

Fadi M. Diya Vice President Nuclear Operations Ameren Missouri Callaway Plant T 573.676.6411 F 573.676.4056

December 21, 2010

ULNRC-05752

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555-0001

> 10 CFR 50.73(a)(2)(i)(B) 10 CFR 50.73(a)(2)(v)(A) 10 CFR 50.73(a)(2)(v)(B) 10 CFR 50.73(a)(2)(v)(D)

Ladies and Gentlemen:

DOCKET NUMBER 50-483 CALLAWAY PLANT UNIT 1 UNION ELECTRIC CO. FACILITY OPERATING LICENSE NPF-30 LICENSEE EVENT REPORT 2010-005-01 EMERGENCY DIESEL GENERATOR "A" SHUTDOWN DURING 24-HOUR SURVEILLANCE

On May 28, 2010 Callaway Plant submitted Licensee Event Report (LER) 2010-005-00 in accordance with 10 CFR 50.73(a)(2)(i)(B), 10 CFR 50.73(a)(2)(v)(A), 10 CFR 50.73(a)(2)(v)(B), and 10 CFR 50.73(a)(2)(v)(D) to report an event in which the "A" emergency diesel generator at Callaway failed and automatically shut down during a 24-hour surveillance run.

The enclosed supplemental Licensee Event Report, LER 2010-005-01, is submitted to revise the causes and corrective actions for the same condition.

This letter does not contain new commitments.

Sincerely. Fadi M Diya

Vice President, Nuclear Operations

Enclosure

Junction CC & Hwy 0 P0 Box 620, MC CA-460

Fulton, MO 65251

ULNRC-05752 December 21, 2010 Page 2

cc: Mr. Elmo E. Collins, Jr. Regional Administrator
U.S. Nuclear Regulatory Commission Region IV
612 E. Lamar Blvd., Suite 400 Arlington, TX 76011-4125

> Senior Resident Inspector Callaway Resident Office U.S. Nuclear Regulatory Commission 8201 NRC Road Steedman, MO 65077

Mr. Mohan C. Thadani (2 copies) Senior Project Manager, Callaway Plant Office of Nuclear Reactor Regulation U. S. Nuclear Regulatory Commission Mail Stop O-8G14 Washington, DC 20555-2738

Mr. James Polickoski Project Manager, Callaway Plant Office of Nuclear Reactor Regulation U. S. Nuclear Regulatory Commission Mail Stop O-8B1A Washington, DC 20555-2738 ULNRC-05752 December 21, 2010 Page 3

Index and send hardcopy to QA File A160.0761

Hardcopy:

Certrec Corporation 4200 South Hulen, Suite 422 Fort Worth, TX 76109 (Certrec receives ALL attachments as long as they are non-safeguards and may be publicly disclosed.)

LEREvents@inpo.org (must send the WORD version of the LER to this address)

Electronic distribution for the following can be made via LER ULNRC Distribution:

A. C. Heflin F. M. Diya C. O. Reasoner III D. W. Neterer L. S. Sandbothe S. A. Maglio S. L. Gallagher T. L. Woodward (NSRB) T. B. Elwood D. E. Dumbacher (NRC) B. D. Brooks (WCNOC) Ms. Diane M. Hooper (WCNOC) Mr. Tim Hope (Luminant Power) Mr. Ron Barnes (APS) Mr. Tom Baldwin (PG&E) Mr. Wayne Harrison (STPNOC) Ms. Linda Conklin (SCE) Mr. John O'Neill (Pillsbury Winthrop Shaw Pittman LLP) Missouri Public Service Commission Records Center (INPO)

