NRC FORM 374

U.S. NUCLEAR REGULATORY COMMISSION

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee	
1. Nuclear Fuel Services, Inc.	3. License Number: SNM-124, Amendment 92
2. 1205 Banner Hill Road Erwin, TN 37650-9718	4. Expiration Date: July 31, 2009
	5. Docket Number.: 70-143 Reference Number.:

6. Byproduct Source, and/or Special Nuclear Material	7. Chemical and/or Physical Form	8. Maximum amount that Licensee May Possess at Any One Time Under This License
A. Uranium enriched up to 100 w/% in the U235 isotope which may contain up to an average of 10^{-6} grams plutonium per gram of uranium, 0.25 millicuries of fission products per gram of uranium, and 1.5×10^{-5} grams transuranic materials (including plutonium), per gram of uranium, as contaminants.	A. As described in Appendix B to Chapter 1 of the NFS license application, excluding pyrophoric forms	A. See Sensitive Conditions
B. Uranium enriched up to 100 w/% in the U233 isotope	B.1 Any form, but only as residual contamination from previous operations	B.1 See Sensitive Conditions

This license contains OFFICIAL USE ONLY information. Upon removal/redaction of the Sensitive Conditions on Pages 22-24, this license is DECONTROLLED.

Enclosure 1

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	B.2 Any form, as received for analysis and/or for input into development studies	B.2 See Sensitive Conditions
C. Plutonium	C.1 As counting and calibration standards	C.1 See Sensitive Conditions
	C.2 As residual contamination and holdup from previous operations	C.2 As described in the license application and an NFS report to the NRC transmitted by letter dated January 21, 1994 (NFS Document No. 28G94-001), and NFS report dated October 17, 1988 (NFS Document No. 28G88-007)
	C.3 As received for analysis or for input into development studies, any form except pyrophoric	C.3 See Sensitive Conditions
	C.4 As waste resulting from decontamination and volume reduction of equipment received from other organizations, any form except pyrophoric	C.4 See Sensitive Conditions
D. Transuranic Isotopes	D. As waste resulting from processing enriched uranium	D. See Sensitive Conditions
E. Fission Products	E. As waste resulting from processing enriched uranium	E. See Sensitive Conditions

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9. Authorized place of use: The licensee's existing facilities in Unicoi County, Tennessee, as described in the referenced application.

10. This license shall be deemed to contain two sections: Safety Conditions and Safeguards Conditions. These sections are part of the license, and the licensee is subject to compliance with all listed conditions in each section.

FOR THE NUCLEAR REGULATORY COMMISSION

Date: _____ By: _____

Brian W. Smith, Acting Deputy Director
Fuel Facility Licensing Directorate
Division of Fuel Cycle Safety
and Safeguards
Office of Nuclear Material Safety
and Safeguards

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SAFETY CONDITIONS

S-1 For use in accordance with the statements, representations, and conditions in Chapters 1 through 8 of the application submitted on the following dates, or as revised pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) 70.32 or 70.72:

July 24, 1996, and supplements dated May 9 and November 14, 1997; March 13, March 25, June 23, July 23, August 7, August 14, August 28, September 4, September 11, September 15, September 25, September 28, October 19, October 21, October 22, October 23, November 6, November 13, November 16, November 20, November 24, December 18, and December 21, 1998; January 29, February 4, February 10, February 16, February 24, April 20, April 23, May 21, July 30 (NFS No. 21G-99-0058), July 30 (NFS No. 21G-99-0093), August 13, December 10, December 21, and December 29, 1999; January 25, March 31, July 6, August 18, August 23, September 1, November 3, December 5, December 8, December 14, December 20, and December 27, 2000; January 11, January 12, March 30, May 11, June 29, October 5, and October 25, 2001; February 21, February 28, March 8, March 12, April 3, April 4, August 23, September 13, October 18, December 17, and December 23, 2002; January 23, February 10, February 14, February 27, March 3, March 6, March 10, March 13, April 14, April 16, April 22, July 31, September 26, and October 27, 2003; January 9, April 5, September 20, November 17, December 3, 2004 (except section 1.7.2.1 and relaxing the review frequency of operating procedures by the safety review committee), and December 16, 2004; February 9, March 30, April 22 (ML051170273), April 22 (ML051260178) April 29, May 23, May 31, June 6, June 10, June 13, June 20, July 18, September 13, September 29, October 21, November 10, December 14, December 16 (ML053530311), December 16 (ML060110519), and December 19, 2005; January 9, January 18, February 20, March 10, March 24, May 2, June 16, October 18, November 8, December 1, and December 13, 2006; January 5, February 15 (except the change to Section 3.0 of the Physical Protection Plan), April 13, April 20, May 15, May 23, June 29, July 3, August 23, August 31, September 28, November 2 (ML073370669), November 2 (ML073370671), December 4, December 14, December 21 (ML073620475), December 21 (ML080160456), 2007; and January 18, February 11, May 2, May 9, May 23, May 30, June 19, July 17, July 29, August 6, August 8, August 12, August 22, October 20, October 29, November 4, November 7 (ML083470300), November 7 (ML083190231), November 10, November 25, December 3, December 5, and December 19, 2008; January 15, January 22, February 6 (ML090680441), April 9, April 15, and November 20, 2009; and March 10, July 19, October 28, and December 6, 2010.

