



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 9, 2011

LICENSEE: PSEG Nuclear, LLC

FACILITY: Salem Nuclear Generating Station, Units 1 and 2

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON
SEPTEMBER 16, 2010, BETWEEN THE U.S. NUCLEAR REGULATORY
COMMISSION AND PSEG NUCLEAR, LLC, CONCERNING QUESTIONS
PERTAINING TO THE SALEM NUCLEAR GENERATING STATION, UNITS 1
AND 2, LICENSE RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of PSEG Nuclear, LLC (the applicant), and Exelon held a telephone conference call on September 16, 2010, to discuss and clarify the staff's questions concerning the Salem Nuclear Generating Station, Units 1 and 2, license renewal application. The telephone conference call was useful in clarifying the intent of the staff's questions.

Enclosure 1 provides a listing of the participants, Enclosure 2 contains a brief summary of the discussion and status of the items, and Enclosure 3 contains the draft request for additional information (RAI).

The applicant had an opportunity to comment on this summary.

A handwritten signature in cursive script that reads "Bennett M. Brady".

Bennett M. Brady, Project Manager
Projects Branch 1
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosures:

1. List of Participants
2. Summary of meeting discussion
3. Draft RAI

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TELEPHONE CONFERENCE CALL
SALEM NUCLEAR GENERATING STATION, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION

LIST OF PARTICIPANTS
September 16, 2010

PARTICIPANTS

AFFILIATIONS

Bennett Brady	U.S. Nuclear Regulatory Commission (NRC)
Allen Hiser	NRC
Rachel Vaucher	NRC
John Hufnagel	Exelon
Christopher Wilson	Exelon
Sam Speer	PSEG Nuclear, LLC
Albert Fulvio	Exelon
Ali Fakhar	PSEG Nuclear, LLC

SUMMARY OF MEETING ON QUESTIONS ON THE
SALEM NUCLEAR GENERATING STATION, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION

SEPTEMBER 16, 2010

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of PSEG Nuclear, LLC held a telephone conference call on September 16, 2010, to discuss and clarify the NRC's request for additional information (RAI) related to potential primary water stress corrosion cracking in steam generator divider plates (RAI 3.1.1-02).

Discussion on Follow-up RAI 3.1.1-02

- NRC explained that without studies that confirm otherwise we do not know, this is a safety-significant issue. We do not have data because no one has ever looked at the issue.
- During the discussion NRC agreed to the following edits in the "Issue" section of the draft RAI
 - To delete the word "likely" in the first sentence and replace it with "potential," and
 - To delete the words "constitutes a condition that is adverse to quality" in the first and replace it with "could lead to the propagation of cracking"
- NRC will send the formal revised RAI.

ENCLOSURE 2

Draft RAI 3.1.1-02

Follow-up to RAI 3.1.1-01 on Salem Steam Generator PWSCC

Background:

Based on recent foreign operating experience, the staff's concern in RAI 3.1.1-01 was about the potential impact of primary water stress corrosion cracking (PWSCC) in steam generator (SG) divider plate assembly on adjacent components, which are part of the reactor coolant pressure boundary (channel head, tubesheet, tubesheet cladding...). In its response to RAI 3.1.1-01, dated July 08, 2010, the applicant describes the materials of its SGs divider plate assembly, which are Alloy 600 for the stub runner and the divider plate, and Alloy 82/182 for the welds that attach the divider plate and stub runner to each other, and to the channel head and to the tubesheet. The applicant also provides additional elements in order to justify why the potential for cracking of its SG divider plate propagating into adjoining components and resulting in loss of the integrity of the reactor coolant pressure boundary would not be expected to occur, and therefore the SGs divider plate does not require an aging management program consisting of inspections for crack propagation.

Issue:

Although not considered to be an immediate safety issue, the likely presence of cracks in Alloy 600 steam generator divider plate assemblies constitutes a condition that is adverse to quality. In addition, this condition could lead to propagation of these cracks into surrounding pressure boundary areas, such as the tube-to-tubesheet welds and the channel head. Although the applicant has provided qualitative arguments for concluding that divider plate cracking is not a concern, the RAI response does not provide a reasonable and sufficient basis for justifying the applicant's conclusions. Further, the use of analytical tools to predict the behavior of service-induced cracking (in other components) has not always bounded actual service performance of these cracks.

Request:

The applicant is requested to provide an aging management program (AMP) that would demonstrate the condition of the steam generator divider plate assembly to support a conclusion that there will be no adverse consequences of divider plate assembly degradation during the renewed license period.

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/RA/

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Memorandum to PSEG Nuclear LLC. From B/ Brady dated February 9, 2011

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