NRC FOF (10-2010)	NRC FORM 366 (10-2010)			U.S. NUCLEAR RE	EGULATOP		SSION						10/31/2013	
		(See r	reverse fo	ENT REPORT or required nun sters for each bl	nber of			reques licensi estima Comm infoco Regula Budge collect not co	ated burden per st: 50 hours. F ing process and ate to the FOIA hiestion, Washin likets@nrc.gov, atory Affairs, Nf et, Washington, ton does not disp onduct or spons nation collection.	Reported less fed back to ind /Privacy Secti igton, DC 20 and to the E COB-10202, (3 DC 20503. If a blay a currently	ons learned are dustry. Send com on (T-5 F53), L D555-0001, or Desk Officer, Of 3150-0104), Offic a means used to valid OMB contr	 incorporat incorporat internet by internet fice of Info ce of Manago impose an ol number, the 	ted into the rding burden r Regulatory t e-mail to rmation and gement and n information he NRC may	
1. FACIL			:1 4					2. DO	OS000483	ER	3. PAGE	OF 7		
4. TITLE		Plant U						<u> </u>	0000405					
		y Diese	I Generat	tor A Shutdowr	n during	24 Hour	· Surve	eilland	се					
5. EVENT DATE 6. LER NUMBER 7. REPORT DATE				ATE			OTHER FAC	CILITIES INVO						
MONTH	DAY	YEAR	YEAR S	EQUENTIAL REV NUMBER NO.	MONTH	DAY	YEAF					DOCKET N	JUMBER	
03	30	2010	2010 -	- 005 - 01	12	21	201		CILITY NAME			DOCKET N	JUMBER	
9. OPER	ATING	MODE	11. T	THIS REPORT IS	SUBMITT	ED PURSI	UANT T	O THE	EREQUIREM	ENTS OF 10	CFR§: (Cheo	k all that a	apply)	
1				 20.2203(a)(3)(i) 20.2203(a)(3)(ii) 20.2203(a)(4) 50.36(c)(1)(i)(A) 		(3)(ii) (4)	□ 50.73(a)(2)(ii)(A) □ 50.73 □ 50.73(a)(2)(ii)(B) □ 50.73 □ 50.73(a)(2)(ii)(B) □ 50.73 □ 50.73(a)(2)(iii) □ 50.73		8(a)(2)(vii) 8(a)(2)(viii)(A) 8(a)(2)(viii)(B) 8(a)(2)(ix)(A)					
10. POW	10. POWER LEVEL		□ 20.220 □ 20.220 □ 20.220	D3(a)(2)(ii) D3(a)(2)(iii) D3(a)(2)(iv) D3(a)(2)(v) D3(a)(2)(vi)	 ☐ 50.36(c)(1)(ii)(A) ☐ 50.36(c)(2) ☐ 50.46(a)(3)(ii) ☐ 50.73(a)(2)(i)(A) ⊠ 50.73(a)(2)(i)(B)))(ii))(i)(A)		 □ 50.73(a)(2)(iv)(A) ⊠ 50.73(a)(2)(v)(A) ⊠ 50.73(a)(2)(v)(B) □ 50.73(a)(2)(v)(C) ⊠ 50.73(a)(2)(v)(D) 		☐ 73.7 ☐ 73.7 ☐ OTH Spec	☐ 73.71(a)(4) ☐ 73.71(a)(5)		
				1	2. LICENS	SEE CONT	FACT F	OR TH	IIS LER					
FACILITY N T.B. EIN		•		ineer, Regulato				-		31	LEPHONE NUMBE	•	∋a Code)	
			13. COMP	LETE ONE LINE F	FOR EAC		NENT	FAILU		ED IN THIS	REPORT			
CAU	SE	SYSTEM	COMPONE	ENT FACTURER	REPOR TO E			JSE	SYSTEM		IT MANU- FACTURER		DRTABLE DEPIX	
A		EK	ENG	6 F010	Ŷ								· · · · · · · · · · · · · · · · · · ·	
🗆 YE	S (If ye			ENTAL REPORT		_	⊠ NO		SUBM	PECTED ISSION ATE	MONTH	DAY	YEAR	
ABSTRA	CT (Lin	nit to 1400	spaces, i.e	e., approximately 1	15 single-s _i	paced type	əwritten	lines)	1				<u> </u>	
The Pos	ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) The A Emergency Diesel Generator (EDG) failed on March 30, 2010, during a 24-hour Technical Specification (TS) run. Post-trip indications showed that the A EDG tripped on reverse power. The B EDG was operable and in its normal standby lineup. Offsite power was available at the time of the event.													
rest	tored to	o operabl	e status w	e and means to re within the TS allo approval letter :	owed time	eframe. E	Enforce	ement	discretion (I	ED) was ree	quested and g	granted. '		

The cause of the A EDG shutdown was due to a failure of the governor to maintain the fuel rack open during operation. The root cause for this event was inattention to detail by craft during fabrication and installation of the EDG governor gear drive assembly gasket which resulted in the failure of the governor drive due to blockage of the lubrication supply port. The fabrication and installation of the gasket occurred eleven years prior to the EDG failure.