For the Blended Low-Enriched Uranium (BLEU) Preparation Facility (BPF) and Oxide Conversion Building (OCB) and Effluent Processing Building (EPB): May 24, August 16, October 11, October 16, November 8, and December 3, 2002; March 8, April 4, June 20, September 3, September 5, October 23 (Attachment 1), October 31, November 5, December 5, and December 10, 2003, February 6, February 11, February 25, March 12, March 15, March 16, March 17, March 18, March 19, April 30, and May 21, 2004; April 13, and May 24, 2006.

S-2 Deleted by Amendment 59, dated January 2005.

S-3 Deleted by Amendment 5, dated May 2000.

S-4 Deleted by Amendment 59, dated January 2005.

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S-5 Deleted by Amendment 59, dated January 2005.

S-6 Deleted by Amendment 2, dated February 2000.

S-7 Deleted by Amendment 2, dated February 2000.

S-8 Nuclear Fuel Services, Inc. (NFS) shall conduct quarterly Nuclear Criticality Safety (NCS) audits of selected plant activities involving Special Nuclear Material (SNM) such that SNM processing or storage areas are audited biennially. The purpose of the audits is to determine that: (a) site operations are conducted in compliance with license conditions, operating procedures, and posted limits, (b) administrative controls and postings are consistent with Nuclear Criticality Safety Evaluation (NCSE), (c) equipment and operations comply with NCSE, and (d) corrective actions relative to findings of NCS inspections are adequate.

S-9 Subcritical parameter values based on experiments, unless they are from the ANSI/ANS Series 8 standards, shall be not less than that corresponding to k_{eff} of 0.98 or, alternatively, the factors in Section 4.2.3.1 of the license application may be applied for uranium-water systems.

S-10 Notwithstanding the description of setting failure limits in Section 4.2.3.2 of the application, when determining subcriticality based on computer code calculations the failure limit shall be no greater than the value corresponding to: k_{eff} = .95 for systems containing uranium enriched in ^{235}U above 20%, k_{eff} = .95 for systems above 10% but below 20% enrichment that are not highly moderated, k_{eff} = .97 for systems above 10% but below 20% enrichment that are highly moderated, and k_{eff} = .97 for systems containing uranium enriched in ^{235}U less than 10%. As one acceptable method, the margin may be based on a validation against applicable benchmark experiments using a one-sided 95% tolerance limit at a 95% confidence level less an additional 0.015 Δk_{eff} . The k_{eff} values of .95 and .97 above are exact limit values, and do not imply that compliance need only be shown to two significant figures. Compliance with them shall allow for purely calculational inaccuracies, such as Monte Carlo variance, by meeting the limit with a margin in the conservative direction of at least two standard deviations. Any rounding shall be in the conservative direction.

S-11 Notwithstanding, Section 4.2.4.7 of the application, for situations in which it is credible, and not unlikely, that critical masses or concentrations may accumulate in a solution confined to a favorable geometry or poisoned vessel, and then be released to vessels of unfavorable geometry, transfer shall be controlled by one of the following three general provisions for double contingency:

(1) multiple engineered hardware controls capable of preventing unsafe transfer; or

(2) at least one engineered hardware control capable of preventing unsafe transfer plus a determination of safe conditions and actuation of transfer by an individual; or

(3) a design requiring independent actions by two individuals before transfer is possible, each action supported by independent measurements of material to be transferred, and a determination of safe conditions. In this case, physical impediments should be included in the system design.

which will prohibit either individual from performing both of the actions intended to be performed independently.

