

DEPARTMENT OF VETERANS AFFAIRS Veterans Health Administration National Health Physics Program 2200 Fort Roots Drive North Little Rock, AR 72114

DEC 13 2010

In Reply Refer To: 598/115HP/NLR

Kevin Null Division of Nuclear Material Safety Nuclear Regulatory Commission (NRC), Region III 2443 Warrenville Road, Suite 210 Lisle, Illinois 60532-4352

Dear Mr. Null:

In reference to NRC Master Materials License 03-23853-01VA, we are requesting release, for unrestricted use, of an incinerator and associated smoke stack adjacent to Building 4 at VA Medical Center, Kansas City, Missouri. The facility holds Permit Number 24-00496-06 under our master materials license. Pending approval by NRC and a permit amendment by VHA National Health Physics Program (NHPP), the permittee plans to demolish the incinerator building and associated stack. We are enclosing supporting information and closeout surveys that were performed by the permittee to support release of the structures. We request an expedited review of this request by NRC, if possible, since the permittee states the stack poses "a safety hazard to approaching and departing ambulance helicopters."

We request that the Derived Concentration Guideline Levels (DCGLs) published in Federal Register Vol. 63, No. 222, and consistent with Federal Register Vol. 65, No. 114, be applied as the release criteria. Using these criteria, the enclosed documents provide information consistent with 10 CFR 30.36 to evaluate these buildings for decommissioning action, and we conclude the enclosed survey results demonstrate that the areas are acceptable for unrestricted use in accordance with regulatory criteria in 10 CFR 20.1402.

If you have any questions or comments or need additional information to support your review, please contact Thomas E. Huston, Ph.D., NHPP, at 501-257-1578.

Sincerely,

GE Williams

Gary E. Williams Director, National Health Physics Program

Enclosures (5)

Permittee letter dated November 22, 2010, requesting release of structures and providing closeout survey information



DEPARTMENT OF VETERANS AFFAIRS Medical Center 4801 Linwood Boulevard Kansas City, MO 64128

November 22, 2010

To: National Health Physics Program (115HP/NLR) Department of Veterans Affairs Veterans Health Administration 2200 Fort Roots Drive, Bldg 101, Room 208 North Little Rock, AR 72114 11-29-10 A11:16 IN

From: Les Morrison, RSO Kansas City VAMC 4801 E. Linwood Blvd Kansas City, MO 64128

Les Morison

Subject: Decommissioning of Incinerator and smoke stack

The Kansas City VA Medical Center, permit number 24-00496-06 is requesting decommissioning approval of an incinerator and an associated brick smoke stack. The approximate dimensions of the incinerator are 7 ft wide x 27 ft long x 8 ft high. The incinerator was installed approximately 1998 and operated until September 2000. This incinerator was operated under a previous NRC Limited Scope License, # 24-00496-06. The incinerator was used for incinerating low level radioactive waste, including animal carcasses. There is no indication that it was used for the incineration of hazardous chemicals.

The associated smoke stack is a red brick stack, approximately 250 ft in height and was in use prior to the installation of the incinerator as part of the boiler plant. The removal of the smoke stack is important because it is a safety hazard for approaching and departing ambulance helicopters.

Richard Poelling, a consultant Health Physicist to the KCVA from 2000 to 2004, performed a close out survey of the incinerator chamber in November, 2003. This was over 3 years since the incinerator was used last. According to Mr. Poelling' report, H-3 and I-125 were the only isotopes which were incinerated during the last 2 years of the incinerator's operation. Samples from the fire brick were taken and analyzed by liquid scintillation. Windows for H-3 and I-125 protocols were used. An open window protocol was used to detect any unknown isotopes which might have been involved. He found no indication of any contamination.

Isotopes which were used included H-3, C-14, P-32, P-33, S-35, Cl-36 and I-125. The data for the H-3 and I-125 is in Mr. Poelling's report. In the event that the other isotope may have been incinerated, P-32, P-33 and S-35 all have half lives of less than 90 days. The C-14 and Cl-36 which have long half lives, would have been detected in the open window channel in the event they were present. The survey procedure was determined with the verbal consultation of the NHPP staff.

