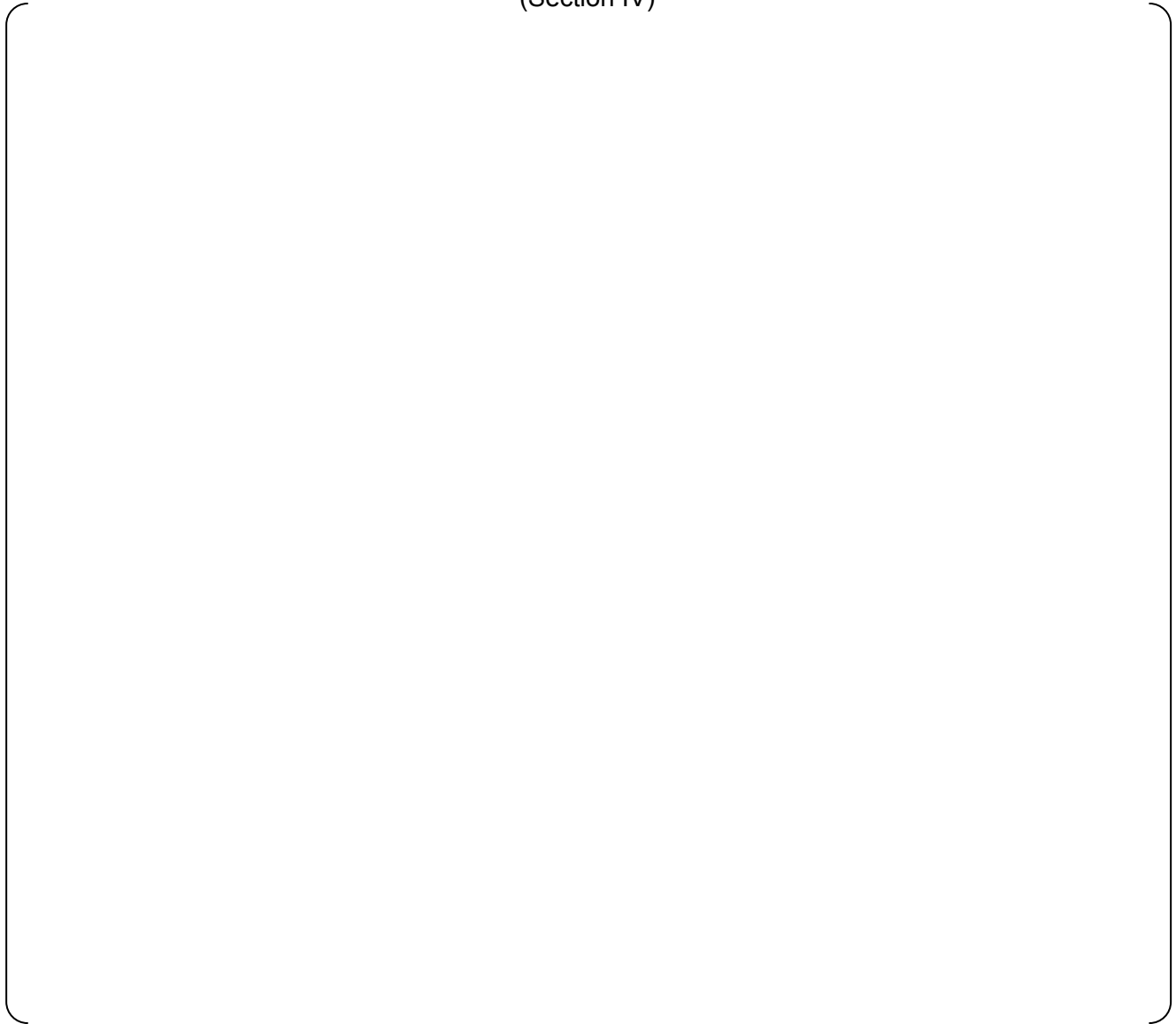




Table A1-14-1-6 Level A, B temperature and pressure input data (12/27)  
(Section IV)

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Table A1-14-1-6 Level A, B temperature and pressure input data (13/27)  
(Section V)

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Table A1-14-1-6 Level A, B temperature and pressure input data (14/27)  
(Section V)

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Table A1-14-1-6 Level A, B temperature and pressure input data (15/27)  
(Section V)

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Table A1-14-1-6 Level A, B temperature and pressure input data (16/27)  
(Section VI)

Table A1-14-1-6 Level A, B temperature and pressure input data (17/27)  
(Section VI)

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**Table A1-14-1-6 Level A, B temperature and pressure input data (18/27)**  
(Section VI)

Table A1-14-1-6 Level A, B temperature and pressure input data (19/27)  
(Section VII)



**Table A1-14-1-6 Level A, B temperature and pressure input data (20/27)  
(Section VII)**

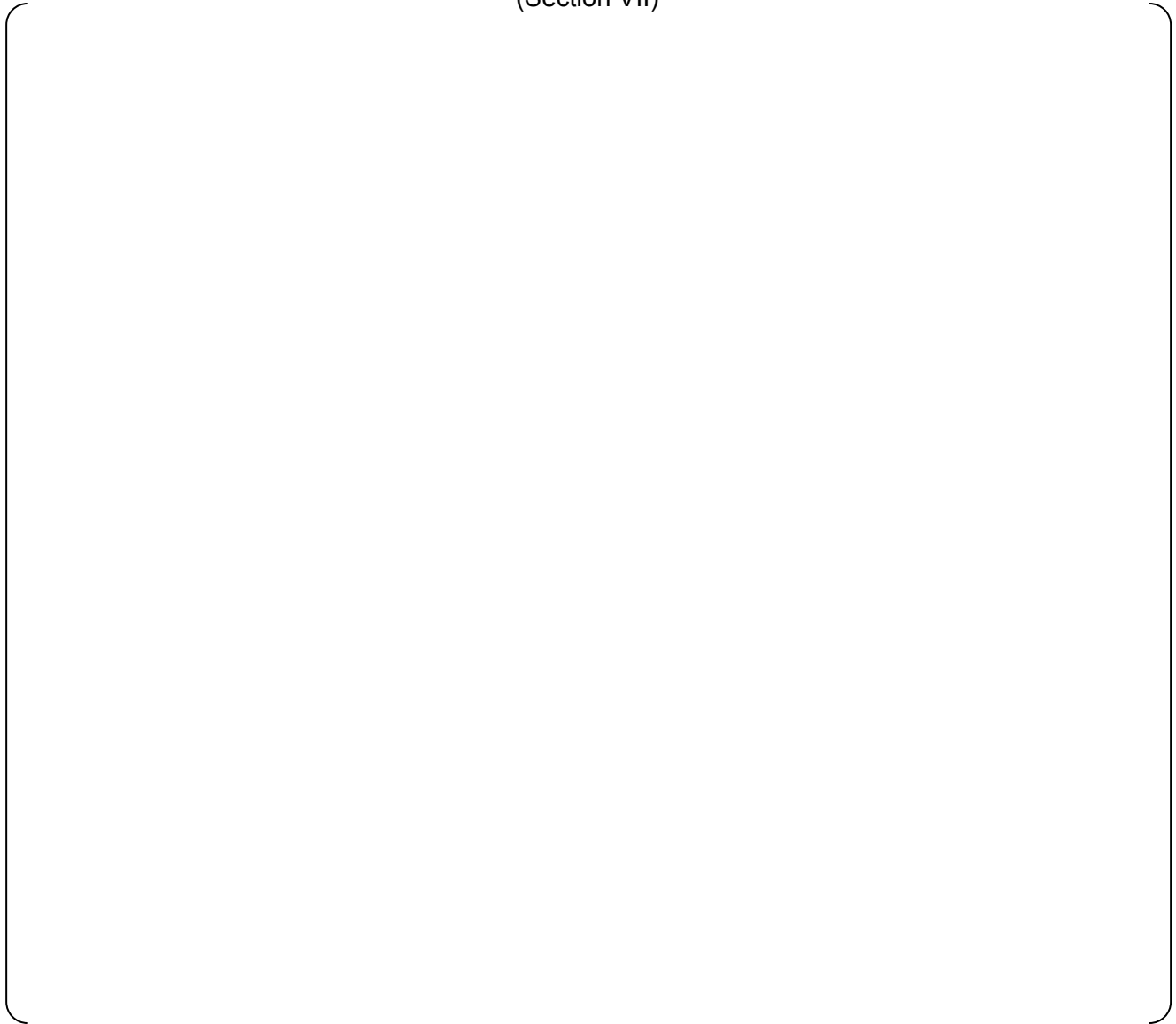
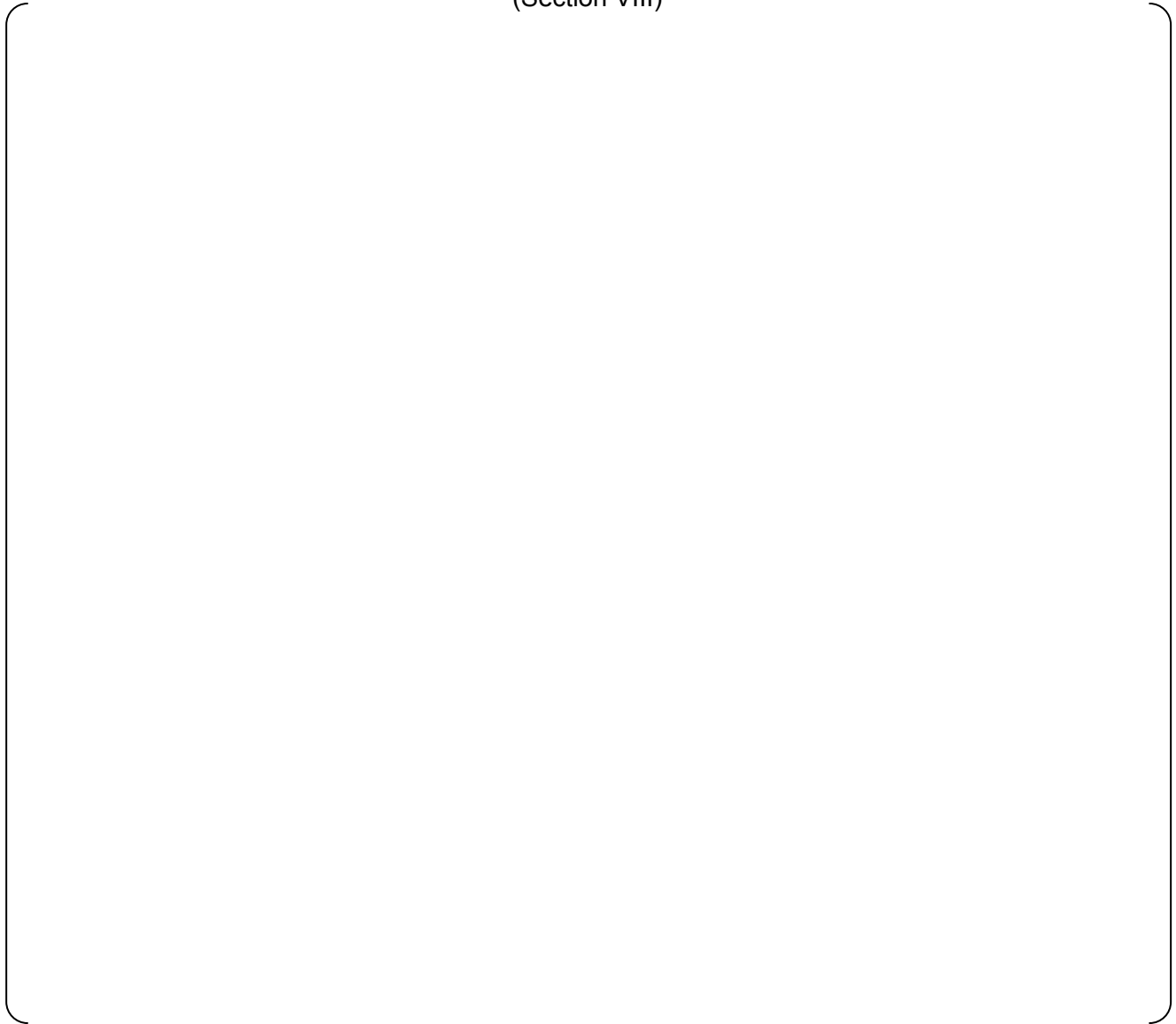


Table A1-14-1-6 Level A, B temperature and pressure input data (21/27)  
(Section VII)

Table A1-14-1-6 Level A, B temperature and pressure input data (22/27)  
(Section VIII)

Table A1-14-1-6 Level A, B temperature and pressure input data (23/27)  
(Section VIII)



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**Table A1-14-1-6 Level A, B temperature and pressure input data (24/27)**  
(Section VIII)

Table A1-14-1-6 Level A, B temperature and pressure input data (25/27)  
(Section IX)

Table A1-14-1-6 Level A, B temperature and pressure input data (26/27)  
(Section IX)

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**Table A1-14-1-6 Level A, B temperature and pressure input data (27/27)**  
(Section IX)



Table A1-14-1-7 Level C, D maximum temperature and pressure input data



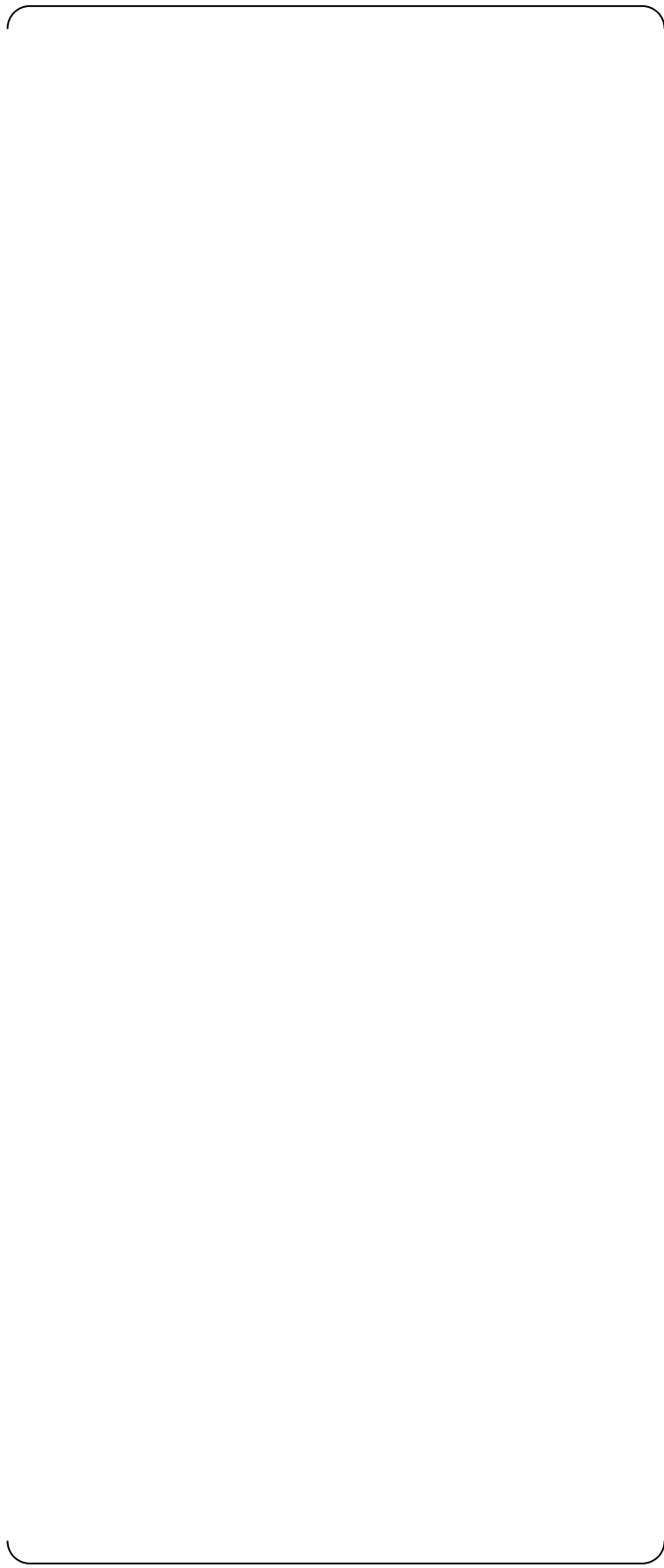
**Figure A1-14-1-2 Floor response curve (1/6)**  
CVCS Letdown (CS02) FRS for MCP Nozzle  
X (EW) direction (damping 4.0%)



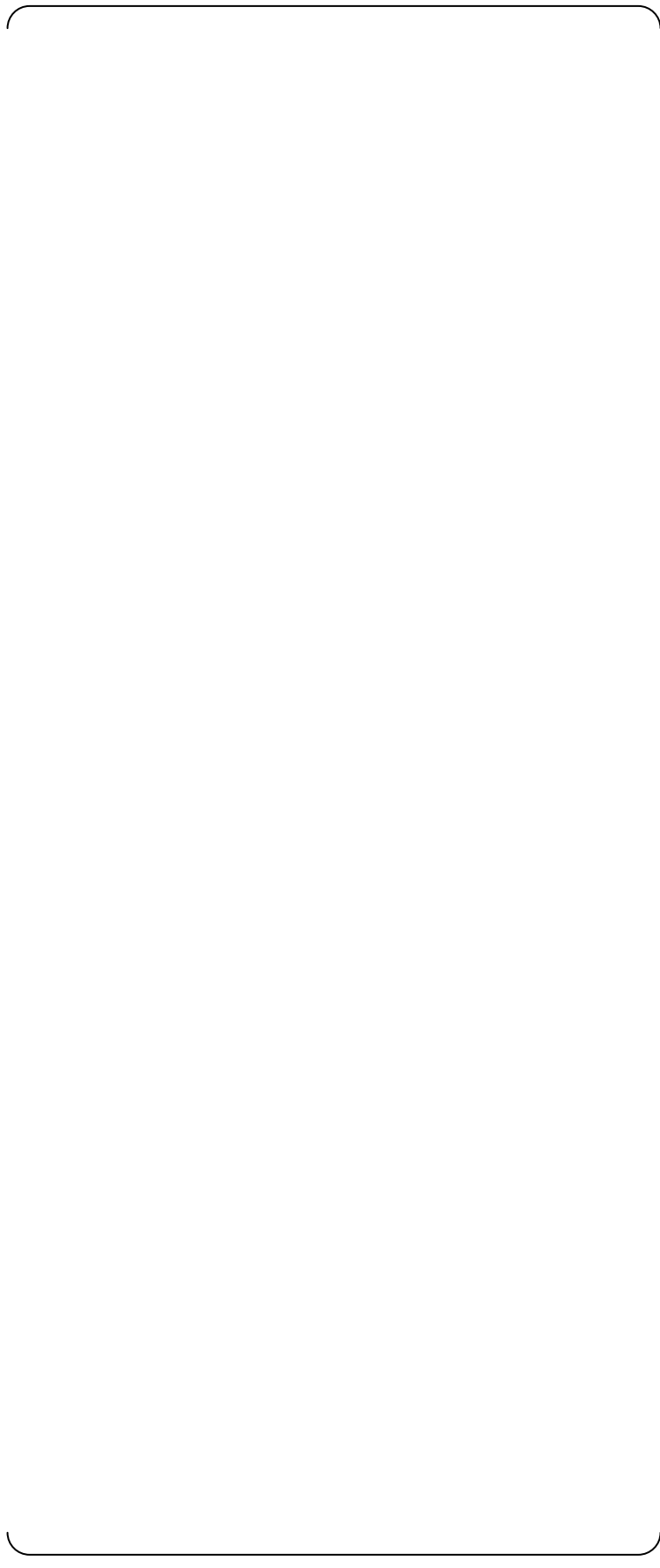
**Figure A1-14-1-2 Floor response curve (2/6)**  
CVCS Letdown (CS02) FRS for MCP Nozzle  
Y (NS) direction (damping 4.0%)



**Figure A1-14-1-2 Floor response curve (3/6)**  
CVCS Letdown (CS02) FRS for MCP Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-14-1-2 Floor response curve (4/6)**  
CVCS Letdown (CS02) FRS for Piping  
X (EW) direction (damping 4.0%)



**Figure A1-14-1-2 Floor response curve (5/6)**  
CVCS Letdown (CS02) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-14-1-2 Floor response curve (6/6)**  
CVCS Letdown (CS02) FRS for Piping  
Z (Vert.) direction (damping 4.0%)

Table A1-14-1-8 Seismic anchor displacement input data





Table A1-14-1-9 DBPB displacement input data

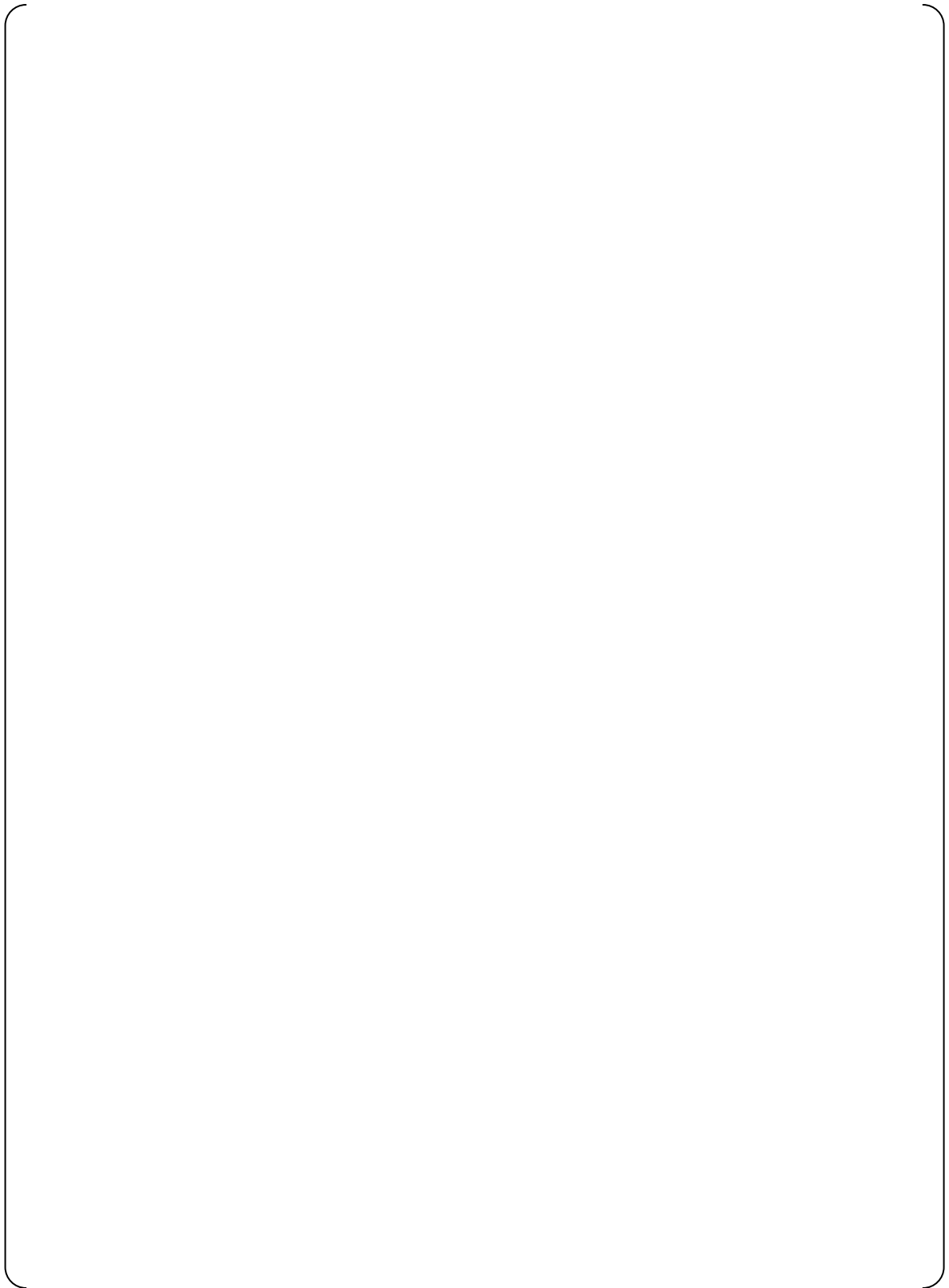


Figure A1-14-2-1 PIPESTRESS analysis model diagram

Table A1-14-2-1 Natural frequency analysis results

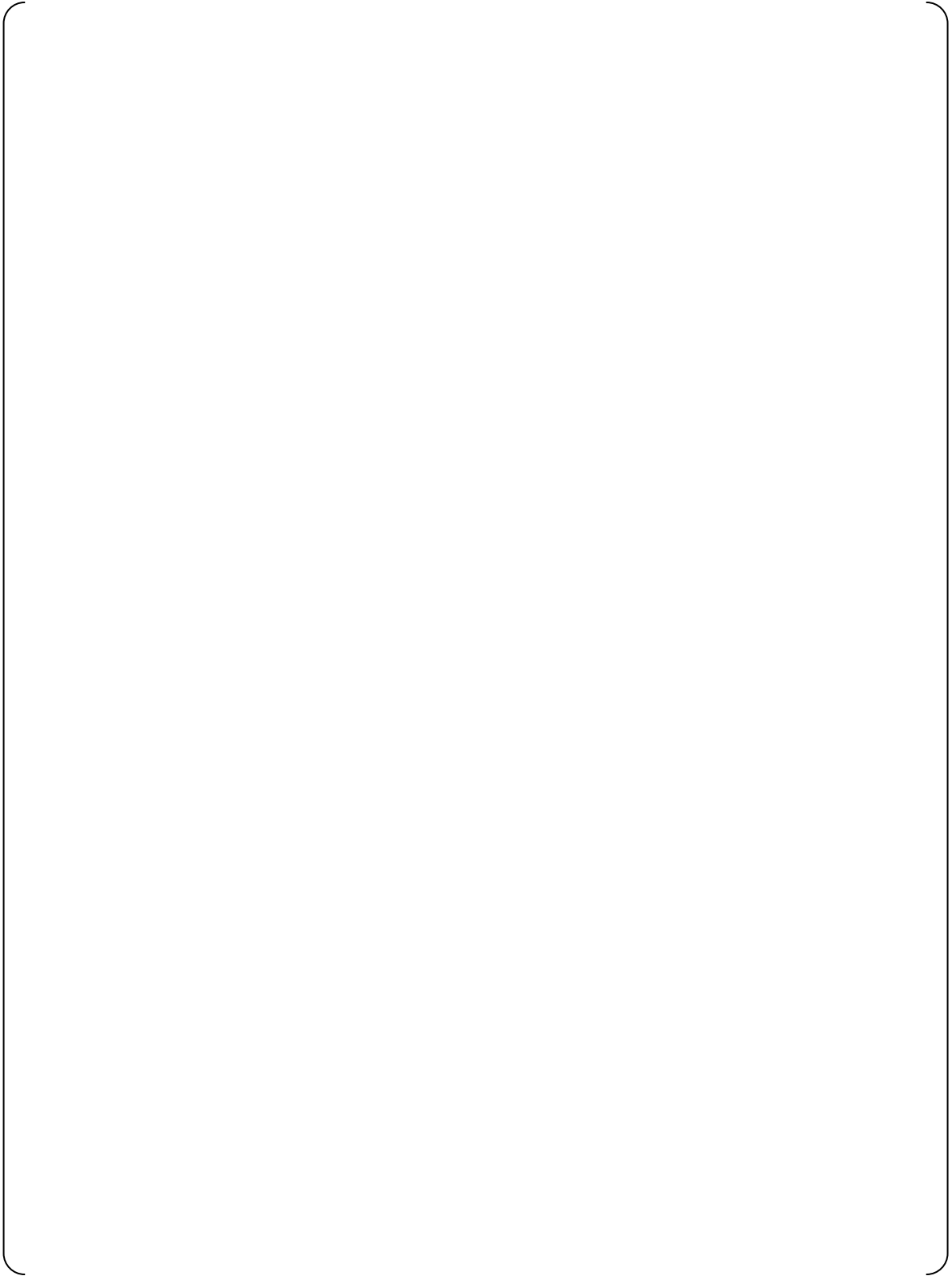


Figure A1-14-2-2 Frequency mode diagram (primary)

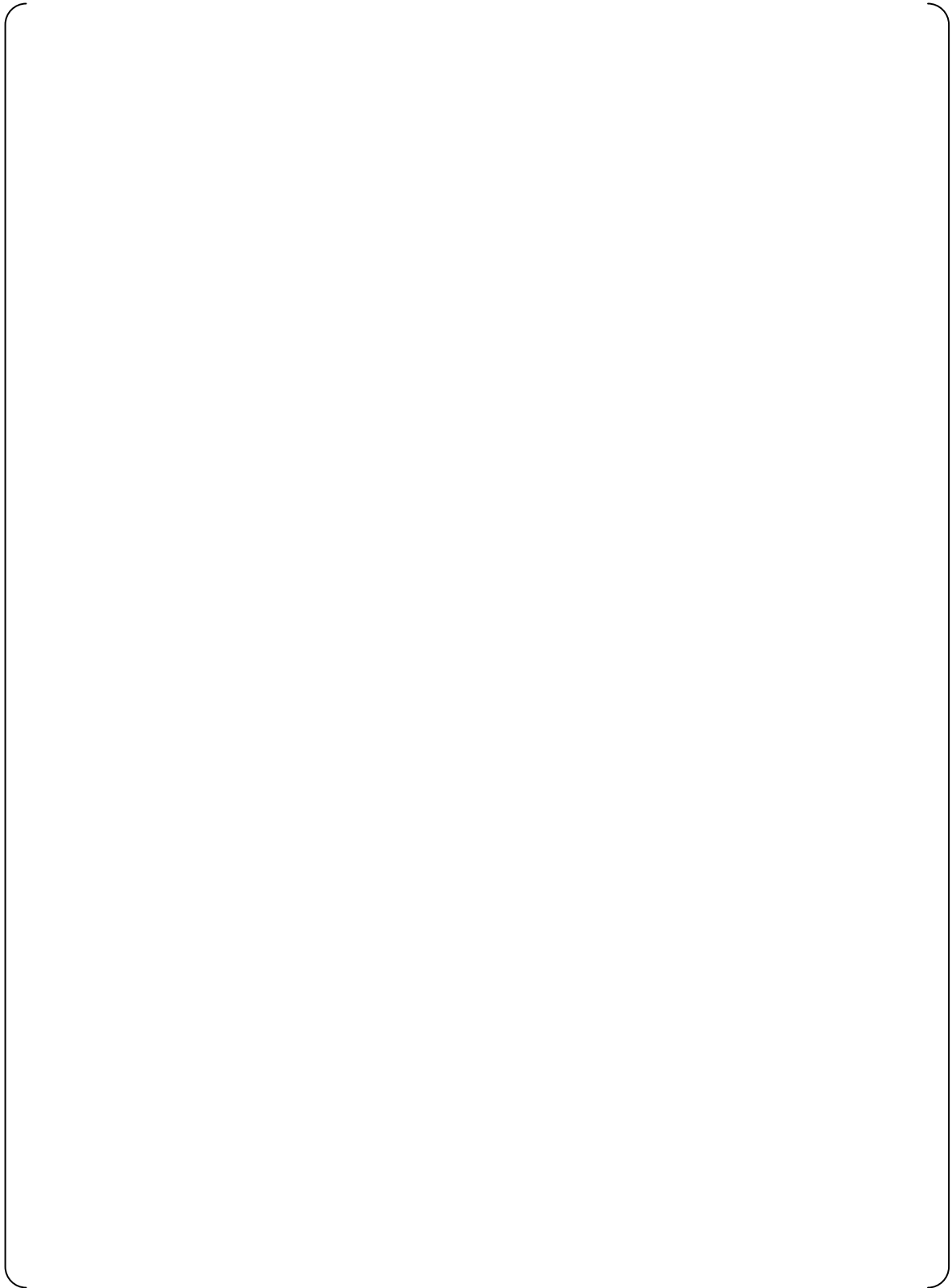


Figure A1-14-2-2 Frequency mode diagram (secondary)