Corrective action includes revising work instructions for the governor drive assembly for the emergency diesel engines to verify the governor drive assembly gasket has an oil supply port and that it is appropriately aligned to the oil port in the governor drive case during installation.

NRC FORM 366A (10-2010)	<u> </u>	LICENSEE EVENT	REPORT (LER) ^{U.S. NUCLEAR R} DN SHEET	EGULATORY COMMISSION
	1. FACILITY NAME	2. DOCKET	6. LER NUMBER	3. PAGE

1. FACILITY NAME	2. DOCKET	6. LER NUMBER				3. PAGE		
Colloway Plant Unit 1	05000483	YEAR	SEQUENTIAL NUMBER	REV NO.	2	OF	7	
Callaway Plant Unit 1	05000465	2010	- 005 -	01	2	OF	1	

1. DESCRIPTION OF STRUCTURE(S), SYSTEM(S) AND COMPONENT(S):

The Class IE AC system at Callaway is divided into two redundant load groups. Either one of the load groups is capable of providing power to the systems/components needed to safely reach cold shutdown. Each AC load group consists of a 4.16-kV bus, 480-V load centers, 480-V motor control centers, and lower voltage AC supplies. Each 4.16-kV load group is supplied by two preferred power supply feeders (offsite power circuit connections) and one diesel generator (standby) supply feeder. One (either) load group is adequate to satisfy minimum engineered safety features demand caused by a LOCA and loss of preferred power supply.

The standby power supply for each safety-related load group consists of one diesel generator [EIIS system EK, EIIS component DG] complete with its accessories and fuel storage and transfer systems. It is capable of supplying essential loads necessary to reliably and safely shut down and isolate the reactor. Each diesel generator is rated at 6,201 kW for continuous operation. One diesel generator is connected exclusively to a single 4.16-kV safety feature bus of a load group. The diesel generators are electrically isolated from each other.

The diesels [EIIS component ENG] are manufactured by the Fairbanks Morse division of Colt Pielstick Industries. Each engine is a four stroke, V-14, 8600 hp, 514 rpm, water cooled engine. The engine control system consists of the governor, auxiliary start and shutdown control servo, overspeed trip, fuel shutdown servo, and linkages to position the fuel injector pump racks. Each diesel uses a Woodward governor [EIIS component 65] Model EGB-50P to regulate engine speed under varying load conditions.

The governor provides the motive force to position the fuel rack of the EDG. The EDG governor operates as a hydraulic actuator and is coupled to the EDG by a splined input shaft. A splined coupling in the governor drive case engages the splined governor input shaft. The splined coupling is supported by an oil lubricated bushing within the governor drive case. The EDG governor regulates the amount of fuel supplied to the engine by manipulating fuel rack position in response to signals from the electronic governor or backup mechanical governor. To maintain constant engine speed, the governor, through linkages to the fuel injection pumps, increases or decreases the fuel supplied to the engine as applied load increases or decreases. The motive force for controlling the fuel rack is provided by a pump within the governor. This shaft-driven pump connects to the governor drive unit at the spline coupling.

2. INITIAL PLANT CONDITIONS:

On March 30, 2010, Callaway plant was in Mode 1 at 100 percent power. Plant personnel were performing a 24-hour loaded run of the train A emergency diesel generator (EDG) per Technical Specification (TS) Surveillance Requirement (SR) 3.8.1.14. The B EDG was operable and in its normal standby lineup as required by TSs. The A EDG was paralled to the

NRC FORM 366A	LICENSEE EVENT R	EPORT (LER) ^{U.S. NUCLEAR R}	EGULATORY COMMISSION
(10-2010)	CONTINUATION	N SHEET	
	2 DOCKET	6. LER NUMBER	3. PAGE

1. FACILITY NAME	2. DOCKET	6. LER NUMBER				3. PAGE		
			SEQUENTIAL	REV				
Callaway Plant Unit 1	05000483	YEAR	NUMBER	NO.	- 3	OF	7	
Callaway Flanc Onic F	00000100	2010	- 005 -	01		01		

grid via the A train 4160-VAC safety-related bus NB01 such that offsite power was available to the train A 4160-VAC safety-related bus. Offsite power was being supplied to the B train 4160-VAC bus NB02 via the normal supply breaker for that bus.

