

## **CHAPTER 2**

### **SITE CHARACTERISTICS**

This chapter defines the site-related parameters for which the AP1000 plant is designed. The site parameters are in Table 2-1. These parameters envelope most potential sites in the United States. The sections of this chapter follow the standard format and discuss how the specific parameters are used in the AP1000 design and how the Combined License applicant is to demonstrate that the site meets the design parameters.

The site is acceptable if the site characteristics fall within the AP1000 plant site design parameters in Table 2-1. Should specific site parameters or characteristics be outside the envelope of assumptions established by Table 2-1, the Combined License applicant referencing the AP1000 will demonstrate that the design satisfies the requirements imposed by the specific site parameters and conforms to the design commitments and acceptance criteria described in the AP1000 Design Control Document.

#### **2.1 Geography and Demography**

The geography and demography are site specific and will be defined by the Combined License applicant.

##### **2.1.1 Combined License Information for Geography and Demography**

Combined License applicants referencing the AP1000 certified design will provide site-specific information related to site location and description, exclusion area authority and control, and population distribution.

Site Information – Site-specific information on the site and its location will include political subdivisions, natural and man-made features, population, highways, railways, waterways, and other significant features of the area.

Exclusion Area – Site-specific information on the exclusion area will include the size of the area and the exclusion area authority and control. Activity that may be permitted within the exclusion area will be included in the discussion.

Population Distribution – Site-specific information will be included on population distribution.

#### **2.2 Nearby Industrial, Transportation, and Military Facilities**

The plant has inherent capability to withstand certain types of external accidents due to the specified design conditions associated with earthquakes, wind loading, and radiation shielding. Acceptability for external accidents associated with a given site will be covered in the Combined License application.

Each Combined License applicant referencing the AP1000 will provide analyses of accidents external to the nuclear plant. The determination of the probability of occurrence of potential accidents which could have severe consequences will be based on analyses of available statistical

data on the occurrence of the accident together with analyses of the effects of the accident on the plant's safety-related structures and components. If an accident is identified for which the probability of severe consequences is unacceptable, specific changes to the AP1000 will be identified in the Combined License safety analysis report. The criteria for not requiring changes to the AP1000 design is that the total annual frequency of occurrence is less than  $10^{-6}$  per year for an external accident leading to severe consequences. The following accident categories will be considered in determining the frequency of occurrence, as appropriate:

**Explosions** – Accidents involving detonations of high explosives, munitions, chemicals, or liquid and gaseous fuels will be considered for facilities and activities in the vicinity of the plant where such materials are processed, stored, used, or transported in quantity.

The AP1000 includes onsite storage facilities for compressed and liquid hydrogen. Accidents involving accidental detonations of hydrogen from these storage facilities are evaluated as part of the AP1000 certified design. Combined License applicants referencing the AP1000 Design Certification are not required to provide analyses of accidents involving these storage facilities provided that the locations and size of the storage facilities are consistent with the safe distances defined by the AP1000 certified design. The bulk gas storage area for the plant gas system (PGS) is located sufficiently far from the nuclear island that an explosion would not result in damage to safety-related structures, systems, and components.

Evaluation of potential explosions due to exposure of chemical storage tanks to exterior fires has determined that all of these postulated accidents are safe distances away from safety-related items.

The AP1000 certified design does not include liquid oxygen or propane storage facilities.

**Flammable Vapor Clouds (Delayed Ignition)** – Accidental releases of flammable liquids or vapors that result in the formation of unconfined vapor clouds in the vicinity of the plant.

A flammable vapor cloud (delayed ignition) due to the accidental release of hydrogen from the PGS bulk gas storage area is evaluated as part of the AP1000 certified design. A detonation of such a hydrogen vapor cloud would not result in damage to safety-related structures, systems, and components. No other chemical has the possibility of developing unconfined flammable vapor clouds.

**Toxic Chemicals** – Accidents involving the release of toxic chemicals from nearby mobile and stationary sources.

**Fires** – Accidents leading to high heat fluxes or smoke, and to nonflammable gas or chemical-bearing clouds from the release of materials as the consequence of fires in the vicinity of the plant.

**Airplane Crashes** – Accidents involving aircraft crashes leading to missile impact or fire in the vicinity of the plant.

The AP1000 safe distance for material in onsite storage facilities for explosions, flammable vapor clouds, and fires is tabulated in Table 2.2-1.

**2.2.1 Combined License Information for Identification of Site-Specific Potential Hazards**

Combined License applicants referencing the AP1000 certified design will provide site-specific information related to the identification of potential hazards within the site vicinity, including an evaluation of potential accidents and verify that the frequency of site-specific potential hazards is consistent with the criteria outlined in Section 2.2. The site-specific information will provide a review of aircraft hazards, information on nearby transportation routes, and information on potential industrial and military hazards.

**2.3 Meteorology**

The AP1000 is designed for air temperatures, humidity, precipitation, snow, wind, and tornado conditions as specified in Table 2-1. The Combined License applicant must provide information to demonstrate that the site parameters are within the limits specified for the standard design.

The design wind is specified as a basic wind speed of 145 mph with an annual probability of occurrence of 0.02. Wind loads are calculated for exposure C, which is applicable to shorelines in hurricane prone areas. The site parameters for the design wind may be demonstrated to be acceptable for other exposures or topographic factors by comparison of the wind loads on the structures. For example, for a site at a location with exposure Category D, the wind speed should be equal to or less than 130 mph.

**2.3.1 Regional Climatology**

The regional climatology is site specific and will be defined by the Combined License applicant.

**2.3.2 Local Meteorology**

The local meteorology is site specific and will be defined by the Combined License applicant.

**2.3.3 Onsite Meteorological Measurement Programs**

The onsite meteorological measurement program is site specific and will be defined by the Combined License applicant. The number and location of meteorological instrument towers are determined by actual site parameters.

**2.3.4 Short-Term Diffusion Estimates**

In the absence of a specific site for use in determining values for short-term diffusion, a study was performed to determine the atmospheric dispersion factors ( $\chi/Q$  values) that would envelope most current plant sites and that could be used to calculate the radiological consequences of design basis accidents. The  $\chi/Q$  values thus derived for offsite are provided in Table 2-1.

This set of offsite  $\chi/Q$  values is representative of potential sites for construction of the AP1000. The values are appropriate for analyses to determine the radiological consequences of accidents. These values were selected to bound 70 to 80 percent of U.S. sites.

The  $\chi/Q$  values for the control room air intake or the door leading to the control room are dependent not only on the site meteorology but also on the plant design and layout. These  $\chi/Q$  values are addressed in Appendix 15A. Separate sets of  $\chi/Q$  values are identified for each combination of activity release location and receptor location.

**2.3.5 Long-Term Diffusion Estimates**

The long-term diffusion estimates are site specific and will be provided by the Combined License applicant. The site boundary annual average  $\chi/Q$  shown in Table 2-1 is used to calculate release concentrations at the site boundary for comparison with the activity release limits defined in 10 CFR 20. The value specified is expected to bound atmospheric conditions at most U.S. sites. If a selected site has a  $\chi/Q$  value that exceeds this reference site value, the release concentrations reported in Section 11.3 would be adjusted proportionate to the change in  $\chi/Q$ .

**2.3.6 Combined License Information****2.3.6.1 Regional Climatology**

Combined License applicants referencing the AP1000 certified design will address site-specific information related to regional climatology.

**2.3.6.2 Local Meteorology**

Combined License applicants referencing the AP1000 certified design will address site-specific local meteorology information.

**2.3.6.3 Onsite Meteorological Measurements Program**

Combined License applicants referencing the AP1000 certified design will address the site-specific onsite meteorological measurements program.

