UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PA 19406-1415

January 25, 2011

Mr. Joseph E. Pollock
Site Vice President
Indian Point Energy Center
450 Broadway, GSB
P.O. Box 249
Buchanan, NY 10511-0249

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT 3 – NOTIFICATION OF CONDUCT OF A TRIENNIAL FIRE PROTECTION BASELINE INSPECTION

Dear Mr. Pollock:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region I staff will conduct a triennial fire protection baseline inspection at the Indian Point Nuclear Generating Unit 3 in May 2011. The inspection team will be led by Dan Orr from the NRC Region I office. The team will be composed of personnel from the NRC Region I. The inspection will be conducted in accordance with NRC inspection procedure 71111.05T, the NRC's baseline fire protection inspection procedure.

The schedule for the inspection is as follows:

- Information gathering visit – Week of April 25, 2011
- Weeks of onsite inspection – May 9, 2011 and May 23, 2011

The purposes of the information gathering visit are to obtain information and documentation needed to support the inspection, to become familiar with the station fire protection programs, fire protection features, post-fire safe shutdown capabilities, plant layout and, as necessary, obtain plant specific site access training and badging for unescorted access. A list of the types of documents the team may be interested in reviewing and possibly obtaining, are listed in Enclosure 1. The team leader will contact you with specific document requests prior to the information gathering visit.

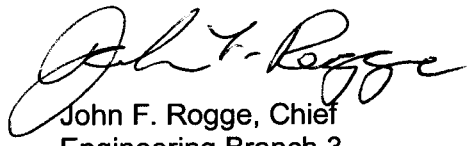
During the information gathering visit, the team will also discuss the following inspection support administrative details: office space size and location; specific documents requested to be made available to the team in their office spaces; arrangements for reactor site access, including radiation protection training, security, safety, and fitness for duty requirements; and the availability of knowledgeable plant engineering and licensing organization personnel to serve as points of contact during the inspection.

We request that during the onsite inspection week you ensure that copies of analyses, evaluations, or documentation regarding the implementation and maintenance of the Indian Point Nuclear Generating Unit 3 fire protection program, including post-fire safe shutdown capability, be readily accessible to the team for their review. Of specific interest for the fire protection portion of the inspection are those documents which establish that your fire protection program satisfies NRC regulatory requirements and conforms to applicable NRC and industry fire protection guidance. Also, personnel should be available at the site during the inspection who are knowledgeable regarding those plant systems required to achieve and maintain safe shutdown conditions from inside and outside the control room, including the electrical aspects of the relevant post-fire safe shutdown analyses, reactor plant fire protection systems and features, and the Indian Point Nuclear Generating Unit 3 fire protection program and its implementation.

This letter does not contain new or amended information collection requirements subject to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 et seq.). Existing information collection requirements were approved by the Office of Management and Budget, control number 3150-0011. The NRC may not conduct or sponsor, and a person is not required to respond to, a request for information or an information collection requirement unless the requesting document displays a currently valid Office of Management and Budget control number.

Your cooperation and support during this inspection will be appreciated. If you have questions concerning this inspection, or the inspection team's information or logistical needs, please contact Dan Orr, the team leader in the Region I Office at (610) 337-5048.

Sincerely,



John F. Rogge, Chief
Engineering Branch 3
Division of Reactor Safety

Docket No: 50-286
License No: DPR-64

Enclosure: List of Reactor Fire Protection Program Supporting Documentation

cc w/Encl: Distribution via ListServ

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Sincerely,
/RA/

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ENCLOSURE

Reactor Fire Protection Program Supporting Documentation

[Note: This is a broad list of the documents the NRC inspection team may be interested in reviewing, and possibly obtaining, during the information gathering site visit.]

1. The current version of the Fire Protection Program and Fire Hazards Analysis.

2. All fire protection program licensing basis documents such as FPP Operating License Condition, applicable UFSAR sections, and FPP SERs with associated correspondence.

3. Current versions of the fire protection program implementing procedures (e.g., administrative controls, surveillance testing, and fire brigade.)

4. Fire brigade training program and pre-fire plans.

5. Post-fire safe shutdown analysis.

6. Piping and instrumentation (flow) diagrams showing the systems and components used to achieve and maintain hot standby and cold shutdown for fires outside the control room and those components used for those areas requiring alternative shutdown capability.

7. Plant layout drawings which identify the physical plant locations of hot standby and cold shutdown equipment.

8. Plant layout drawings which identify plant fire area delineation, areas protected by automatic fire suppression and detection, and the locations of fire protection equipment.

9. Plant layout drawings which identify the location of post-fire emergency lighting units.

10. Plant operating procedures which would be used for shutdown from the control room with a postulated fire occurring in any plant area outside the control room, and procedures which would be used to implement alternative shutdown capability in the event of a fire in either the control or cable spreading room.

11. Copy of most recently completed maintenance and surveillance testing procedures for alternative shutdown capability components, fire barriers, fire detectors, fire pumps and suppression systems.

12. Maintenance procedures which routinely verify fuse and circuit breaker coordination in accordance with the post-fire safe shutdown coordination analysis.

13. A listing of significant fire protection and post-fire safe shutdown related design change packages (including their associated 10 CFR 50.59 evaluations) and Generic Letter 86-10 evaluations.

14. The reactor plant's IPEEE, results of any post-IEEE reviews, and listings of actions taken/plant modifications conducted in response to IEEE information.

15. Organization charts of site personnel down to the level of fire protection staff personnel.

16. Procedures/instructions that control the configuration of the reactor plant's fire protection program, features and post-fire safe shutdown methodology and system design.

17. A list of applicable codes and standards related to the design of plant fire protection features and evaluations of all code deviations.

18. The three most recent fire protection QA audits and/or fire protection self-assessments.

19. Recent (last 12 months) QA surveillances of fire protection activities.

20. A listing of open fire protection and fire safe shutdown related condition reports and a listing of fire protection and fire safe shutdown condition reports closed in the past three years.

21. Fire protection system health reports (last 2).

22. Lesson plans and related training information for licensed and non-licensed operators for post-fire safe shutdown including alternative shutdown.

23. Copies of AC and DC electrical system one line diagrams. (From offsite power grid connections down to the 120 volt level.)