3. EVENT DESCRIPTION:

The event addressed by this LER is a failure of the A EDG to continue to run during performance of a surveillance (24-hour run) on March 30, 2010.

The 24-hour run was begun with a slow start of the A EDG at approximately 0125 on March 30, 2010. Nothing unusual was observed at that time. The EDG load was subsequently ramped up to six megawatts such that full load was achieved at approximately 0145. There were no alarms or indication present while the EDG continued to run.

Shortly after 1800, the Reactor Operator (RO) observed both megawatts and voltage lowering, so some slight adjustments to power and voltage were made. Load and voltage parameters were maintained within the required range.

At 1808, alarms unexpectedly came on for an automatic diesel shutdown. The following annunciators came in:

Main Control Board Annunciator: 20D - DG NE01 TROUBLE 20B – alarmed and cleared on under voltage/under frequency

Local annunciators: 6D - ENGINE TROUBLE SHUTDOWN 7A - GENERATOR UNDERFREQUENCY 7D - GENERATOR PROTECTIVE RELAY

Flags dropped locally for Protective Relays: NE107 132 DG REVERSE POWER NE107 130 DG UNIT S/D TARGET RELAY NE107 124 1DG VOLT HZ NE107 186 1DG LOCK OUT RELAY

The Field Supervisor inspected the A EDG. He saw no apparent cause for the trip. A System Engineer came on site at approximately 1915 and conducted a walk down of the XNB01 transformer, the NB03 capacitor bank, the diesel generator output breaker, and the diesel room. There were no flags dropped on NB0111, the diesel generator output breaker. There were no other lockouts on the NB01 bus and no indications of any faulted equipment with lockouts.

NRC FORM 366A LICENSEE EVENT REPORT (LER) ^{U.S.} NUCLEAR REGULATORY COMMISSION (10-2010) CONTINUATION SHEET									
1. FACILITY NAME	2. DO	СКЕТ	6. LER NUMBER	3. PAGE					
			SEQUENTIAL	REV					

Callaway Plant Unit 1	05000483	YEAR	NUMBER	NO.	4	OF	7
	00000100	2010	- 005 -	01			
NARRATIVE							

Post-trip indications showed that the A EDG tripped on reverse power. A reverse power scenario occurs if the diesel engine is no longer supplying sufficient power (torque) to the generator. If insufficient power is generated by the diesel engine, the generator will act as a motor and drive the diesel engine.

A one-hour run of the B EDG was completed to meet the common-cause determination requirement of TS Required Action B.3.2.

4. ASSESSMENT OF SAFETY CONSEQUENCES:

This event resulted in the loss of the safety function of the A EDG to provide emergency 4160-VAC power to the NB01 bus in the event of a loss of offsite power (LOOP). This event was evaluated with the Callaway PRA model. The evaluation determined the conditional core damage probability (CCDP) of this event was less than 1E-6; therefore, this event was of very low risk significance. Use of the PRA model to evaluate the event provides for a comprehensive, quantitative assessment of the potential safety consequences and implications of the event, including consideration of alternative conditions beyond those analyzed in the FSAR.

It was determined that the A EDG was inoperable between May 11, 2007 and April 4, 2010. (See section 5 below.) Review of plant documents indicates that the B EDG was taken out-of-service during the timeframes listed below (while it was unknown that the A EDG was inoperable). During these timeframes both EDGs were inoperable.

