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U.S. Nuclear Regulatory Commission  
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Your ref: Docket No. 52-006  
 Our ref: DCP\_NRC\_003104

December 10, 2010

Subject: Commitment to Perform Validation Testing of RCP Flywheel Material

This letter is a follow-up to a discussion that occurred during the December 3, 2010 ACRS meeting regarding questions associated with the Reactor Coolant Pump flywheel retainer ring material.

During the ACRS discussion, Westinghouse reviewed and justified the basis for selection of the flywheel retainer ring material. While the material was evaluated for service in aggressive environments, it is not expected to be exposed to primary water because it is encased within a corrosion resistant material, the ACRS expressed a concern that a defect in the casing could occur which could then expose the retainer ring material to primary water. While Westinghouse considers the flywheel material and design acceptable as-is without additional testing and asserts that existing NRC safety conclusions related to this material remain valid, to be responsive to the ACRS concern, Westinghouse made a verbal commitment to validate the expected resistance of this material to stress corrosion cracking by performing a stress corrosion cracking test of the flywheel material in primary water environment. This letter formalizes the commitment to perform this testing.

Results from this testing are expected to provide added confidence with the material selection in addition to the basis used to select the material. Westinghouse will provide the results of this testing when it is complete.

Questions or requests for additional information related to the content and preparation of this letter should be directed to Westinghouse. Please send copies of such questions or requests to the prospective applicants for combined licenses referencing the AP1000 Design Certification. A representative for each applicant is included on the cc: list of this letter.

Very truly yours,

R. F. Ziesing  
 Director, U.S. Licensing

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