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United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Hope Creek Generating Station
Facility Operating License No. NPF-57
NRC Docket No. 50-354

Subject: **Supplement to License Amendment Request H10-01, APPLICATION FOR TECHNICAL SPECIFICATION CHANGE REGARDING RISK-INFORMED JUSTIFICATION FOR THE RELOCATION OF SPECIFIC SURVEILLANCE FREQUENCY REQUIREMENTS TO A LICENSEE CONTROLLED PROGRAM**

References: (1) Letter from PSEG to NRC, "Application for Technical Specification change regarding risk-informed justification for the relocation of specific surveillance frequency requirements to a licensee controlled program," dated March 19, 2010 (ML 100900224)

(2) Letter from PSEG to NRC, "Response to Request for Additional Information-Risk Informed Justification for the Relocation of Specific Surveillance Frequency Requirements to a Licensee Controlled Program," dated July 28, 2010 (ML 102230417)

(3) Email from NRC to PSEG, "Request for Camera Ready Pages - Hope Creek TSTF-425," dated November 22, 2010

In Reference 1, PSEG Nuclear LLC (PSEG) submitted a license amendment request for Hope Creek Generating Station (HCGS). The request would modify HCGS Technical Specifications (TS) by relocating specific surveillance frequencies to a licensee-controlled program, the Surveillance Frequency Control Program, with the implementation of Nuclear Energy Institute (NEI) 04-10, "Risk Informed Method for Control of Surveillance Frequencies."

PSEG requested that the proposed license amendment be approved by the NRC by March 31, 2011 with a 120 day implementation period. In Reference 2, PSEG responded to the NRC's request for additional information regarding the license amendment request. In Reference 3, the NRC requested camera-ready pages. While PSEG was reviewing the significant number of technical specification pages affected by this change, a few editorial corrections were identified. By this letter, PSEG is submitting page corrections to the NRC for review. The revised pages are editorial in nature and do not modify the intent of the license amendment request.

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PSEG has reviewed the information supporting a finding of no significant hazards consideration that was provided to the NRC in Reference 1. The additional information provided in this letter does not affect the conclusion that the proposed license amendment does not involve a significant hazards consideration.

There are no regulatory commitments contained in this submittal.

If you have any questions or require additional information, please do not hesitate to contact Mrs. Erin West at (856) 339-5411.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on 1/10/11
(Date)

Sincerely,

John F. Perry
Site Vice President – Hope Creek

Attachment

Marc L. Dapas, Acting Regional Administrator - NRC Region I
R. Ennis, Project Manager - USNRC
NRC Senior Resident Inspector – Hope Creek
P. Mulligan, Manager IV, NJBNE
Commitment Coordinator – Hope Creek
PSEG Commitment Coordinator - Corporate

ATTACHMENT 1
TECHNICAL SPECIFICATION PAGES WITH PROPOSED CHANGES:
LICENSE AMENDMENT TO ADOPT TSTF-425, REVISION 3,
"RELOCATE SURVEILLANCE FREQUENCIES TO LICENSEE CONTROL"

The following HCGS Technical Specifications pages (Facility Operating License NPF-57) are affected by this supplement:

3/4 3-31	3/4 3-66
3/4 3-55	3/4 3-67

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK (6)	CHANNEL FUNCTIONAL TEST (c)	CHANNEL CALIBRATION (6)	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
HIGH PRESSURE COOLANT INJECTION SYSTEM ISOLATION (Continued)				
h. HPCI Torus Compartment Temperature - High	NA	Q	R	1, 2, 3
i. Drywell Pressure - High	NA	Q	R	1, 2, 3
j. Manual Initiation	NA	R	NA	1, 2, 3
7. RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION				
a. Reactor Vessel Water Level - Low, Level 3	S	Q	R	1, 2, 3
b. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	NA	Q	R	1, 2, 3
c. Manual Initiation	NA	Q(a)	NA	1, 2, 3

* When handling recently irradiated fuel in the secondary containment and during operations with a potential for draining the reactor vessel.

** When any turbine stop valve is greater than 90% open and/or when the key-locked bypass switch is in the Norm position.

(a) Manual initiation switches shall be tested ~~at least once per 18 months~~ INSERT 7. All other circuitry associated with manual initiation shall receive a CHANNEL FUNCTIONAL TEST ~~at least once per 92 days~~ as part of circuitry required to be tested for automatic system isolation. INSERT 7

(b) Each train or logic channel shall be tested ~~at least every other 92 days~~. INSERT 7

(c) Frequencies are specified in the Surveillance Frequency Control Program unless otherwise noted in the table.

TABLE 4.3.5.1-1

REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNITS	CHANNEL CHECK (b)	CHANNEL FUNCTIONAL TEST (b)	CHANNEL CALIBRATION (b)
a. Reactor Vessel Water Level - Low Low, Level 2	~~S~~	~~Q~~	~~R~~
b. Reactor Vessel Water Level - High, Level 8	~~S~~	~~Q~~	~~R~~
c. Condensate Storage Tank Level - Low	NA	~~Q~~	~~R~~
d. Manual Initiation	NA	~~Q~~(a)	NA

(a) Manual initiation switches shall be tested ~~at least once per 18 months~~ INSERT 1. All other circuitry associated with manual initiation shall receive a CHANNEL FUNCTIONAL TEST ~~at least once per 92 days~~ INSERT 1 as part of circuitry required to be tested for automatic system actuation.

(b) Frequencies are specified in the Surveillance Frequency Control Program unless otherwise noted in the table.

TABLE 4.3.7.1-1

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENTATION	CHANNEL CHECK(a)	CHANNEL FUNCTIONAL TEST(a)	CHANNEL CALIBRATION(a)	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
1. Control Room Ventilation Radiation Monitor	S	Q	R	1, 2, 3, and *
2. Area Monitors				
a. Criticality Monitors				
1) New Fuel Storage Vault	S	Q	R	#
2) Spent Fuel Storage Pool	S	Q	R	##
b. Control Room Direct Radiation Monitor	S	Q	R	At all times
3. Reactor Auxiliaries Cooling Radiation Monitor	S	Q	R	At all times
4. Safety Auxiliaries Cooling Radiation Monitor	S	Q	R	At all times
5. Offgas Pre-treatment Radiation Monitor	S	Q	R	**

TABLE 4.3.7.1-1 (Continued)

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TABLE NOTATION

#With fuel in the new fuel storage vault.
##With fuel in the spent fuel storage pool.
*When recently irradiated fuel is being handled in the secondary containment and during operations with the potential for draining the reactor vessel.
**When the offgas treatment system is operating.

(a) Frequencies are specified in the Surveillance Frequency Control Program unless otherwise noted in the table.