Finish	Duration (HH:mm)
3/19/2010 5:46	41:45
10/30/2009 1:21	69:11
6/4/2009 0:53	20:27
4/9/2009 0:00	217:00
12/25/2008 20:46	28:22
10/30/2008 9:43	129:41
10/1/2008 16:03	36:00
6/11/2008 1:41	21:11
5/7/2008 19:55	39:37
3/18/2008 23:16	12:41
3/1/2008 2:52	21:52
12/17/2007 23:47	14:58
10/4/2007 13:25	57:23
8/12/2007 21:49	39:53
	3/19/2010 5:46 10/30/2009 1:21 6/4/2009 0:53 4/9/2009 0:00 12/25/2008 20:46 10/30/2008 9:43 10/1/2008 16:03 6/11/2008 1:41 5/7/2008 19:55 3/18/2008 23:16 3/1/2008 2:52 12/17/2007 23:47 10/4/2007 13:25

NRC FORM 366A (10-2010)	LICENSEE EVENT F			LEAR REG	GULATO		MISSION	
1. FACILITY NAME	2. DOCKET		6. LER NUMBER			3. PAGE		
Colloway Plant Linit	1 05000492	YEAR	SEQUENTIAL NUMBER	REV NO.		OF	7	
Callaway Plant Unit	1 05000483				1 3	0F	1	

REPORTING F	REQUIREMEN	ſS:
	• •	

This LER is submitted pursuant to 10CFR50.73(a)(2)(i)(B) to report a condition prohibited by the Technical Specifications and 10CFR50.73(a)(2)(v) as an event or condition that could have prevented fulfillment of a safety function.

2010

005

01

10CFR50.73(a)(2)(i)(B): The A EDG failed at approximately 16.7 hours into its 24-hour TS surveillance run. TS SR 3.8.1.14 requires verifying that "each DG operates at a power factor less than or equal to 0.9 and greater than or equal 0.8 and operates for greater than or equal to 24 hours ..." Since this was not met, the condition constituted a failure to meet the TS Limiting Condition for Operation (LCO).

TS 3.8.1 requires both EDGs to be operable during plant operation (i.e., Modes 1, 2, 3, and 4). With one EDG inoperable during plant operation, Condition B in the ACTIONS section of TS 3.8.1 applies, and per associated Required Action B.4, the inoperable EDG must be restored to operable status within 72 hours. If the Completion Time of 72 hours cannot be met, Condition G must be entered, wherein Required Action G.1 requires commencement of a controlled plant shutdown such that the plant is required to be in Mode 3 within 6 hours AND in Mode 5 within 36 hours.

After the test failure, and once the cause of the failure and the means to restore the EDG were identified, Callaway Plant determined that the EDG would not be restored to operable status within the timeframe allowed by the TS. Callaway Plant requested and NRC granted enforcement discretion. This is documented in AmerenUE ULNRC-05693 and in the NRC's approval letter for Notice of Enforcement Discretion (NOED) 10-4-001. The A EDG was declared operable on April 4, 2010 at 1220. The plant was not required to be shut down because the EDG was repaired and declared operable within the timeframe allowed by the NOED.

Results from the cause investigation indicate that the EDG was not able to meet its 24-hour surveillance time prior to the failure on March 30, 2010. Based on evaluation of the failure mechanism that caused the A EDG to become inoperable, it is estimated that the last time the EDG would have been able to meet the 24-hour surveillance test was on February 3, 2010. That timeframe exceeds the 72-hour allowed out-of-service time permitted by TS 3.8.1 per Required Action B.4.

Beyond the above, an evaluation was performed to determine when the A EDG became incapable of meeting its 7-day mission time (which is also considered to be an operability requirement). Based on the failure mechanism identified for the EDG (as described in section 6), and as determined by extrapolation of the wear rate associated with the failure mechanism (as related to the number of operating hours accumulated by the EDG), it was determined that the EDG became incapable of meeting its 7-day mission in May 2007. (Specifically, it was during the performance of a 24-hour run on May 11, 2007 that the accumulated hours of operation exceeded the threshold established as the point when the EDG became incapable of

NARRATIVE 5.