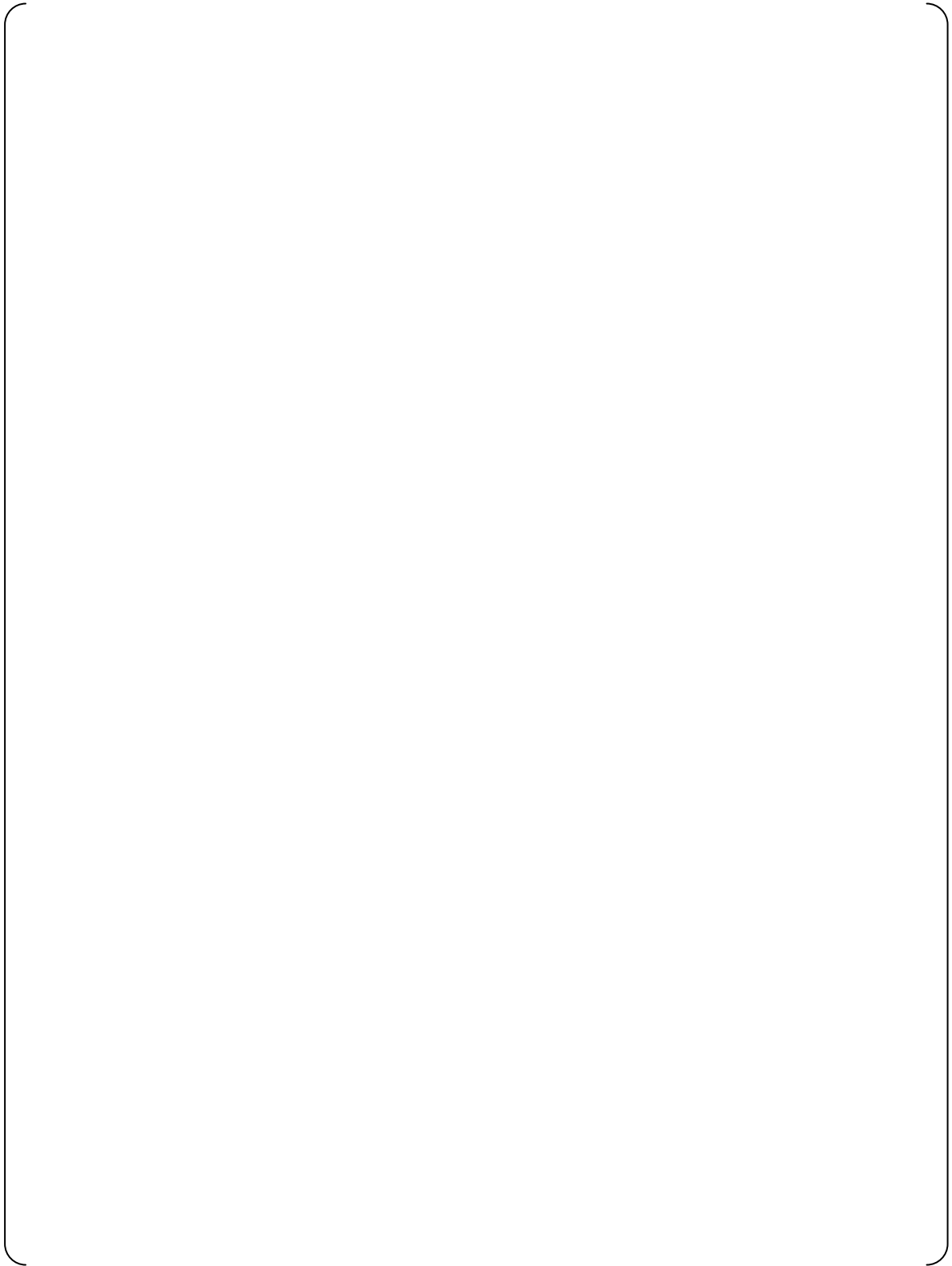


Figure A1-14-2-2 Frequency mode diagram (tertiary)

Table A1-14-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (1/18)  
(Section I)





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-14-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (2/18)  
(Section I)

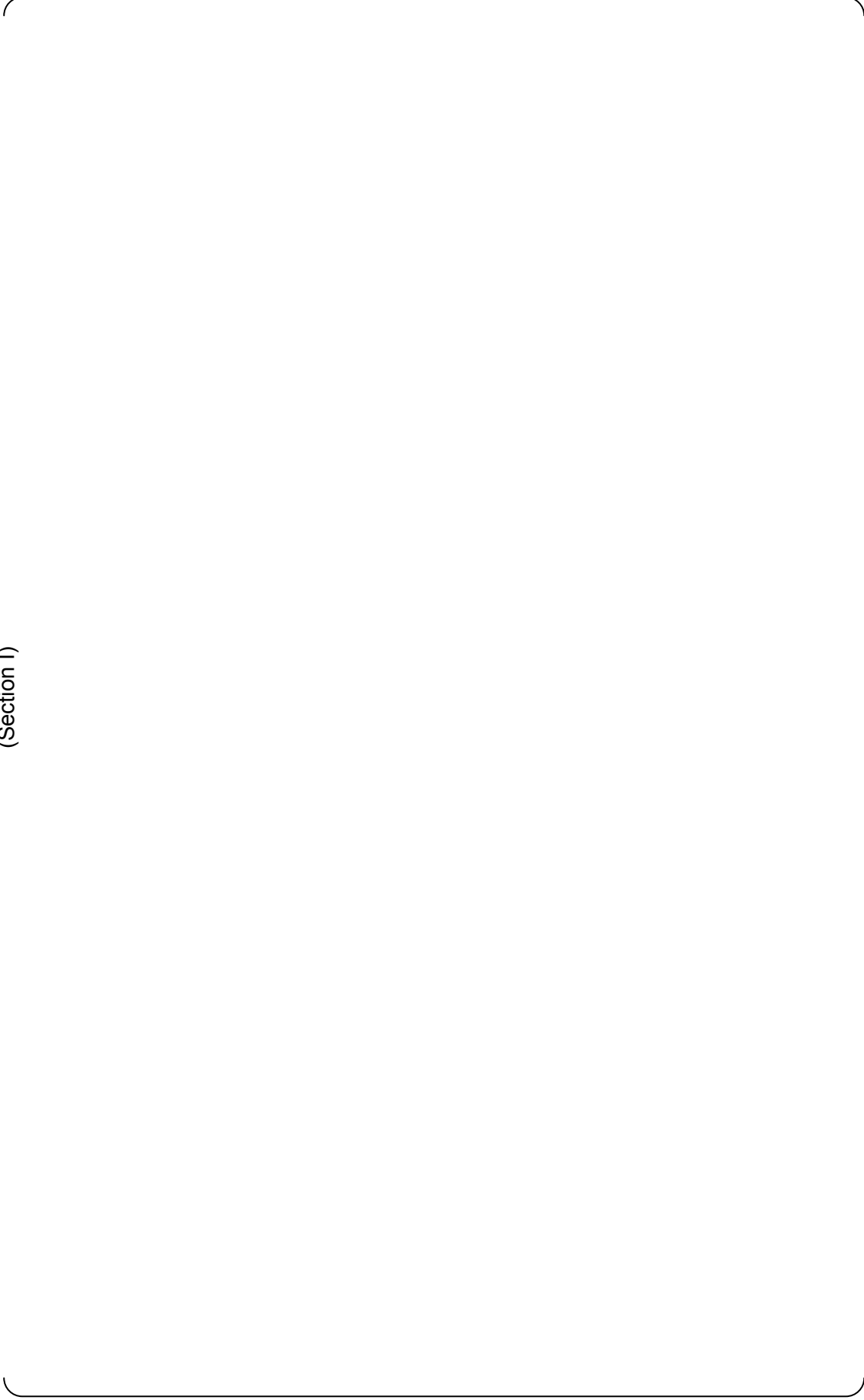


Table A1-14-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (3/18)  
(Section I)

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-14-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (4/18)  
(Section II)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-14-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (5/18)  
(Section II)









Table A1-14-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (7/18)  
(Section III)





Table A1-14-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (8/18)  
(Section III)





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



**Table A1-14-2-2 Thermal analysis results ( $\Delta T_1$ ,  $\Delta T_2$ , Ta-Tb) (1018)**  
(Section IV)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-14-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta$ - $Tb$ ) (11/18)  
(Section IV)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-14-2-2 Thermal analysis results ( $\Delta T_1$ ,  $\Delta T_2$ ,  $T_a$ - $T_b$ ) (13/18)  
(Section V)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-14-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (14/18)  
(Section V)

| Section V |
|-----------|
|-----------|

Table A1-14-2-2 Thermal analysis results ( $\Delta T_1$ ,  $\Delta T_2$ ,  $T_a$ - $T_b$ ) (15/18)  
(Section V)

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





Table A1-14-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (16/18)  
(Section VI)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-14-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta$ - $Tb$ ) (17/18)  
(Section VI)

Table A1-14-2-2 Thermal analysis results ( $\Delta T_1$ ,  $\Delta T_2$ , Ta-Tb) (18/18)  
(Section VI)

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



**Table A1-14-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-14-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

## Appendix 1-15

### CS04 CVCS Seal Injection A Line Piping Analysis Results



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|  |                  |
|--|------------------|
| 1. INPUT   |                  |
| 1.1 Used for creating the pipe structural model                              |                  |
| 1.1.1 Block division and piping specifications                               | Table A1-15-1-1  |
| 1.1.2 Piping isometrics  | Figure A1-15-1-1 |
| 1.1.3 Concentrated mass  | Table A1-15-1-2  |
| 1.1.4 Support point rigidity   | Table A1-15-1-3  |
| 1.1.5 Valve rigidity   | Table A1-15-1-4  |
| 1.2 Used for creating load conditions  |                  |
| 1.2.1 Level A/B design transient   | see main text    |
| 1.2.2 Level A/B thermal displacement input data                              | Table A1-15-1-5  |
| 1.2.3 Level A, B temperature and pressure input data                         | Table A1-15-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data                 | Table A1-15-1-7  |
| 1.2.5 Floor response curve   | Figure A1-15-1-2 |
| 1.2.6 Seismic anchor displacement input data                                 | Table A1-15-1-8  |
| 1.2.7 DBPB displacement input data   | Table A1-15-1-9  |
| 2. OUTPUT  |                  |
| 2.1 PIPESTRESS analysis model diagram  | Figure A1-15-2-1 |
| 2.2 Natural frequency analysis results                                       | Table A1-15-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                             | Figure A1-15-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a$ - $T_b$ ) | Table A1-15-2-2  |
| 2.5 Piping stress and fatigue evaluation results                             | Table A1-15-2-3  |

Table A1-15-1-1 Block division and piping specifications

US-APWR CS04  
CVCS Seal Injection A Line  
Figure A1-15-1-1 Piping Isometrics

Table A1-15-1-2 Concentrated mass

Table A1-15-1-3 Support point rigidity



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Table A1-15-1-4 Valve rigidity

Table A1-15-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9100)

| Point | Level A | Level B |
|-------|---------|---------|
| 9100  |         |         |



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-15-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9100)

| Point | Level A | Level B |
|-------|---------|---------|
| 9100  |         |         |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-15-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9100)

| Point | Level A | Level B |
|-------|---------|---------|
| 9100  |         |         |

Table A1-15-1-6 Level A, B temperature and pressure input data (1/18)  
(Section I)

Table A1-15-1-6 Level A, B temperature and pressure input data (2/18)  
(Section I)

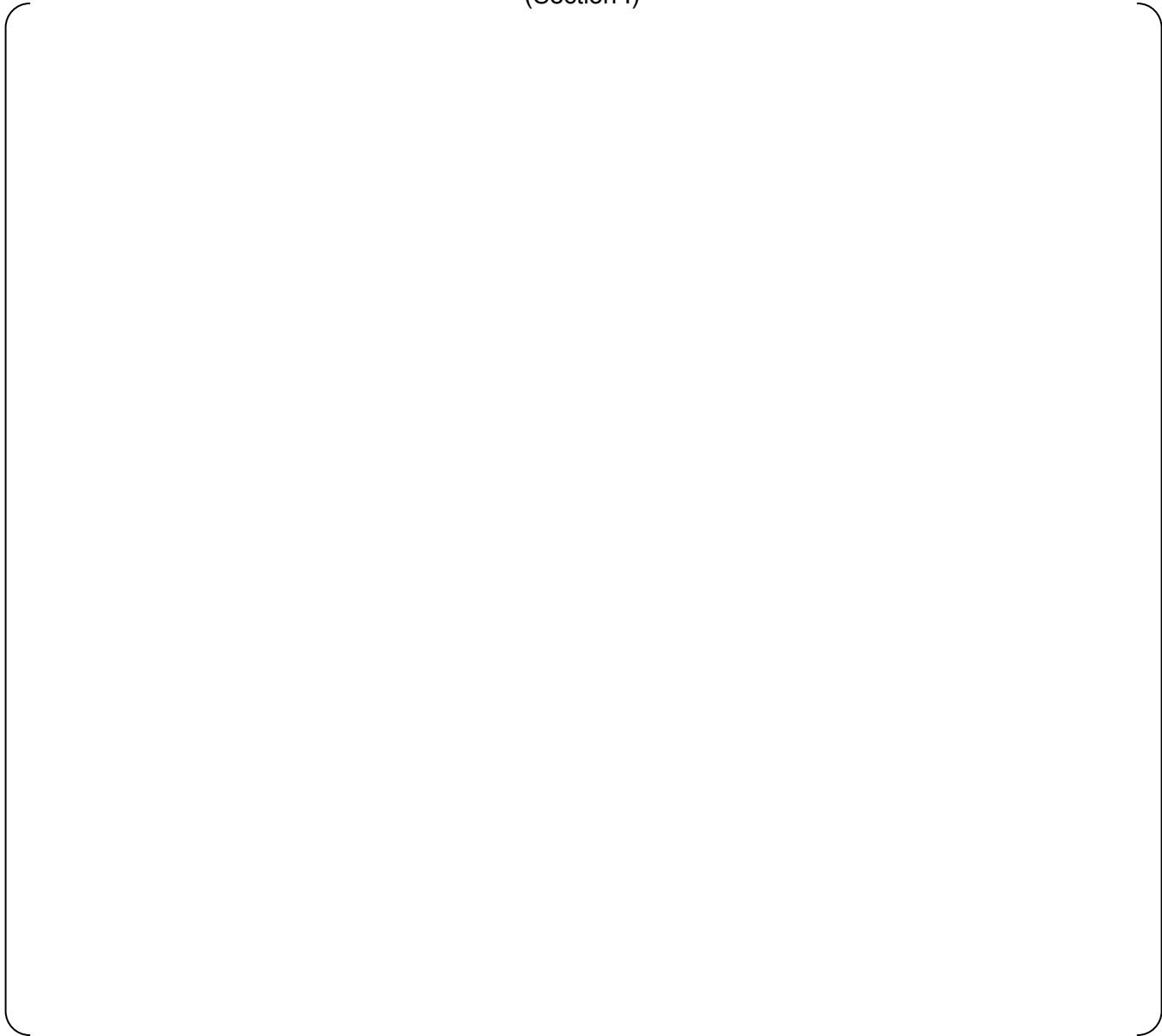


Table A1-15-1-6 Level A, B temperature and pressure input data (3/18)  
(Section I)

Table A1-15-1-6 Level A, B temperature and pressure input data (4/18)  
(Section II)

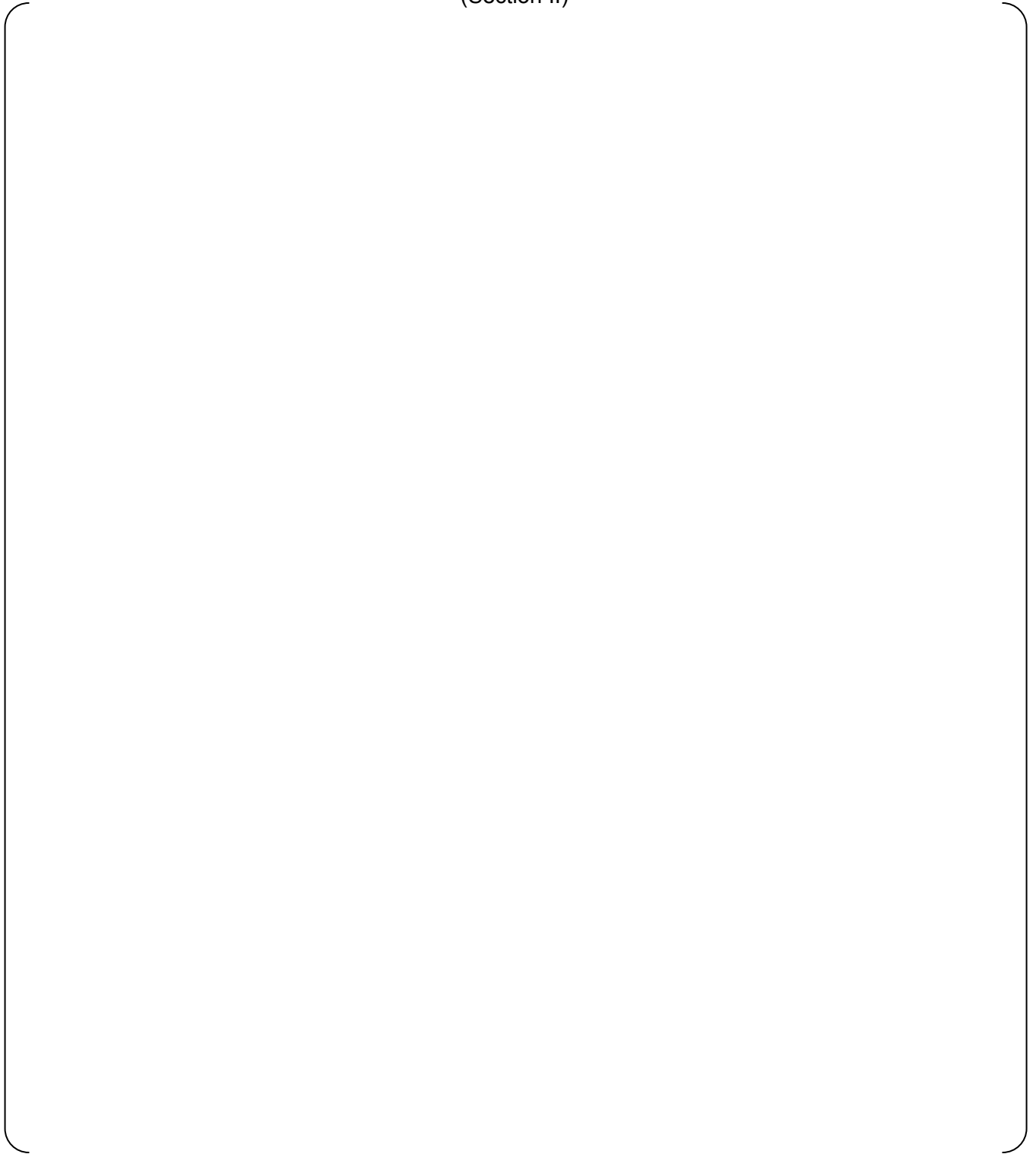
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Table A1-15-1-6 Level A, B temperature and pressure input data (5/18)  
(Section II)

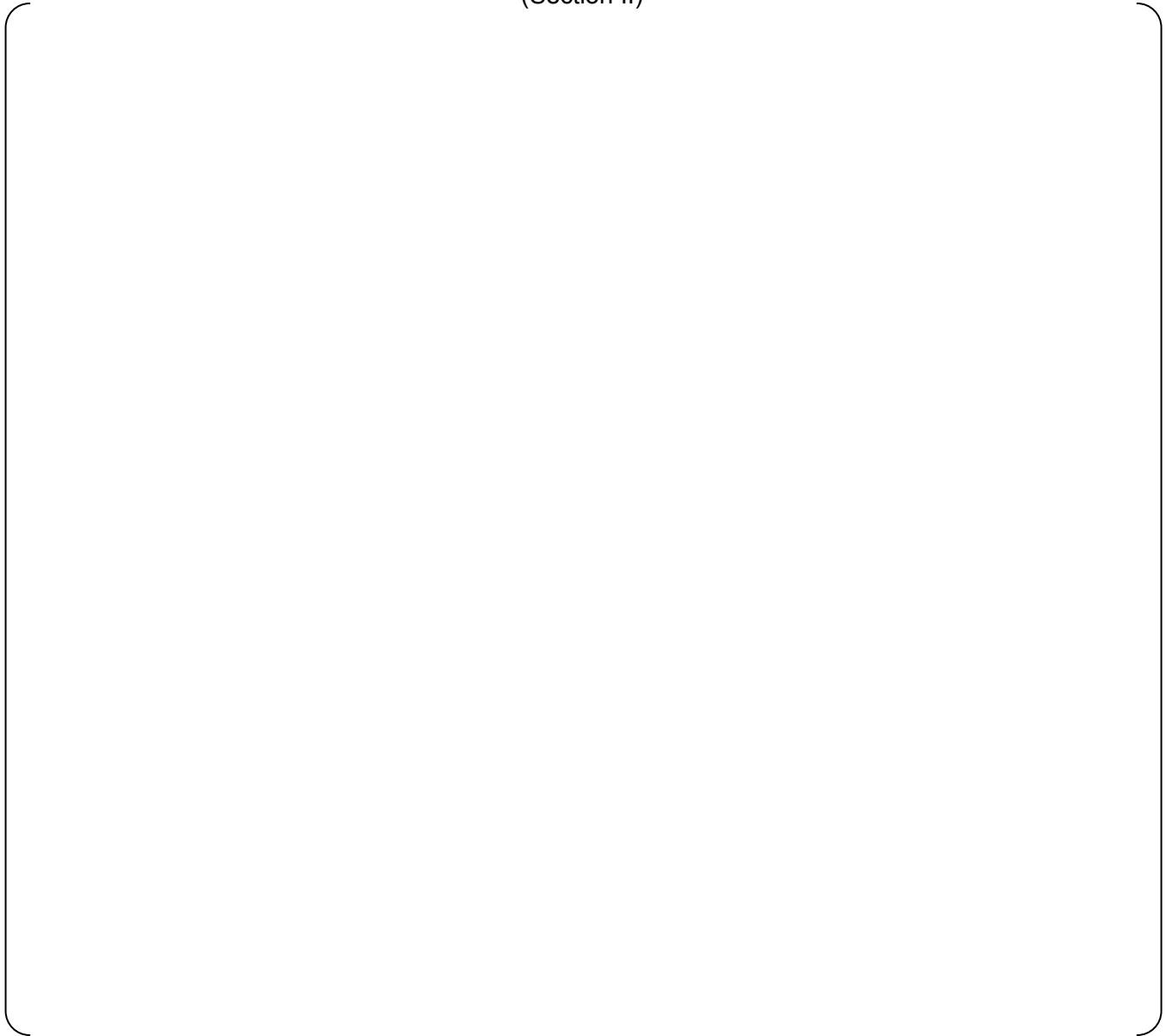
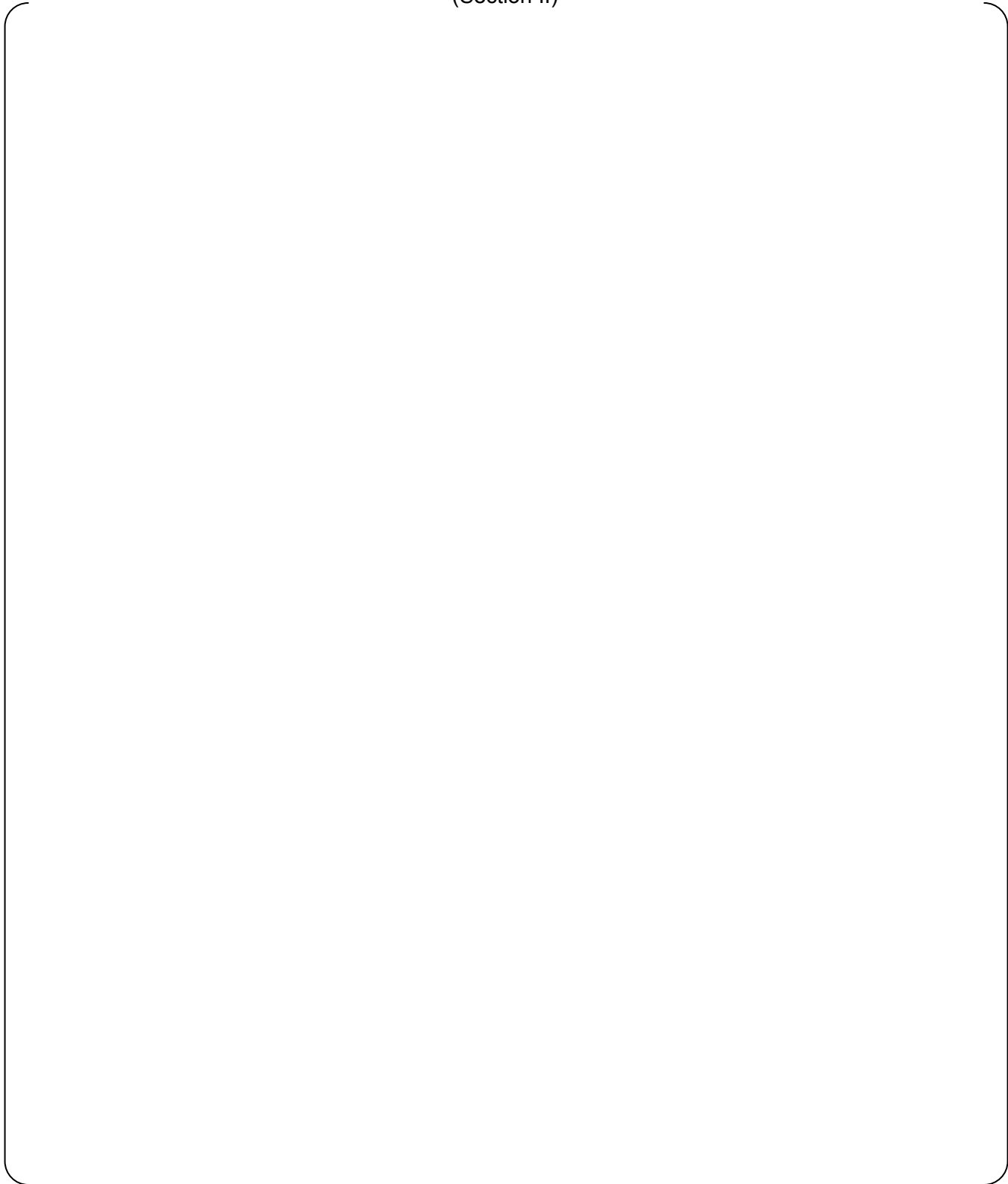
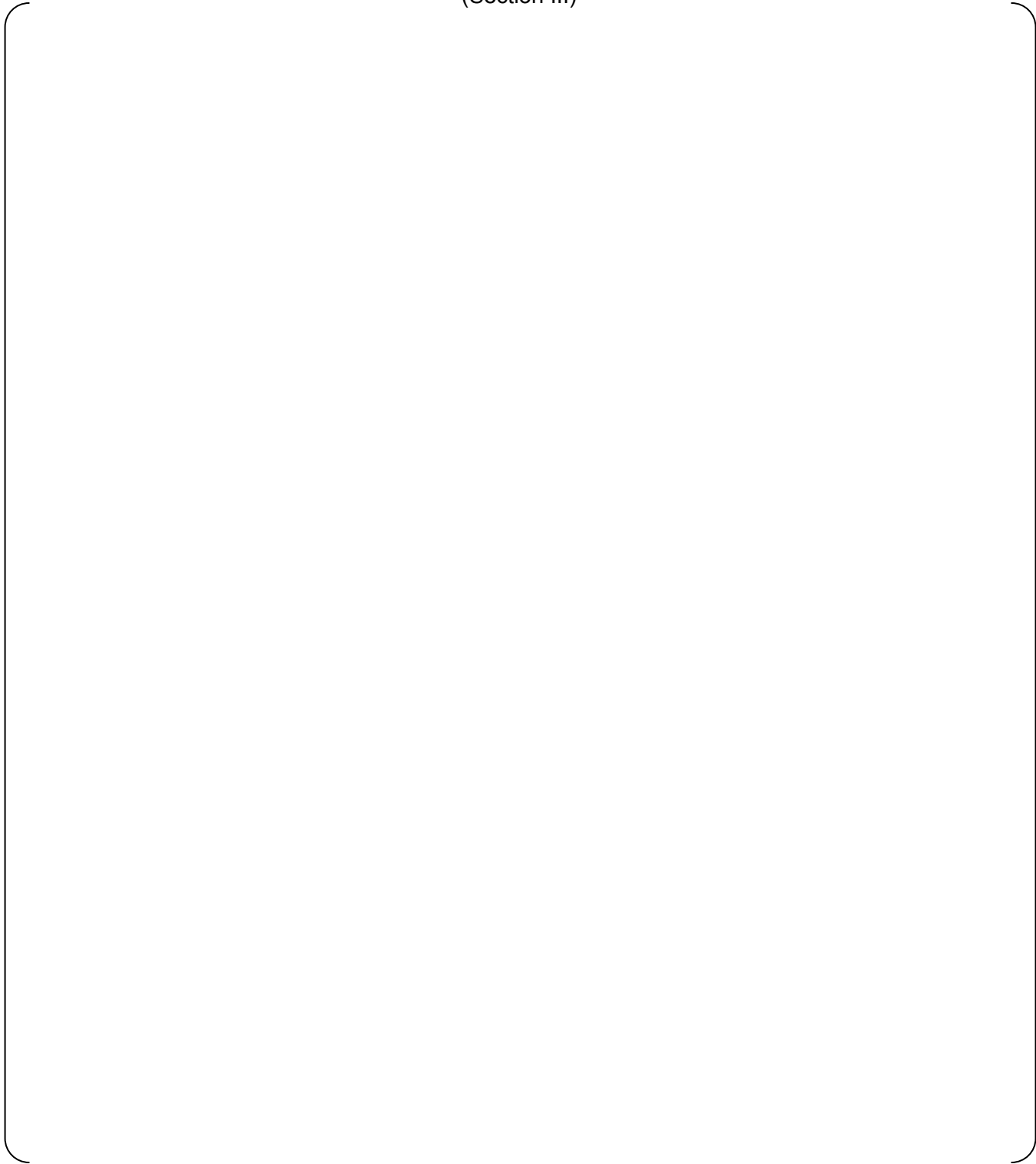
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Table A1-15-1-6 Level A, B temperature and pressure input data (6/18)  
(Section II)

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**Table A1-15-1-6 Level A, B temperature and pressure input data (7/18)**  
(Section III)

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**Table A1-15-1-6 Level A, B temperature and pressure input data (8/18)**  
(Section III)

Table A1-15-1-6 Level A, B temperature and pressure input data (9/18)  
(Section III)