The removal and demolition of the incinerator and smoke stack is anticipated as soon as funding is available.

If you have any questions or need further information, please contact me at <u>Leslie.Morrison@VA.GOV</u> or at 816-861-4700 ext 57699.

Attached: Richard Poelling's Incinerator Close out Survey report dated November 20, 2003

Department of Veterans Affairs

Memorandum

Date: November 20, 2003

From: Richard E. Poelling (Consulting Health Physicist)

To: Radiation Safety Officer

Subj: Close out survey of Building 4 Incinerator Chamber

1. On November 6, 2003 a confined space entry was performed with the support of Jeff Muehlmann, Industrial Hygienist and Robert Kent, Safety Technician into the incinerator chamber to collect scrappings of the fire brick surface and measure for residual radioactive material. The incinerator was taken out of service on September 6, 2000.

2. The exposure rate of the incinerator room and out side of the steel chamber was less than or equal to 0.02 mR/hr using a ludlum-3 GM survey meter with pancake probe. The internal chamber exposure was consistently about 0.03 to 0.04 mR/hr on the surface of the firebrick throughout the unit. Firebricks from the same lot that line the chamber were not available to determine background exposure readings. The consistent exposure rate about twice background was uniform throughout the chamber and problably respresents a natural radioactive material within the firebrick.

3. The surface of the chamber firebricks were scraped and the particles were analyzed in a liquid scintillation counter along with calibrated standards. No residual radioactive material contamination was note on any of the twenty seven samples.

4. The incinerator chamber does not appear to have any residual radioactive material contamination.

RICHARD E. POELLING Consulting Health Physicist

Attachments: Closeout Survey Questionnaire Closeout Survey Grid Information Sample data and raw counts

cc:

Decommissioning File (Radiation Safety Office)

Closeout Survey Questionnaire

Building 4: Boiler Plant Incinerator Chamber

1. List radionuclides used in the room, circle form used (Sealed, Unsealed, or Gas): \star

Radionuclides Used	Form Used	First Use Date	Last Use Date
Hydrogen-3	s 🛈 G	Unknown	Sept 6, 2000
Iodine-125	s (U) G	Unknown	Sept 6, 2000
	SUG		
	SUG		
	SUG	· · · · · · · · · · · · · · · · · · ·	

A review of the available documentation for the last two years of radioactive material incineration indicated that microcurie quantities of I-125 from the radioimmunoassay laboratory and some H-3 were incinerated.

- 2. Did a major spill occur in the room which resulted in residual radioactivity? [X] No [] Yes (if yes, attach description)
- Did any sealed sources stored or used in the room leak or fail a leak test?
 No [] Yes (if yes, attach description)

4. Were sealed sources which required a leak test transferred or relocated from the room? [χ] No [] Yes (if yes, attach regulatory current test results)

5. Were all radioactive materials, sources, and equipment removed?
[] No [X] Yes (if no, attach description)

Incinerator was completely cleaned out of residual ash

Department of Veterans Affairs Kansas City VA Medical Center 4801 Linwood Blvd Kansas City, Missouri Page 1 of 3

* See NHPP reviewer notes, in Enclosure 5. 721 Autor 12/13/2010

- 6. What survey instruments were used?
 - A) Exposure Rate Measurements: Ludlum-3 with pancake probe SN 119621 Calibrated: Jan 31, 2003 Background reading: <0.02 mR/hr
 - B) Surface Scans for Fixed Radioactivity: Ludlum-3 with pancake probe SN 119621 Calibrated: Jan 31, 2003 Background reading: <0.02 mR/hr
 - C) Swipe Surveys for Removable Radioactive Contamination: Beckman LS-6000 Liquid Scintillation Counter (EE589-10841) Sample Container: 20 ml LS glass vials Counting Cocktail: Packard OptiFluor (10 ml) Sample: 0.2 to 0.6 g of firebrick surface scrapings Area Scraped: 100 cm²