**2.3.6.4 Short-Term Diffusion Estimates**

Combined License applicants referencing the AP1000 certified design will address the site-specific  $\chi/Q$  values specified in subsection 2.3.4. For a site selected that exceeds the bounding  $\chi/Q$  values, the Combined License applicant will address how the radiological consequences associated with the controlling design basis accident continue to meet the dose reference values given in 10 CFR Part 50.34 and control room operator dose limits given in General Design Criteria 19 using site-specific  $\chi/Q$  values. The Combined License applicant should consider topographical characteristics in the vicinity of the site for restrictions of horizontal and/or vertical plume spread, channeling or other changes in airflow trajectories, and other unusual conditions affecting atmospheric transport and diffusion between the source and receptors. No further action is required for sites within the bounds of the site parameters for atmospheric dispersion.

With regard to assessment of the postulated impact of an accident on the environment, the COL applicant will provide  $\chi/Q$  values for each cumulative frequency distribution which exceeds the median value (50 percent of the time).

**2.3.6.5 Long-Term Diffusion Estimates**

Combined License applicants referencing the AP1000 certified design will address long-term diffusion estimates and  $\chi/Q$  values specified in subsection 2.3.5. The Combined License applicant should consider topographical characteristics in the vicinity of the site for restrictions of horizontal and/or vertical plume spread, channeling or other changes in airflow trajectories, and other unusual conditions affecting atmospheric transport and diffusion between the source and receptors. No further action is required for sites within the bounds of the site parameter for atmospheric dispersion.

With regard to environmental assessment, the COL applicant will also provide estimates of annual average  $\chi/Q$  values for 16 radial sectors to a distance of 50 miles from the plant.

**2.4 Hydrologic Engineering**

The AP1000 is designed for a normal groundwater elevation up to plant elevation 98' and for a flood level up to plant elevation 100'. For structural analysis purposes, grade elevation is also established as plant elevation 100'. Actual grade will be a few inches lower to prevent surface water from entering doorways.

For a portion of the annex building the site grade will be 107 feet to permit truck access at the elevation of the floor in the annex building and inside containment. Subsection 3.4.1 describes design provisions for groundwater and flooding.

The Combined License applicant will evaluate events leading to potential flooding to demonstrate that the site meets the site parameter for flood level. As necessary, the Combined License applicant may propose measures to protect the plant according to the Standard Review Plan, Section 2.4.10. Events to be considered are those identified in Standard Review Plan, Section 2.4.2.

Adverse effects of flooding due to high water or ice effects do not have to be considered for site-specific nonsafety-related structures and water sources outside the scope of the certified design. Flooding of water intake structures, cooling canals, or reservoirs or channel diversions would not prevent safe operation of the plant.

**2.4.1 Combined License Information****2.4.1.1 Hydrological Description**

Combined License applicants referencing the AP1000 certified design will describe major hydrologic features on or in the vicinity of the site including critical elevations of the nuclear island and access routes to the plant.

**2.4.1.2 Floods**

Combined License applicants referencing the AP1000 certified design will address the following site-specific information on historical flooding and potential flooding factors, including the effects of local intense precipitation.

- Probable Maximum Flood on Stream and Rivers – Site-specific information that will be used to determine the design basis flooding at the site. This information will include the probable maximum flood on streams and rivers.
- Dam Failures – Site-specific information on potential dam failures.
- Probable Maximum Surge and Seiche Flooding – Site-specific information on probable maximum surge and seiche flooding.
- Probable Maximum Tsunami Loading – Site-specific information on probable maximum tsunami loading.
- Flood Protection Requirements – Site-specific information on flood protection requirements or verification that flood protection is not required to meet the site parameter for flood level.

No further action is required for sites within the bounds of the site parameter for flood level.

**2.4.1.3 Cooling Water Supply**

Combined License applicants will address the water supply sources to provide makeup water to the service water system cooling tower.

**2.4.1.4 Groundwater**

Combined License applicants referencing the AP1000 certified design will address site-specific information on groundwater. No further action is required for sites within the bounds of the site parameter for ground water.

**2.4.1.5 Accidental Release of Liquid Effluents in Ground and Surface Water**

Combined License applicants referencing the AP1000 certified design will address site-specific information on the ability of the ground and surface water to disperse, dilute, or concentrate accidental releases of liquid effluents. Effects of these releases on existing and known future use of surface water resources will also be addressed.

**2.4.1.6 Emergency Operation Requirement**

Combined License applicants referencing the AP1000 certified design will address any flood protection emergency procedures required to meet the site parameter for flood level.

**2.5 Geology, Seismology, and Geotechnical Engineering**

Combined License applicants referencing the AP1000 certified design will address site specific information related to basic geological, seismological, and geotechnical engineering of the site and the region, as discussed in the following subsections.

**2.5.1 Basic Geological and Seismic Combined License Information**

Combined License applicants referencing the AP1000 certified design will address the following regional and site-specific geological, seismological, and geophysical information as well as conditions caused by human activities:

- Structural geology of the site
- Seismicity of the site
- Geological history
- Evidence of paleoseismicity
- Site stratigraphy and lithology
- Engineering significance of geological features
- Site groundwater conditions
- Dynamic behavior during prior earthquakes
- Zones of alteration, irregular weathering, or structural weakness
- Unrelieved residual stresses in bedrock
- Materials that could be unstable because of mineralogy or unstable physical properties
- Effect of human activities in the area

**2.5.2 Vibratory Ground Motion**

The AP1000 is designed for an earthquake defined by a peak ground acceleration (PGA) of 0.30g and the design response spectra specified in subsection 3.7.1.1, and Figures 3.7.1-1 and 3.7.1-2. The AP1000 design earthquake is referred to as the AP1000 Certified Seismic Design Response Spectra (CSDRS). The AP1000 CSDRS was developed using the Regulatory Guide 1.60 response spectra as the base and modified to include additional high frequency amplification at a control point at 25 Hz. The peak ground accelerations in the two horizontal and the vertical directions are equal. The CSDRS also represents the AP1000 Foundation Input Response Spectra (FIRS) at a hard rock site.

The AP1000 is evaluated for high frequency input using the response spectra specified in Appendix 3I, Figures 3I.1-1 and 3I.1-2. The seismic response spectra given in Figures 3I.1-1 and 3I.1-2 are envelope response spectra with high frequency content.

**2.5.2.1 Combined License Seismic and Tectonic Characteristics Information**

Combined License applicants referencing the AP1000 certified design will address the following site-specific information related to the vibratory ground motion aspects of the site and region:

- Seismicity
- Geologic and tectonic characteristics of site and region
- Correlation of earthquake activity with seismic sources
- Probabilistic seismic hazard analysis and controlling earthquakes
- Seismic wave transmission characteristics of the site
- SSE ground motion

The site-specific ground motion response spectra (GMRS) for comparison against the CSDRS are determined in the free-field on the ground surface. For sites with soil layers that will be completely excavated to expose competent material, the GMRS is specified on an outcrop or a hypothetical outcrop that will exist after excavation. Motions at this hypothetical outcrop are developed as a free-surface motion, not as an in-column motion with no soil or backfill soil layers above the outcrop. Competent material may be defined as in-situ material having a low strain shear wave velocity equal to or greater than 1000 fps. The Combined License applicant must demonstrate that the proposed site meets the following requirements:

1. The free field peak ground acceleration at the finished grade level is less than or equal to a 0.30g SSE.
2. The site-specific ground motion response spectra (GMRS) at the finished grade level in the free-field are less than or equal to the AP1000 certified seismic design spectra (CSDRS) given in Figures 3.7.1-1 and 3.7.1-2.
3. In lieu of (1) and (2) above, for a site where the nuclear island is founded on hard rock with shear wave velocity greater than 8,000 feet per second, the site-specific ground motion response spectra (GMRS) may be defined at the foundation level and shown to be less than or equal to the CSDRS given in Figures 3.7.1-1 and 3.7.1-2.
4. In lieu of (1) and (2) above, for a site where the nuclear island is founded on hard rock defined by a shear wave velocity at the bottom of the basemat equal to or higher than 7,500 fps, while maintaining a shear wave velocity equal to or above 8,000 fps at the lower depths, site-specific spectra may be developed at the top of the competent rock and shown at the foundation level to be less than or equal to those given in Figures 3I.1-1 and 3I.1-2 over the entire frequency range. If a COL applicant has site GMRS exhibiting hard rock high frequency (HRHF) characteristics, but is not enveloped by the AP1000 HRHF envelope response spectra or the AP1000 CSDRS, or has shear wave velocities that are not associated with hard rock, the COL applicant may perform site-specific studies to demonstrate that the high frequency is not damaging. This may be accomplished by the following:
  - a. Demonstrating that the site floor response spectra, developed at the locations of the spectra given in APP-GW-GLR-115, Sections 5.2 and 6.3 (Reference 3) using the seismic input defined by the site GMRS, are enveloped by the AP1000 HRHF envelope response spectra or CSDRS spectra.
  - b. If it is shown in step one that the spectra are not enveloped, evaluations similar to those described in Appendix 3I (documented in Reference 3) would be made to demonstrate that the high frequency input is non-damaging.
5. Foundation material layers are approximately horizontal (dip less than 20 degrees), and the minimum estimate of the low strain shear wave velocity of the soil below the foundation of the nuclear island is greater than or equal to 1000 feet per second.
6. For sites where the nuclear island is founded on soil, the minimum estimate of the strain-compatible soil shear modulus (G), the shear wave velocity (Vs), and hysteretic

damping is compared to the values used in the AP1000 generic analyses shown in Table 3.7.1-4 and Figure 3.7.1-17. Properties of soil layers within a depth of 120 feet below finished grade are compared to those in the generic soil site analyses (soft soil [SS], soft-to-medium soil [SM], and upper bound soft-to-medium soil [UBSM]). The shear wave velocity should generally increase with depth. The average low strain shear wave velocity in any layer should not be less than 80 percent of the average shear wave velocity in any layer at higher elevation. For the SS, SM, and UBSM soil profile, the shear wave velocity of each layer is to be within the lower bound and upper bound of the soil profiles. The lower bound and upper bound of shear wave velocity correspond to  $G_{max}/1.5$  and  $1.5*G_{max}$ . The lower bound shear wave velocity for the SS and SM soil profile is to be greater than or equal to 1000 fps.

7. In lieu of (1) to (6) above, a site-specific evaluation can be performed as described in subsection 2.5.2.3.

#### 2.5.2.2 Site-Specific Seismic Structures

The AP1000 includes all seismic Category I structures, systems and components in the scope of the design certification.

#### 2.5.2.3 Site-Specific Seismic Evaluation

The Combined License applicant may identify site-specific features and parameters that are not clearly within the guidance provided in subsection 2.5.2.1. These features and parameters may be demonstrated to be acceptable by performing site-specific seismic analyses. These analyses may be either 2D or 3D. Where 2D or 3D analyses apply are as follows:

- The 3D SASSI analyses will be used to quantify the effects of exceedances of site-specific GMRS compared to the CSDRS, or the HRHF GMRS at a hard rock site (DCD Figures 3I.1-1 and 3I.1-2), or in cases where the site-specific velocity soil profiles do not fall within the range evaluated for the standard design.
- For site-specific cases outside of the certified design, such as loads not evenly applied on the foundation that can be caused by soil conditions not evenly applied throughout the AP1000 foundation, the site-specific analysis should consider 3D effects.
- The 2D analyses are performed for parameter studies.

Results will be compared to the corresponding 2D or 3D generic analyses.

##### 2.5.2.3.1 2D Analyses

Where features of the site are not within the parameters specified for the AP1000, site-specific soil structure interaction analyses may be performed using the 2D SASSI models described in Appendix 3G for variations in site conditions that can be represented in these models. Results should be compared to the results of the 2D SASSI analyses described in Appendix 3G. Such analyses may be used to demonstrate that local features, such as soil degradation properties or backfill, are well within the bounds established by the design cases. If the results are not clearly

enveloped at the significant frequencies of response at the six key locations compared with the floor response spectra of the certified design at 5-percent damping, then a 3D SASSI analysis may be required. These evaluations and comparisons will be provided and reviewed as part of the Combined License application.

#### 2.5.2.3.2 3D Analyses

If required, a 3D evaluation will consist of a site-specific dynamic analysis and generation of in-structure response spectra at six key locations to be compared with the floor response spectra of the certified design at 5-percent damping. The certified seismic site design response spectra at the foundation level in the free-field given in Figures 3.7.1-1 and 3.7.1-2 were used to develop the floor response spectra. They were applied at foundation level for the hard rock site and at finished grade level for the soil sites. The site is acceptable if the floor response spectra from the site-specific evaluation do not exceed the AP1000 spectra for each of the locations identified below or the exceedances are justified:

|  |                             |
|--|-----------------------------|
| Containment internal structures at elevation of reactor vessel support | Figure 3G.4-5X to 3G.4-5Z   |
| Containment operating floor  | Figure 3G.4-6X to 3G.4-6Z   |
| Auxiliary building NE corner at elevation 116'-6"                      | Figure 3G.4-7X to 3G.4-7Z   |
| Shield building at fuel building roof                                  | Figure 3G.4-8X to 3G.4-8Z   |
| Shield building roof   | Figure 3G.4-9X to 3G.4-9Z   |
| Steel containment vessel at polar crane support                        | Figure 3G.4-10X to 3G.4-10Z |

Site-specific soil structure interaction analyses are performed using the 3D SASSI models described in Appendix 3G. The site-specific soil structure interaction analyses use the site-specific soil conditions (including variation in soil properties in accordance with Standard Review Plan 3.7.2 and site-specific soil degradation models). The three components of the site-specific ground motion time history must satisfy the regulatory requirements for statistical independence and enveloping of the site design spectra at 5% damping. Floor response spectra determined from the site-specific analyses should be compared against the design basis of the AP1000 described above. These evaluations and comparisons will be provided and reviewed as part of the Combined License application.

If the site-specific spectra at foundation level at a rock site exceed the response spectra in Figures 3I.1-1 and 3I.1-2 at any frequency, a site-specific evaluation can be performed similar to that described in Appendix 3I.

#### 2.5.3 Surface Faulting Combined License Information

Combined License applicants referencing the AP1000 certified design will address the following surface and subsurface geological, seismological, and geophysical information related to the potential for surface or near-surface faulting affecting the site:

- Geological, seismological, and geophysical investigations

- Geological evidence, or absence of evidence, for surface deformation
- Correlation of earthquakes with capable tectonic sources
- Ages of most recent deformation
- Relationship of tectonic structures in the site area to regional tectonic structures
- Characterization of capable tectonic sources
- Designation of zones of quaternary deformation in the site region
- Potential for surface tectonic deformation at the site

The AP1000 design has not been evaluated for a site where there is a fault displacement potential. A COL applicant will satisfy the requirement for no surface or near surface tectonic structure capable of displacements beneath the nuclear island and adjacent seismic Category II structures by completing geological, seismological, and geophysical investigations that are consistent with the guidance of Regulatory Guide 1.206.

**2.5.4****Stability and Uniformity of Subsurface Materials and Foundations**

Combined License applicants referencing the AP1000 certified design will address the following site-specific information related to the stability and uniformity of subsurface materials and foundations.