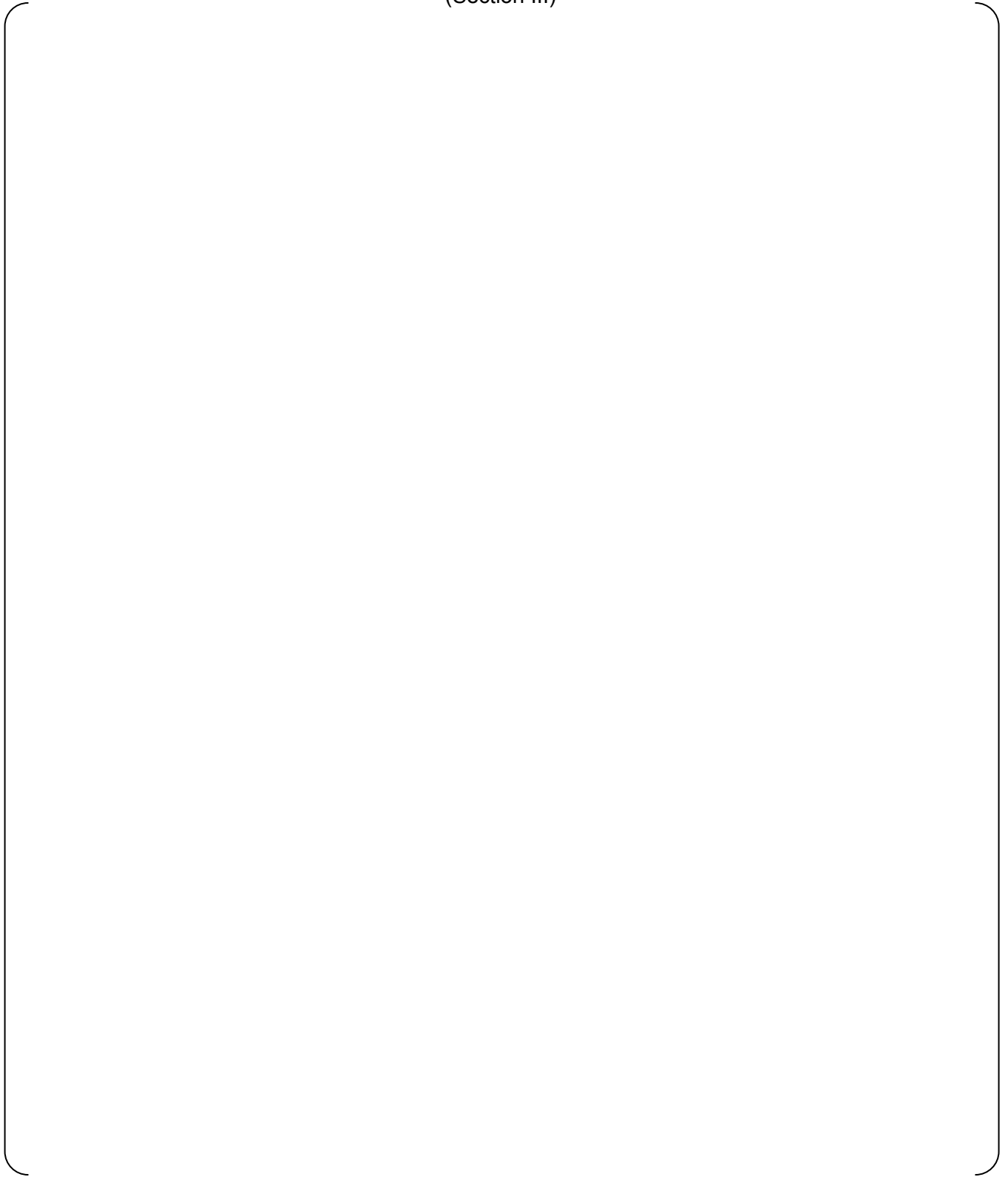
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Table A1-15-1-6 Level A, B temperature and pressure input data (10/18)  
(Section IV)

Table A1-15-1-6 Level A, B temperature and pressure input data (11/18)  
(Section IV)

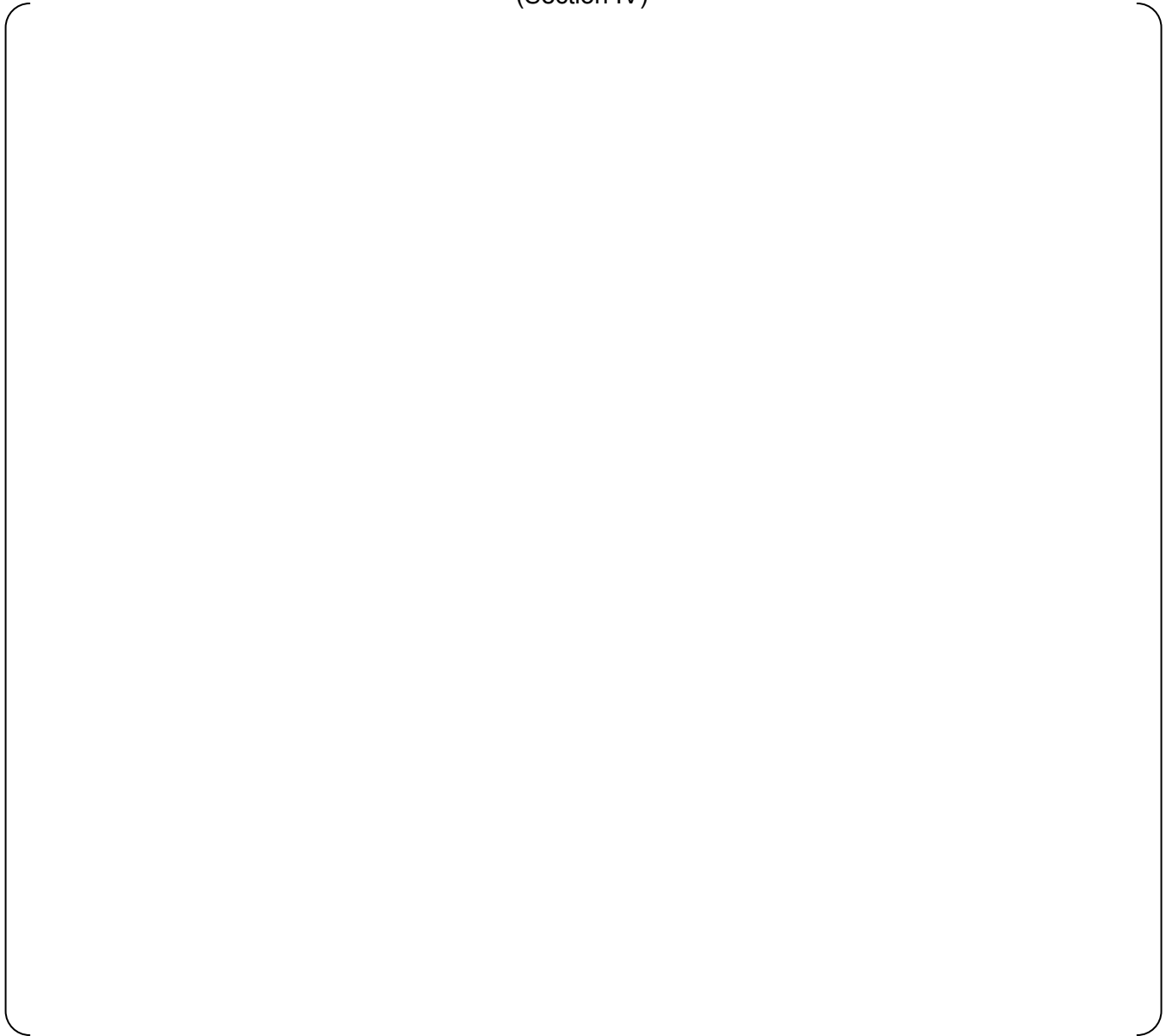


Table A1-15-1-6 Level A, B temperature and pressure input data (12/18)  
(Section IV)

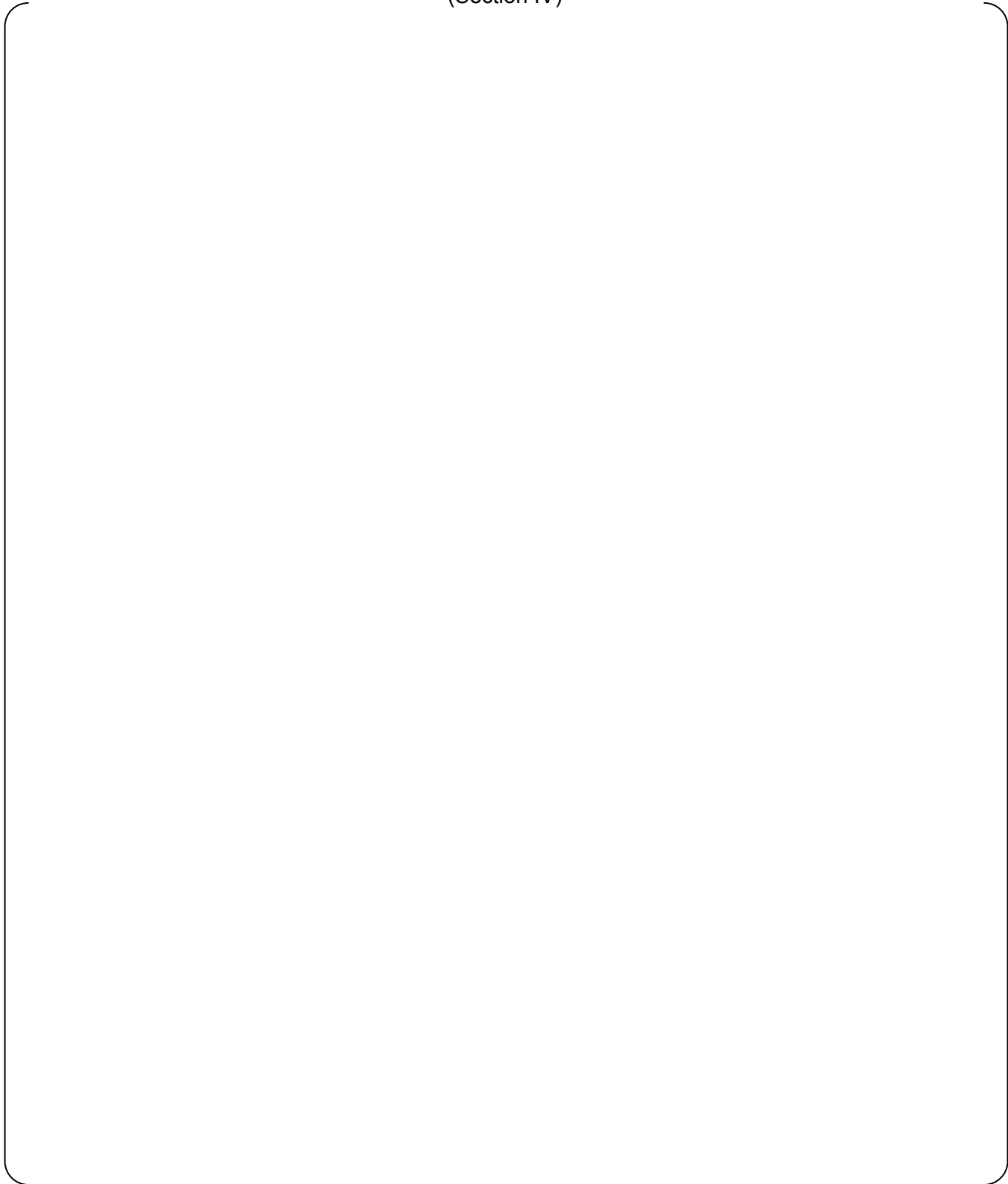
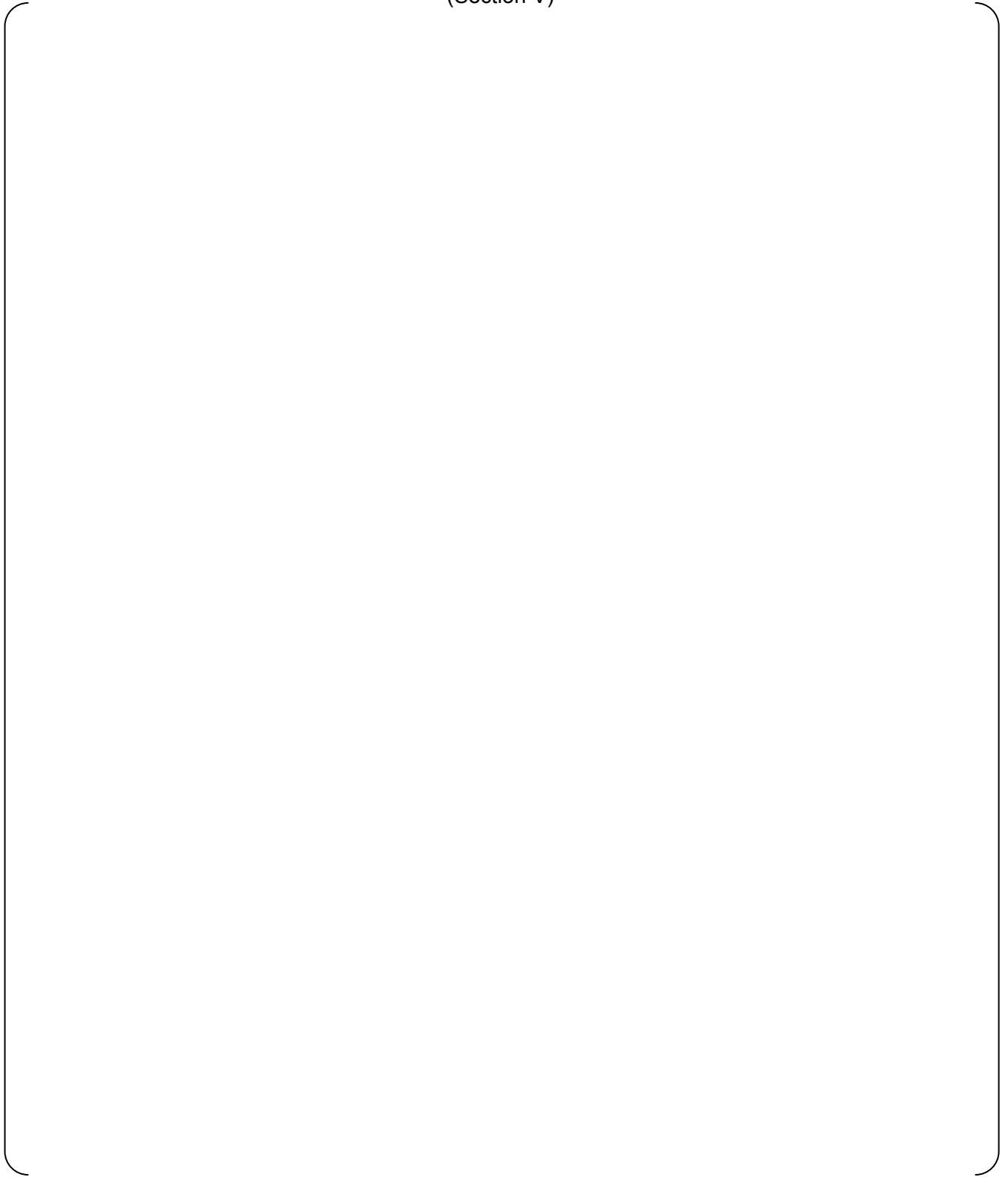
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Table A1-15-1-6 Level A, B temperature and pressure input data (13/18)  
(Section V)

Table A1-15-1-6 Level A, B temperature and pressure input data (14/18)  
(Section V)

Table A1-15-1-6 Level A, B temperature and pressure input data (15/18)  
(Section V)

A large, empty rectangular frame with rounded corners, intended for the data from Table A1-15-1-6. The frame is currently blank.

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**Table A1-15-1-6 Level A, B temperature and pressure input data (16/18)**  
(Section VI)

Table A1-15-1-6 Level A, B temperature and pressure input data (17/18)  
(Section VI)

Table A1-15-1-6 Level A, B temperature and pressure input data (18/18)  
(Section VI)

Table A1-15-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-15-1-2 Floor response curve (1/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
X (EW) direction (damping 4.0%)





**Figure A1-15-1-2 Floor response curve (2/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
Y (NS) direction (damping 4.0%)



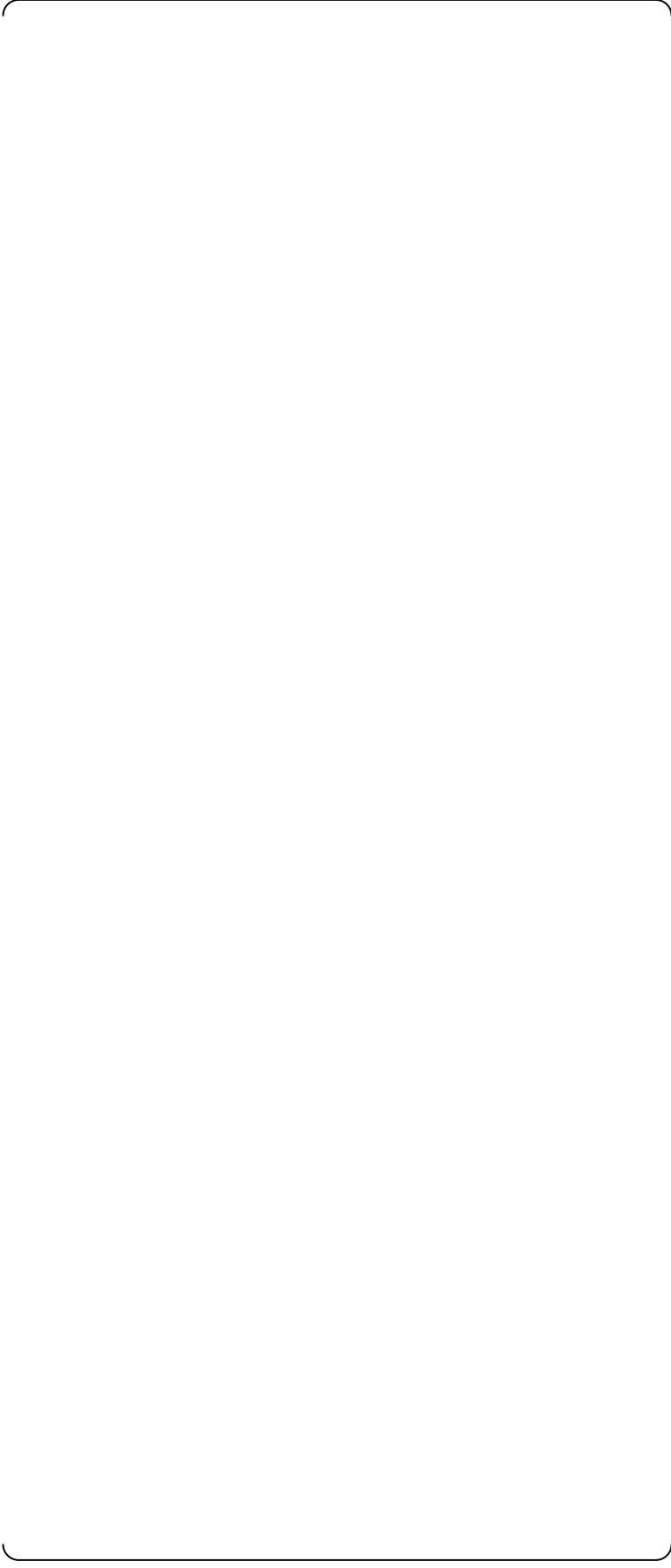
**Figure A1-15-1-2 Floor response curve (3/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-15-1-2 Floor response curve (4/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
X (EW) direction (damping 4.0%)



**Figure A1-15-1-2 Floor response curve (5/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-15-1-2 Floor response curve (6/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
Z (Vert.) direction (damping 4.0%)

Table A1-15-1-8 Seismic anchor displacement input data

Table A1-15-1-9 DBPB displacement input data

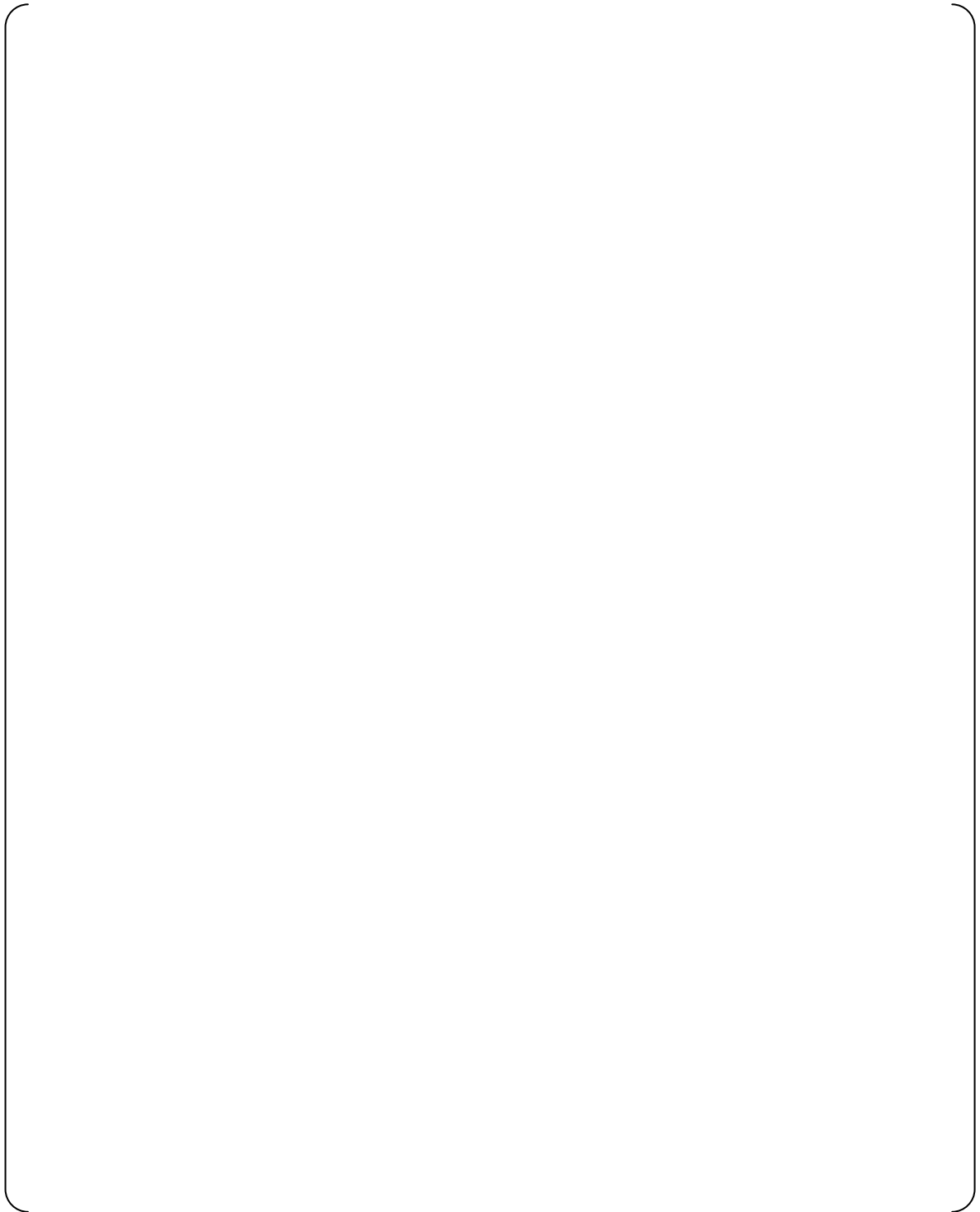


Figure A1-15-2-1 PIPESTRESS analysis model diagram



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**Table A1-15-2-1 Natural frequency analysis results**

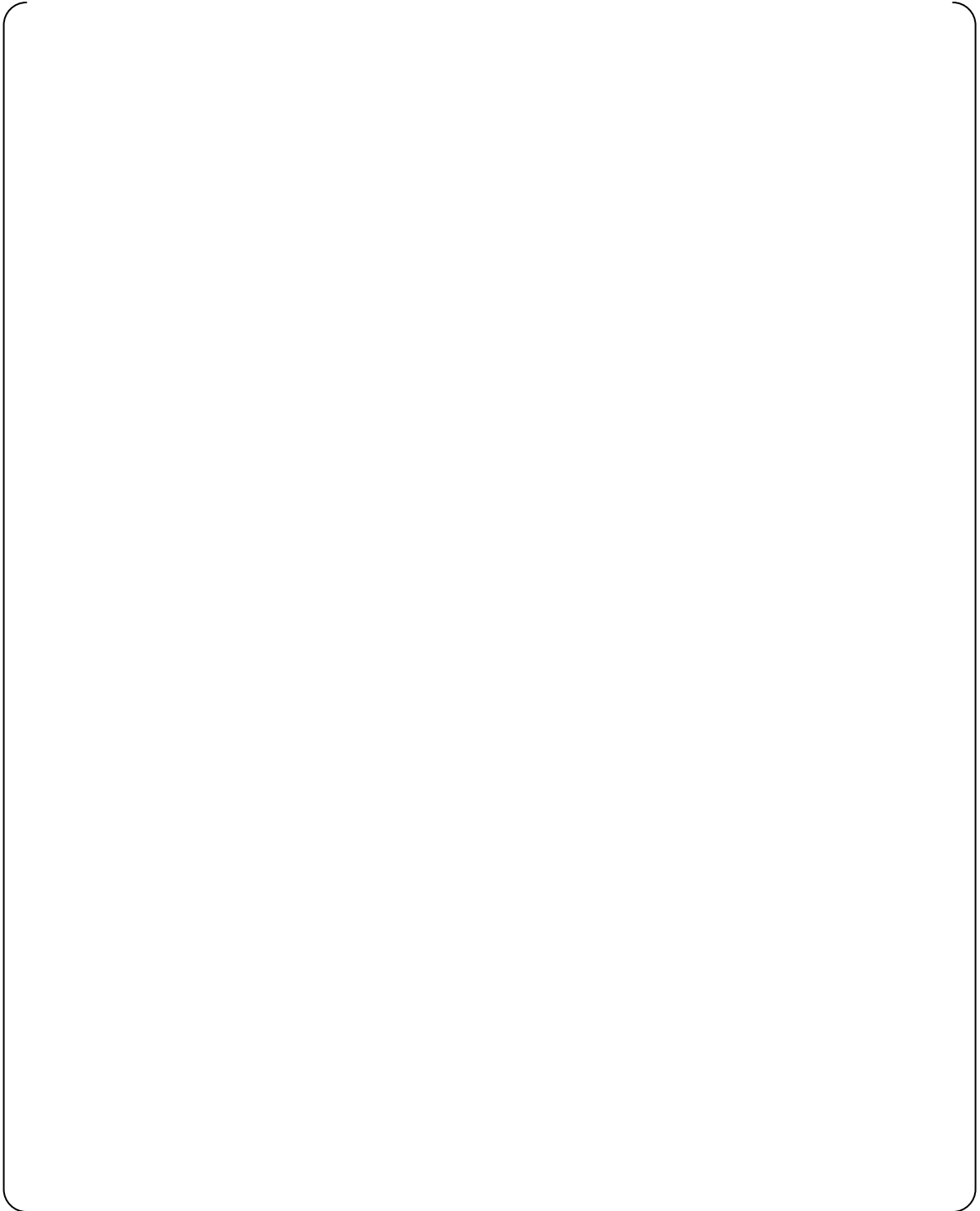


Figure A1-15-2-2 Frequency mode diagram (primary)

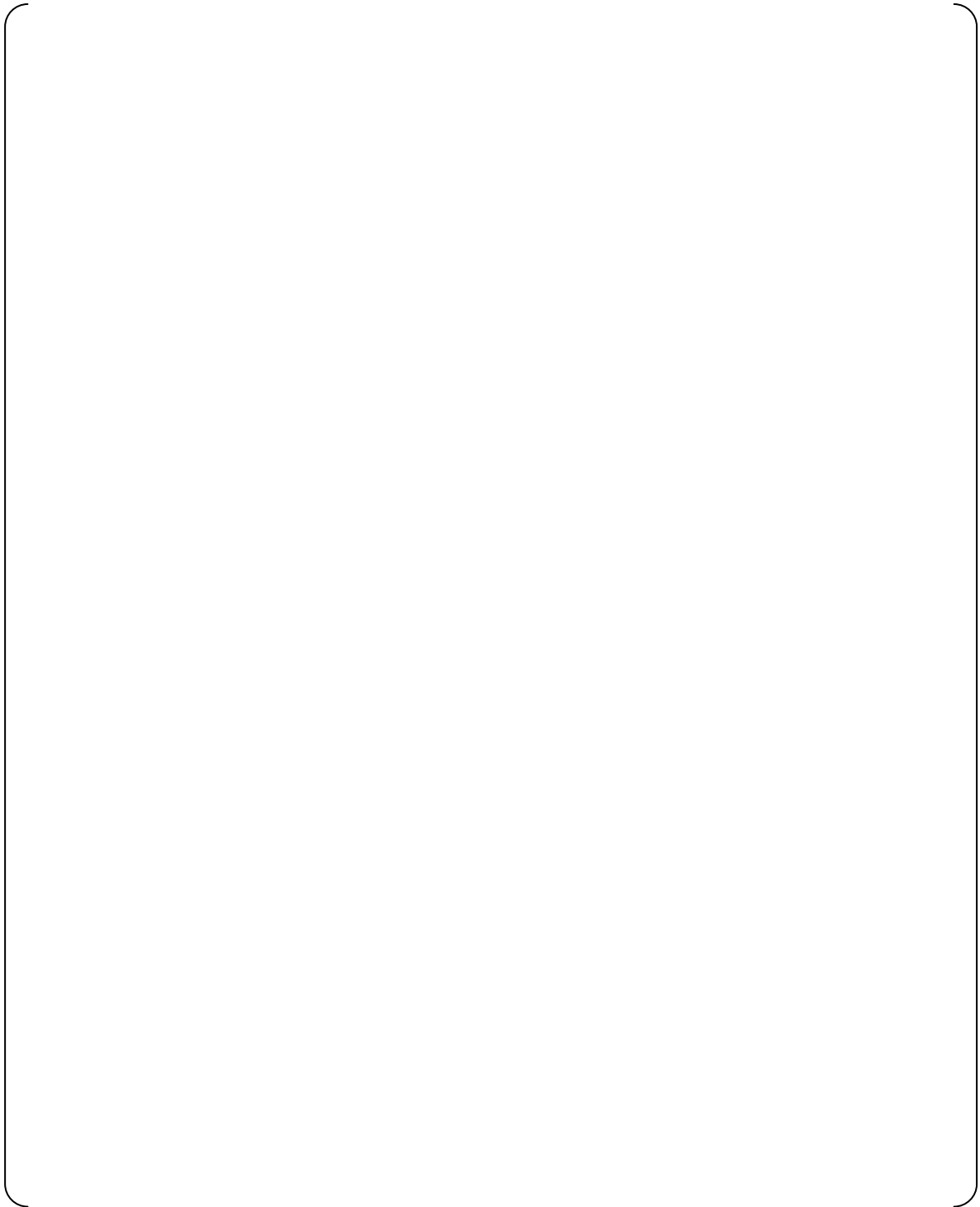


Figure A1-15-2-2 Frequency mode diagram (secondary)

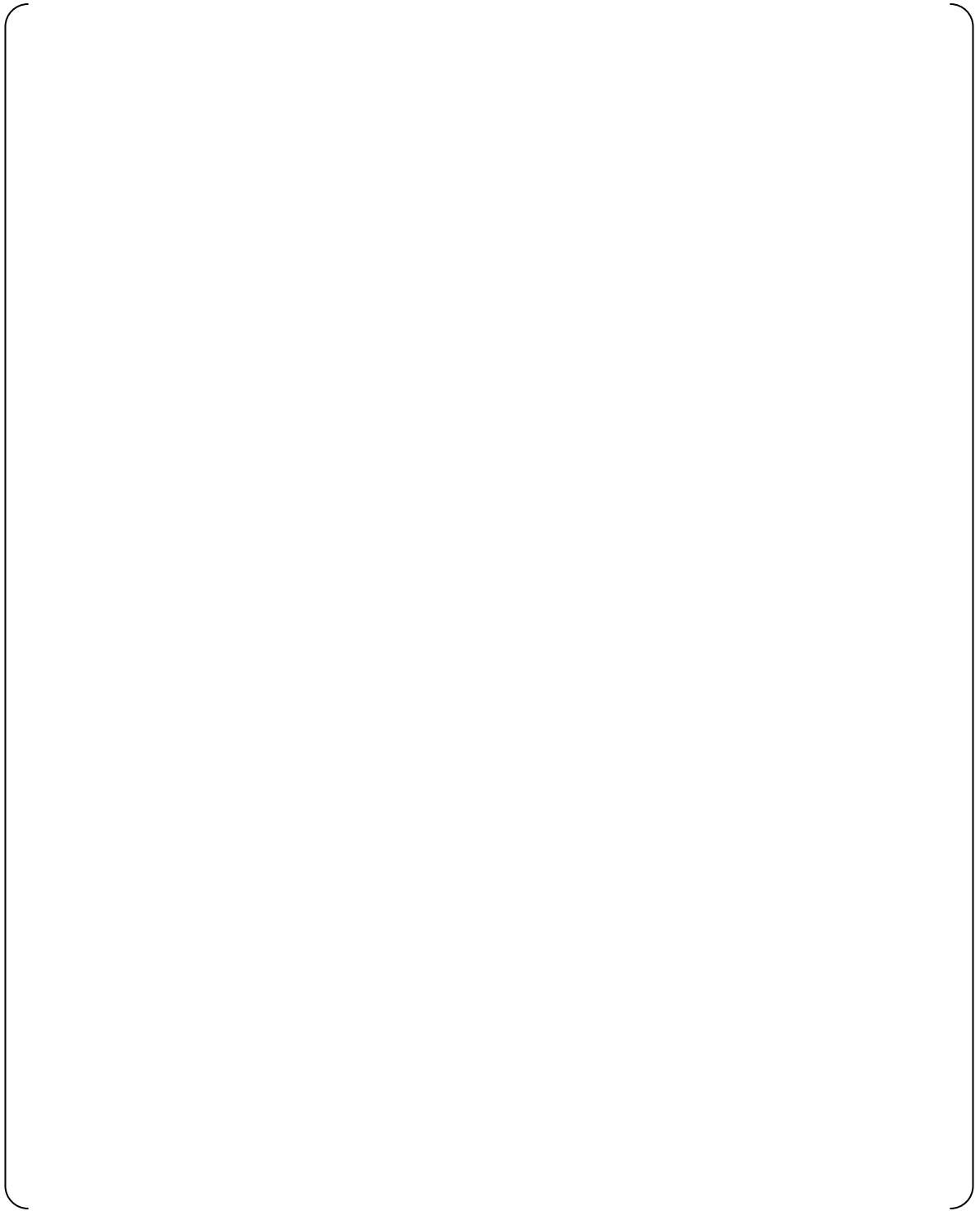


Figure A1-15-2-2 Frequency mode diagram (tertiary)

