NRC FORM 366A (10-2010) LICENSEE EVENT REPORT (LER) ^{U.S. NUCLEAR REGULATORY COMMISSION} CONTINUATION SHEET									
1. FACILITY NAME	2. DOCKET		6. LER NUMBER 3. PAGE						
Collowov Plant Linit	1 05000483	YEAR	SEQUENTIAL NUMBER	REV NO.	6	OF	7		
Callaway Plant Unit 1	1 05000465	2010	- 005 -	01		OF	1		

meeting its 7-day mission time). On this basis the EDG was determined to be inoperable from May 11, 2007 to April 4, 2010 (when it was restored to operable status following the March 30, 2010 test failure).

10CFR50.73(a)(2)(v): As discussed above, the A EDG was determined to be inoperable between May 11, 2007 and April 4, 2010. Review of plant documents indicates that the B EDG was taken out-of-service during the timeframes listed above in the Assessment of Safety Consequences section (as it was not known that the A EDG was inoperable during those times). During these timeframes both EDGs were inoperable such that the EDG safety function(s) could not be fulfilled.

During a design basis accident, the EDGs are designed and intended to supply power to the ECCS systems and residual heat removal (RHR) pumps and valves, as well as the component cooling water systems and essential service water systems. As such, the EDGs support safety functions A, B, and D of 10CFR50.73(a)(2)(v). Those safety functions are:

(A) Shut down the reactor and maintain it in a safe shutdown condition,

(B) Remove residual heat,

(D) Mitigate the consequences of an accident.

6. CAUSE OF THE EVENT:

The cause of the A EDG shutdown was due to a failure of the governor to maintain the fuel rack open during operation. The governor operates as a hydraulic actuator. Hydraulic pressure within the governor was lost when the splined coupling on the drive shaft failed. Troubleshooting, disassembly and inspection of the governor drive assembly revealed an improperly cut gasket which blocked the oil port to the gear case. Operation without forced oil lubrication to the governor drive gear case resulted in increased wear of the vertical shaft bearing. The bearing wear resulted in the radial movement of the vertical gear shaft in the gear case which caused wear on the teeth on the spline sleeve and mating male spline on the governor drive shaft. The worn teeth on the spline resulted in the governor failing to rotate with engine speed and subsequent diesel shutdown on March 30, 2010. The damaged parts were replaced and a post-maintenance test was conducted to restore the EDG to an operable condition.

The root cause for this event was determined to be inattention to detail by craft during fabrication and installation of the EDG governor gear drive assembly gasket which resulted in the failure of the governor drive due to blockage of the lubrication supply port. This fabrication and installation of the gasket occurred eleven years prior to the EDG failure event. In addition, a contributing cause was determined to be the failure to implement industry recommended preventive maintenance practices which if implemented may have identified the lack of forced oil flow in the governor drive or the degrading governor spline and degraded

NRC FORM 366A (10-2010) LICENSEE EVENT REPORT (LER) ^{U.S.} NUCLEAR REGULATORY COMMISSION CONTINUATION SHEET									
1. FACILITY NAME	2. DOCKET		6. LER NUMBER				3. PAGE		
Colleway Plant Linit 1	05000482	YEAR	SEQUENTIAL NUMBER	REV NO.	7	OF	7		
Callaway Plant Unit 1	05000483	2010	- 005 -	01	′	OF	1		

governor drive components prior to the failure event.

As an extent of condition review, oil flow to the B EDG governor drive assembly was verified on April 6, 2010. Additionally, the B EDG governor was removed and inspected on April 23, 2010. The spline on the shaft was found to be in like new condition with no evidence of wear.

7. CORRECTIVE ACTIONS:

The Corrective Action to Prevent Recurrence is to revise the standard work instructions for the governor drive assembly for the emergency diesel engines to verify the governor drive assembly gasket has an oil supply port and that it is appropriately aligned to the oil port in the governor drive case during installation. In addition, an independent verification for this configuration was added to the standard work instructions. The revision of the standard work instructions has been implemented.

Corrective actions to ensure proper material stocking levels and to more effectively implement vendor maintenance recommendations for the EDGs and other risk-significant equipment were implemented.

8. PREVIOUS SIMILAR EVENTS:

The EDG failure event causal factor was: Governor Drive Assembly Gasket did not have the oil supply port. None of the internal operating experience reviewed had events originating in this Causal Factor.