- Excavation
- Bearing capacity
- Settlement
- Liquefaction

**2.5.4.1****Excavation**

Excavation for the nuclear island structures below grade may use either a sloping excavation or a vertical face as described in subsequent paragraphs. If sloping excavations are to be used on a soil site, the Combined License applicants must evaluate the 3D effects on the site response and perform site-specific SSI analyses using either or a combination of 2D or 3D SASSI models that reflect the sloping excavations. If backfill is to be placed adjacent to the exterior walls of the nuclear island, the Combined License applicant will provide information on the properties of backfill and its compaction requirements as described in subsection 2.5.4.6.3 and will evaluate its properties against those used in the seismic analyses described in subsection 3.7.2.

For the vertical face alternative, excavation in soil for the nuclear island structures below grade will establish a vertical face with lateral support of the adjoining undisturbed soil or rock. This vertical face will be covered by a waterproof membrane as described in subsection 3.4.1.1.1 and is used as the outside form for the exterior walls below grade of the nuclear island. Alternative methods include a soil nailing and mechanically stabilized earth (MSE) walls.

**2.5.4.1.1****Vertical Face Using Soil Nails**

Soil nailing is a method of retaining earth in-situ. As the nuclear island excavation progresses vertically downward, holes are drilled horizontally into the adjoining undisturbed soil, a metal rod

is inserted into the hole, and grout is pumped into each hole to fill the hole and to anchor the “nail” rod.

As each increment of the nuclear island excavation is completed, nominal eight to ten inch diameter holes are drilled horizontally through the vertical face of the excavation into adjacent undisturbed soil. These “nail” holes, spaced horizontally and vertically on five to six feet centers, are drilled slightly downward to the horizontal. A “nail”, normally a metal bar/rod, is center located for the full length of the hole. The nominal length of soil nails is 60 percent to 70 percent of the wall height, depending upon soil conditions. The hole is filled with grout to anchor the rod to the soil. A metal face plate is installed on the exposed end of the rod at the excavated wall vertical surface. Welded wire mesh is hung on the wall surface for wall reinforcement and secured to the soil nail face plates for anchorage. A 4,000 psi to 5,000 psi non-expansive pea gravel shotcrete mix is blown onto the wire mesh to form a nominal four to six inch thick soil retaining wall. Installation of the soil retaining wall closely follows the progress of the excavation and is from the top down, with each wire mesh-reinforced, shotcreted wall section being supported by the soil “nails” and the preceding elevations of soil nailed wall placements.

Soil nailing as a method of soil retention has been successfully used on excavations up to 55 feet deep on projects in the U.S. Soils have been retained for up to 90 feet in Europe. The state of California CALTRANS uses soil nailing extensively for excavations and soil retention installations. Soil nailing design and installation has a successful history of application which is evidenced by its excellent safety record.

The soil nailing method produces a vertical surface down to the bottom of the excavation and is used as the outside forms for the exterior walls below grade of the nuclear island. Concrete is placed directly against the vertical concrete surface of the excavation.

For methods of soil retention other than soil nailing, such as for excavation in rock, four to six inches of shotcrete are blown on to the vertical surface. The concrete for the exterior walls is placed against the shotcrete.

#### **2.5.4.1.2 Vertical Face Using Mechanically Stabilized Earth Walls**

Mechanically stabilized earth walls (MSE) are flexible retaining wall systems that use strip, grid, or sheet type of tensile reinforcements in the soil mass, and a discrete modular pre-cast concrete, which is vertical. MSE walls function like, and are generally more economical than, conventional retaining walls. The tensile strength of the reinforcements and the slip at the interface of the reinforcement and the soil provide great internal stability to MSE walls. These walls may be used where the side soils have to be removed or the grade elevation needs to be raised. The walls and backfill are placed prior to construction of the nuclear island.

#### **2.5.4.1.3 Mudmat**

The mudmat provides a working surface prior to initiating the placement of reinforcement for the foundation mat structural concrete. The lower and upper mudmats are as follows:

- Lower mudmat – (minimum 6 inches thick) of un-reinforced concrete, with a minimum compressive strength of 2,500 psi, as discussed in subsection 2.5.4.6.3. The lower mudmat will be used as the final dental concrete layer on the underlying foundation media.
- Upper mudmat – (minimum 6 inches thick) of un-reinforced concrete with a minimum compressive strength of 2,500 psi. This upper mudmat will support the chairs that, in turn, support the reinforcing steel.

The waterproofing system is described in subsection 2.5.4.6.12.

#### 2.5.4.2 Bearing Capacity

The maximum bearing reaction determined from the 3D SASSI analyses described in Appendix 3G is less than 35,000 lb/ft<sup>2</sup> under all combined loads, including the safe shutdown earthquake. The maximum dynamic bearing demand of 35 ksf occurs under the west edge of the shield building and is primarily due to the response to the east-west component of the earthquake. The east edge of the nuclear island lifts off the soil. The Combined License applicant will verify that the site-specific allowable soil bearing capacities for static and dynamic loads at the site will exceed the static and dynamic bearing demand given in Table 2-1.

The evaluation of the allowable capacity of the soil is based on the properties of the underlying materials (see subsection 2.5.4.5.2), including appropriate laboratory test data to evaluate strength, and considering local site effects, such as fracture spacing, variability in properties, and evidence of shear zones. The allowable bearing capacity should provide a factor of safety appropriate for the design load combination, including safe shutdown earthquake loads.

If the shear wave velocity or the allowable bearing capacity is outside the range evaluated for AP1000 design certification, a site-specific evaluation can be performed using the AP1000 basemat model and methodology described in subsection 3.8.5. The safe shutdown earthquake loads are those from the AP1000 analyses described therein. Alternatively, bearing pressures may be determined from a site-specific analysis using site-specific inputs as described in subsection 2.5.2.3. For the site to be acceptable, the bearing pressures from the site-specific analyses, including static and dynamic loads, need to be less than the capacity of each portion of the basemat.

#### 2.5.4.3 Settlement

The Combined License applicant will address short-term (elastic) and long-term (heave and consolidation) settlement for soil sites for the history of loads imposed on the nuclear island foundation and adjacent buildings consistent with the construction sequence. The resulting time-history of settlements includes construction activities such as dewatering, excavation, bearing surface preparation, placement of the basemat, and construction of the superstructure.

The AP1000 does not rely on structures, systems, or components located outside the nuclear island to provide safety-related functions. Differential settlement between the nuclear island foundation and the foundations of adjacent buildings does not have an adverse effect on the safety-related functions of structures, systems, and components. Differential settlement under the

nuclear island foundation could cause the basemat and buildings to tilt. Much of this settlement occurs during civil construction prior to final installation of the equipment. Differential settlement of a few inches across the width of the nuclear island would not have an adverse effect on the safety-related functions of structures, systems, and components. Table 2.5-1 provides guidance to the Combined License applicant on predictions of absolute and differential settlement that are acceptable without further evaluation. The predicted settlements will cover the periods before construction begins through the construction phase, and for the subsequent plant operating period or otherwise justified. The predicted settlements will be based on conservative assumptions of soil properties. If the predicted settlements exceed the limits of Table 2.5-1, a detailed evaluation and construction plan will be described by the Combined License applicant. During construction and plant operation at a soil site, settlements would be measured and compared to the predicted settlement values and any exceedances would require additional investigation.