**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-15-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (5/9)  
(Section II)

| Location | $\Delta T1$ (°F) | $\Delta T2$ (°F) | $Ta-Tb$ (°F) |
|----------|------------------|------------------|--------------|
|----------|------------------|------------------|--------------|













Table A1-15-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (8/9)  
(Section III)









**Table A1-15-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-15-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)



## Appendix 1-16

### CS05 CVCS Seal Injection B Line Piping Analysis Results

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|  |                  |
|--|------------------|
| 1. INPUT   |                  |
| 1.1 Used for creating the pipe structural model                            |                  |
| 1.1.1 Block division and piping specifications                             | Table A1-16-1-1  |
| 1.1.2 Piping isometrics  | Figure A1-16-1-1 |
| 1.1.3 Concentrated mass  | Table A1-16-1-2  |
| 1.1.4 Support point rigidity   | Table A1-16-1-3  |
| 1.1.5 Valve rigidity   | Table A1-16-1-4  |
| 1.2 Used for creating load conditions                                      |                  |
| 1.2.1 Level A/B design transient   | see main text    |
| 1.2.2 Level A/B thermal displacement input data                            | Table A1-16-1-5  |
| 1.2.3 Level A, B temperature and pressure input data                       | Table A1-16-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data               | Table A1-16-1-7  |
| 1.2.5 Floor response curve   | Figure A1-16-1-2 |
| 1.2.6 Seismic anchor displacement input data                               | Table A1-16-1-8  |
| 1.2.7 DBPB displacement input data   | Table A1-16-1-9  |
| 2. OUTPUT  |                  |
| 2.1 PIPESTRESS analysis model diagram                                      | Figure A1-16-2-1 |
| 2.2 Natural frequency analysis results                                     | Table A1-16-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                           | Figure A1-16-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a - T_b$ ) | Table A1-16-2-2  |
| 2.5 Piping stress and fatigue evaluation results                           | Table A1-16-2-3  |

Table A1-16-1-1 Block division and piping specifications



US-APWR CS05  
CVCS Seal Injection B Line  
Figure A1-16-1-1 Piping Isometrics

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Table A1-16-1-2 Concentrated mass

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Table A1-16-1-3 Support point rigidity



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Table A1-16-1-4 Valve rigidity



Table A1-16-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9100)

| Point | Level A | Level B |
|-------|---------|---------|
| 9100  |         |         |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-16-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9100)

| Point | Level A | Level B |
|-------|---------|---------|
| 9100  |         |         |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-16-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9100)

| Point | Level A | Level B |
|-------|---------|---------|
| 9100  |         |         |

---

Table A1-16-1-6 Level A, B temperature and pressure input data (1/18)  
(Section I)

Table A1-14-1-6 Level A, B temperature and pressure input data (2/18)  
(Section I)

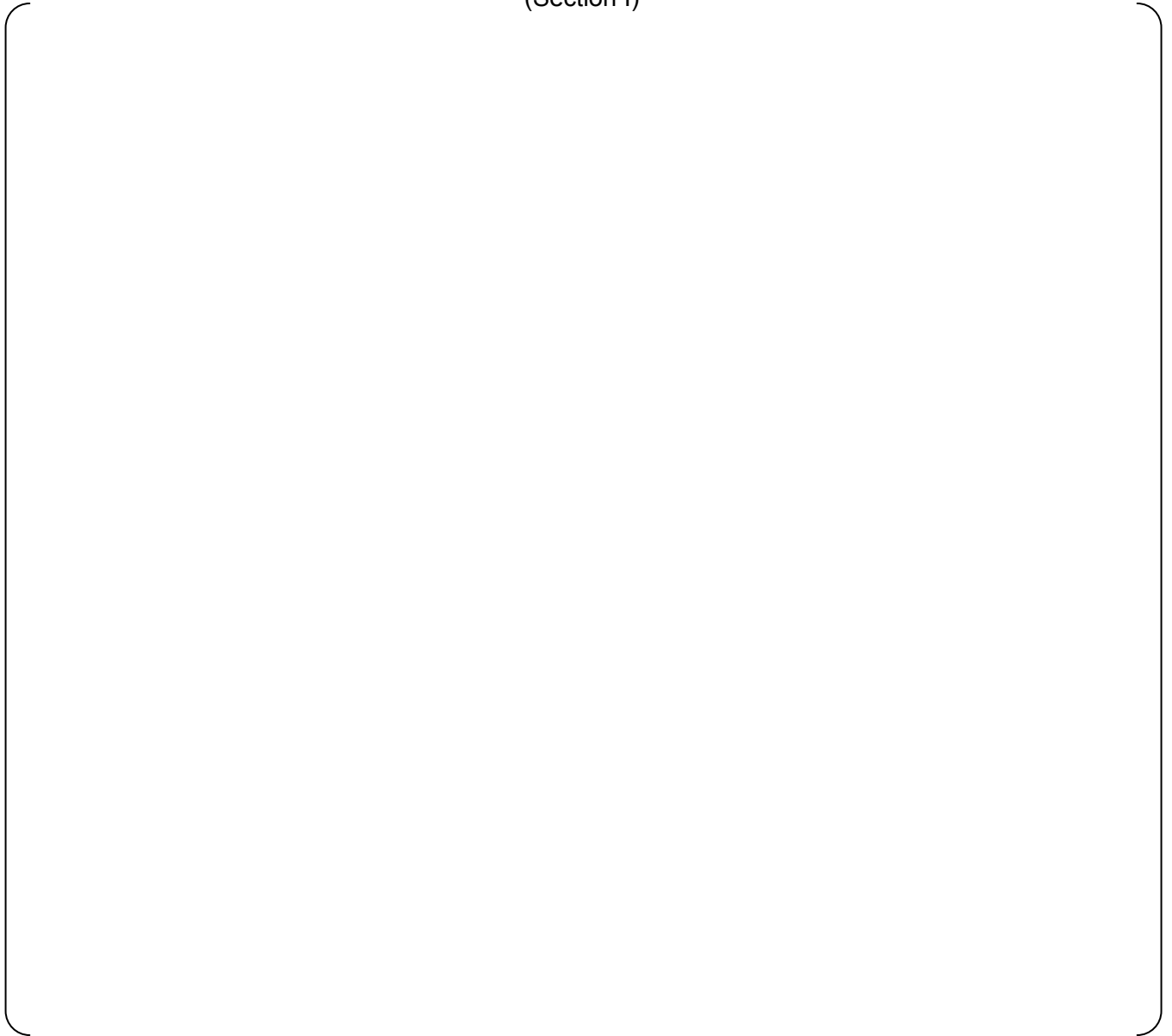


Table A1-16-1-6 Level A, B temperature and pressure input data (3/18)  
(Section I)

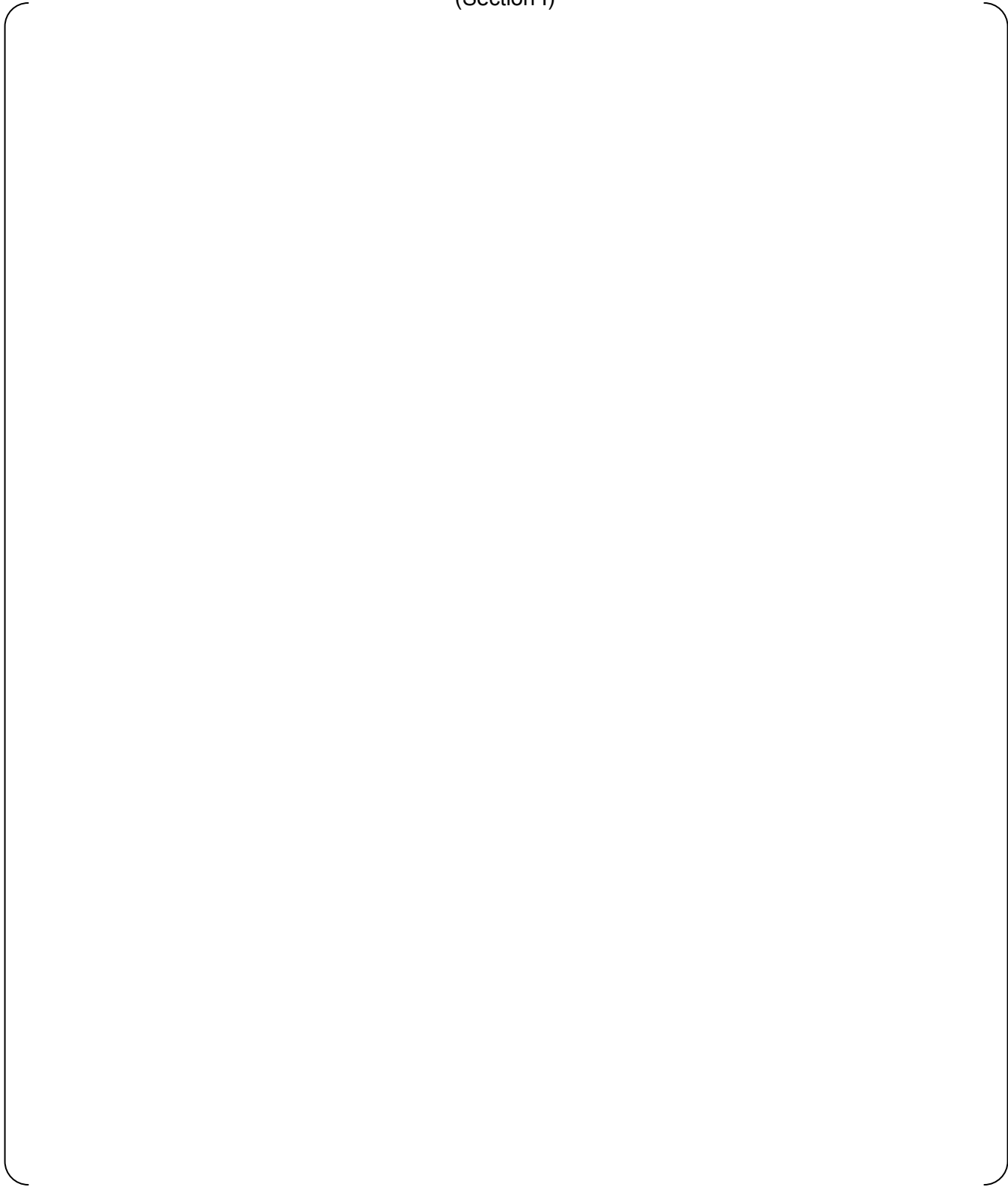




Table A1-16-1-6 Level A, B temperature and pressure input data (4/18)  
(Section II)

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Table A1-16-1-6 Level A, B temperature and pressure input data (5/18)  
(Section II)

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Table A1-16-1-6 Level A, B temperature and pressure input data (6/18)  
(Section II)

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Table A1-16-1-6 Level A, B temperature and pressure input data (7/18)  
(Section III)

Table A1-16-1-6 Level A, B temperature and pressure input data (8/18)  
(Section III)

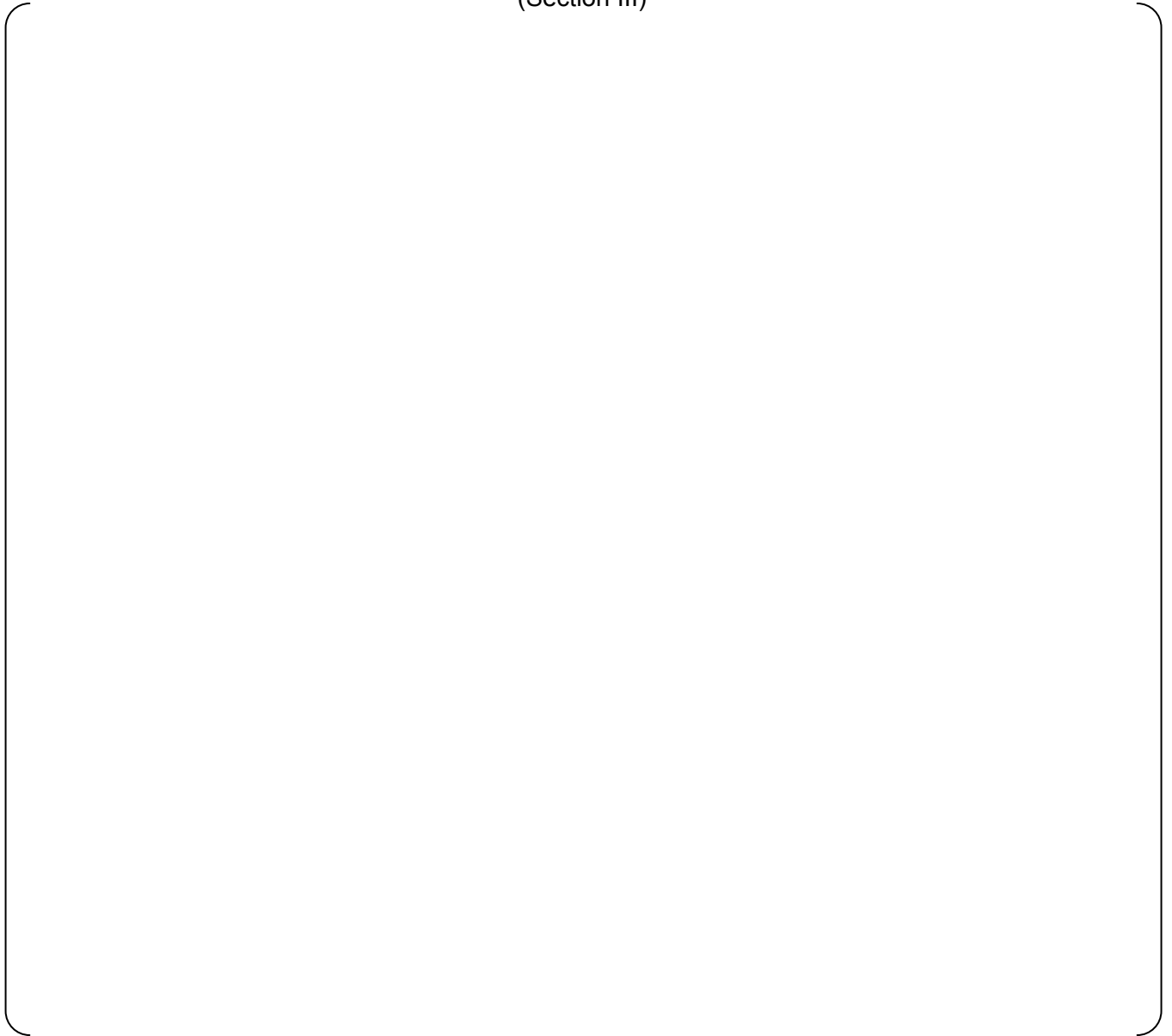
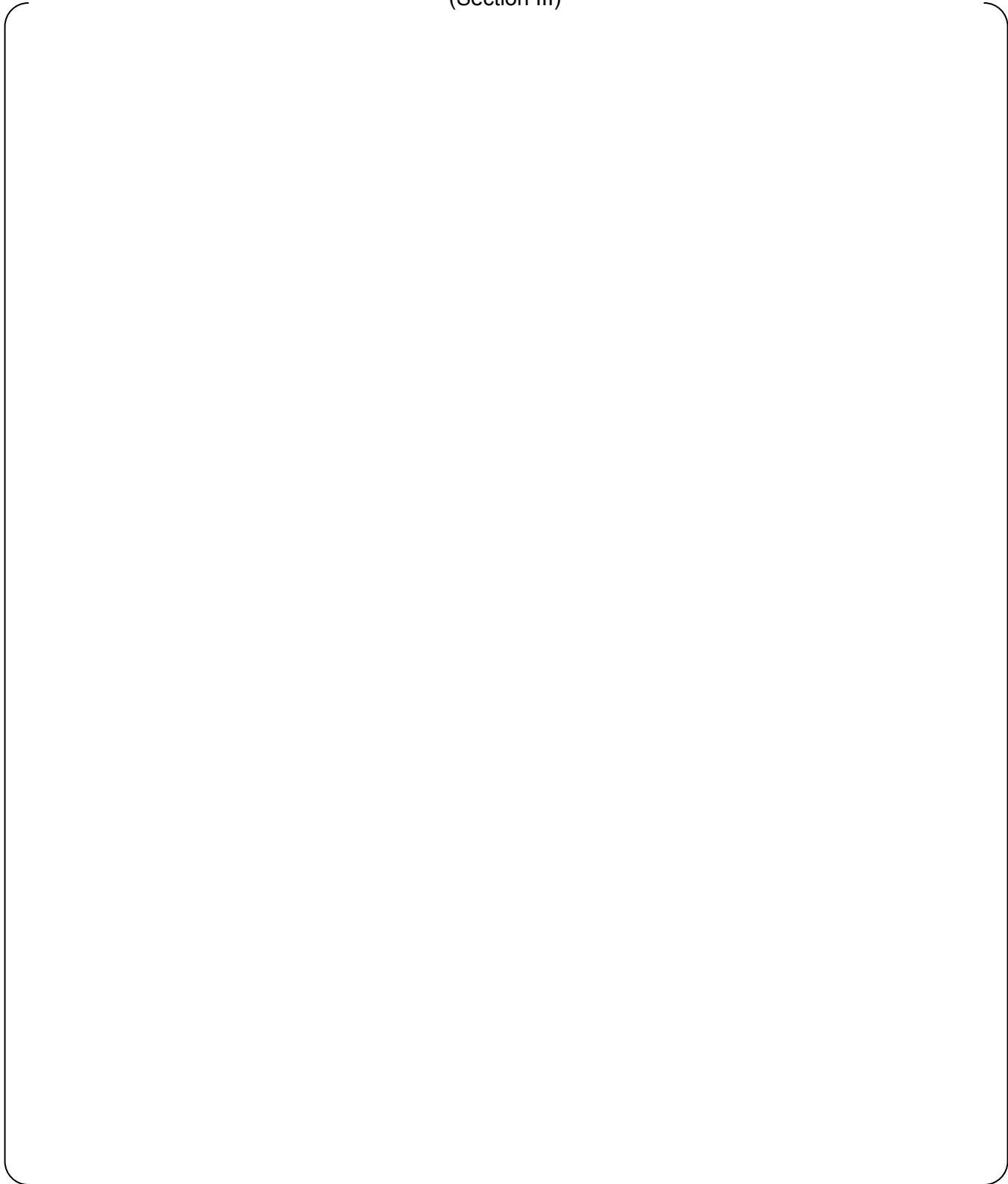
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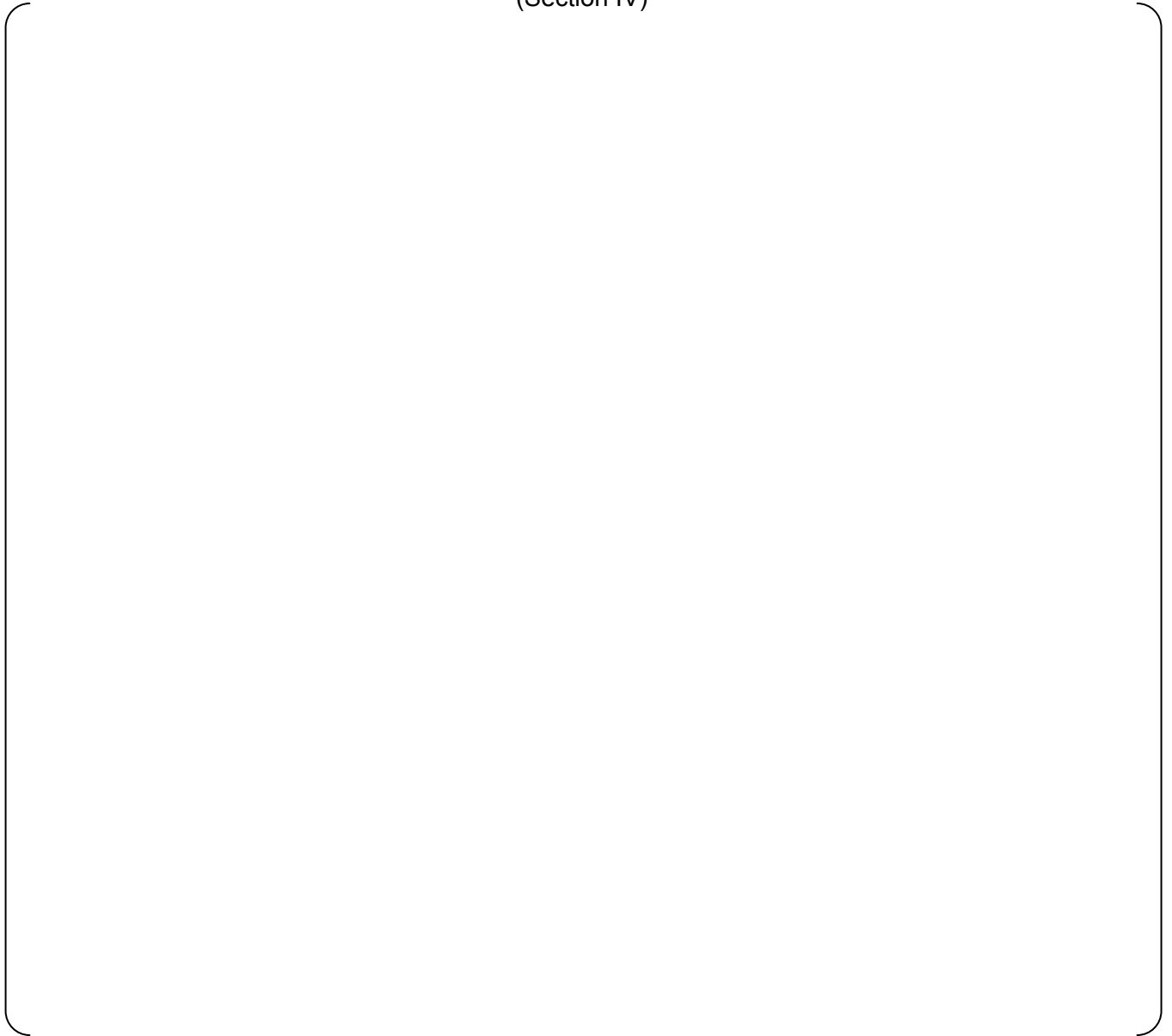
Table A1-16-1-6 Level A, B temperature and pressure input data (9/18)  
(Section III)



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**Table A1-16-1-6 Level A, B temperature and pressure input data (10/18)**  
(Section IV)

Table A1-16-1-6 Level A, B temperature and pressure input data (11/18)  
(Section IV)

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**Table A1-16-1-6 Level A, B temperature and pressure input data (12/18)**  
(Section IV)

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Table A1-16-1-6 Level A, B temperature and pressure input data (13/18)  
(Section V)

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Table A1-16-1-6 Level A, B temperature and pressure input data (14/18)  
(Section V)

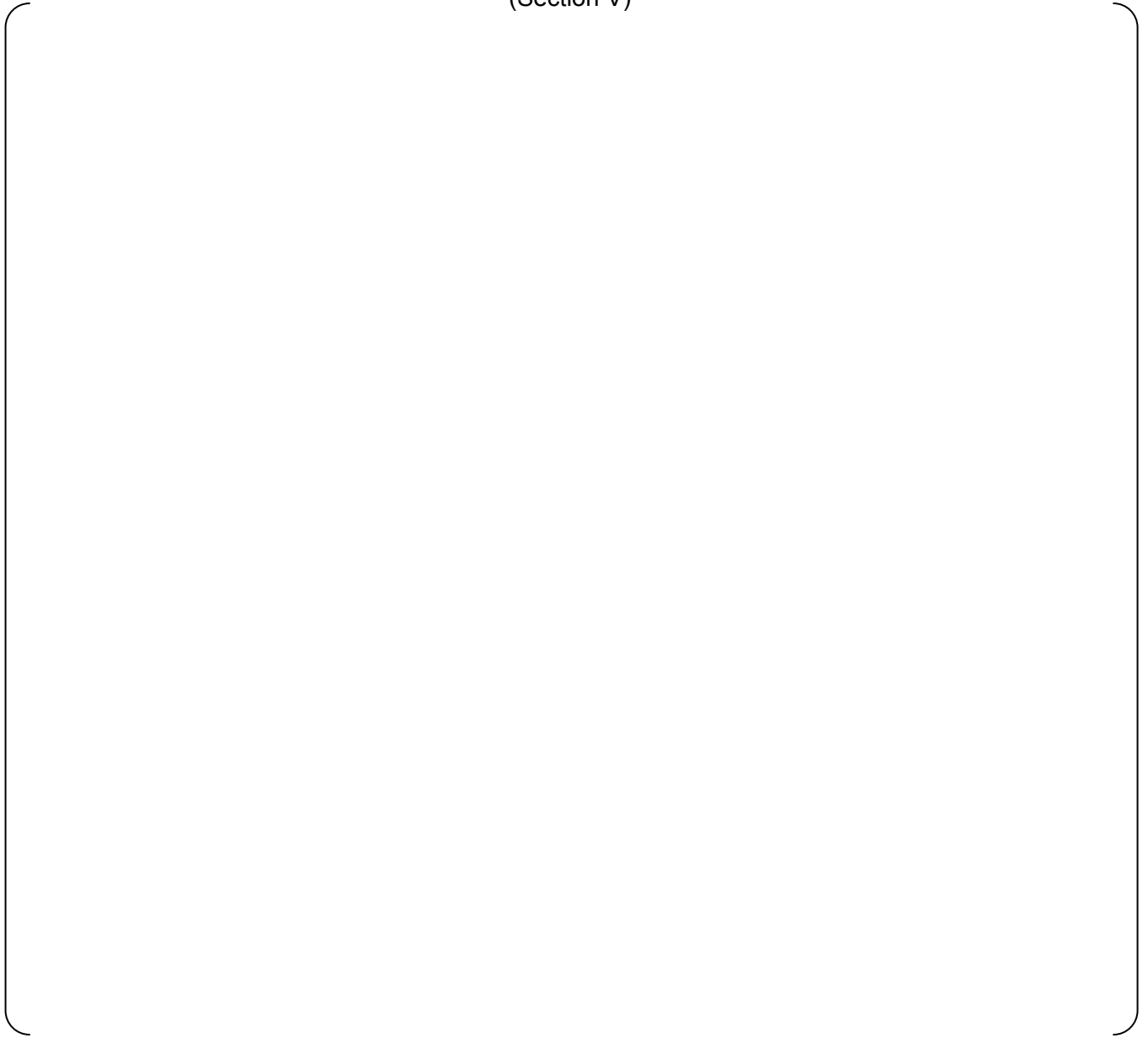


Table A1-16-1-6 Level A, B temperature and pressure input data (15/18)  
(Section V)

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**Table A1-16-1-6 Level A, B temperature and pressure input data (16/18)**  
(Section VI)

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Table A1-16-1-6 Level A, B temperature and pressure input data (17/18)  
(Section VI)

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**Table A1-16-1-6 Level A, B temperature and pressure input data (18/18)**  
(Section VI)

Table A1-16-1-7 Level C, D maximum temperature and pressure input data





**Figure A1-16-1-2 Floor response curve (1/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-16-1-2 Floor response curve (2/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
Y (NS) direction (damping 4.0%)



**Figure A1-16-1-2 Floor response curve (3/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-16-1-2 Floor response curve (4/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
X (EW) direction (damping 4.0%)



**Figure A1-16-1-2 Floor response curve (5/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-16-1-2 Floor response curve (6/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
Z (Vert.) direction (damping 4.0%)

