



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

January 10, 2011

Site Vice President
Entergy Nuclear Operations, Inc.
Pilgrim Nuclear Power Station
600 Rocky Hill Road
Plymouth, MA 02360-5508

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - PILGRIM NUCLEAR POWER
STATION - TO SUPPORT THE REVIEW OF SETPOINT AND SETPOINT
TOLERANCE INCREASES FOR SAFETY RELIEF VALVES AND SPRING
SAFETY VALVES (TAC NO. ME3543)

Dear Sir or Madam:

By letter dated March 15, 2010 (Agencywide Documents Access Management System (ADAMS) Accession No. ML100770450), Entergy Nuclear Operations, Inc. (Entergy), the licensee for Pilgrim Nuclear Power Station, submitted a license amendment request (LAR) to revise its Technical Specifications to modify the setpoint and setpoint tolerances for Safety Relief Valves (SRVs) and Spring Safety Valves (SSVs) and changes related to the replacement of (i) four (4) Target Rock Two-Stage SRVs with Three-Stage SRVs, and (ii) two existing Dresser 3.749 inch-throat diameter SSVs with Dresser 4.956 inch-throat diameter SSVs.

On June 3, 2010, the Nuclear Regulatory Commission (NRC) staff issued a request for additional information (RAI) concerning the LAR. The RAI included two requests (RAI-1 and RAI-2) related to the Setpoint Methodology for the proposed Anticipated Transient Without Scram/High Pressure Recirculation Pump Trip trip setpoint. By letter dated August 30, 2010 (ADAMS Accession No. ML102510137), Entergy provided responses to the staff's RAIs. The NRC staff has reviewed Entergy's response in that letter, and has determined that the supplemental information did not completely address the staff's June 3, 2010, RAIs. Enclosed is the NRC staff's supplemental RAIs which provide the additional information needed by the staff to complete this review.

The informational needs were discussed with your staff on January 6, 2011, and it was agreed that your response would be provided by January 31, 2011.

Sincerely,

A handwritten signature in black ink, appearing to read "R. Guzman", followed by a horizontal line.

Richard V. Guzman, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-293

Enclosure:
As stated

cc w/encl: Distribution via Listserv

SUPPLEMENTAL REQUEST FOR ADDITIONAL INFORMATION (RAI)

ENTERGY NUCLEAR OPERATIONS, INC.

PILGRIM NUCLEAR POWER STATION

REVIEW OF SETPOINT AND SETPOINT TOLERANCE INCREASES

FOR SAFETY RELIEF VALVES AND SPRING SAFETY VALVES

DOCKET NO. 50-293

The Nuclear Regulatory Commission (NRC) staff is reviewing the Entergy Nuclear Operations, Inc. (Entergy or the licensee) submittal dated March 15, 2010. By letter dated August 30, 2010, Entergy provided responses to the staff's RAIs. The NRC staff has reviewed that letter, and has determined that the supplemental information did not completely address the staff's concerns. In order for the NRC staff to complete its review; the clarifying information items described below will need to be addressed.

1. Definition of "Trip Level Setting" in TS Table 3.2-G:

Clarify what the column title, "Trip Level Setting," in TS Table 3.2-G, "Instrumentation That Initiate Recirculation Pump Trip and Alternate Rod Insertion," represents compared to the setpoint terms listed in Figure 1 of Regulatory Guide (RG) 1.105, "Setpoints for Safety-Related Instrumentation," Revision 3, dated December 3, 1999.

2. Setpoint Calculation Methodology:

Provide documentation of the methodology used for establishing the limiting trip setpoint or normal trip setpoint and limiting acceptable values for the as-found and as-left setpoints as measured during the periodic surveillance testing. Documentation should:

- Include calculations.
- Indicate Analytical Limits and/or other limiting design values.
- Indicate the source of the parameters (tolerances) used in calculating these three setpoint values and how they comply with RG 1.105 to ensure the 95/95 confidence level is achieved and that these setpoints are adequate for the surveillance test interval specified in the plant surveillance requirements.
- As applicable, provide reference(s) to NRC approved documents related to this setpoint calculation.

3. Test Procedures:

Describe the measures to be taken to ensure that the associated instrument channels are capable of performing specified functions in accordance with applicable design requirements and associated analysis, including:

- Information on the controls employed to ensure that the as left trip setting after completion of periodic surveillance is consistent with the setpoint methodology.
- Discussion on the plant corrective action process for restoring the channels to operable status.

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/RA/

Richard V. Guzman, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
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ADAMS Accession No.: ML110090027 *Concurrence via e-mail. NRR-088

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