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S-12 Prior to August 15, 1999, NFS will implement fire protection procedures to minimize the threat of fire, explosions, or related perils to process control and safety systems which could lead to an unacceptable release of hazardous material related to SNM or radiation that would threaten workers, the public health and safety, or the environment, as committed to in Section 6.2 of the license application.

S-13 Deleted by Amendment No. 4, March 2000.

S-14 See Sensitive Conditions.

S-15 Active and administrative controls for flammable liquids and gasses must be operable in the fire area where flammable liquids and gases are present during KAST processing.

S-16 Prior to August 15, 1999, KAST Process fire walls will be upgraded to meet FHA recommendations, as described in NFS Document No. 21G-98-0198, *NFS Response to Request for Additional Fire Safety Information for the KAST Process*, dated December 8, 1998.

S-17 Prior to December 31, 1999, NFS shall protect KAST process areas and SNM vaults from lightning by installing a lightning protection system in accordance with the standard “Lightning Protection Code,” NFPA 780.

S-18 See Sensitive Conditions.

S-19 Prior to December 31, 1999, NFS will upgrade all process area sprinkler systems to alarm at a constantly manned location.

S-20 Deleted by Amendment 24, April 2001.

S-21 Deleted by Amendment No. 64, dated August 2005.

S-22 See Sensitive Conditions.

S-23 NFS shall inform the NRC within 30 days of receipt of a violation notice from the State of Tennessee Division of Air Pollution or Water Pollution Control, or receipt of modified requirements of the State-issued National Pollutant Discharge Elimination System permit.

S-24 The licensee shall maintain and execute the response measures in the Emergency Plan, Revision 10, transmitted by letter dated May 22, 2006, or as further revised by the licensee consistent with 10 CFR 70.32(i).

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S-25 NFS may make changes (modifications, additions, or removals) to the site, structures, processes, systems, equipment, components, computer programs, and activities of personnel without license amendment, provided that the proposed change does not involve:

(1) the creation of new types of accident sequences that, unless mitigated or prevented, would exceed the performance requirements of 10 CFR 70.61 and have not previously been described in the Integrated Safety Analysis (ISA) summary;

(2) the usage of new processes, technologies, or controls for which NFS has no prior experience;

(3) the removal, without at least an equivalent replacement of the safety function, of an item relied on for safety (IROFS) that is listed in the ISA summary;

(4) the alteration of any IROFS, listed in the ISA summary, that is the sole item preventing or mitigating an accident sequence that exceeds the performance requirements of 10 CFR 70.61; and

(5) a change to the conditions of this license or Part I of the license application.

Proposed changes not meeting all of the above criteria shall be deemed to require NRC approval by amendment. As part of the application for amendment, NFS shall perform an ISA for the change and submit either an ISA summary or applicable changes to a prior existing ISA summary. NFS shall also provide any necessary revisions to its environmental report.

Proposed changes requiring revision of applicable safety or environmental bases, but not requiring an amendment to the license in accordance with the above criteria, shall be reviewed and approved by the NFS safety review committee. The internally authorized change documentation shall provide the basis for determining that the change will be consistent with the criteria (1) through (5) above.

For any internally authorized change implemented by NFS without NRC approval pursuant to this license condition, NFS shall submit annually to the NRC applicable changes to the ISA summary of a prior existing ISA. In addition, NFS will submit annually a brief summary of all internally authorized changes not requiring prior NRC approval. NFS will submit by January 30 of each calendar year the revisions to the ISA summary and the summary of all internally authorized changes not requiring NRC approval.

S-26 Prior to engaging in the decommissioning activities specified in Section 1.6.6 of the license application dated November 16, 1998, NFS must determine the status of the procedures and activities planned with respect to 10 CFR 70.38(g)(1). If required, NFS must submit a decommissioning plan to the NRC for review and approval prior to initiating such actions.

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S-27 By January 30 of each calendar year, the licensee shall update the safety demonstration sections of the license application to reflect the licensee's current operations and evaluations. The updates shall, as a minimum, include information for the health and safety section of the application as required by 10 CFR 70.22(a) through 70.22(f) and 70.22(i) and operational data on environmental releases as required by 70.21.

S-28 Deleted by Amendment 31, October 2001.

S-29 Deleted by Amendment 31, October 2001.

S-30 Deleted by Amendment 31, October 2001.

S-31 Deleted by Amendment 31, October 2001.