Table A1-16-1-8 Seismic anchor displacement input data

Table A1-16-1-9 DBPB displacement input data



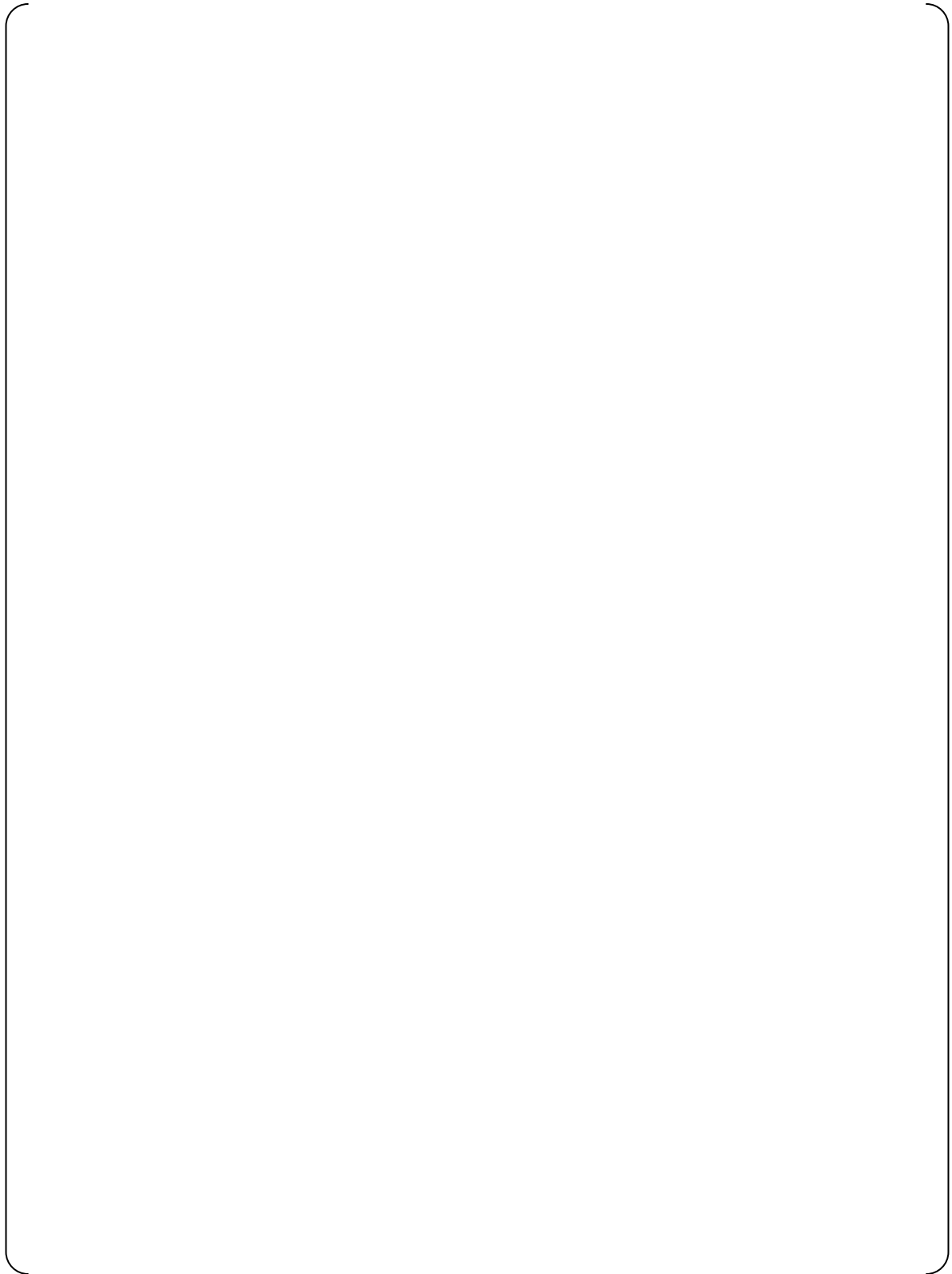


Figure A1-16-2-1 PIPESTRESS analysis model diagram

Table A1-16-2-1 Natural frequency analysis results

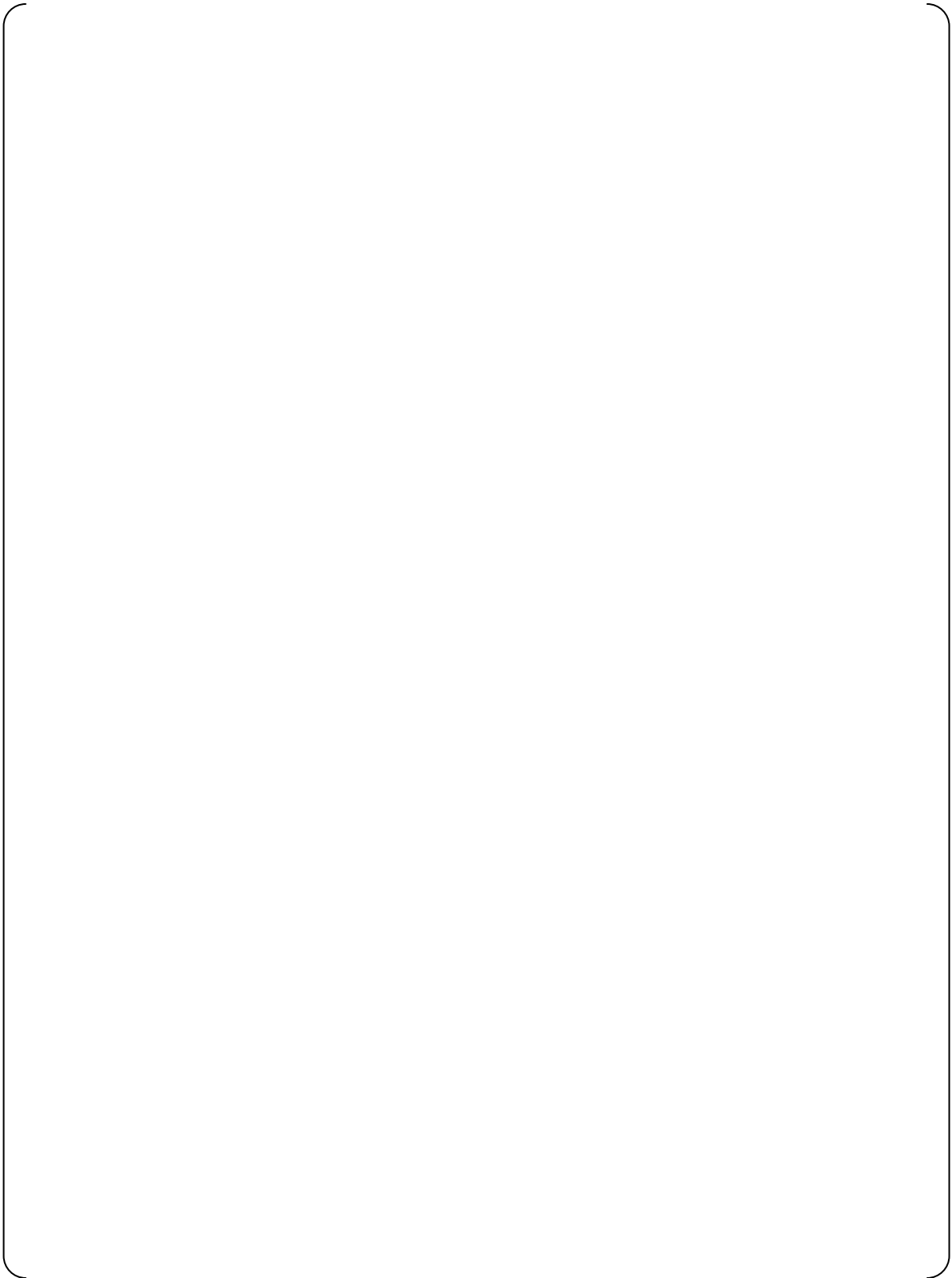


Figure A1-16-2-2 Frequency mode diagram (primary)

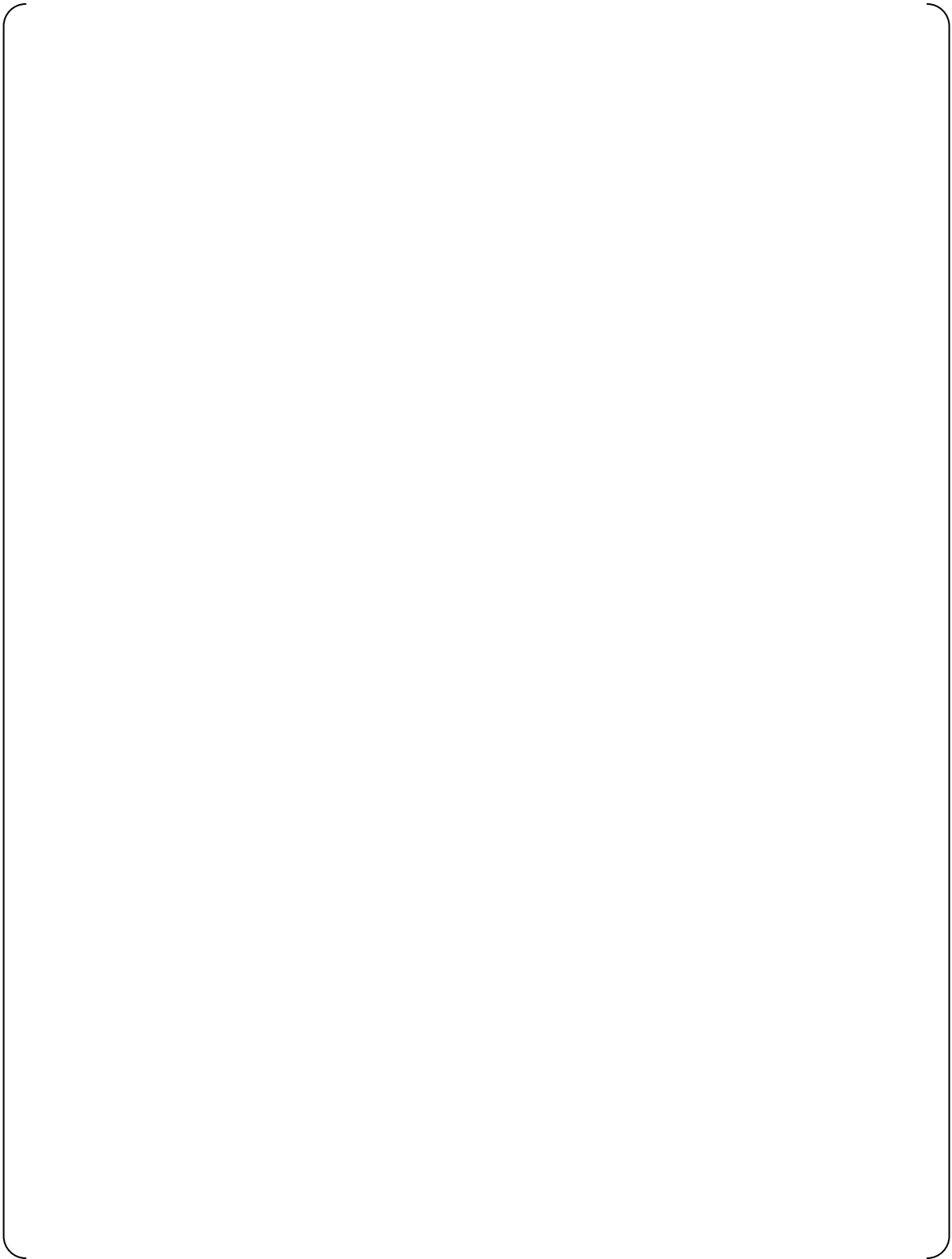


Figure A1-16-2-2 Frequency mode diagram (secondary)

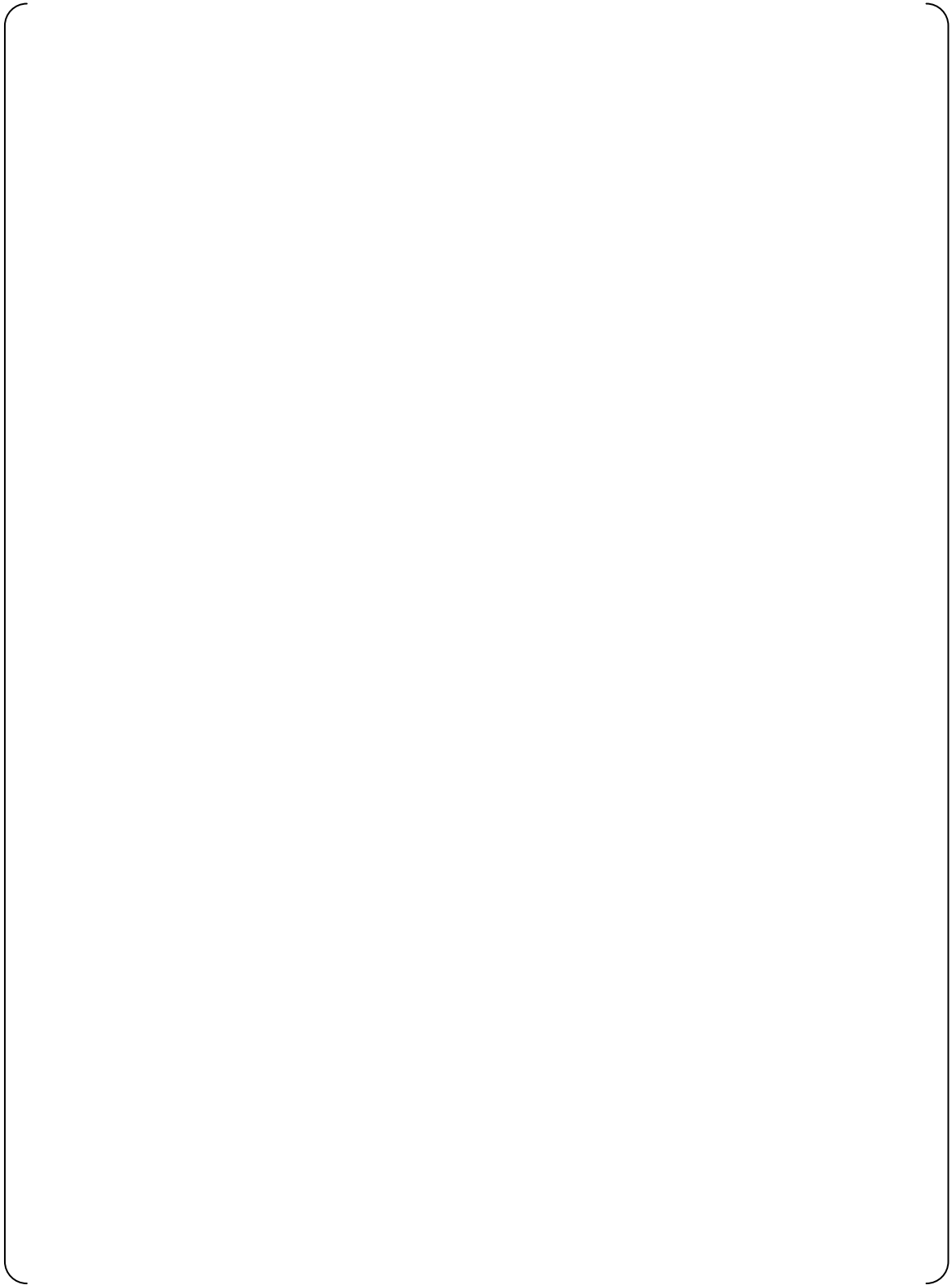


Figure A1-16-2-2 Frequency mode diagram (tertiary)









**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







Table A1-16-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (5/9)  
(Section II)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-16-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (7/9)  
(Section III)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-16-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (9/9)  
(Section III)

| Location | $\Delta T1$ | $\Delta T2$ | Ta-Tb |
|----------|-------------|-------------|-------|
|----------|-------------|-------------|-------|

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



**Table A1-16-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)



**Table A1-16-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

## Appendix 1-17

### CS06 CVCS Seal Injection C Line Piping Analysis Results

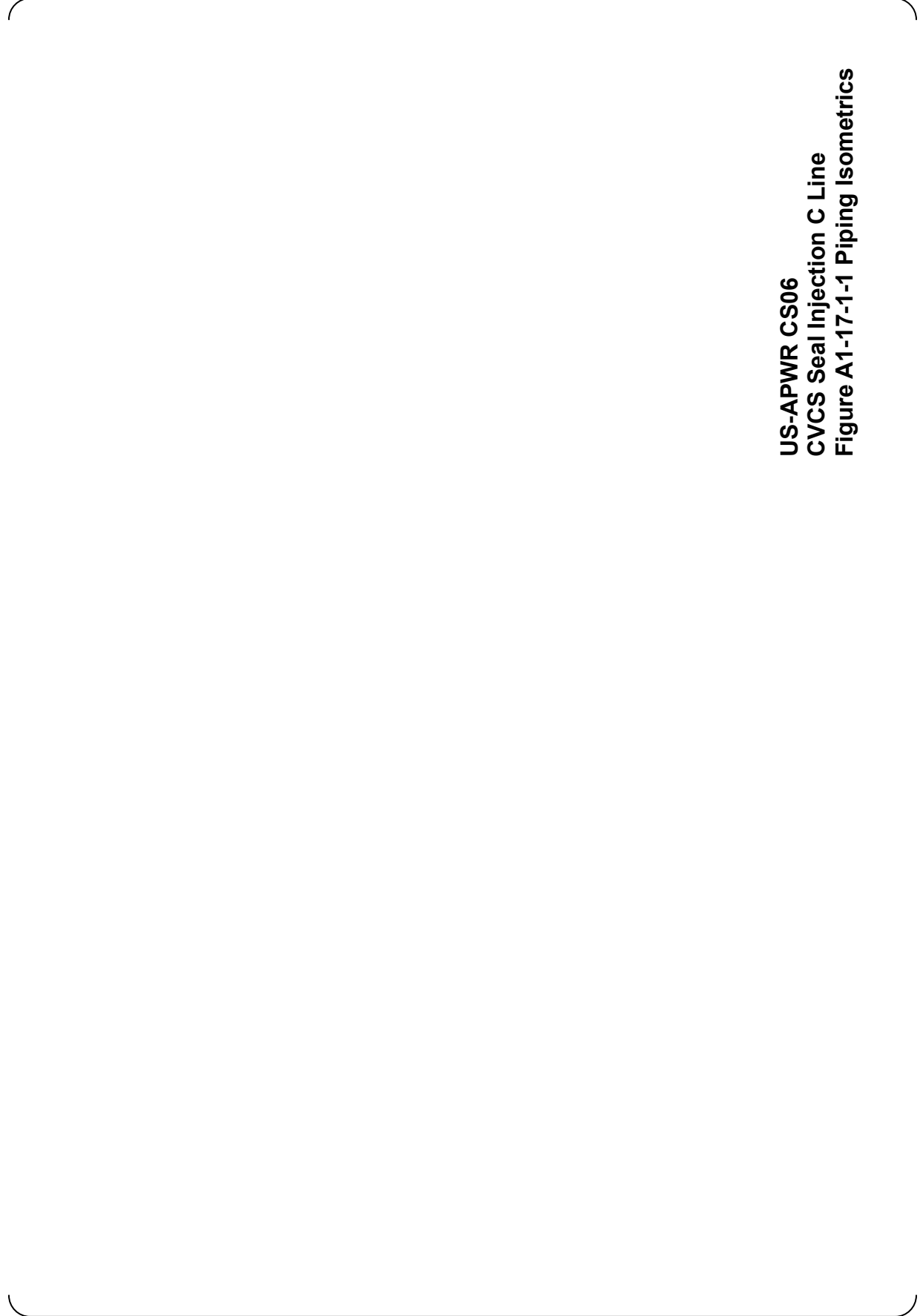
1. INPUT

|  |                  |
|--|------------------|
| 1.1 Used for creating the pipe structural model              |                  |
| 1.1.1 Block division and piping specifications               | Table A1-17-1-1  |
| 1.1.2 Piping isometrics                                      | Figure A1-17-1-1 |
| 1.1.3 Concentrated mass                                      | Table A1-17-1-2  |
| 1.1.4 Support point rigidity                                 | Table A1-17-1-3  |
| 1.1.5 Valve rigidity   | Table A1-17-1-4  |
| 1.2 Used for creating load conditions                        |                  |
| 1.2.1 Level A/B design transient                             | see main text    |
| 1.2.2 Level A/B thermal displacement input data              | Table A1-17-1-5  |
| 1.2.3 Level A, B temperature and pressure input data         | Table A1-17-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data | Table A1-17-1-7  |
| 1.2.5 Floor response curve                                   | Figure A1-17-1-2 |
| 1.2.6 Seismic anchor displacement input data                 | Table A1-17-1-8  |
| 1.2.7 DBPB displacement input data                           | Table A1-17-1-9  |

2. OUTPUT

|  |                  |
|--|------------------|
| 2.1 PIPESTRESS analysis model diagram  | Figure A1-17-2-1 |
| 2.2 Natural frequency analysis results                                       | Table A1-17-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                             | Figure A1-17-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a$ - $T_b$ ) | Table A1-17-2-2  |
| 2.5 Piping stress and fatigue evaluation results                             | Table A1-17-2-3  |

Table A1-17-1-1 Block division and piping specifications



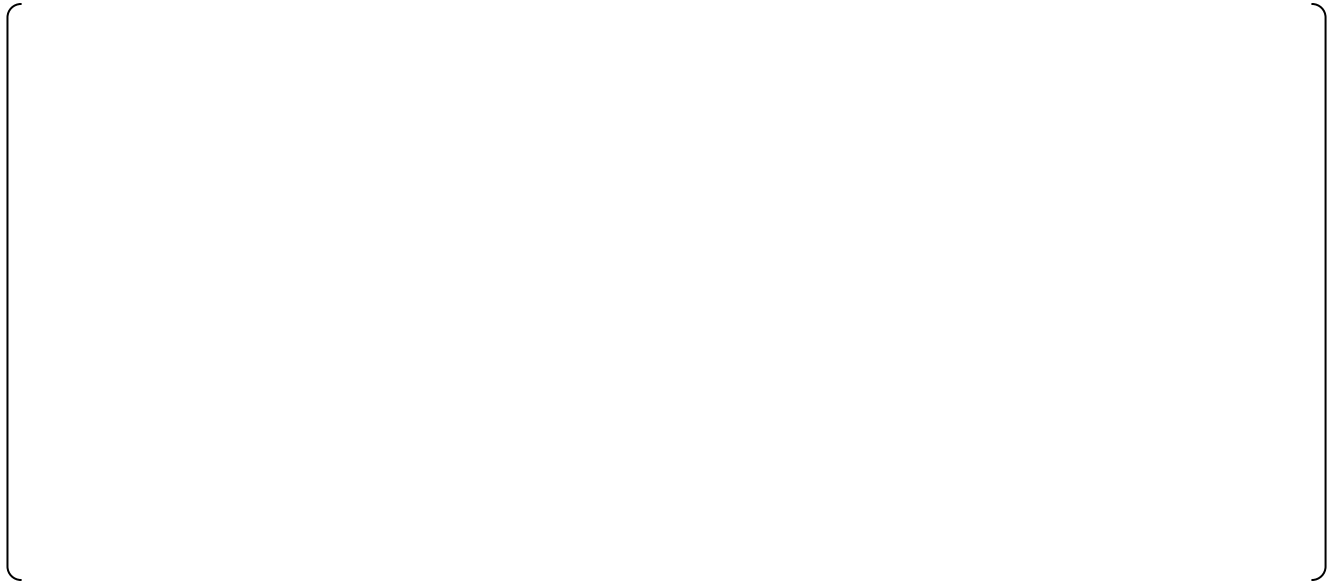
US-APWR CS06  
CVCS Seal Injection C Line  
Figure A1-17-1-1 Piping Isometrics

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Table A1-17-1-2 Concentrated mass

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Table A1-17-1-3 Support point rigidity





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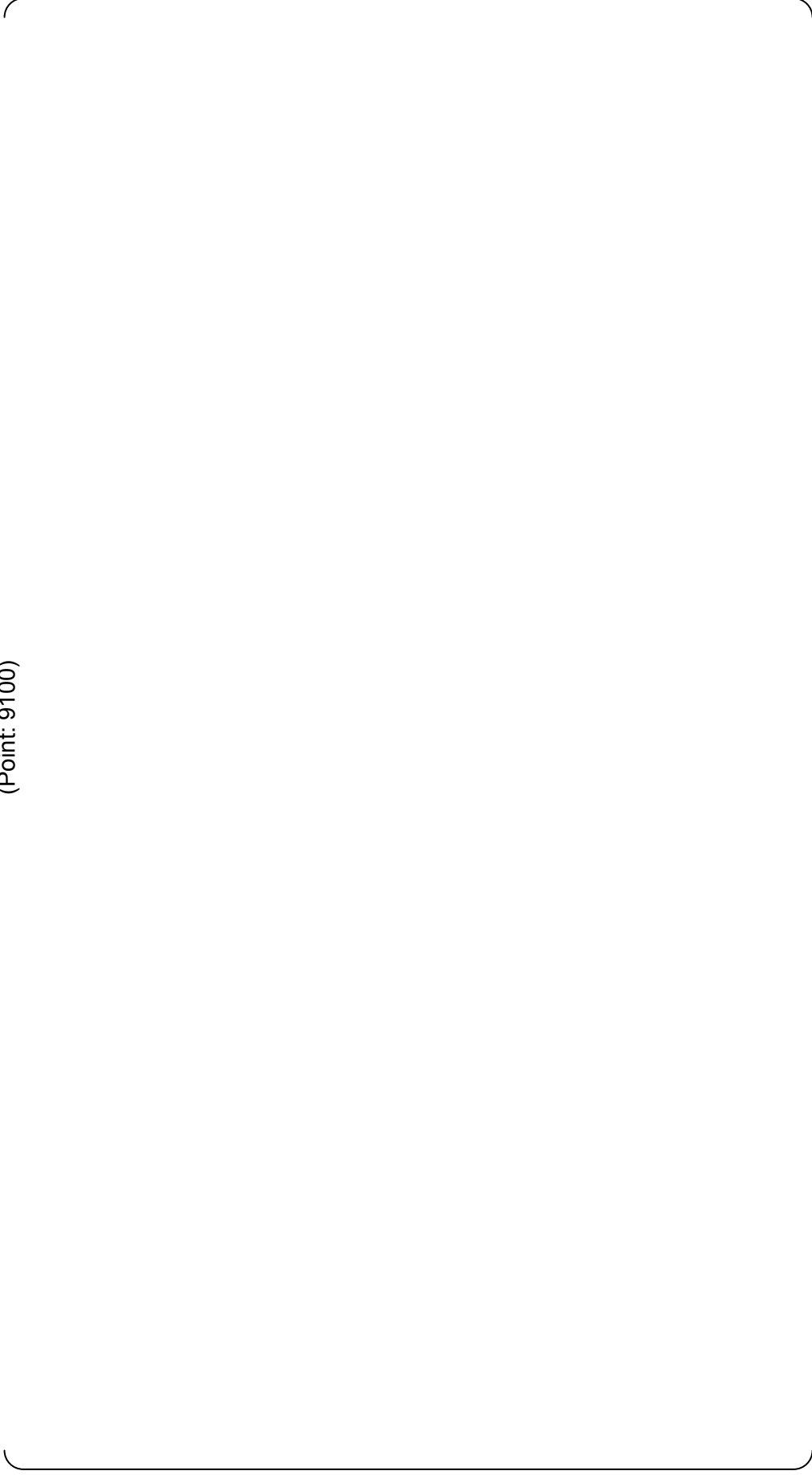
Table A1-17-1-4 Valve rigidity

Table A1-17-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9100)

| Point | Level A/B | Thermal Displacement Input Data       |
|-------|-----------|---------------------------------------|
| 9100  | Level A/B | Thermal displacement input data (1/3) |



Table A1-17-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9100)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-17-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9100)



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Table A1-17-1-6 Level A, B temperature and pressure input data (1/18)  
(Section I)

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Table A1-17-1-6 Level A, B temperature and pressure input data (2/18)  
(Section I)



Table A1-17-1-6 Level A, B temperature and pressure input data (3/18)  
(Section I)

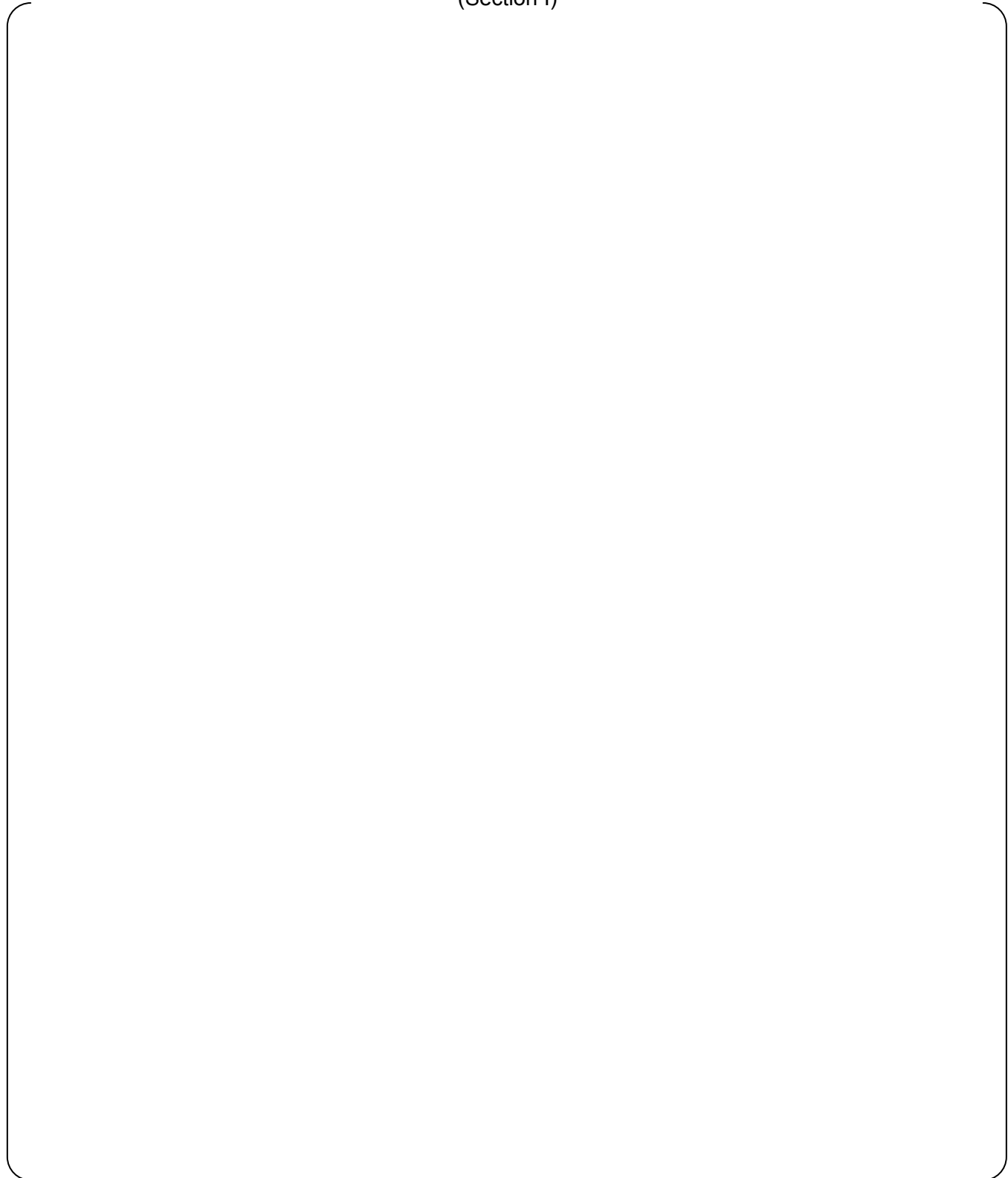


Table A1-17-1-6 Level A, B temperature and pressure input data (4/18)  
(Section II)

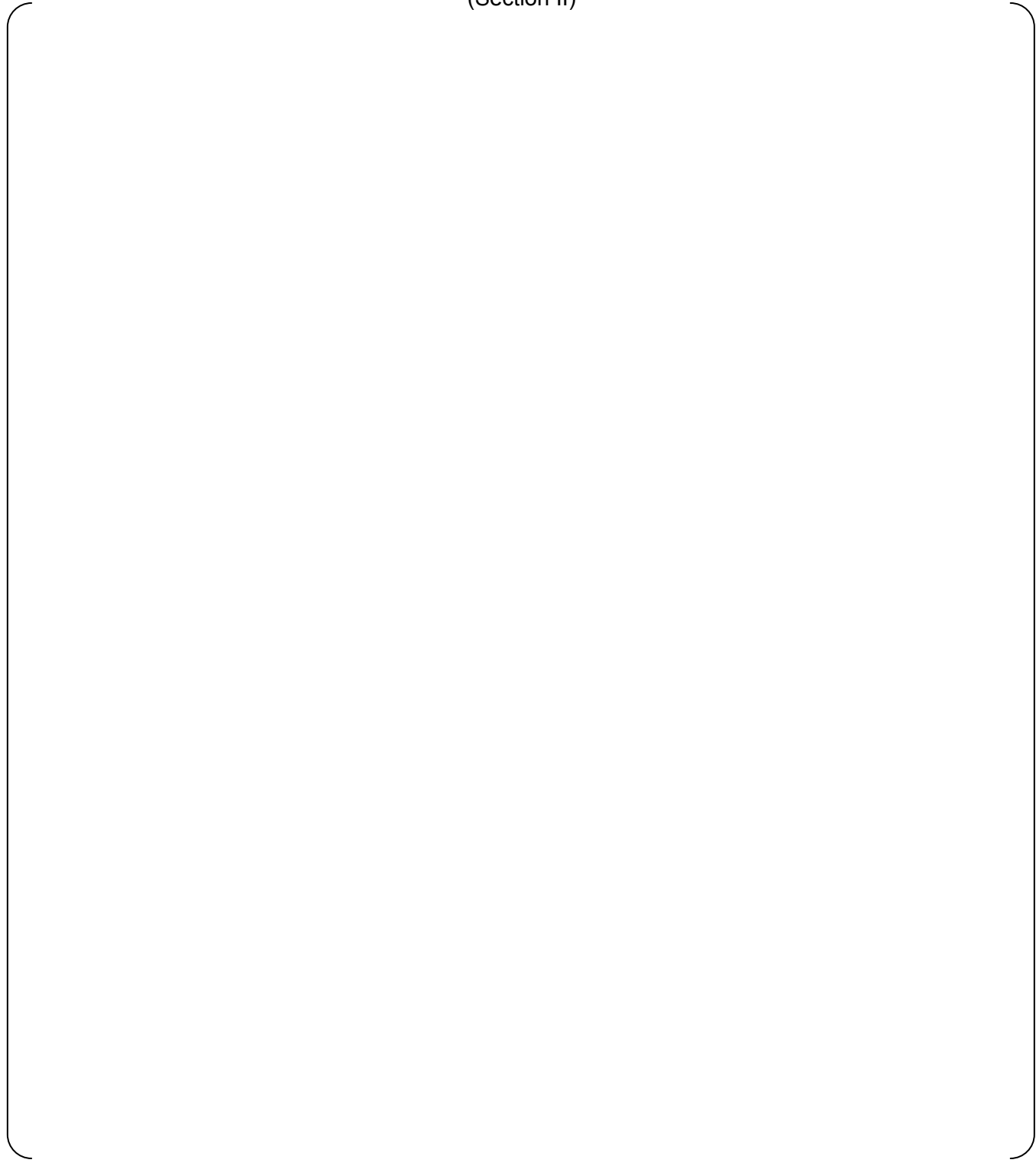


Table A1-17-1-6 Level A, B temperature and pressure input data (5/18)  
(Section II)