Alternatives for the additional evaluations are provided as follows:

1. Evaluate the impact of the elevated estimated settlement values on the critical components of the AP1000 including, but not limited to, piping spanning between the nuclear island and the adjacent structures, the equipment support pads, the construction gap between the nuclear island and adjacent buildings, and the stresses on the basemat (along with influences to the underlying soil).
2. Submit a construction sequence to control the predicted settlement behavior. A revised sequence should follow the specific schedule to distribute construction loads as necessary in order to obtain acceptable values. Depending on soil conditions, a significant amount of the settlement could occur during construction and can be controlled through the construction sequence.
3. Provide a uniform excavation and engineered backfill to manage static building rotation and differential settlement between the nuclear island and adjacent structures.
4. Implement an active settlement monitoring system throughout the entire construction sequence as well as a long-term (plant operation) plan. By monitoring the settlement throughout construction, the Combined License applicant will be able to modify the construction sequences of adjacent buildings to conform to the site's settlement characteristics and minimize differential settlement. For soil sites, the potential heave or rebound of the excavation bottom, the effect of dewatering, and the effect of foundation loading during construction should be monitored by the Combined License applicant. The monitoring system shall consist of three primary elements as follows:
  - Piezometers to measure pore pressures in a soil layer prone to consolidation type settlement. Vibrating wire piezometers are preferred for this purpose because they are adequately sensitive and responsive and easily record positive and negative changes on a real time basis.
  - Settlement monuments placed directly on concrete, preferably on the mudmat for early construction monitoring and on the corners of the structures at grade once the mudmat

monuments have been covered by backfill to be used for long-term monitoring. Monuments at grade are to be accessible with conventional surveying equipment.

- Settlement telltales if monuments are not practical or if fills are used over consolidation type soils and it is necessary to monitor settlement of the consolidation type in-situ soils independent of the consolidation of the engineered fill soil. Most soil sites will not need this particular form of monitoring.

Develop graphs and plots of the field measurements to:

- Show movement (settlement or heave) versus time.
- Estimate construction loads versus time.
- Measure ground water levels from the dewatering activities versus time.

This data should be maintained during construction and post-construction as needed depending on the field measurement results.

#### **2.5.4.4 Liquefaction**

The Combined License applicant will demonstrate for soil sites that the potential for liquefaction is negligible for both the soil underneath the nuclear island foundation and the soil of the side embedment engaged in passive resistance adjacent to the nuclear island.

The AP1000 design has not been evaluated for a site where there is a liquefaction potential of the soil below the nuclear island. A COL applicant will satisfy the requirement that liquefaction beneath the nuclear island and adjacent seismic Category II structures needs not be considered by providing information concerning the properties and stability of supporting soils and rock consistent with the guidance of Regulatory Guide 1.206.

#### **2.5.4.5 Subsurface Uniformity**

Soil structure interaction and foundation design are a function of the uniformity of the soil or rock below foundation. Although the design and analysis of the AP1000 is based on soil or rock conditions with uniform properties within horizontal layers, it includes provisions and design margins to accommodate many non-uniform sites. This subsection identifies the requirements for site investigation that may be used to demonstrate that:

- A site is “uniform” based on the criteria outlined in subsection 2.5.4.5.3. If the site can be demonstrated to be “uniform,” no further site specific analysis is required to qualify the site for the AP1000.
- A “non-uniform” site is acceptable to locate the AP1000 based on the criteria for acceptability outlined in subsection 2.5.4.5.3. Non-uniform sites may be shown to be acceptable as described in subsection 2.5.4.5.3.1 using site-specific evaluation as part of the Combined License application.

Considerations with respect to the materials underlying the nuclear island are the type of site, such as rock or soil, and whether the site can be considered uniform. If the site is non-uniform, the non-uniform soil characteristics, such as the location and profiles of soft and hard spots, should be considered. These considerations can be assessed with the information developed in response to Regulatory Guides 1.132 and 1.138. The geological investigations of subsections 2.5.1 and 2.5.4.6.1 provide information on the uniformity of the site, whether it may be geologically impacted, and whether the bedrock may be sloping or undulatory.

**2.5.4.5.1 Not Used****2.5.4.5.2 Not Used****2.5.4.5.3 Site Foundation Material Evaluation Criteria**

The AP1000 is designed for application at a site where the foundation conditions do not have extreme variation within the nuclear island footprint. This subsection provides criteria for evaluation of soil variability. The subsurface may consist of layers and these layers may dip with respect to the horizontal. If the dip is less than 20 degrees, the generic analysis using horizontal layers is applicable as described in NUREG/CR-0693 (Reference 2). The physical properties of the foundation medium may or may not vary systematically across a horizontal plane. The recommended methodology for checking uniformity is to calculate from the boring logs a series of “best-estimate” planes beneath the nuclear island footprint that define the top (and bottom) of each layer. The planes could represent stratigraphic boundaries, lithologic changes, and unconformities, but most important, they should represent boundaries between layers having different shear wave velocities. Shear wave velocity is the primary property used for defining uniformity of a site.

The distribution of bearing reactions under the basemat is a function of the subgrade modulus, which in turn is a function of the soil properties. The Combined License applicant shall demonstrate that the variation of subgrade modulus across the footprint is within the range considered for design of the nuclear island basemat. The farther that the non-uniform layer is located below the foundation, the less influence it has on the bearing pressures at the basemat. Lateral variability of the shear wave velocity at depths greater than 120 feet below grade (80 feet below the foundation) do not significantly affect the subgrade modulus.

Subsurface conditions should be evaluated by the Combined License applicant based on the geologic investigation in accordance with Regulatory Guide 1.132. Subsurface conditions should be evaluated within the nuclear island footprint and 40 feet beyond the boundaries of the nuclear island footprint at depths less than 120 feet below grade. Subsurface conditions may be considered uniform if the geologic and stratigraphic features can be correlated from one boring or sounding location to the next with relatively smooth variations in thicknesses or properties of the geologic units. An occasional anomaly or a limited number of unexpected lateral variations may occur. If a site can be classified as uniform, it qualifies for the AP1000 based on analyses and evaluations performed to support design certification without additional site-specific analyses.

As an example of sites that are considered uniform, the variation of soil properties in the material below the foundation to a depth of 120 feet below finished grade within the nuclear island footprint and 40 feet beyond the boundaries of the nuclear island footprint meets the criteria outlined below:

- The depth to a given layer indicated on each boring log may not fall precisely on the postulated “best-estimate” plane. The deviation of the observed layers from the “best-estimate” planes should not exceed 5 percent of the observed depths from the ground surface to the plane. If the deviation is greater than 5 percent, additional planes may be appropriate or additional borings may be required. This thereby diminishes the spacing.
- For a layer with a low strain shear wave velocity greater than or equal to 2500 feet per second, the layer should have approximately uniform thickness and should have a dip no greater than 20 degrees, and the shear wave velocity at any location within any layer should not vary from the average velocity within the layer by more than 20 percent.

#### 2.5.4.5.3.1 Site-Specific Subsurface Uniformity Design Basis

Many sites that do not meet the above criteria for a uniform site are acceptable for the AP1000. The key attribute for acceptability of the site for an AP1000 is the bearing pressure on the underside of the basemat. A site having local soft or hard spots within a layer or layers does not meet the criteria for a uniform site. Non-uniform soil conditions may also require evaluation of the AP1000 seismic response as described in subsection 2.5.2.3.

As described in subsection 3.8.5, the nuclear island foundation is designed specifically for bearing pressures of 120 percent of those of the uniform soil properties case. Evaluation criteria are defined to evaluate sites that do not satisfy the site parameters directly. The design basis provided below is included to provide a clear specification of the design commitment and evaluation criteria required to demonstrate that a site-specific application satisfies AP1000 requirements. Application of the AP1000 to sites using this site-specific evaluation is not approved as part of the AP1000 design certification and the evaluation should be provided and reviewed as part of the Combined License application.