³ H Sta	ndard (Unqu	enched)		¹⁴ C Standard (Unquenched)						
106,0	00 dpm	on 4/1	1/2001			101,600 dpm on 4/11/2001					
Count Window:	: ⁽³ H) ¹²⁵	I ¹⁴ C	³⁵ S ³² P \	Vide	Count	Window:	³ H ¹²	²⁵ I ¹⁴ C	³⁵ S ³² P Wide		
Background	Background 13.				Back	ground	3	6.1 cpm			
Standard	5482	27 cpm	cpm 91657 dpm		Sta	ndard	970	074 cpm	101568 dpm		
Effic:	59.8 %	MI	DA 9.0 d	pm	Effic	9	5.5 %	MDA	9.3 dpm		
			Isotope	-	Count Window						
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				(0 1000						

7. Were any results for exposure rate measurements or surface scans for fixed radioactivity greater than background readings? (>20 dpm/100 cm²)
 [V] No
 [] Yes (if yes, attach description)

Department of Veterans Affairs Kansas City VA Medical Center 4801 Linwood Blvd Kansas City, Missouri Page 2 of 3

* See NHPP reviewer note for item 7 in Enclosure 5. 7914 to 12/13/2010

8. Were any swipe survey results greater than applicable release criteria in NUREG 1556, Volume 11, Table S.5?

[X] No [] Yes (if yes, attach description)

Acceptable Surface Contamination Levels in Unrestricted Areas											
Nuclide	Average	Maximum	Removable								
I-125, I-129	100 dpm/100cm ²	300 dpm/100cm ²	20 dpm/100cm ²								
I-126, I-131, I-133, Sr-90	1,000 dpm/100cm ²	3,000 dpm/100cm ²	200 dpm/100cm ²								
Beta-Gamma Emitters	5,000 dpm/100cm ²	15,000 dpm/100cm ²	1,000 dpm/100cm ²								

Contamination found in unrestricted areas should be immediately decontaminated to background levels. When it is not possible to get to background levels, the licensee must ensure that the amounts do not exceed the contamination levels listed above.

9. Information attached (other than listed above):

- [] Room diagram
- [X] Survey grid
- [X] Counting system calibration / quality assurance results

[X] Other (specify) Raw data counts

10. Point of contact Name: Richard E. Poelling, Health Physicist Consultant

- Phone: (573) 814-6000 Ext. 2590
- Fax: (573) 814-6600

E-mail: richard.poelling@med.va.gov

R. Chelling 11/20/2003 11. Survey performed by: Date: 12-19-03 12. Survey approved by:

Department of Veterans Affairs Kansas City VA Medical Center 4801 Linwood Blvd Kansas City, Missouri Page 3 of 3 For item 8 in Enclosure 5. Julian 12/13/240

Closeout Survey Grid Information For Incinerator Chamber

Location: Building 4, Boiler Plant

Date Surveyed: November 6, 2003

1. The sample number in each box corresponds to a grid number on the chamber diagram.

2. Exposure rate measurements in mR per hour. Background readings: <0.02 mR/hr

3. Dpm swipe survey results, in disintegrations per minute per 100 cm² for three energy levels. The firebrick surface was scraped in the specified areas in an area of about 4" x 4". The scraped particles were placed in individual glass vials for future processing. Each sample was weighed and placed in a clean 20 ml glass scintillation vial with 10 ml of Packard "Opti-Fluor counting cocktail and allowed to settle in the dark for 2 days. A Beckman liquid scintillation counter was used to count each sample for 10 minutes in a ³H, ¹²⁵I, and wide window setting. Each sample was counted 3 times and the data was averaged.

4. See attached diagram and spread sheet for data.

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Kansas City VA Medical Center Kansas City, Missouri