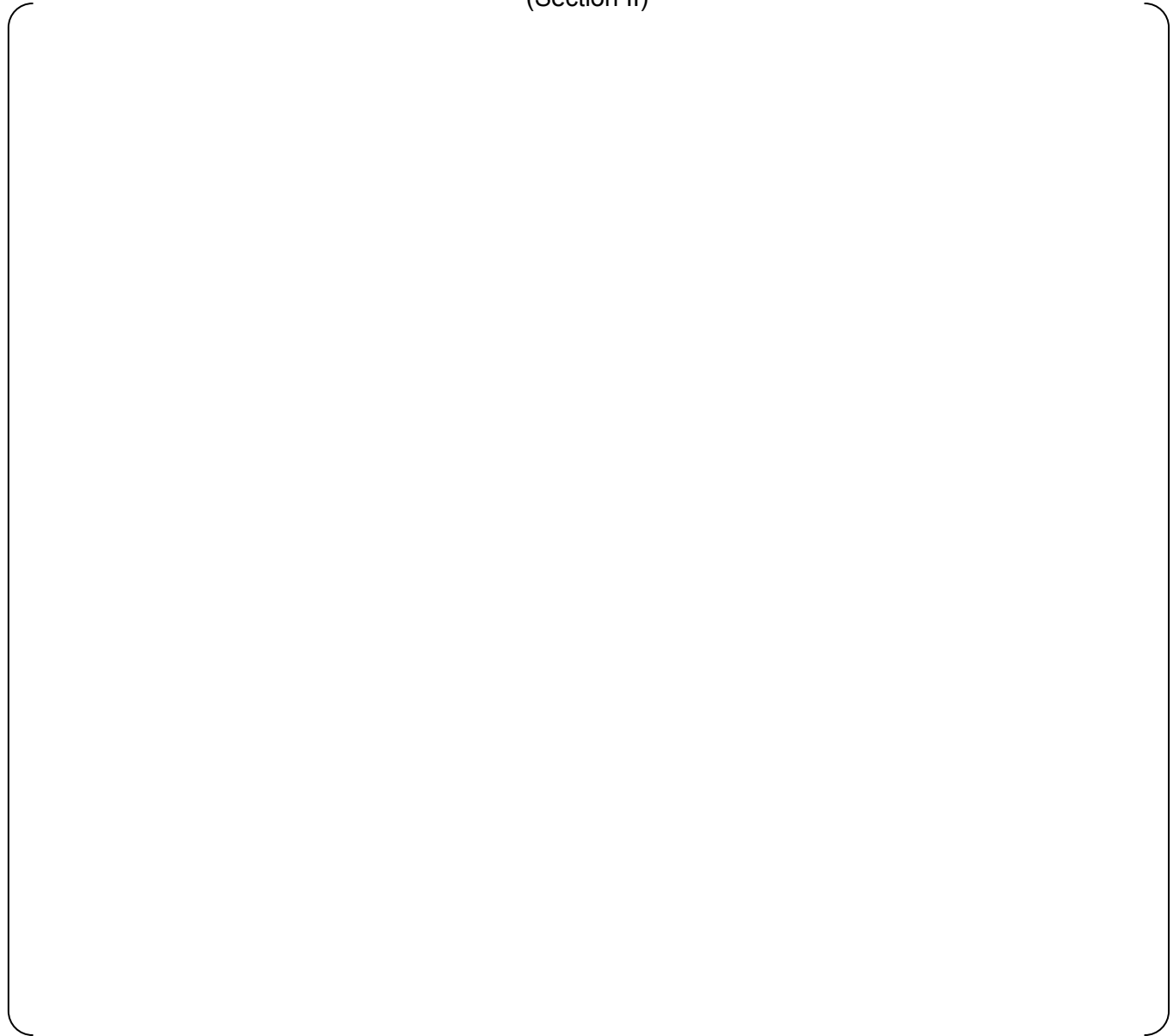
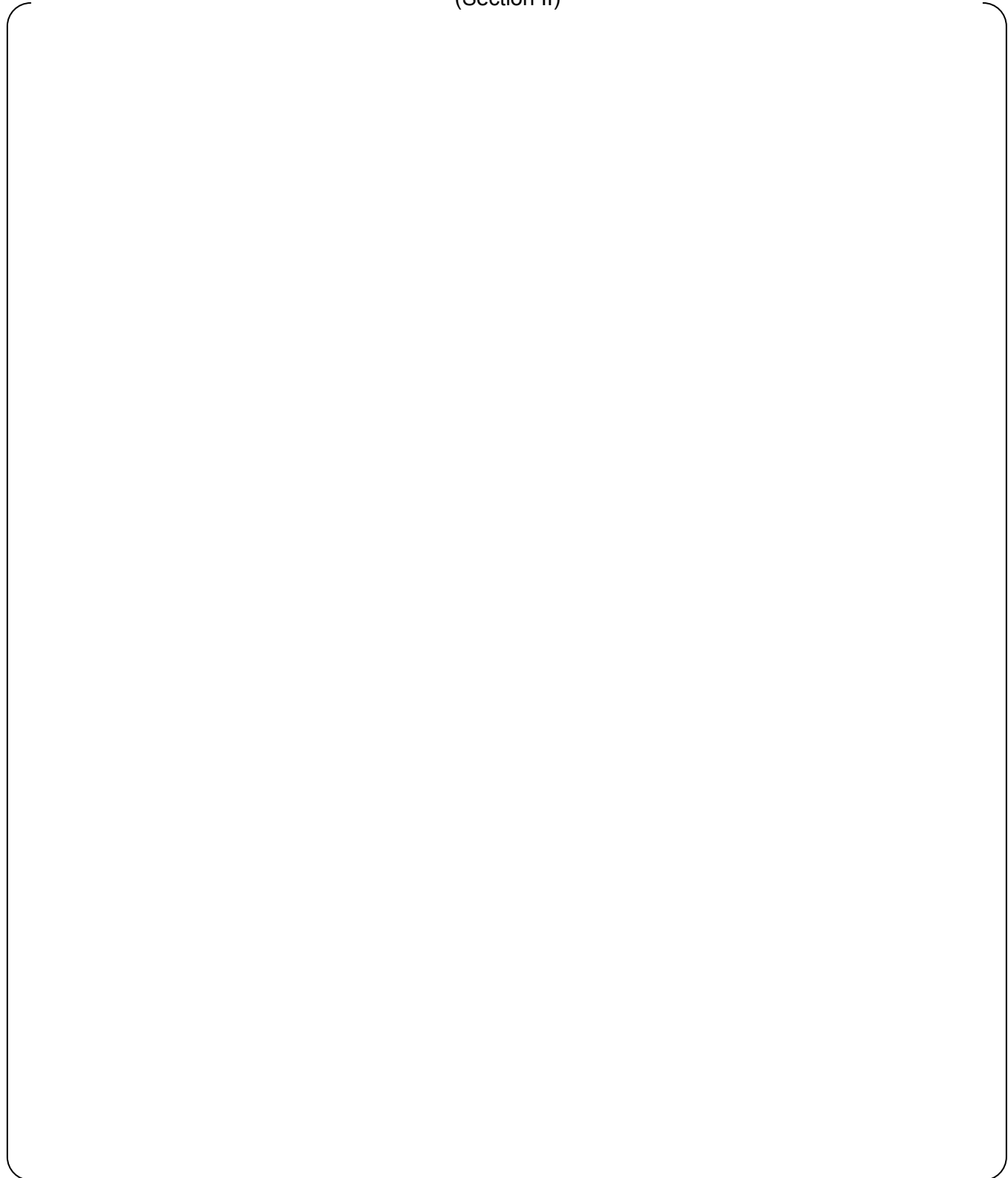


Table A1-17-1-6 Level A, B temperature and pressure input data (6/18)  
(Section II)



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Table A1-17-1-6 Level A, B temperature and pressure input data (7/18)  
(Section III)

Table A1-17-1-6 Level A, B temperature and pressure input data (8/18)  
(Section III)

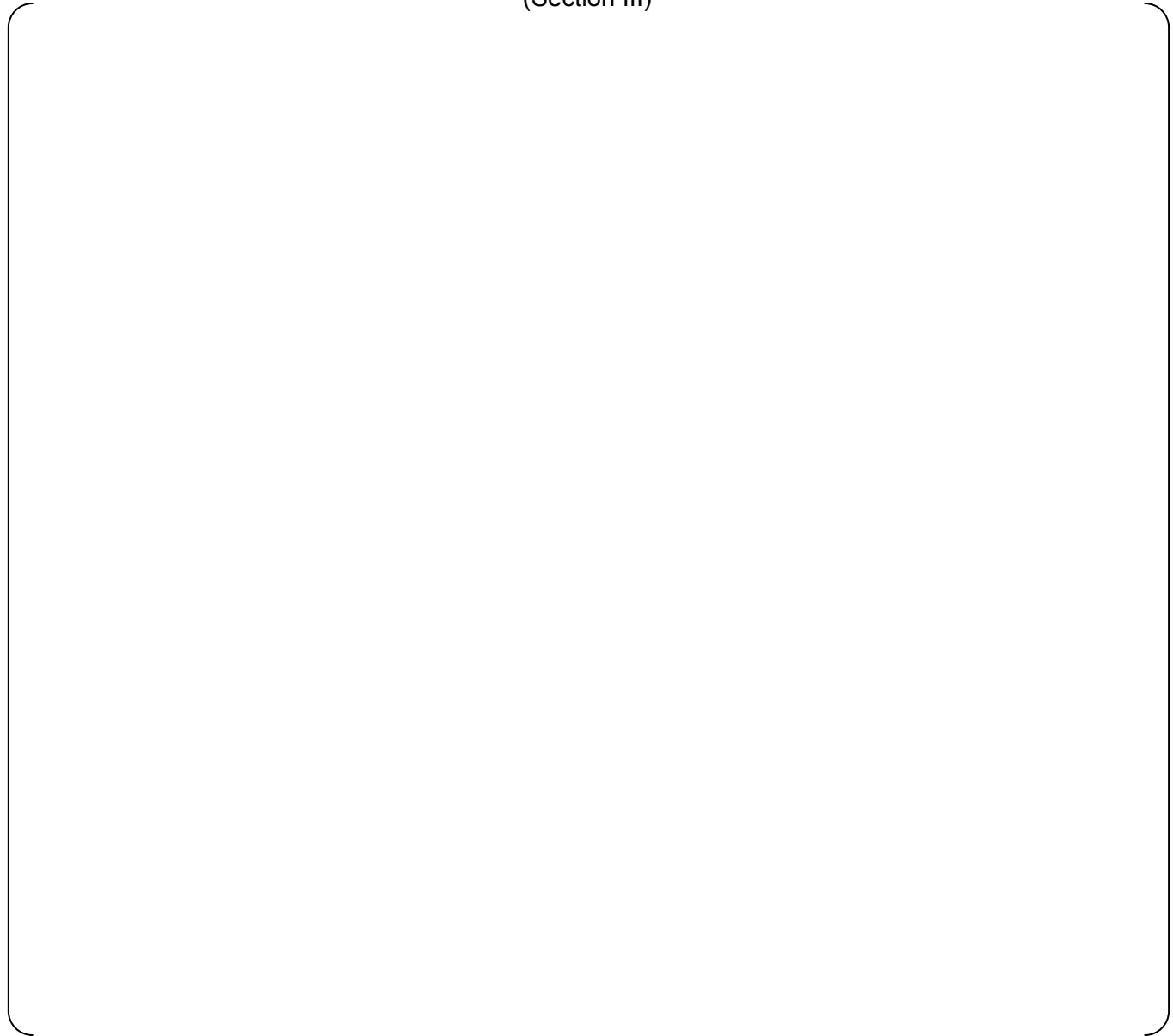
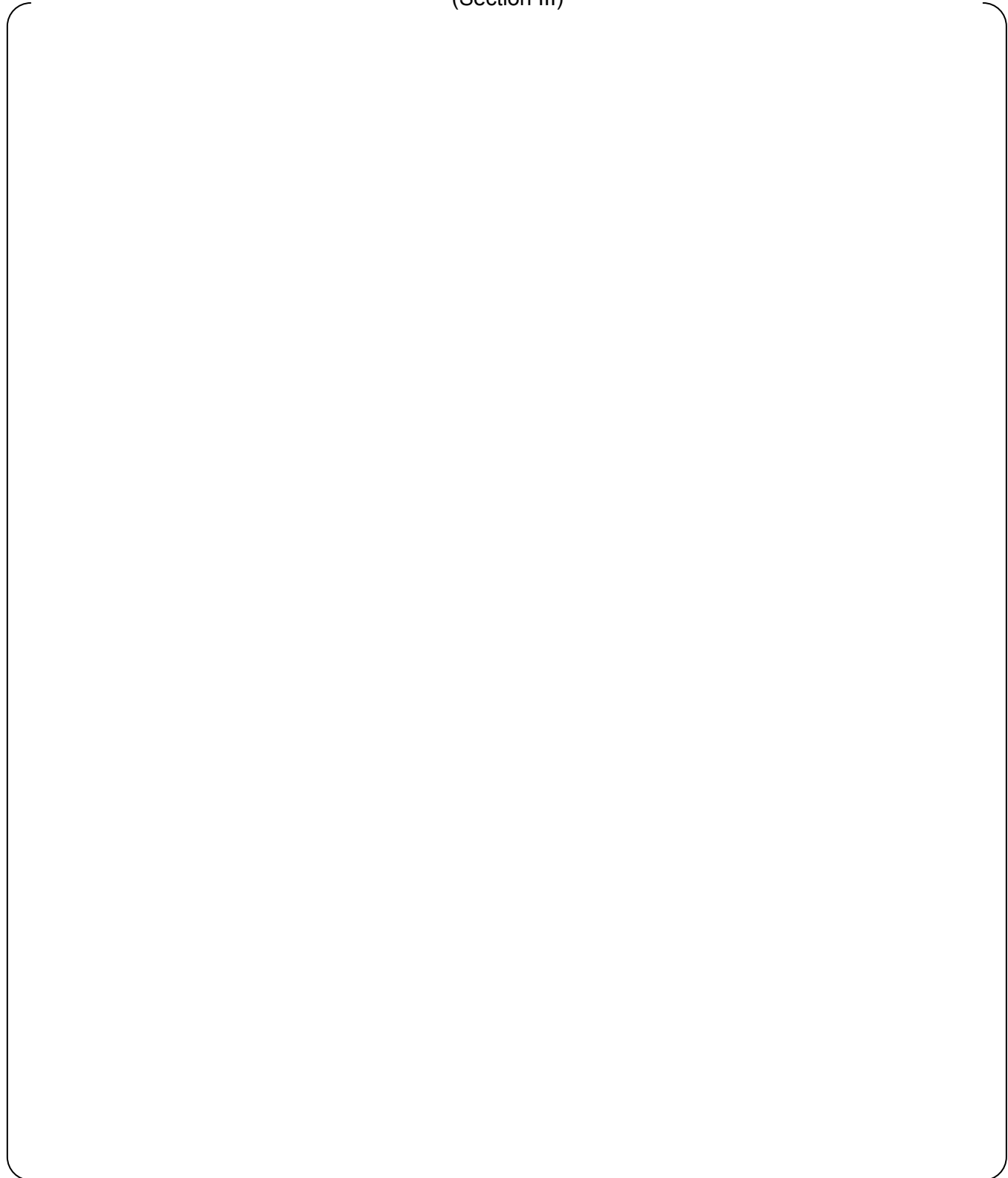


Table A1-17-1-6 Level A, B temperature and pressure input data (9/18)  
(Section III)



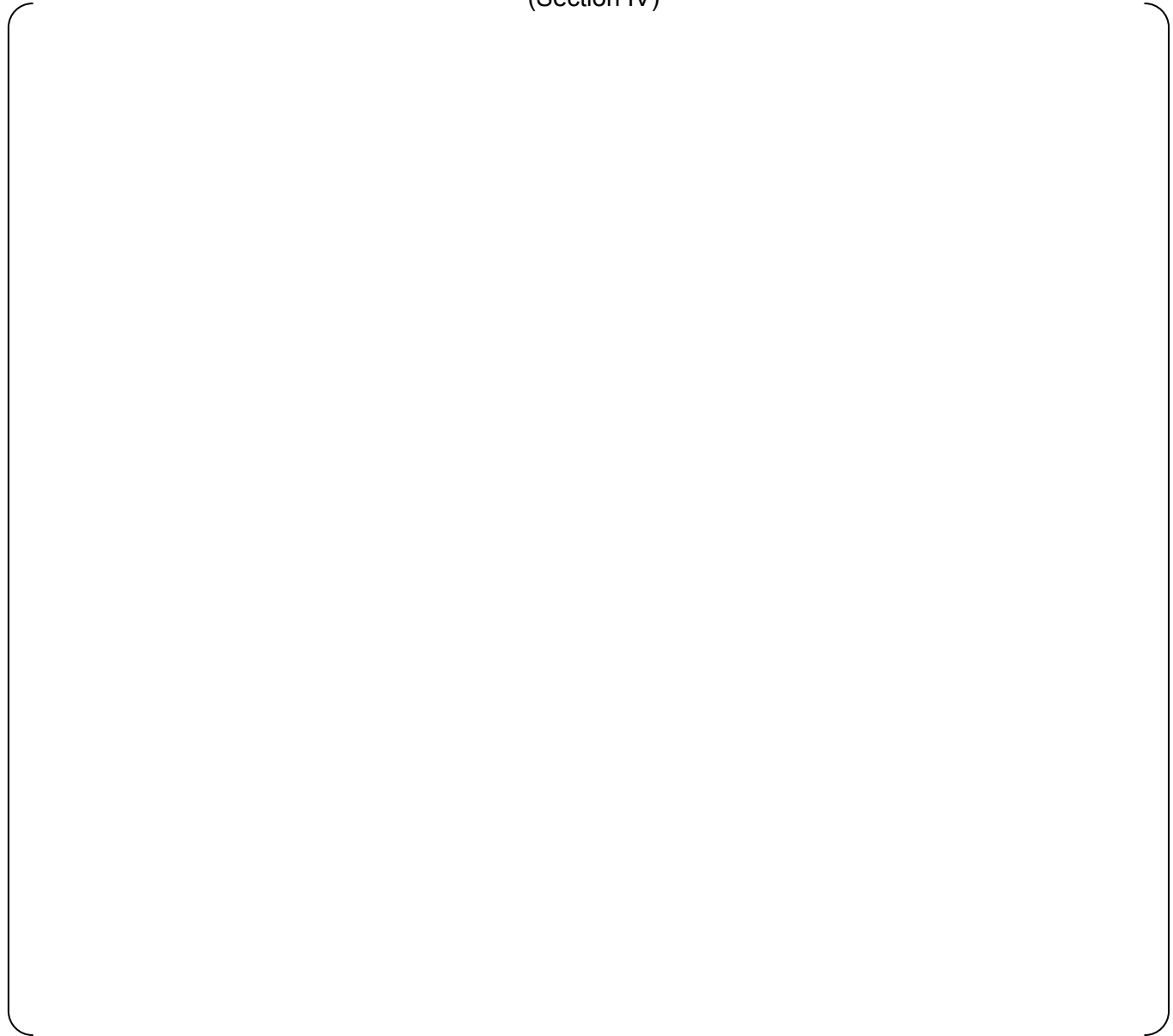
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**Table A1-17-1-6 Level A, B temperature and pressure input data (10/18)**  
(Section IV)



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**Table A1-17-1-6 Level A, B temperature and pressure input data (11/18)**  
(Section IV)



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**Table A1-17-1-6 Level A, B temperature and pressure input data (12/18)**  
(Section IV)

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Table A1-17-1-6 Level A, B temperature and pressure input data (13/18)  
(Section V)

Table A1-17-1-6 Level A, B temperature and pressure input data (14/18)  
(Section V)

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Table A1-17-1-6 Level A, B temperature and pressure input data (15/18)  
(Section V)

Table A1-17-1-6 Level A, B temperature and pressure input data (16/18)  
(Section VI)

Table A1-17-1-6 Level A, B temperature and pressure input data (17/18)  
(Section VI)

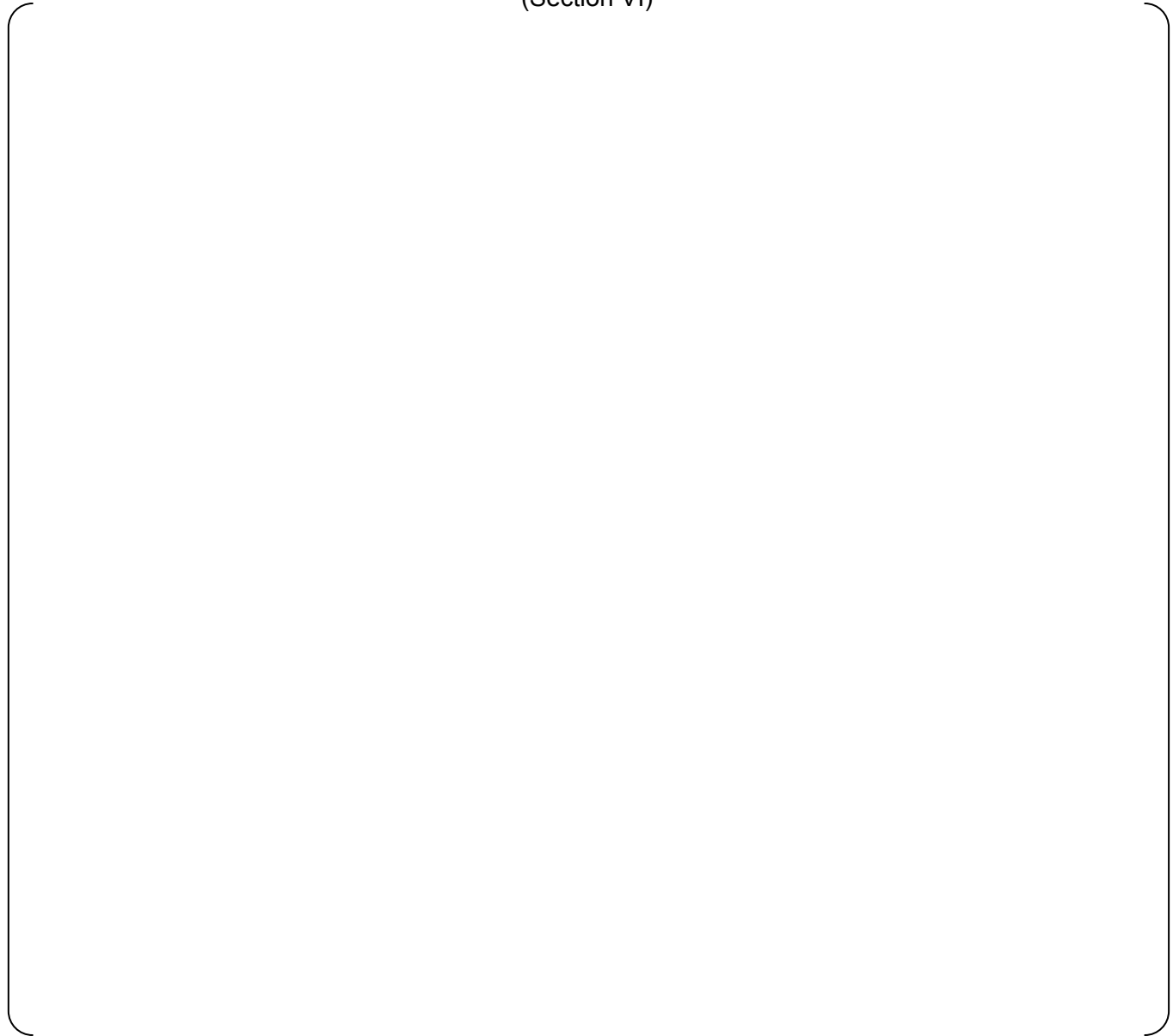


Table A1-17-1-6 Level A, B temperature and pressure input data (18/18)  
(Section VI)

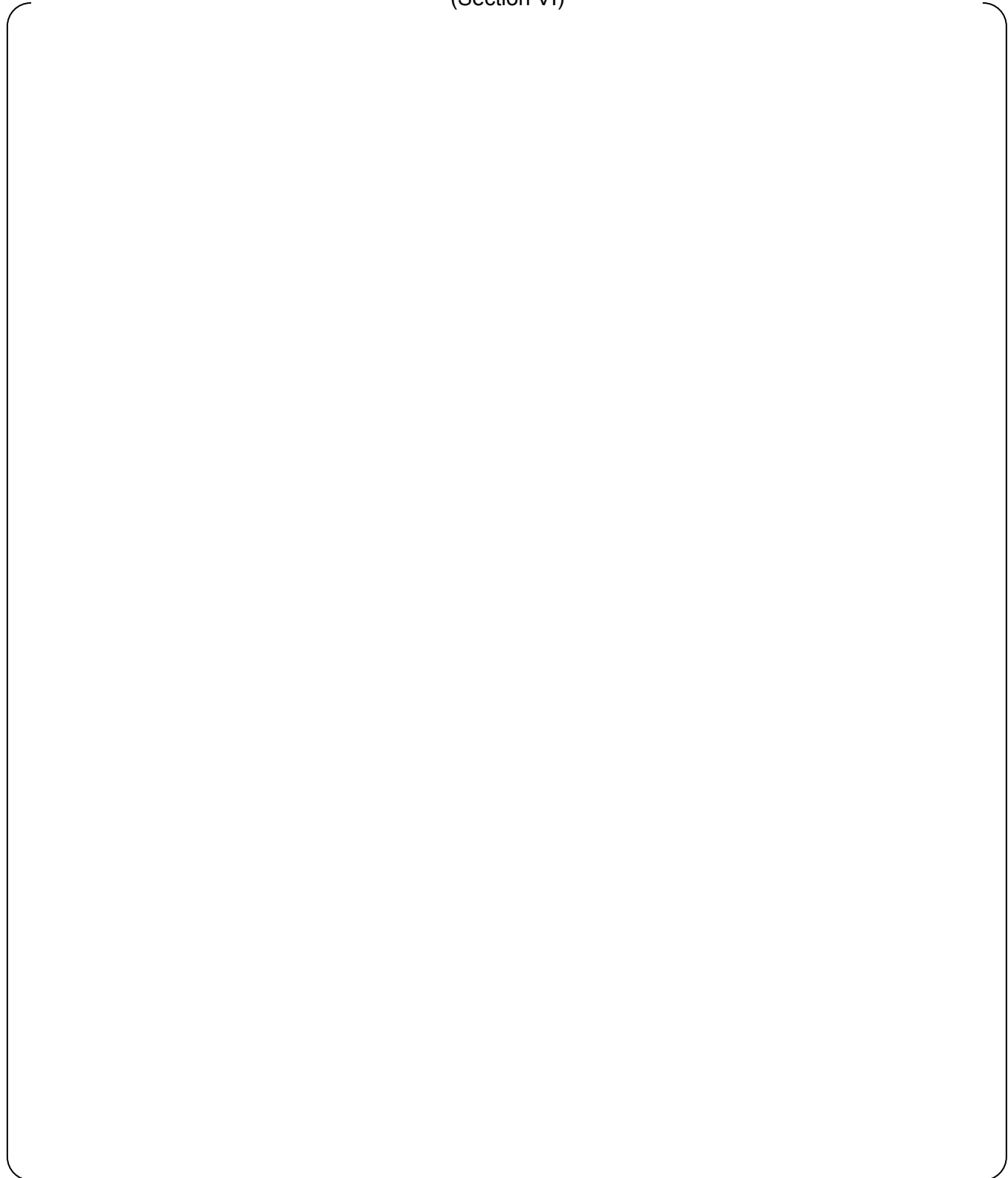




Table A1-17-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-17-1-2 Floor response curve (1/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-17-1-2 Floor response curve (2/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
Y (NS) direction (damping 4.0%)



**Figure A1-17-1-2 Floor response curve (3/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-17-1-2 Floor response curve (4/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
X (EW) direction (damping 4.0%)



**Figure A1-17-1-2 Floor response curve (5/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-17-1-2 Floor response curve (6/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
Z (Vert.) direction (damping 4.0%)

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Table A1-17-1-8 Seismic anchor displacement input data



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Table A1-17-1-9 DBPB displacement input data

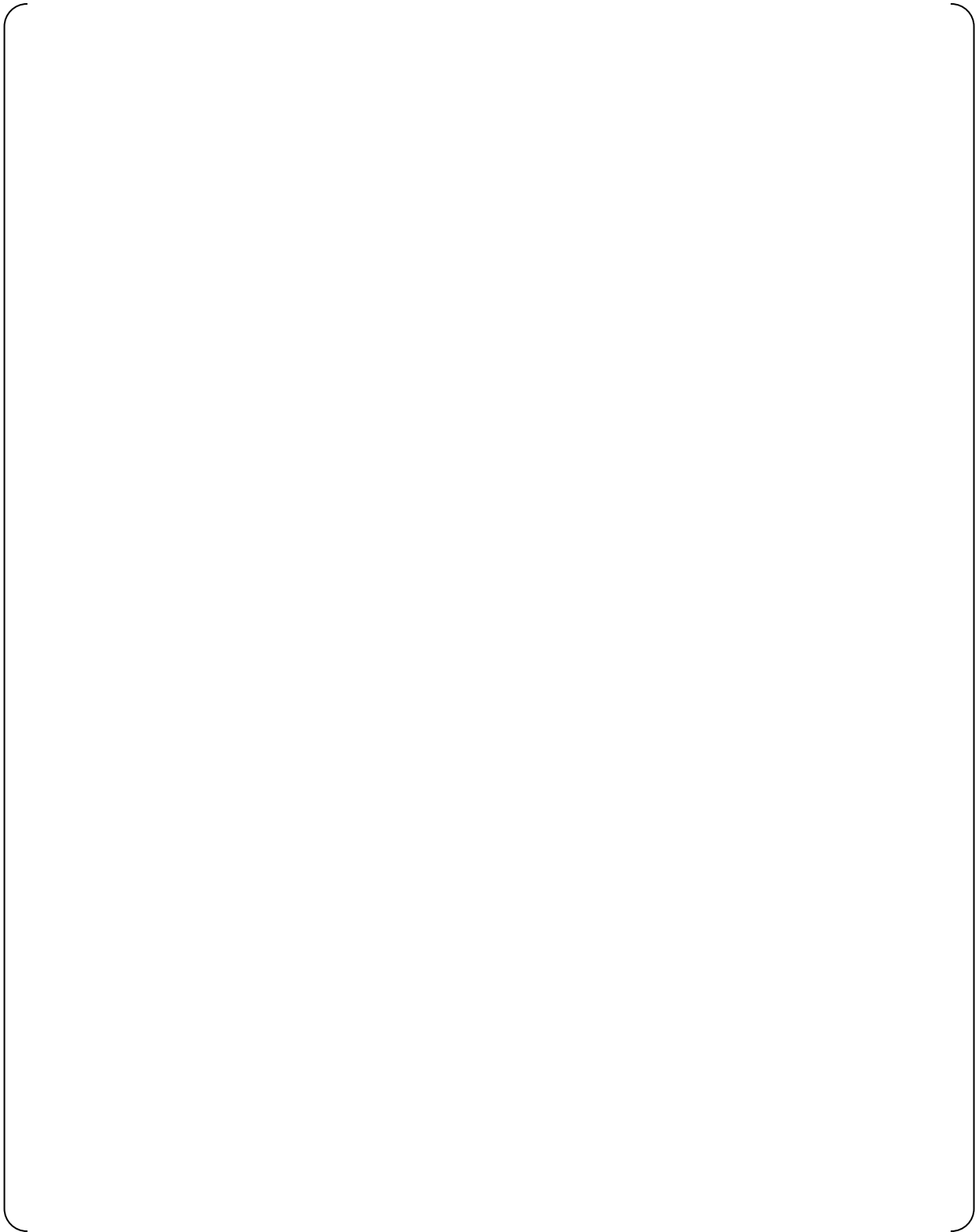


Figure A1-17-2-1 PIPESTRESS analysis model diagram

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Table A1-17-2-1 Natural frequency analysis results