#### Rigid Basemat Evaluation

A site with nonuniform soil properties may be demonstrated to be acceptable by evaluation of the bearing pressures on the underside of a rigid rectangular basemat equivalent to the nuclear island. The soils identified in the site investigation may be included in a finite element model of the soil to analyze the effect of the lateral variability. When the variability identified at the site can be modeled in two dimensions (there is not significant variability in one horizontal direction), 2D analyses may be used. Where the variability occurs in both horizontal directions, a 3D analysis should be performed. Bearing pressures are calculated in a linear analysis for unit vertical load and overturning moments. For the site to be acceptable, the bearing pressures from this analysis need to be less than or equal to 120 percent of the bearing pressures calculated in similar analyses for a site having uniform soil properties.

Alternatively, the safe shutdown earthquake loads may be determined from a site-specific seismic analysis of the nuclear island using site-specific inputs as described in subsections 2.5.2.1 or 2.5.2.3.

For the site to be acceptable, the bearing pressures from the site-specific analyses (with site-specific response and site-specific soil properties) need to be less than or equal to 120 percent of the bearing pressures calculated in similar rigid basemat analyses using the AP1000 design ground motion at a site having uniform soil properties.

### **Flexible Basemat Evaluation**

For sites having bedrock close to the foundation level, the assumption of a rigid basemat may be overly conservative because local deformation of the basemat will reduce the effect of local soil variability. For such sites, a site-specific analysis may be performed using the AP1000 basemat model and methodology described in subsection 3.8.5. The soils may be represented by soil springs or by a finite element model of the soil depending on the type of variability identified at the site. The safe shutdown earthquake loads are those from the AP1000 design soil case representative of the site-specific soil. Alternatively, bearing pressures may be determined from a site-specific soil structure interaction analysis using site-specific inputs as described in subsection 2.5.2.3. For the site to be acceptable, the bearing pressures from the site-specific analyses, including static and dynamic loads, need to be less than the design bearing strength of each portion of the basemat.

#### **2.5.4.6 Combined License Information**

Combined License applicants referencing the AP1000 design will address the following site specific information related to the geotechnical engineering aspects of the site. No further action is required for sites within the bounds of the site parameters.

**2.5.4.6.1** Site and Structures – Site-specific information regarding the underlying site conditions and geologic features will be addressed. This information will include site topographical features, as well as the locations of seismic Category I structures.

**2.5.4.6.2** The Combined License applicant will establish the properties of the foundation soils to be within the range considered for design of the nuclear island basemat.

Properties of Underlying Materials – A determination of the static and dynamic engineering properties of foundation soils and rocks in the site area will be addressed. This information will include a discussion of the type, quantity, extent, and purpose of field explorations, as well as logs of borings and test pits. Results of field plate load tests, field permeability tests, and other special field tests (e.g., bore-hole extensometer or pressuremeter tests) will also be provided. Results of geophysical surveys will be presented in tables and profiles. Data will be provided pertaining to site-specific soil layers (including their thicknesses, densities, moduli, and Poisson's ratios) between the basemat and the underlying rock stratum. Plot plans and profiles of site explorations will be provided. The Combined License applicant is to demonstrate that the minimum soil angle of internal friction is greater than or equal to 35 degrees below the footprint of the nuclear island at its excavation depth as specified in Table 2-1. If the minimum soil angle of internal friction is below 35 degrees, then the Combined License applicant will evaluate stability as described in subsections 3.8.5.5.3 and 3.8.5.5.5 using the site-specific soil properties.

Laboratory Investigations of Underlying Materials – Information about the number and type of laboratory tests and the location of samples used to investigate underlying materials will be provided. Discussion of the results of laboratory tests on disturbed and undisturbed soil and rock samples obtained from field investigations will be provided.

- 2.5.4.6.3** Excavation and Backfill – Information concerning the extent (horizontal and vertical) of seismic Category I excavations, fills, and slopes, if any will be addressed. The sources, quantities, and static and dynamic engineering properties of borrow materials will be described in the site-specific application. The compaction requirements, results of field compaction tests, and fill material properties (such as moisture content, density, permeability, compressibility, and gradation) will also be provided. Information will be provided concerning the specific soil retention system, for example, the soil nailing system or mechanically stabilized earth walls, including the length and size of the soil nails or tension reinforcement, which is based on actual soil conditions and applied construction surcharge loads. If backfill is to be placed adjacent to the exterior walls of the nuclear island, information will be provided concerning compaction of the backfill and any additional loads on the exterior walls of the nuclear island.
- 2.5.4.6.4** Ground Water Conditions – Groundwater conditions will be described relative to the foundation stability of the safety-related structures at the site. The soil properties of the various layers under possible groundwater conditions during the life of the plant will be compared to the range of values assumed in the standard design in Table 2-1.
- 2.5.4.6.5** Liquefaction Potential – Soils under and around seismic Category I structures will be evaluated for liquefaction potential for the site specific SSE ground motion. This should include justification of the selection of the soil properties, as well as the magnitude, duration, and number of excitation cycles of the earthquake used in the liquefaction potential evaluation (e.g., laboratory tests, field tests, and published data). Liquefaction potential will also be evaluated to address seismic margin.
- 2.5.4.6.6** Bearing Capacity – The Combined License applicant referencing the AP1000 certified design will verify that the site-specific allowable soil bearing capacities for static and dynamic loads are equal to or greater than the values documented in Table 2-1, or will provide a site-specific evaluation as described in subsection 2.5.4.2. The acceptance criteria for this evaluation are those of Standard Review Plan 2.5.4 as follows:
- The static and dynamic loads, and the stresses and strains induced in the soil surrounding and underlying the nuclear island, are conservatively and realistically evaluated.
  - The consequences of the induced soil stresses and strains, as they influence the soil surrounding and underlying the nuclear island, have been conservatively assessed.
- 2.5.4.6.7** Earth Pressures – The Combined License applicant will describe the design for static and dynamic lateral earth pressures and hydrostatic groundwater pressures acting on plant safety-related facilities using soil parameters as evaluated in previous subsections.
- 2.5.4.6.8** Soil Properties for Seismic Analysis of Buried Pipes – The AP1000 does not utilize safety-related buried piping. No additional information is required on soil properties.

- 2.5.4.6.9** Static and Dynamic Stability of Facilities – Soil characteristics affecting the stability of the nuclear island will be addressed including foundation rebound, settlement, and differential settlement.
- 2.5.4.6.10** Subsurface Instrumentation – Data will be provided on instrumentation, if any, proposed for monitoring the performance of the foundations of the nuclear island. This will specify the type, location, and purpose of each instrument, as well as significant details of installation methods. The location and installation procedures for permanent benchmarks and markers for monitoring the settlement will be addressed.
- 2.5.4.6.11** Settlement of Nuclear Island – Data will be provided on short-term (elastic) and long-term (heave and consolidation) settlement for soil sites for the history of loads imposed on the nuclear island foundation and adjacent buildings consistent with the construction sequence. The resulting time-history of settlements includes construction activities such as dewatering, excavation, bearing surface preparation, placement of the basemat, and construction of the superstructure. Subsection 3.8.5.4.2 includes analyses of settlement during construction completed to support the design certification and the required limitations on construction sequence for some sites. The limitations on construction sequence impose limits on the placement of concrete for the shield building and the auxiliary building prior to completion of both buildings at elevation 82'-6". Special construction requirements will be described, if required, to accommodate site-specific settlement predicted to exceed the values shown in Table 2.5-1.
- 2.5.4.6.12** Waterproofing System – The Combined License applicant will provide a waterproofing system used for the foundation mat (mudmat) and below grade exterior walls exposed to flood and groundwater under seismic Category I structures. The waterproofing membrane should be placed immediately beneath the upper mudmat, and on top of the lower mudmat. The performance requirements to be met by the COL applicant for the waterproofing system are described in subsection 3.4.1.1.1.1.