Sample	Sample	Firebrick	H-3 Window		I-125 Wir	I-125 Window		Wide Window		
Number	туре	Wt (a)	com	dpm	com	dpm	cpm	mqb	mR/hr	
L	I		1			1	1	1		
1	Scraping	0.25	13.9	0	10.7	0	44.9	0	< 0.04	
2	Scraping	0.14	11.4	0	8.5	0	38.2	0	< 0.04	
3	Scraping	0.54	16.0	0	10.8	0	54.3	0	< 0.04	
4	Scraping	0.27	14.2	0	11.0	0	45.0	0	< 0.04	
5	Scraping	0.16	10.9	0	7.7	0	35.3	0	< 0.04	
6	Scraping	0.23	11.3	0	7.4	0	39.4	0	< 0.04	
7	Scraping	0.19	13.3	0	8.0	0	40.4	0	< 0.04	
8	Scraping	0.32	11.3	0	7.8	0	36.6	0	< 0.04	
9	Scraping	0.26	12.2	0	7.6	0	42.0	0	< 0.04	
10	Scraping	0.41	14.2	0	12.3	0	54.8	0	< 0.04	
11	Scraping	0.60	13.5	0	11.3	0	52.5	0	<0.04	
12	Scraping	0.53	14.2	0	8.1	0	47.1	0	<0.04	
13	Scraping	0.24	9.7	0	6.6	0	35.5	0	<0.04	
14	Scraping	0.17	8.8	0	7.0	0	33.3	0	<0.04	
15	Scraping	0.45	8.6	0	6.9	0	35.4	0	<0.04	
16	Scraping	0.26	14.9	0	10.5	0	46.4	0	<0.04	
17	Scraping	0.26	12.0	0	8.3	0	38.7	0	<0.04	
18	Scraping	0.28	12.0	0	9.2	0	43.4	0	<0.04	
19	Scraping	0.71	12.0	0	8.8	0	41.8	0	<0.04	
20	Scraping	0.52	14.4	0	11.0	0	51.0	0	<0.04	
21	Scraping	0.32	12.3	0	7.7	0	38.8	0	<0.04	
22	Scraping	0.52	13.3	0	8.4	0	43.7	0	< 0.04	
23	Scraping	0.30	9.0	0	7.0	0	35.2	0	< 0.04	
24	Scraping	0.28	11.7	0	8.7	0	40.6	0	<0.04	
25	Scraping	0.40	13.5	0	8.3	0	41.9	0	< 0.04	
26	Scraping	0.46	12.3	0	8.8	0	44.2	0	<0.04	
27	Scraping	0.43	9.5	0	8.6	0	38.2	0	<0.04	
28	Bkg	0.22	17.5		9.7		49.3		0.02	
29	Bkg	0.43	21.5		11.9		60.6		0.02	
30	Bkg	0.64	27.5		15.8		69.7		0.02	
31	H-3 Std		54827.0	59.80%	816.0	0.90%	55663.0	60.70%		
32	C-14 Std		17669.0	17.40%	49337.0	48.60%	97074.0	95.60%		
33	Unq Bkg		13.3		4.9		36.1			

Building 4 Incinerator Close Out Survey Data Sheet

Sample Quench Range (H#)114-158Background Quench Range (H#)104-109

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Exposure rate of the incinerator room was consistently <0.02 mR/hr The outside of the incinerator housing was consistently <0.02 mR/hr Inside the incinerator chamber next to the firebrick was consistently 0.03 to 0.04 mR/hr The firebrick appears to contain sometype of natural radioactive material.

BECKMAN LS-6000 LIQUÍO SCINTILLATION COUNTER. PABEL 1 SN EE 10,841 TRUMANVA. PABEL 1

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LEER: 6		COMMENT:F	TRE BRICK SCRAF				
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COUNT BLANK :	NO.	ICH : N	REPLICATES	: 1	. F65232	3	» ()
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SCINTILLATOR:	LIGUID	LLMEX:YES	LOW SAMPLE RE	Ju C)		
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2	**-2	10.00	124.4	10.30	19.71	9.CO	21.08	38.30	10.22	0.03	52.70
23	**-2	10.00	123.0	12.80	17.68	8.40	21.82	40.40	9.95	o,, o3	63.14
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		Repeat	Average	CFM for	WIDE	35,30	CŒF. O	F VAR:	4.173		
Ś	来来一台	10.00	123.7	12.20	18,11	8.00	22.36	39.50	10.06	0.05	168.77
6	**	10.00	125.6	10.10	19,90	6.10	25.61	38.30	10,22	0.*04	179.20
6	**6	10.00	124.8	11.50	18.65	8.10		40.30	9.96	0.04	189.62
		Repeat	Average	(CPM for		11.27	COEF. O	F VAR:	9,491		
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Radioactive Decay Calculator For Sealed Sources