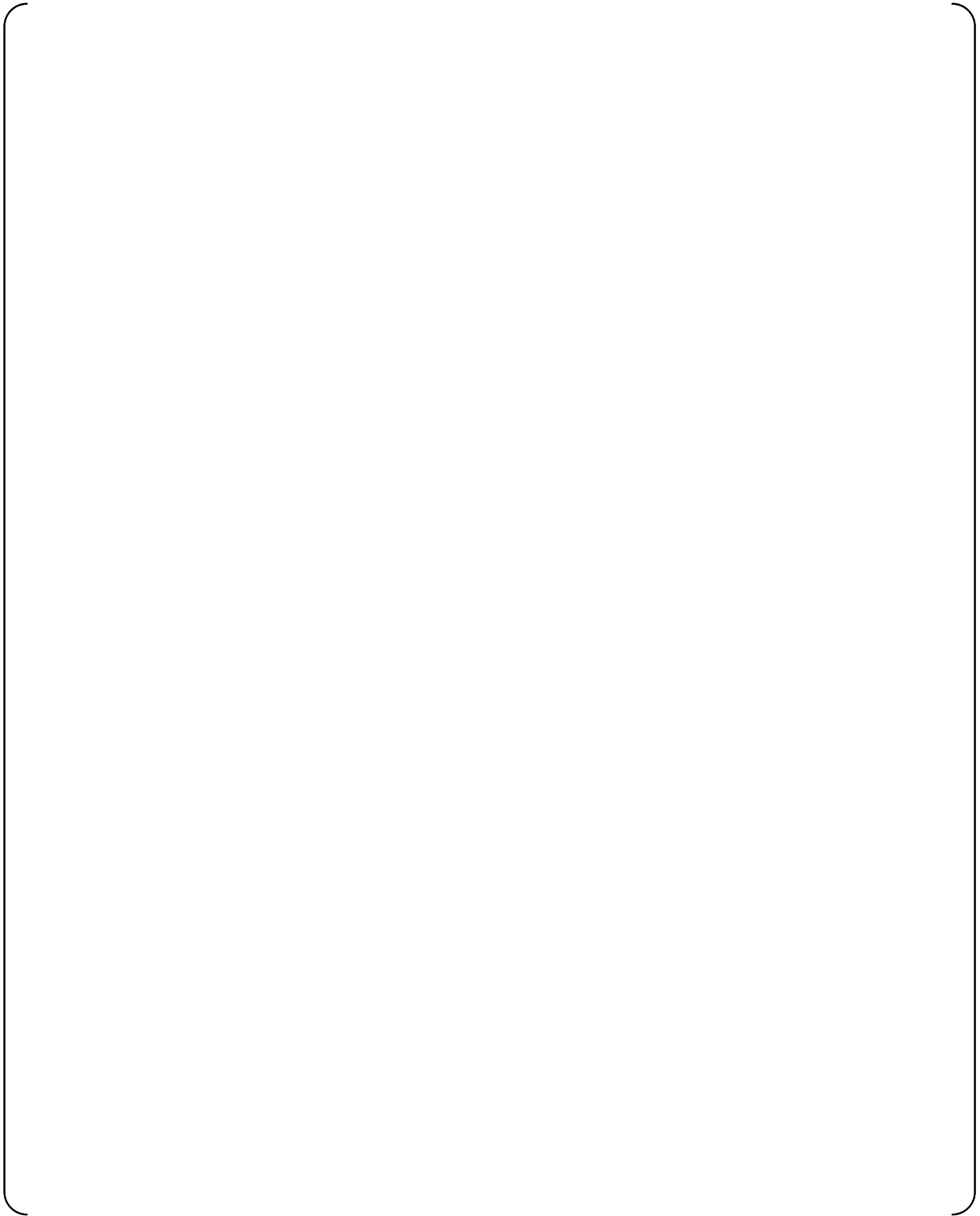
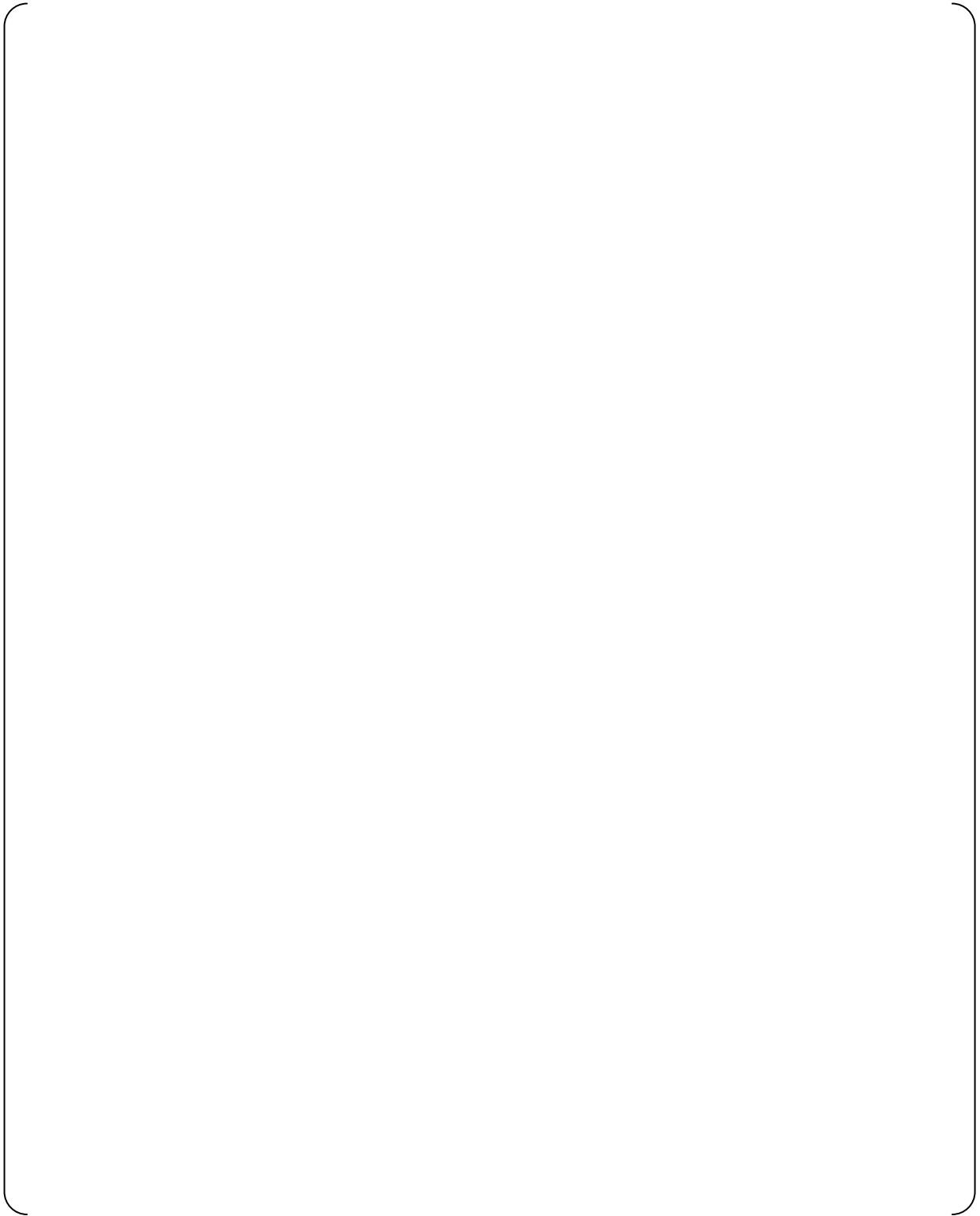


Figure A1-17-2-2 Frequency mode diagram (primary)



**Figure A1-17-2-2 Frequency mode diagram (secondary)**

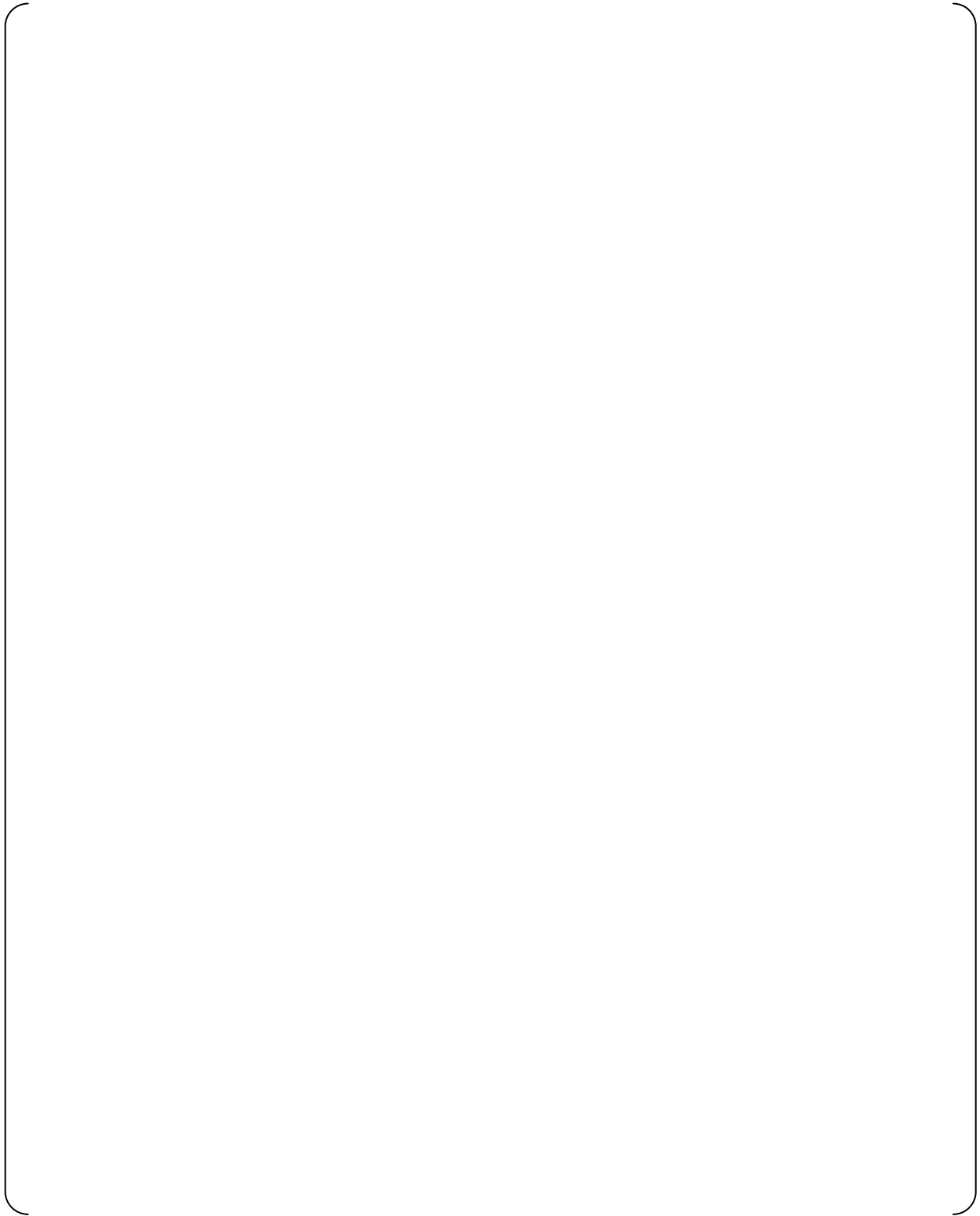


Figure A1-17-2-2 Frequency mode diagram (tertiary)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





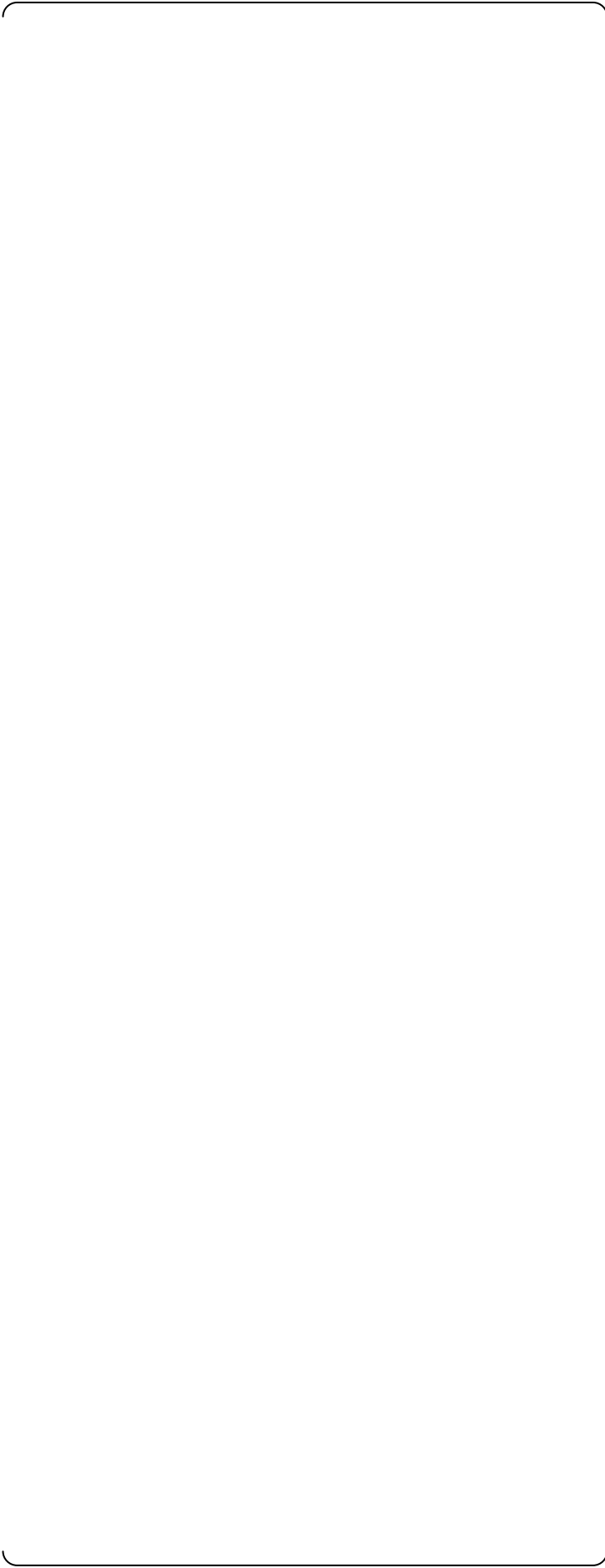


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**













**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**











**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-17-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (8/9)  
(Section III)

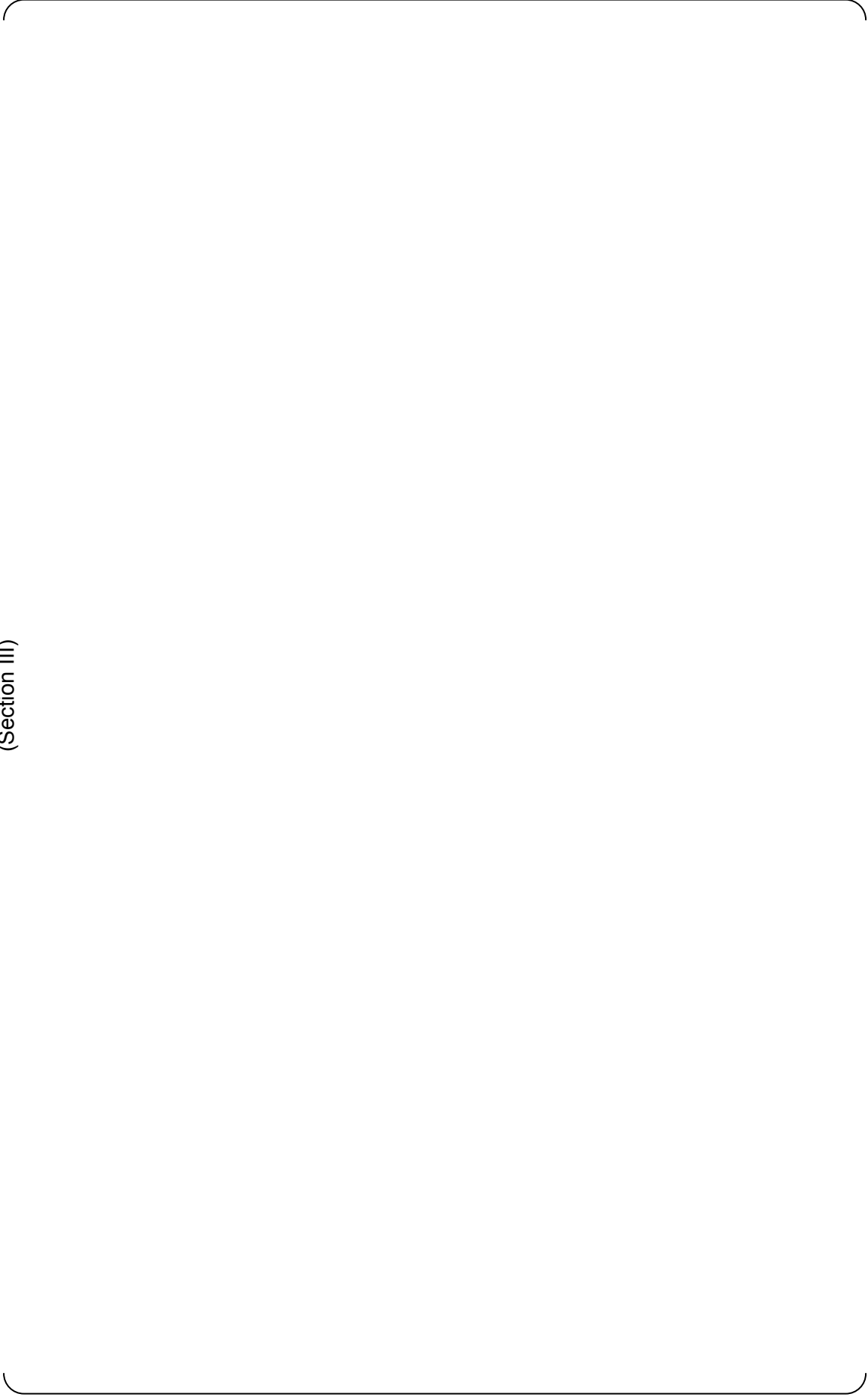


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-17-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (9/9)  
(Section III)







**Table A1-17-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-17-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

## Appendix 1-18

### CS07 CVCS Seal Injection D Line Piping Analysis Results

1. INPUT

|  |                  |
|--|------------------|
| 1.1 Used for creating the pipe structural model              |                  |
| 1.1.1 Block division and piping specifications               | Table A1-18-1-1  |
| 1.1.2 Piping isometrics                                      | Figure A1-18-1-1 |
| 1.1.3 Concentrated mass                                      | Table A1-18-1-2  |
| 1.1.4 Support point rigidity                                 | Table A1-18-1-3  |
| 1.1.5 Valve rigidity   | Table A1-18-1-4  |
| 1.2 Used for creating load conditions                        |                  |
| 1.2.1 Level A/B design transient                             | see main text    |
| 1.2.2 Level A/B thermal displacement input data              | Table A1-18-1-5  |
| 1.2.3 Level A, B temperature and pressure input data         | Table A1-18-1-6  |
| 1.2.4 Level C, D maximum temperature and pressure input data | Table A1-18-1-7  |
| 1.2.5 Floor response curve                                   | Figure A1-18-1-2 |
| 1.2.6 Seismic anchor displacement input data                 | Table A1-18-1-8  |
| 1.2.7 DBPB displacement input data                           | Table A1-18-1-9  |

2. OUTPUT

|  |                  |
|--|------------------|
| 2.1 PIPESTRESS analysis model diagram  | Figure A1-18-2-1 |
| 2.2 Natural frequency analysis results                                       | Table A1-18-2-1  |
| 2.3 Frequency mode diagram (primary to tertiary)                             | Figure A1-18-2-2 |
| 2.4 Thermal analysis results ( $\Delta T_1$ , $\Delta T_2$ , $T_a$ - $T_b$ ) | Table A1-18-2-2  |
| 2.5 Piping stress and fatigue evaluation results                             | Table A1-18-2-3  |

Table A1-18-1-1 Block division and piping specifications

US-APWR CS07  
CVCS Seal Injection D Line  
Figure A1-18-1-1 Piping Isometrics

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Table A1-18-1-2 Concentrated mass

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Table A1-18-1-3 Support point rigidity





Table A1-18-1-4 Valve rigidity

Table A1-18-1-5 Level A/B thermal displacement input data (1/3)  
(Point: 9100)

| Point | Level A/B | Thermal Displacement Input Data       |
|-------|-----------|---------------------------------------|
| 9100  | Level A/B | Thermal displacement input data (1/3) |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-18-1-5 Level A/B thermal displacement input data (2/3)  
(Point: 9100)

| Point | Level A/B | Thermal Displacement Input Data       |
|-------|-----------|---------------------------------------|
| 9100  | Level A/B | Thermal displacement input data (2/3) |

**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-18-1-5 Level A/B thermal displacement input data (3/3)  
(Point: 9100)

| Point | Level A | Level B |
|-------|---------|---------|
| 9100  |         |         |

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Table A1-18-1-6 Level A, B temperature and pressure input data (1/18)  
(Section I)



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Table A1-18-1-6 Level A, B temperature and pressure input data (2/18)  
(Section I)

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Table A1-18-1-6 Level A, B temperature and pressure input data (3/18)  
(Section I)

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Table A1-18-1-6 Level A, B temperature and pressure input data (4/18)  
(Section II)

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Table A1-18-1-6 Level A, B temperature and pressure input data (5/18)  
(Section II)

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Table A1-18-1-6 Level A, B temperature and pressure input data (6/18)  
(Section II)

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Table A1-18-1-6 Level A, B temperature and pressure input data (7/18)  
(Section III)

Table A1-18-1-6 Level A, B temperature and pressure input data (8/18)  
(Section III)

Table A1-18-1-6 Level A, B temperature and pressure input data (9/18)  
(Section III)



Table A1-18-1-6 Level A, B temperature and pressure input data (10/18)  
(Section IV)

Table A1-18-1-6 Level A, B temperature and pressure input data (11/18)  
(Section IV)

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**Table A1-18-1-6 Level A, B temperature and pressure input data (12/18)**  
(Section IV)

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Table A1-18-1-6 Level A, B temperature and pressure input data (13/18)  
(Section V)

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Table A1-18-1-6 Level A, B temperature and pressure input data (14/18)  
(Section V)

Table A1-18-1-6 Level A, B temperature and pressure input data (15/18)  
(Section V)

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**Table A1-18-1-6 Level A, B temperature and pressure input data (16/18)**  
(Section VI)

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**Table A1-18-1-6 Level A, B temperature and pressure input data (17/18)**  
(Section VI)



Table A1-18-1-6 Level A, B temperature and pressure input data (18/18)  
(Section VI)

Table A1-18-1-7 Level C, D maximum temperature and pressure input data



**Figure A1-18-1-2 Floor response curve (1/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
X (EW) direction (damping 4.0%)



**Figure A1-18-1-2 Floor response curve (2/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
Y (NS) direction (damping 4.0%)



**Figure A1-18-1-2 Floor response curve (3/6)**  
CVCS Seal Injection (CS04-07) FRS for RCP Nozzle  
Z (Vert.) direction (damping 4.0%)



**Figure A1-18-1-2 Floor response curve (4/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
X (EW) direction (damping 4.0%)



**Figure A1-18-1-2 Floor response curve (5/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
Y (NS) direction (damping 4.0%)



**Figure A1-18-1-2 Floor response curve (6/6)**  
CVCS Seal Injection (CS04-07) FRS for Piping  
Z (Vert.) direction (damping 4.0%)



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Table A1-18-1-8 Seismic anchor displacement input data

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Table A1-18-1-9 DBPB displacement input data

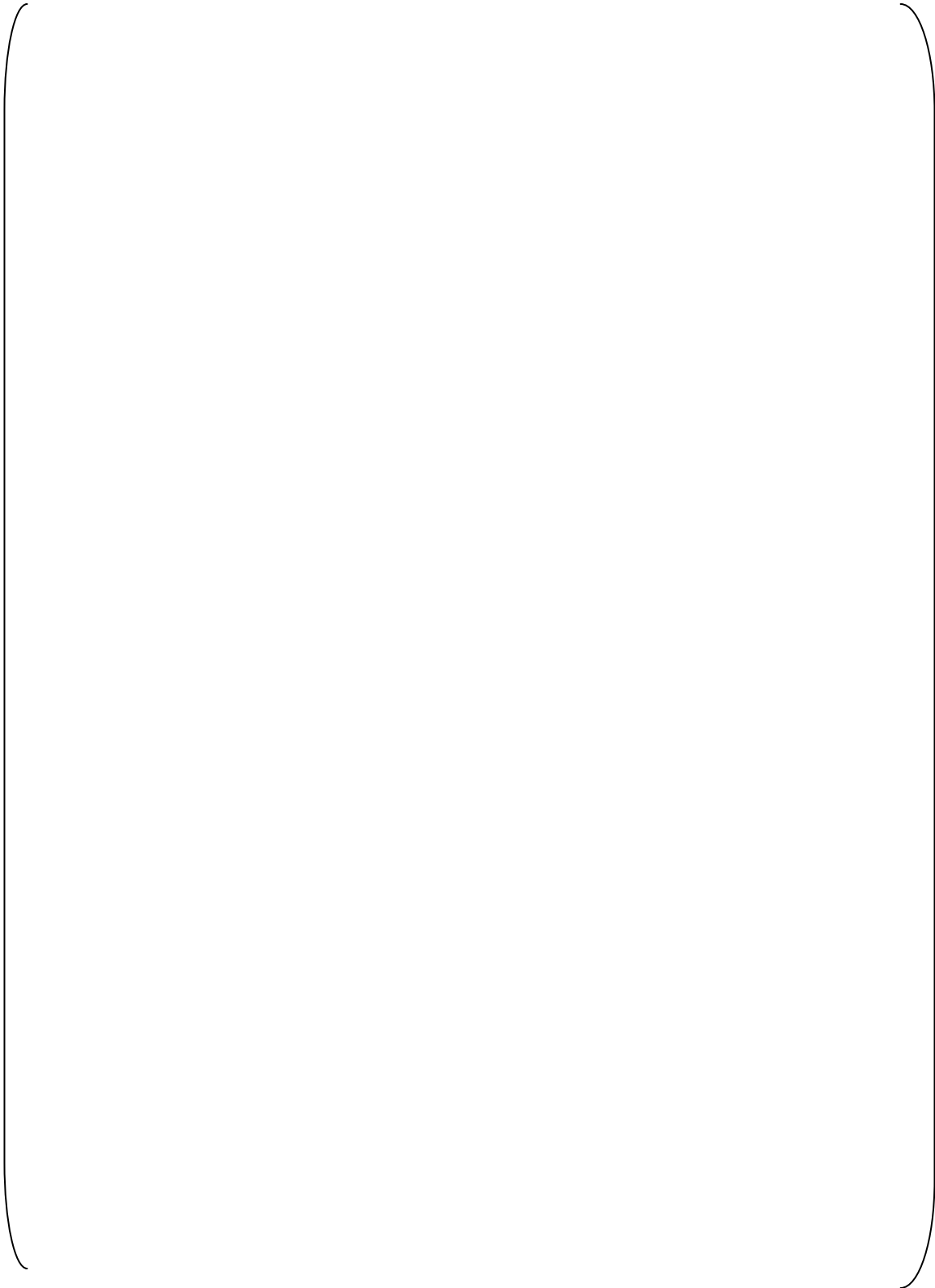


Figure A1-18-2-1 PIPESTRESS analysis model diagram

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Table A1-18-2-1 Natural frequency analysis results

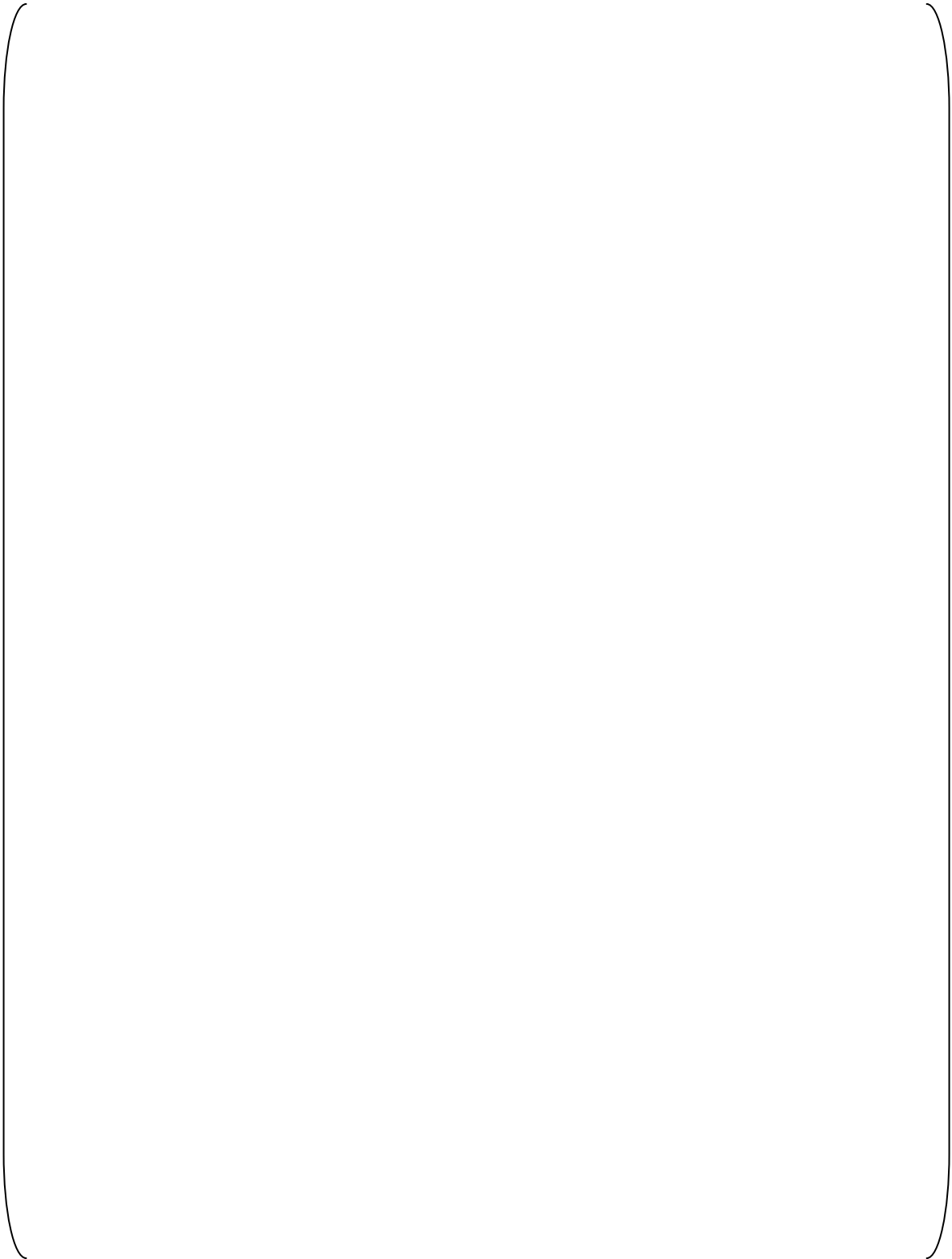


Figure A1-18-2-2 Frequency mode diagram (primary)