**2.5****Combined License Information for Stability of Slopes**

Combined License applicants referencing the AP1000 design will address site-specific information about the static and dynamic stability of soil and rock slopes, the failure of which could adversely affect the nuclear island.

**2.5.6****Combined License Information for Embankments and Dams**

Combined License applicants referencing the AP1000 design will address site-specific information about the static and dynamic stability of embankments and dams, the failure of which could adversely affect the nuclear island.

**2.6****References**

1. American Concrete Institute (ACI), "Building Code Requirements for Structural Concrete," ACI 318-02.
2. NUREG/CR-0693, "Seismic Input and Soil Structure Interaction," February 1979.

- 3. APP-GW-GLR-115, "Effect of High Frequency Seismic Content on SSCs," Westinghouse Electric Company LLC.

| Table 2-1 (Sheet 1 of 4)                     |   |
|--|---|
| <b>SITE PARAMETERS</b>                       |   |
| <b>Air Temperature</b>                       |   |
| Maximum Safety <sup>(a)</sup>                | 115°F dry bulb/86.1°F coincident wet bulb <sup>(g)</sup><br>86.1°F wet bulb (noncoincident)   |
| Minimum Safety <sup>(a)</sup>                | -40°F   |
| Maximum Normal <sup>(b)</sup>                | 101°F dry bulb/80.1°F coincident wet bulb<br>80.1°F wet bulb (noncoincident) <sup>(d)</sup>   |
| Minimum Normal <sup>(b)</sup>                | -10°F   |
| <b>Wind Speed</b>                            |   |
| Operating Basis                              | 145 mph (3 second gust); importance factor 1.15 (safety),<br>1.0 (nonsafety); exposure C; topographic factor 1.0  |
| Tornado                                      | 300 mph   |
| <b>Seismic</b>                               |   |
| CSDRS  | 0.30g peak ground acceleration <sup>(c)(f)</sup>  |
| Fault Displacement Potential                 | No potential fault displacement considered beneath the seismic Category I and seismic Category II structures and immediate surrounding area. The immediate surrounding area includes the effective soil supporting media associated with the seismic Category I and seismic Category II structures.   |
| <b>Soil</b>                                  |   |
| Average Allowable Static Bearing Capacity    | The allowable bearing capacity, including a factor of safety appropriate for the design load combination, shall be greater than or equal to the average bearing demand of 8,900 lb/ft <sup>2</sup> over the footprint of the nuclear island at its excavation depth.  |
| Dynamic Bearing Capacity for Normal Plus SSE | The allowable bearing capacity, including a factor of safety appropriate for the design load combination, shall be greater than or equal to the maximum bearing demand of 35,000 lb/ft <sup>2</sup> at the edge of the nuclear island at its excavation depth, or Site-specific analyses demonstrate factor of safety appropriate for normal plus safe shutdown earthquake loads. |
| Shear Wave Velocity                          | Greater than or equal to 1,000 ft/sec based on minimum low-strain soil properties over the footprint of the nuclear island at its excavation depth  |

| Table 2-1 (Sheet 2 of 4)                |   |
|---|---|
| <b>SITE PARAMETERS</b>                  |   |
| Lateral Variability                     | <p>Soils supporting the nuclear island should not have extreme variations in subgrade stiffness. This may be demonstrated by one of the following:</p> <ol style="list-style-type: none"> <li>1. Soils supporting the nuclear island are uniform in accordance with Regulatory Guide 1.132 if the geologic and stratigraphic features at depths less than 120 feet below grade can be correlated from one boring or sounding location to the next with relatively smooth variations in thicknesses or properties of the geologic units, or</li> <li>2. Site-specific assessment of subsurface conditions demonstrates that the bearing pressures below the nuclear island do not exceed 120% of those from the generic analyses of the nuclear island at a uniform site, or</li> <li>3. Site-specific analysis of the nuclear island basemat demonstrates that the site-specific demand is within the capacity of the basemat.</li> </ol> <p>As an example of sites that are considered uniform, the variation of shear wave velocity in the material below the foundation to a depth of 120 feet below finished grade within the nuclear island footprint and 40 feet beyond the boundaries of the nuclear island footprint meets the criteria in the case outlined below:</p> <p>Case 1: For a layer with a low strain shear wave velocity greater than or equal to 2500 feet per second, the layer should have approximately uniform thickness, should have a dip not greater than 20 degrees, and should have less than 20 percent variation in the shear wave velocity from the average velocity in any layer.</p> |
| Liquefaction Potential                  | No liquefaction considered beneath the seismic Category I and seismic Category II structures and immediate surrounding area. The immediate surrounding area includes the effective soil supporting media associated with the seismic Category I and seismic Category II structures.   |
| Minimum Soil Angle of Internal Friction | Greater than or equal to 35 degrees below footprint of nuclear island at its excavation depth   |
| <b>Missiles</b>                         |   |
| Tornado                                 | 4000 - lb automobile at 105 mph horizontal, 74 mph vertical<br>275 - lb, 8 in. shell at 105 mph horizontal, 74 mph vertical<br>1 inch diameter steel ball at 105 mph horizontal and vertical  |

| Table 2-1 (Sheet 3 of 4)   |   |
|--|---|
| SITE PARAMETERS  |   |
| <b>Flood Level</b>   | Less than plant elevation 100'  |
| <b>Ground Water Level</b>  | Less than plant elevation 98'   |
| <b>Plant Grade Elevation</b>                                     | Less than plant elevation 100' except for portion at a higher elevation adjacent to the annex building                      |
| <b>Precipitation</b>   |   |
| Rain   | 20.7 in./hr [1-hr 1-mi <sup>2</sup> PMP]  |
| Snow/Ice   | 75 pounds per square foot on ground with exposure factor of 1.0 and importance factors of 1.2 (safety) and 1.0 (non-safety) |
| <b>Atmospheric Dispersion Values - <math>\chi/Q^{(e)}</math></b> |   |
| Site boundary (0-2 hr)   | $\leq 5.1 \times 10^{-4}$ sec/m <sup>3</sup>  |
| Site boundary (annual average)                                   | $\leq 2.0 \times 10^{-5}$ sec/m <sup>3</sup>  |
| Low population zone boundary                                     |   |
| 0 - 8 hr   | $\leq 2.2 \times 10^{-4}$ sec/m <sup>3</sup>  |
| 8 - 24 hr  | $\leq 1.6 \times 10^{-4}$ sec/m <sup>3</sup>  |
| 24 - 96 hr   | $\leq 1.0 \times 10^{-4}$ sec/m <sup>3</sup>  |
| 96 - 720 hr  | $\leq 8.0 \times 10^{-5}$ sec/m <sup>3</sup>  |
| <b>Population Distribution</b>                                   |   |
| Exclusion area (site)  | 0.5 mi  |

**Notes:**

- (a) Maximum and minimum safety values are based on historical data and exclude peaks of less than 2 hours duration.
- (b) The maximum normal value is the 1-percent seasonal exceedance temperature. The minimum normal value is the 99-percent seasonal exceedance temperature. The minimum temperature is for the months of December, January, and February in the northern hemisphere. The maximum temperature is for the months of June through September in the northern hemisphere. The 1-percent seasonal exceedance is approximately equivalent to the annual 0.4-percent exceedance. The 99-percent seasonal exceedance is approximately equivalent to the annual 99.6-percent exceedance.
- (c) With ground response spectra as given in Figures 3.7.1-1 and 3.7.1-2. Seismic input is defined at finished grade except for sites where the nuclear island is founded on hard rock.
- (d) The noncoincident wet bulb temperature is applicable to the cooling tower only.
- (e) For AP1000, the terms "site boundary" and "exclusion area boundary" are used interchangeably. Thus, the  $\chi/Q$  specified for the site boundary applies whenever a discussion refers to the exclusion area boundary.
- (f) Sites that fall within the hard rock high frequency envelope response spectra given in Figure 3I.1-1 and Figure 3I.1-2, and satisfy the limitation on shear wave velocity in subsection 2.5.2.1, are acceptable.
- (g) The containment pressure response analysis is based on a conservative set of dry-bulb and wet-bulb temperatures. These results envelop any conditions where the dry-bulb temperature is 115°F or less and wet-bulb temperature of less than or equal to 86.1°F.