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Todays Date 11/9/2003

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	isotope	Activity	Campiation	Dayson	Kemanning	ACTIVITY	1.D.	Decay racior
			Date	Decay	Activity	Units	No.	(Days)

E-Vial Reference Sources

Co-57	5.329	10/1/2002	404	1.902	mCi	146	271.79
Co-57	5.608	10/15/2000	1120	0.323	mCi	133	271.79
Ba-133	250	5/9/1985	6758	74.001	uCi	H-5	3847
Ba-133	272	11/3/1998	1832	195.544	uCi	126	3847
Cs-137	201	8/14/1984	7026	128.850	uCi	B-28	10950

Gamma Reference Disc Source Set (NES-101S)

Na-22	1.1	2/28/1994	3541	0.08310	uCi	950
Mn-54	1.16	8/2/1994	3386	0.00064	uCi	312.7
Co-57	0.9959	9/15/2002	420	0.34129	uCi	271.79
Co-57	1.18	6/29/1994	3420	0.00019	uCi	271.79
Co-60	0.84	6/29/1993	3785	0.21488	uCi	1924
Cd-109	8.5	2/23/1994	3546	0.04260	uCi	464
Ba-133	1.05	6/18/1993	3796	0.52992	uCi	3847
Cs-137	1.02	1/31/1994	3569	0.81378	uCi	10950

GM Calibration Source

Cs-137	135	1/31/1984	7222	85.474	mCi	10950
Sr-90	9.879	9/15/2002	420	9.606	uCi	10402.5
Sr-90	0.022	2/11/1976	10133	0.011	uCi	10402.5
C-14	0.182	2/9/1976	10135	0.181	uCi	2091450

12x75 Well Counter Sources

Co-57	213000	4/21/2003	202	126827	dpm	270
1-125	264000	4/17/2003	206	24645	dpm	60.2
I-129	107000	1/27/2003	286	107000	dpm	6205000000
Co-57	213000	8/14/2002	452	66764	dpm	270
I-125	213000	8/14/2002	452	1098	dpm	59.46
Co-57	222000	4/24/2000	1294	8016	dpm	270
I-125	262000	4/12/2000	1306	0	dpm	59.46

Unquenched Liquid Scintillation Standards (Beckman 9696)

H-3	106000	4/11/2001	942	91657	dpm	4490
C-14	101600	4/11/2001	942	101568	dpm	2091450

NHPP memorandum dated December 3, 2010, requesting additional information

Memorandum

DEPARTMENT OF VETERANS AFFAIRS

Date 0EC 0.3 2010

From: Director, VHA National Health Physics Program (NHPP) (115HP/NLR)

Subj. Request for Information for VHA Permit Number 24-00496-06 (for radioactive materials use)

To: Director (589/00), VA Medical Center, Kansas City, Missouri

1. We are responding to a memorandum from your Radiation Safety Officer (RSO) that was dated November 22, 2010, and requested decommissioning approval (i.e., release for unrestricted use) of an incinerator and associated smoke stack which were historically used to incinerate wastes containing radioactivity.

2. Due to the circumstances of the release (viz., historical use of radionuclides with half-lives greater than 120 days and plans for demolition of these structures), we will need to defer this request to the Nuclear Regulatory Commission (NRC) for their review and approval before we can approve a permit amendment to release the structures for unrestricted use. The NRC review process often takes up to 60 days, depending on their priorities and any follow-up questions.

3. As additional information for our review, please provide a site diagram showing the location of the structures to be released relative to other existing buildings and site boundaries. The diagram may be provided in hardcopy format or electronically to our group e-mail at <u>vhconhpp@va.gov</u>. Please provide this information within 30 days from the date of this memorandum. After we receive this information, we will forward a request to the NRC to release structures. We will provide a copy of the submittal to your RSO as information.