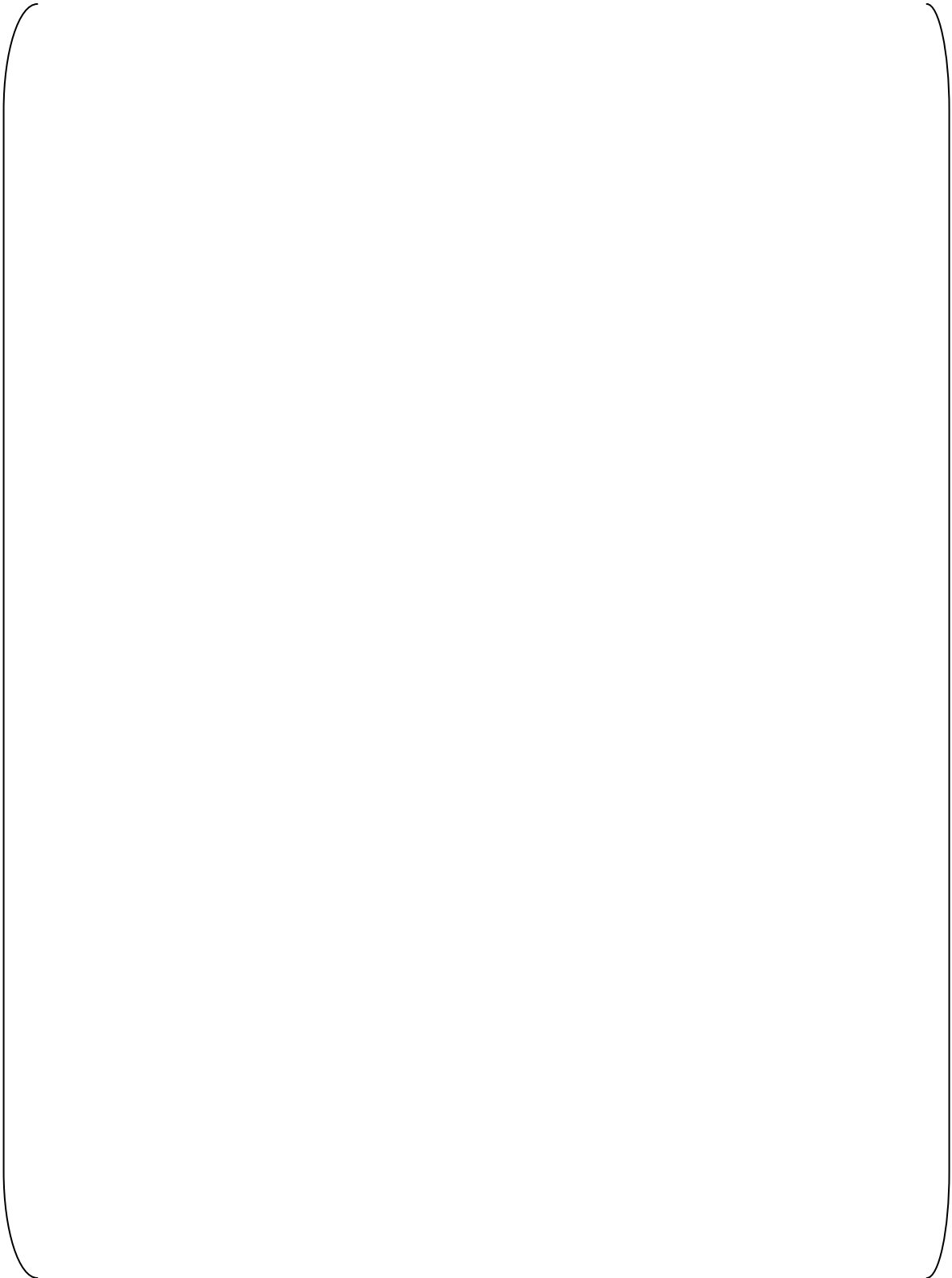


Figure A1-18-2-2 Frequency mode diagram (secondary)

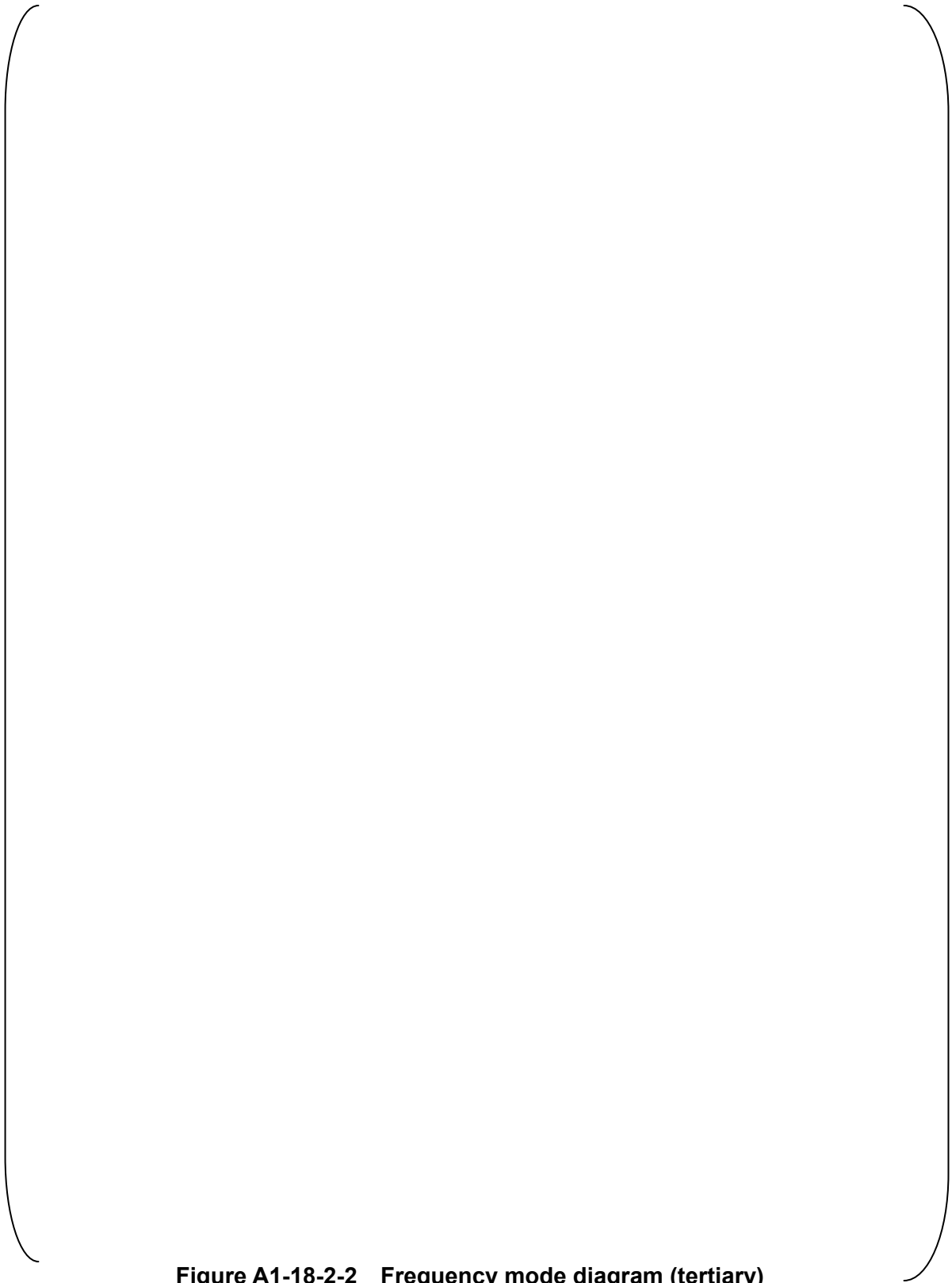
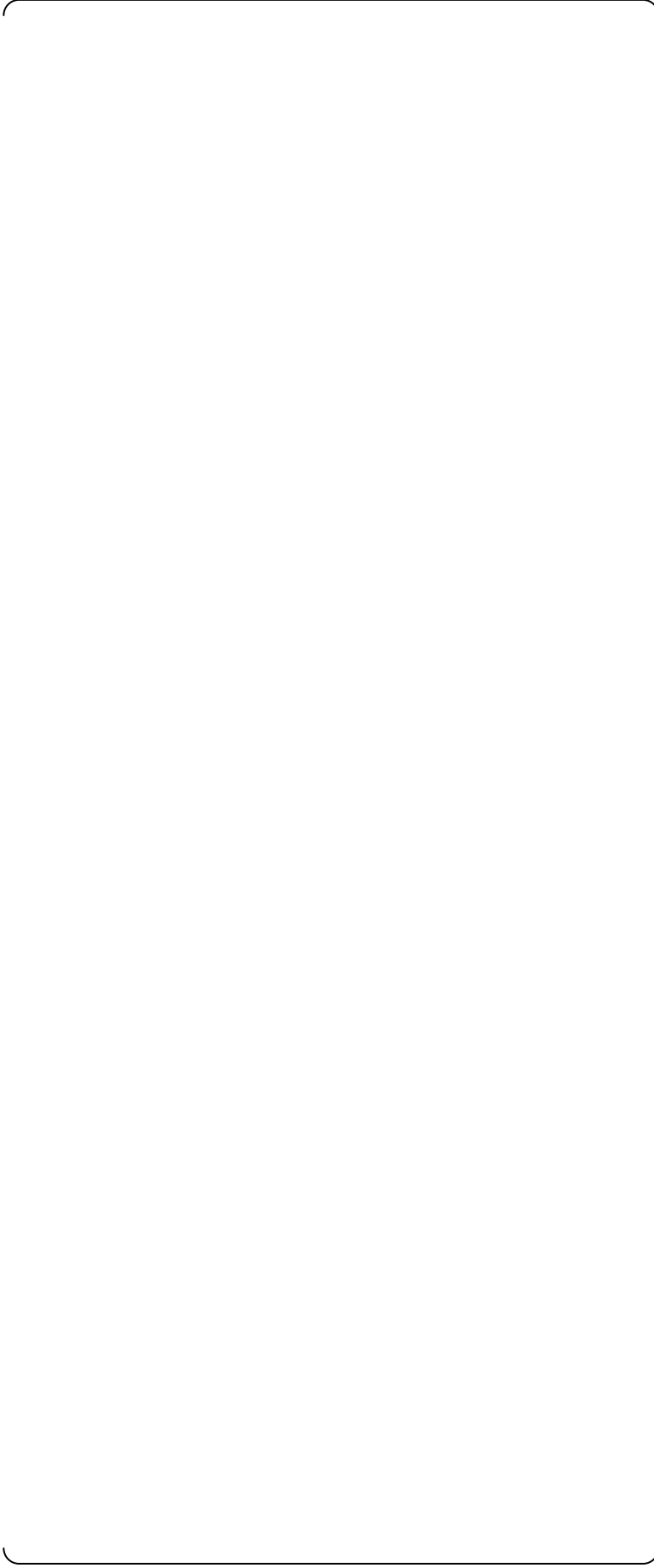


Figure A1-18-2-2 Frequency mode diagram (tertiary)







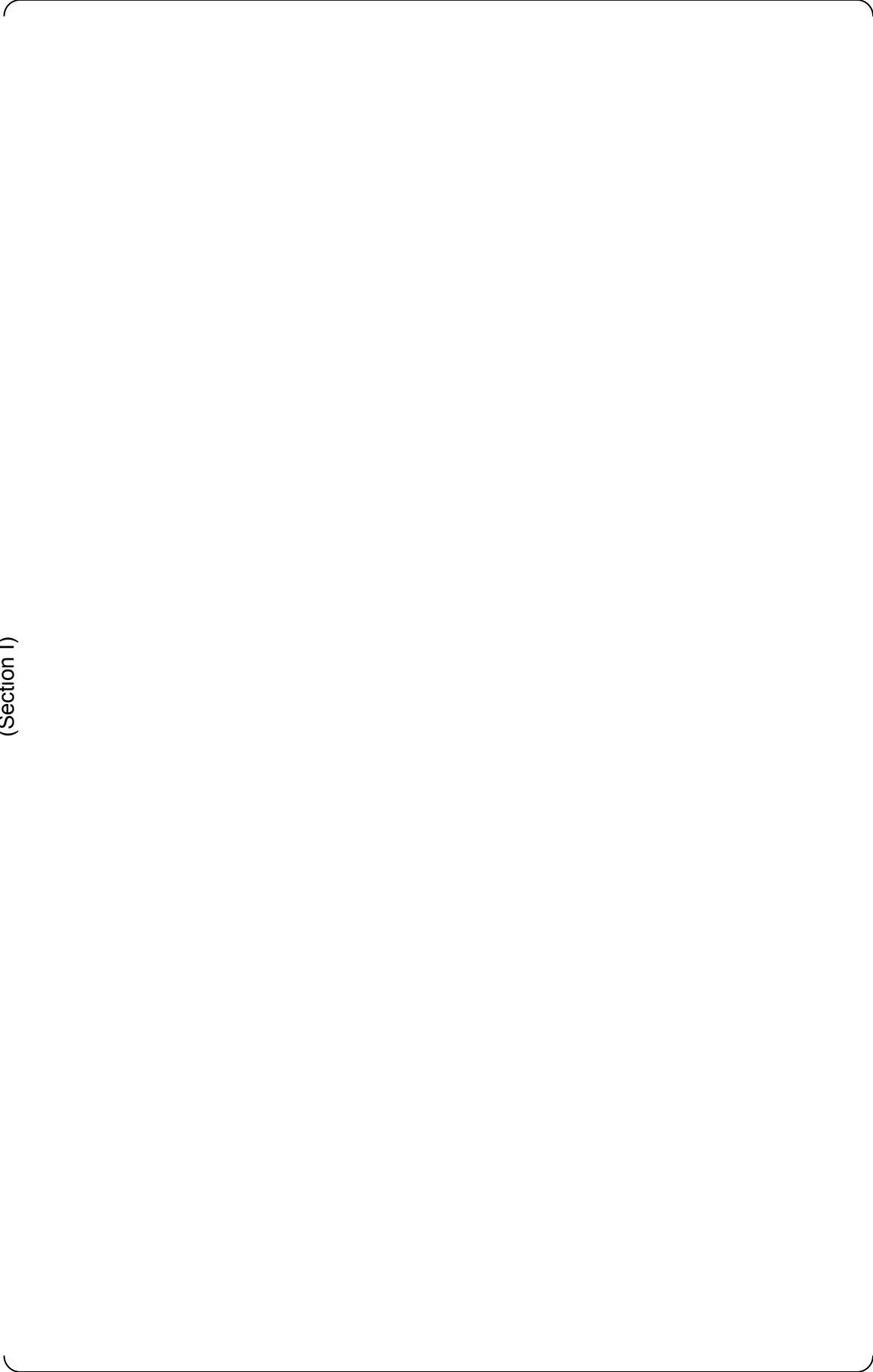


**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-18-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (3/9)  
(Section I)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



Table A1-18-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ ,  $Ta-Tb$ ) (5/9)  
(Section II)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







Table A1-18-2-2 Thermal analysis results ( $\Delta T1$ ,  $\Delta T2$ , Ta-Tb) (7/9)  
(Section III)



**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**





**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**







**Summary of Stress Analysis Results for the  
US-APWR Reactor Coolant Loop Branch Piping**

**MUAP-09011-NP (R2)**



**Table A1-18-2-3 Piping stress and fatigue evaluation results**  
(Piping that exceeds 1 inch NB-3650 evaluation)

**Table A1-18-2-3 Piping stress and fatigue evaluation results**  
(Piping of 1 inch or less NC-3650 evaluation)

Appendix 2

**LEAK BEFORE BREAK EVALUATION**

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## **A2 LEAK BEFORE BREAK EVALUATION**

### **A2.1 Introduction**

The leak-before-break (LBB) evaluation of the US-APWR follows the methodology in accordance with General Design Criteria (GDC) 4 of 10CFR50, Appendix A (Ref. 1), NUREG-0800, Standard Review Plan (SRP) 3.6.3, Rev. 1, (Ref. 2), and NUREG-1061, Volume 3 (Ref. 3). The evaluation includes an assessment of all potential failure mechanisms, and development of bounding analysis curves (BACs) that define the allowable maximum stress as a function of the normal operating stress for each piping systems or subsystem with different materials.

The LBB analysis is described in detail in Section 3.6.3 of the US-APWR Design Control Document (DCD), with details in Appendix 3B. That appendix provides the development of the BACs for the piping.

A LBB analysis was done for the pressurizer surge line and accumulator line. Table A2.0-1 shows the conditions used for development of the BAC.

### **A2.2 Modified Bounding Analysis Curve Approach**

Work has currently been completed to update the BAC analysis for Appendix 3B of the DCD, Rev.1. The following methods will be used for the LBB evaluation:

- (1) Calculation of a leakage rate and determination of leakage flow sizes as a function of normal operating conditions
- (2) Calculation of critical flaw sizes as a function of applied stress
- (3) Development of the BACs from the above

#### **A2.2.1 Leak Rate Determination**

The DCD described the thermal-hydraulics model used to develop the DCD curves. The fundamental equations for calculation of flow through a circumferential crack in a pipe are described. For the revised leakage rate calculations, the following revisions have been made:

- (1) The EPRI-developed PICEP computer program (Ref. 4) was used to calculate leakage. PICEP incorporates the thermal-hydraulic model described in the DCD but has improved methods for calculation of the crack opening area based on the EPRI-developed methods for elastic plastic fracture mechanics (Ref. 5).
- (2) For conservatism, the crack opening area was calculated without taking credit for the plastic opening. This is consistent with the approach in the DCD. In addition, the plastic zone correction factor was conservatively based on a flow stress (not yield stress) of 51 ksi (consistent with the maximum flow stress in SRP 3.6.3), minimizing the effects of plastic zone correction on the crack opening area increase due to plasticity effects.

- (3) The coefficient of discharge ( $C_D$ ) was taken as 0.61. Consistent with testing conducted at the time of the PICEP development and verification that it would adequately calculate leakage, the crack roughness for assumed fatigue cracks was taken as 0.000197 inches and no turns were included. These assumptions are consistent with those made at the time that SRP 3.6.3 and NUREG-1061 Volume were published that required a factor of 10 between the calculated leakage and the plant leakage detection system to cover various uncertainties (Ref. 6).
- (4) Based on a review of revisions to Regulatory Guide 1.45 (Ref. 7), it was determined that the sensitivity of the US-APWR leakage detection system can be reduced to 0.5 gpm, allowing the leakage flow sizes to be based on a leak of 5 gpm.

### A2.2.2. Fracture Mechanics Analysis

Because austenitic stainless steel has high fracture toughness, limit load methodology can be applied to evaluate the fracture behavior of the piping. The methods for limit load evaluation as described in SRP 3.6.3 and in the DCD are used. The flow stress used in the analysis is based on ASME Code minimum values at temperature, conservatively applying these same values for SMAW weldments, since this is less than the specified value of 51 ksi in SRP 3.6.3. Since stresses for the various loadings will be combined by absolute sum methods, the factor of safety for maximum load is 1.0, such that the critical flaw size was determined to be twice the leakage flaw size.

### A2.2.3 Generation of BAC

The BAC methodology uses a LBB assessment diagram to show that LBB requirements are met for all weld locations in each piping system. In the BAC diagram  $\sigma_{nor} = |P_m| + |P_b|$ , the sum of the membrane stress and the bending stress under normal operation, is plotted along the abscissa, and  $\sigma_{max} = |P_{m,max}| + |P_{b,max}|$ , the absolute sum of the membrane stress and the bending stress under the maximum load, is plotted along the ordinate. The procedure used in developing the BAC diagram was as follows:

- (1) Determine the leakage crack length for a crack with a leak rate 10 times as large as the detectable leak rate by applying the abscissa's normal stress  $\sigma_{nor}$ .
- (2) Based on a critical crack size of twice the leakage crack length, determine the maximum stress  $\sigma_{max}$  that is required to produce this critical crack size.
- (3) Perform the above steps at a sufficient number of points of normal operating stress to develop of smooth curve of the maximum stress  $\sigma_{max}$  as a function of the abscissa's normal stress  $\sigma_{nor}$ .
- (4) For the modified BACs, the normal operating stress was varied from that due to pressure up to a limit of 50 ksi. This upper bound is arbitrary and is a stress greater than will be limited by the ASME Code stress limits for the piping that also must be satisfied.

Per the requirements in SRP 3.6.3, the maximum stress is a combination of the effects of pressure + dead weight + maximum seismic stress if the weld is TIG. If the weld is SMAW or

SAW, the maximum stress is a combination of the effects of pressure + dead weight + thermal expansion + maximum seismic stress.

For the BAC curves, the membrane stresses were calculated based on the axial force divided by the metal area. For the piping evaluated in this report, the axial loads due to loads other than pressure are not significant. The axial pressure force was based on the internal pipe pressure times the internal area of the weld. For convenience, the bending stresses included in the BAC curves were based on the piping moment divided by the weld section modulus, effectively using the stress at the outside of the piping. All of the BACs were developed using the nominal thickness and diameter of the welds.

If the actual stresses in the piping system, calculated using the same methods as above, fall in the regions below the BAC, then LBB requirements are satisfied.

### A2.3 Calculation of LBB Evaluation Points

The assessment of LBB acceptability was performed based on the calculated stresses at each weld in the piping system being evaluated. For each weld, stresses for normal operation and the maximum stress conditions were calculated from the piping stress analysis.

The stress for normal operation along the abscissa of the BAC was calculated for each weld in the piping system as follows.

- 1) For all types of welds, calculate the algebraic sum of the axial force, the bending and torque moment due to deadweight, the internal pressure, and the thermal expansion. Thermal expansion is always included since it will contribute to the crack opening area.

$$F = F_{DW} + F_{Th} + F_P$$

$$M = \sqrt{\left( (M_X)^2 + (M_Y)^2 + (M_Z)^2 \right)}$$

$$M_X = (M_X)_{DW} + (M_X)_{Th}$$

$$M_Y = (M_Y)_{DW} + (M_Y)_{Th}$$

$$M_Z = (M_Z)_{DW} + (M_Z)_{Th}$$

Where  $F$  = Axial force

$M$  = Bending moment

Subscripts indicate the loads shown below

$DW$  = Deadweight

$Th$  = Thermal expansion

$P$  = Internal pressure



$x$ ,  $y$  and  $z$  = Component of  $x, y$  and  $z$  direction.

- 2) Calculate the cross sectional area  $A$  and the section modulus  $Z$  assuming the minimum wall thickness.
- 3) Calculate the stress  $\sigma_{nor}$  at the evaluation point under normal operation.

$$\sigma_{nor} = P_m + P_b = F/A + M/Z$$

The maximum stress for each weld in the piping system was evaluated as follows:

- 1) For SMAW and SAW welds, calculate the absolute sum of the axial force, the bending and torque moment due to deadweight, the internal pressure, the thermal expansion, and earthquake using the following equations:

$$|F| = |F_{DW}| + |F_{Th}| + |F_P| + |F_{SSE}| + |F_{SAM}|$$

$$|M| = \sqrt{((M_X)^2 + (M_Y)^2 + (M_Z)^2)}$$

$$M_X = |(M_X)_{DW}| + |(M_X)_{Th}| + |(M_X)_{SSE}| + |(M_X)_{SAM}|$$

$$M_Y = |(M_Y)_{DW}| + |(M_Y)_{Th}| + |(M_Y)_{SSE}| + |(M_Y)_{SAM}|$$

$$M_Z = |(M_Z)_{DW}| + |(M_Z)_{Th}| + |(M_Z)_{SSE}| + |(M_Z)_{SAM}|$$

Where subscripts indicate the following loads.

SSE = Inertia load due to SSE

SAM = Seismic anchor motion load due to SSE.

- 2) If the weld is a TIG weld, the loads due to thermal expansion and seismic anchor movements may be excluded per SRP 3.6.3.
- 3) Calculate stress under the maximum load  $\sigma_{max}$  at the weld joint.

$$|\sigma_{max}| = |P_{m\_max}| + |P_{b\_max}| = (|F|/A + |M|/Z)$$

The BAC assessment points were then plotted on the BAC to determine LBB acceptance. In some cases, the welds may not be acceptable if the assessment is based on the assumption of a SMAW weld joint. In this case, the weld joint can be qualified as a TIG weld, and this can be implemented in the piping system fabrication/construction on a location-unique basis.

#### A2.4 BAC Setting for LBB Evaluation

Table A2.0-1 lists the piping property of Pressurizer surge line and accumulator line selected for setting the BAC. The detailed BACs are shown in Figure A2.0-1 and Figure A2.0-2. Table A2.0-2 and Table A2.0-3 is the tabulated BAC points.

## **A2.5 References**

1. 'General Design Criteria for Nuclear Power Plants,' "Domestic Licensing of Production and Utilization Facilities," Energy. Title 10, Code of Federal Regulation, Part 50, Appendix A, U.S. Nuclear Regulatory Commission, Washington, D.C.
2. Leak-Before-Break Evaluation Procedures,' "Design of Structures, Components, Equipment, and Systems," Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants. NUREG-0800, Standard Review Plan 3.6.3, Rev. 1, U.S. Nuclear Regulatory Commission, Washington, DC, March 2007.
3. "Evaluation of Potential for Pipe Breaks," Report of U.S. NRC Piping Review Committee. NUREG-1061, Vol. 3, U.S. Nuclear Regulatory Commission< Washington, DC, 1984.
4. PICEP: Pipe Crack Evaluation Program. NP-3596-SR, Rev. 1, Electric Power Research Institute, 1987.
5. Kumar, V, and German, M. D., "Elastic-Plastic Fracture Analysis of Through-Wall and Surface Flaws in Cylinders," EPRI NP-5596, January 1988.
6. D. Abdollahian and B. Chexal, "Calculation of Leak Rates Through Cracks in Pipes and Tubes," EPRI NP-3395, Electric Power Research Institute, Palo Alto, CA, December 1983.
7. Regulatory Guide 1.45, "Guidance On Monitoring And Responding To Reactor Coolant System Leakage." U. S. Nuclear Regulatory Commission, May 2008.

Table A2.0-1 List of piping property for setting the BAC

| Subsystem                 | OD,<br>inches | t,<br>inches | Material     | Temp,<br>°F <sup>(1)</sup> | Pressure,<br>psig <sup>(1)</sup> | Axial.<br>Stress,<br>ksi | BAC<br>Figure<br>No. | BAC<br>Table<br>No. |
|---------------------------|---------------|--------------|--------------|----------------------------|----------------------------------|--------------------------|----------------------|---------------------|
| Surge Line <sup>(2)</sup> | 16            | 1.594        | SA-312 TP316 | 653                        | 2,248 <sup>(2)</sup>             | 4.017                    | A2-1                 | A2-2                |
| Accumulator Line          | 14            | 1.406        | SA-312 TP316 | 551                        | 2,296                            | 4.058                    | A2-2                 | A2-3                |

Note:

1. Conditions from Reactor Coolant System DCD, Table 5.1-2
2. Used conservative lower 2243 psig for leakage which is the pressurizer end pressure.

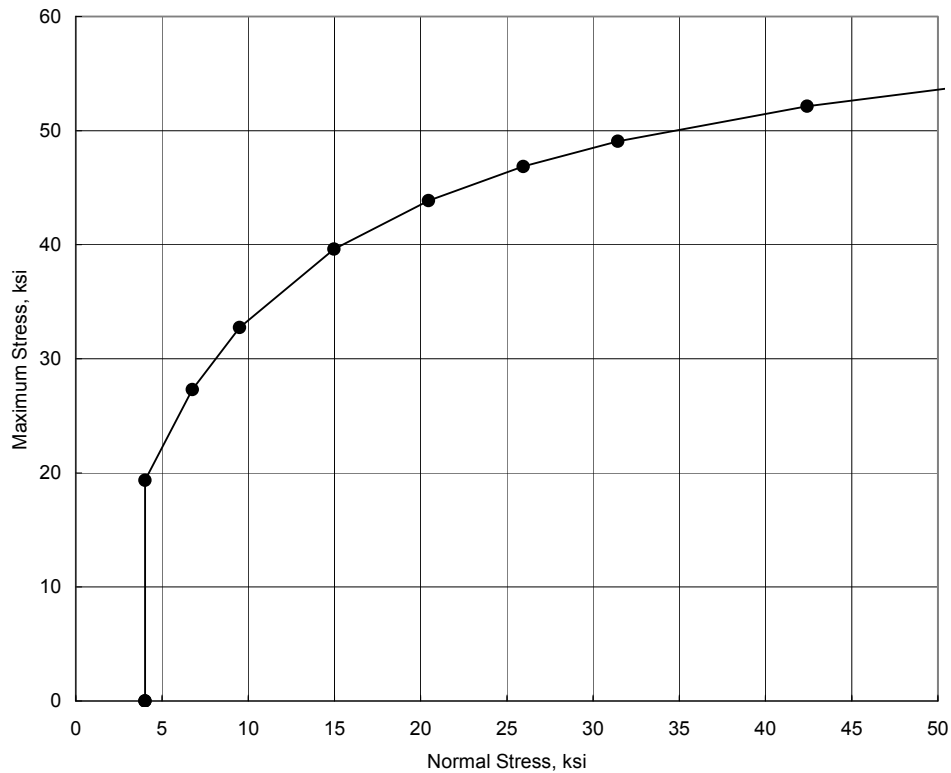


Figure A2.0-1 BAC for Surge Line

Table A2.0-2 The tabulated BAC points

| Normal Stress, ksi | Maximum Stress, ksi |
|--------------------|---------------------|
| 4.008              | 0                   |
| 4.008              | 19.355              |
| 6.751              | 27.288              |
| 9.494              | 32.738              |
| 14.98              | 39.619              |
| 20.466             | 43.873              |
| 25.952             | 46.847              |
| 31.438             | 49.059              |
| 42.411             | 52.148              |
| 53.383             | 54.242              |

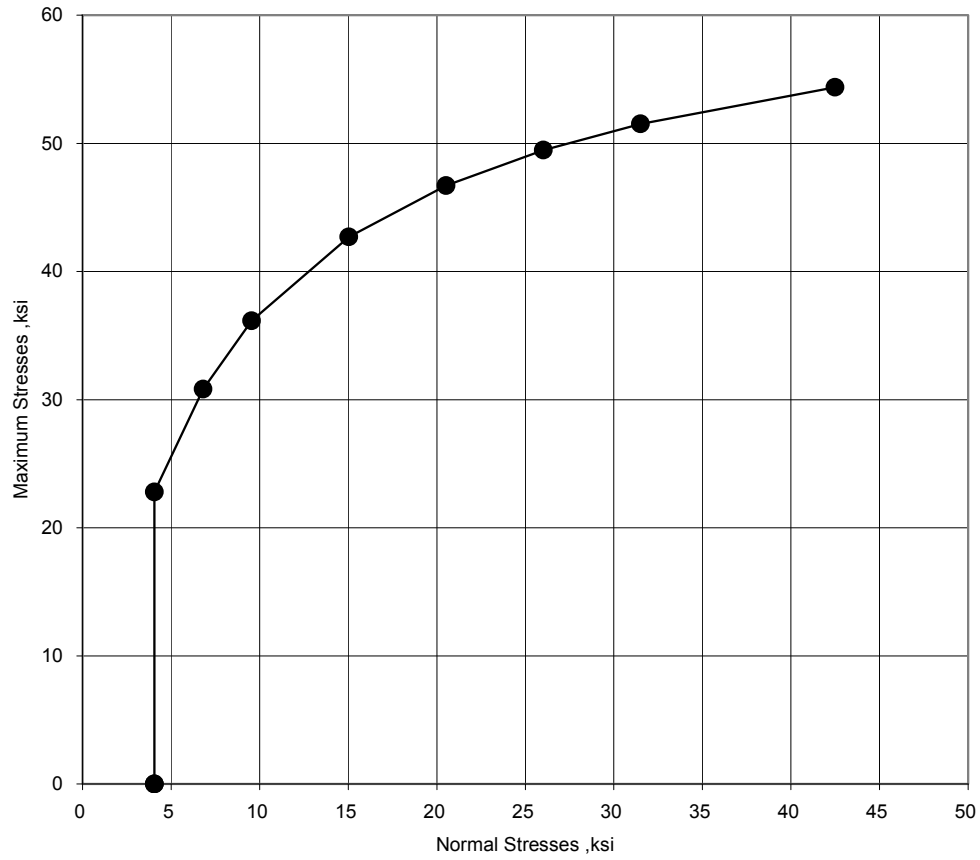


Figure A2.0-2 BAC for Accumulator Line

Table A2.0-3 The tabulated BAC points

| Normal Stress, ksi | Maximum Stress, ksi |
|--------------------|---------------------|
| 4.058              | 0.000               |
| 4.058              | 22.784              |
| 6.802              | 30.821              |
| 9.547              | 36.151              |
| 15.037             | 42.709              |
| 20.527             | 46.715              |
| 26.017             | 49.475              |
| 31.506             | 51.522              |
| 42.486             | 54.365              |
| 53.465             | 56.290              |