| Table 2-1 (Sheet 4 of 4)   |  |   |                           |                                   |  |        |
|--|--|---|---------------------------|-----------------------------------|--|--------|
| SITE PARAMETERS  |  |   |                           |                                   |  |        |
| Control Room Atmospheric Dispersion Factors ( $\chi/Q$ ) for Accident Dose Analysis                  |  |   |                           |                                   |  |        |
| $\chi/Q$ (s/m <sup>3</sup> ) at HVAC Intake for the Identified Release Points <sup>(1)</sup>         |  |   |                           |                                   |  |        |
| Plant Vent or PCS Air Diffuser <sup>(3)</sup>  | Ground Level Containment Release Points <sup>(4,8)</sup> | PORV and Safety Valve Releases <sup>(5)</sup> | Steam Line Break Releases | Fuel Handling Area <sup>(6)</sup> | Condenser Air Removal Stack <sup>(7)</sup> |        |
| 0 - 2 hours  | 3.0E-3   | 6.0E-3  | 2.0E-2                    | 2.4E-2                            | 6.0E-3                                     | 6.0E-3 |
| 2 - 8 hours  | 2.5E-3   | 3.6E-3  | 1.8E-2                    | 2.0E-2                            | 4.0E-3                                     | 4.0E-3 |
| 8 - 24 hours   | 1.0E-3   | 1.4E-3  | 7.0E-3                    | 7.5E-3                            | 2.0E-3                                     | 2.0E-3 |
| 1 - 4 days   | 8.0E-4   | 1.8E-3  | 5.0E-3                    | 5.5E-3                            | 1.5E-3                                     | 1.5E-3 |
| 4 - 30 days  | 6.0E-4   | 1.5E-3  | 4.5E-3                    | 5.0E-3                            | 1.0E-3                                     | 1.0E-3 |
| $\chi/Q$ (s/m <sup>3</sup> ) at Annex Building Door for the Identified Release Points <sup>(2)</sup> |  |   |                           |                                   |  |        |
| Plant Vent or PCS Air Diffuser <sup>(3)</sup>  | Ground Level Containment Release Points <sup>(4)</sup>   | PORV and Safety Valve Releases <sup>(5)</sup> | Steam Line Break Releases | Fuel Handling Area <sup>(6)</sup> | Condenser Air Removal Stack <sup>(7)</sup> |        |
| 0 - 2 hours  | 1.0E-3   | 1.0E-3  | 4.0E-3                    | 4.0E-3                            | 6.0E-3                                     | 2.0E-2 |
| 2 - 8 hours  | 7.5E-4   | 7.5E-4  | 3.2E-3                    | 3.2E-3                            | 4.0E-3                                     | 1.8E-2 |
| 8 - 24 hours   | 3.5E-4   | 3.5E-4  | 1.2E-3                    | 1.2E-3                            | 2.0E-3                                     | 7.0E-3 |
| 1 - 4 days   | 2.8E-4   | 2.8E-4  | 1.0E-3                    | 1.0E-3                            | 1.5E-3                                     | 5.0E-3 |
| 4 - 30 days  | 2.5E-4   | 2.5E-4  | 8.0E-4                    | 8.0E-4                            | 1.0E-3                                     | 4.5E-3 |

**Notes:**

1. These dispersion factors are to be used 1) for the time period preceding the isolation of the main control room and actuation of the emergency habitability system, 2) for the time after 72 hours when the compressed air supply in the emergency habitability system would be exhausted and outside air would be drawn into the main control room, and 3) for the determination of control room doses when the non-safety ventilation system is assumed to remain operable such that the emergency habitability system is not actuated.
2. These dispersion factors are to be used when the emergency habitability system is in operation and the only path for outside air to enter the main control room is that due to ingress/egress.
3. These dispersion factors are used for analysis of the doses due to a postulated small line break outside of containment. The plant vent and PCS air diffuser are potential release paths for other postulated events (loss-of-coolant accident, rod ejection accident, and fuel handling accident inside the containment); however, the values are bounded by the dispersion factors for ground level releases.

4. The listed values represent modeling the containment shell as a diffuse area source, and are used for evaluating the doses in the main control room for a loss-of-coolant accident, for the containment leakage of activity following a rod ejection accident, and for a fuel handling accident occurring inside the containment.
5. The listed values bound the dispersion factors for releases from the steam line safety & power-operated relief valves. These dispersion factors would be used for evaluating the doses in the main control room for a steam generator tube rupture, a main steam line break, a locked reactor coolant pump rotor, and for the secondary side release from a rod ejection accident.
6. The listed values bound the dispersion factors for releases from the fuel storage and handling area. The listed values also bound the dispersion factors for releases from the fuel storage area in the event that spent fuel boiling occurs and the fuel building relief panel opens on high temperature. These dispersion factors are used for the fuel handling accident occurring outside containment and for evaluating the impact of releases associated with spent fuel pool boiling.
7. This release point is included for information only as a potential activity release point. None of the design basis accident radiological consequences analyses model release from this point.
8. The LOCA dose analysis models the ground level containment release point HVAC intake atmospheric dispersion factors. Other analyses model more conservative values.

| Table 2.2-1<br>AP1000 ONSITE EXPLOSION SAFE DISTANCES           |   |   |                               |
|---|---|---|-------------------------------|
| Material  | Explosion Minimum Safe Distance <sup>(1)</sup> (feet) | Flammable Vapor Cloud Safe Distance <sup>(1)</sup> (feet) | AP1000 Distance to SSC (feet) |
| Liquid Hydrogen, H <sub>2</sub>                                 | 577   | 175   | 635                           |
| Pressurized Gaseous Hydrogen, H <sub>2</sub>                    | 6   | Not Applicable  | 10                            |
| Hydrazine, N <sub>2</sub> H <sub>4</sub>                        | 45  | Not Applicable  | 176                           |
| Morpholine, O(CH <sub>2</sub> CH <sub>2</sub> ) <sub>2</sub> NH | 66  | Not Applicable  | 176                           |
| 3-Methoxy propylamine (MOPA), C <sub>4</sub> H <sub>11</sub> NO | 87  | Not Applicable  | 176                           |
| No. 2 Diesel Fuel Oil   | 280   | Not Applicable  | 318                           |
| Waste Oil   | 102   | Not Applicable  | 201                           |

**Note:**

- Safe distance is to nearest point of nuclear island SSC.

| LIMITS OF ACCEPTABLE SETTLEMENT WITHOUT ADDITIONAL EVALUATION |   |   |  |
|---|---|---|--|
| Differential Across Nuclear Island Foundation Mat             | Total for Nuclear Island Foundation Mat | Differential Between Nuclear Island and Turbine Building <sup>(1)</sup> | Differential Between Nuclear Island and Other Buildings <sup>(1)</sup> |
| 1/2 inch 50 ft  | 6 inches                                | 3 inches  | 3 inches   |

**Note:**

1. Differential settlement is measured at the center of the nuclear island and the center of the adjacent structures.