4. Please note 10 CFR 30.36(d) requires notifications to NHPP/NRC within specific timeframes for circumstances involving an inactive separate building where residual radioactivity above release levels could exist. In this case, we received no notification about status of these inactive structures until receipt of the RSO memorandum dated November 22, 2010. Also, the release survey for the structures appears to have been completed about a year after the notification window closed. As a mitigating factor, the radiation surveys that were provided by the RSO appear to support the absence of residual radioactivity in these structures. To identify whether other similar circumstances might exist and to prevent recurrence of this apparent deficiency, you should evaluate whether additional corrective actions are warranted. NHPP will likely follow up on this item during your next routine facility inspection.

Page 2

Request for Information for VHA Permit Number 24-00496-06 (for radioactive materials use)

5. Please note that, as a limited-scope research permittee, a permit amendment from NHPP is required to add new areas of radioactive material use and to release, for unrestricted use, areas in which radioactive materials were previously used.

6. If you have any questions about this memorandum, please contact Thomas E. Huston, Ph.D., NHPP, at 501-257-1578.

GEILDE

Gary E. Williams

Permittee e-mail dated December 3, 2010, with attached facility diagram and requesting expedited review

(Note: Item 2 in that e-mail references a different area and is not included in this request.)

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From:	Morrison, Leslie (KCVA)
Sent:	Friday, December 03, 2010 2:00 PM
То:	VACO NHPP
Cc:	Hill, Kent D. (SES- KCVA)
Subject:	Requested diagram plus more information
Attachments:	KANSAS CITY VA SITE PLAN.pdf

Attached is a copy of the site diagram that you requested. It is a PDF format so you should be able to view and print the diagram easily. There are 2 points that I need to bring to your attention on this diagram.

1. The smoke stack that we are requesting decommissioning of is the circle, located in the lower right hand corner. It is just above the upper left corner of Building 4. The incinerator that we are requesting to be decommissioned is located in an extension of Building 4, just to the right of the smoke stack. The structure between the smoke stack and the extension of Building 4 is an above ground walkway.

2. If you look at the bottom of the diagram and towards the middle, you will see a small building structure that is labeled, "Incinerator". This building is a small 2 level building. I have found out that this building housed an incinerator, which was used for the incineration of radioactive waste prior to the current incinerator that we are requesting decommissioning of. This previous incinerator started operation approximately 1975, was located in the lower part of the building. This incinerator was approved by the NRC for incineration of radioactive material in May of 1980, as part of Amendment #41. This incinerator was removed many years ago and the building has been used for storage. I have not been able to find any records of what materials were actually burned. According to a H-3, C-14, I-125, March 28, 1980 request, the anticipated isotopes to be incinerated were: P-32, V-48, S-35, Fe-55, Fe-59 and Se-75. Animal carcasses (rabbits, rats and mice) would be used with an anticipated activity range of 1- 10 uCi per animal would be incinerated as well as flammable liquid scintillation cocktail. I have not been able to find, at this point in time, any information pertaining to the release of this building by the NRC.

We would like to request, if possible, for an expedited review of our original request so that the contract for the removal of the current incinerator in Building 4 can proceed.

We are also requesting guidance on how to procedure with the former incinerator building.

Thank you.

Les Morrison, Radiation Safety Officer Kansas City VAMC 816-861-4700 ext 57699



Additional e-mail correspondence between NHPP and permittee dated December 6, 2010, with clarifying details to identify structures involved in this request

----Original Message----From: Morrison, Leslie (KCVA) Sent: Monday, December 06, 2010 9:49 AM To: Huston, Thomas E. Cc: Hill, Kent D. (SES- KCVA); VACO NHPP Subject: RE: Requested diagram plus more information

Yes, the two areas are marked appropriately.

Les Morrison, RSO

-----Original Message-----From: Huston, Thomas E. Sent: Monday, December 06, 2010 9:43 AM To: Morrison, Leslie (KCVA) Cc: Hill, Kent D. (SES- KCVA); VACO NHPP Subject: RE: Requested diagram plus more information

Les,

Thank you for the information.

I am attaching an annotated version of your drawing to identify more clearly for NRC the stack and incinerator subject to the current request for release. Please confirm (by email back to me) that I have correctly marked those two areas.

We will submit your request to release, for unrestricted use, the incinerator and stack adjacent to Building 4, to NRC for review.

In our submittal to NRC, we will note our desire for an expedited review; however, we cannot control NRC's timetable for these types of review.

NRC might have follow-up questions or requests for information that could delay the review/approval process.

For the other former incinerator building (referenced in item 2 of your the email below), if you are not able to find any information that the building was released for unrestricted use by NRC (NHPP's permit file information only dates back to around 1984), then our advice is that you all perform a decommissioning quality survey of the area and submit that information to NHPP for review for unrestricted use.

Thomas E. Huston, PhD, CHP National Health Physics Program (115/HP) Veterans Health Administration Wk: 501-257-1578; Cell: 501-454-7264; Fax: 501-257-1570



Prepared December 6, 2010

NHPP reviewer notes on specified items in closeout survey report included as Enclosure 1.

NHPP reviewer notes on survey report dated November 20, 2003:

Item 1. Comments about radionuclides of concern:

Only H-3 and possibly C-14 and Cl-36 appear to be radionuclides of concern. Assuming a residual hold-up activity of 1000 uCi for other short-lived nuclides (< 120 day half life) at the time of last use (i.e., P-32 (14.3 d), P-33 (25.3 d), S-35 (87.5 d), and I-125 (59.4 d), no residual activity from these nuclides would be expected after 10 years decay (10 year decay of 1000 uCi for longest half-life of 87.5 days \rightarrow 2.8E-10 uCi).

Item 7. Comments about exposure rate and surface scan results:

Please disregard the comments in the survey report for this item. Exposure rate measurements and surface scans were less than or equal to about <u>two</u> times background, or 0.04 mR/hr, using a GM pancake detector. Typical background level for a Ludlum Model 44-9 GM pancake probe is around 0.02 mR/hr or about 60 cpm. It is very likely that the brick and construction materials used for the incinerator provided a natural background level higher than simple ambient instrument background measured outside the incinerator. This opinion is supported by the lack of measureable activity for the wipe samples. However, the surveyor did not identify a natural background level for the incinerator construction materials. Using the lower instrument background of 0.02 mR/hr does not alter the overall conclusions that the area appears free from residual contamination above unrestricted release levels.

For C-14, the two-times-background level corresponds to a contamination level of around 1200 dpm-net for C-14 assuming a 5% (4-pi) efficiency (i.e., 120 cpm gross \rightarrow (60 cpm net/0.05 cpm/dpm) \rightarrow 1200 dpm-net). For a typical window area of 15 cm², the "two times background" level corresponds to about 8000 dpm/100cm² (1200 dpm/15cm² * 100cm²). The two-times-background level is well below 1% of the NRC screening level for C-14 in NUREG-1757, Vol. 1, Rev. 2 (3,700,000 dpm/100cm²). For higher energy beta emitters (e.g., Cl-36), the detection efficiency is significantly higher than that for C-14, and two-times-background would correspond to a much lower level. A general conclusion is that the total residual activity is well below the screening levels for C-14 (3,700,000 dpm/100cm²) and Cl-36 (500,000 dpm/100cm²).

While the scanning technique would not have detected H-3, the wipe results did not indicate any removable H-3 activity. It is unlikely that elevated H-3 activity would be present at or near screening levels in Federal Register Vol. 63, No. 222 (i.e., 120,000,000 dpm/100 cm²) and none be removable. Therefore, a conclusion is that no removable or fixed H-3 is present at levels approaching the screening level release criteria.

Item 8. Comments about release criteria for removable activity:

Please disregard reference to NUREG-1556, Volume 11, Table S.5. We are currently requesting use of screening level criteria published in Federal Register Vol. 63, No. 222, and consistent with Federal Register Vol. 65, No. 114, as basis for release of the area for unrestricted use. The

screening level values are: H-3 (120,000,000 dpm/100cm2), C-14 (3,700,000 dpm/100cm2), and Cl-36 (500,000 dpm/100cm2). The screening level derivation assumes 10% of the activity is removable. No removable activity was detected in wipe surveys.

Additional Comment about Stack:

We acknowledge that the surveys pertain to the incinerator area only. The absence of any measureable activity in the incinerator supports a position that the stack is also free of residual activity.

NHPP reviewer: _______ Thomas E. Huston, Ph.D., CHP Date: 12/13/2010