S-32 Deleted by Amendment 31, October 2001.

S-33 Deleted by Amendment 31, October 2001.

S-34 Deleted by Amendment 31, October 2001.

S-35 Deleted by Amendment 31, October 2001.

S-36 Deleted by Amendment 31, October 2001.

S-37 Deleted by Amendment 31, October 2001.

S-38 Deleted by Amendment 31, October 2001.

S-39 See Sensitive Conditions.

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S-40 By December 31, 1999, for KAST process structures and equipment, NFS shall classify all items relied on for nuclear criticality safety as either safety-related or configuration-controlled equipment. Safety-related equipment (SRE) is defined as active or passive engineered-controls that are relied on to prevent nuclear criticality in accordance with the double contingency principle, and whose operation can change with time such that the equipment might not perform its function. Configuration-controlled equipment (CCE) is defined as structures, systems, or components for which either:

(i) some characteristic is relied on for double contingency, which characteristic will not change with time as a result of accidents identified in the ISA, or

(ii) the control is supplemented by one or more controls as one leg of the double contingency principle.

For SRE items, maintenance, calibration, testing, and/or inspection shall be performed in accordance with written, approved procedures to assure continued reliability and functional performance. SRE that has undergone maintenance will be functionally tested, calibrated, or inspected (as applicable) prior to restart.

CCE will be functionally tested, maintained, calibrated, and/or inspected periodically in accordance with written, approved procedures, with the following exceptions:

CCE that has no credible mechanism to fail beyond the conditions assumed in the bounding normal case does not require functional testing, calibration, or preventive maintenance.

CCE that is tested by every use and that is used with sufficient frequency to ensure adequate reliability does not require functional testing or preventive maintenance, unless it contains parts that degrade over time.

CCE items will be inspected after initial installation, replacement, and by periodic NCS audits.

S-41 Deleted by Amendment 32, dated February 2002.

S-42 Deleted by Amendment 5, dated April 2000.

S-43 Deleted by Amendment 22, dated March 2001.

S-44 Deleted by Amendment 22, dated March 2001.

S-45 Deleted by Amendment 32, dated February 2002.

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S-46 By August 1, 2000, NFS shall submit a Criticality Safety Upgrade Program (CSUP) Plan to NRC for review and approval. This CSUP shall address the following elements, at a minimum:

1. All Nuclear Criticality Safety Analyses (NCSAs) performed or revised after May 1, 2000, shall be upgraded as follows:

   (a) the criticality safety basis shall be consolidated in a single integrated and self-consistent document;

   (b) all engineered structures, systems, and components and operator actions relied on to meet the double contingency principle shall be clearly identified for each accident sequence leading to criticality;

   (c) the basis for double contingency shall be clearly documented, including technical documentation of the independence and unlikelihood of control failure;

   (d) normal and credible abnormal operating conditions shall be clearly identified; and

   (e) all assumptions credited for criticality safety shall be supported by documentation consisting of a technical demonstration of the adequacy of the assumptions rather than reliance on engineering judgement or historical practices.

2. By August 1, 2001, management procedures defining the criticality safety program shall be upgraded to the following standards:

   (a) the NCSAs consist of self-contained safety basis documents, sufficiently detailed to permit independent reconstruction of results by a knowledgeable criticality safety specialist without reliance on additional site-specific or historical knowledge;

   (b) the standard technical practices used in designing calculational models are specified in sufficient detail to ensure that the resulting NCSAs are uniform with respect to modeling reflection, determining the optimal range of moderation, treating interactions, accounting for dimensional tolerances, and any bounding approximations in models;

   (c) evaluation of accident sequences take potential interaction between fire and chemical safety and criticality safety into account;

   (d) the scope, conduct, and documentation of independent reviews of NCSAs are specified;

   (e) the applicability of code validation(s) to the specific cases being modeled is evaluated, including a determination of the adequacy of the subcritical margin;

(f) engineered as opposed to administrative controls are used as the preferred method of ensuring criticality safety, wherever practicable.

(g) the basis for using administrative instead of engineered controls is documented as part of the NCSA; and

(h) a problem reporting and corrective action program is established to ensure the effectiveness of the criticality safety program and criticality controls, and to ensure that effective corrective actions and lessons learned are flowed down into appropriate implementing documents. This program shall include the re-evaluation of the unlikelihood of control failure, as part of the double contingency safety basis, as control failure data is generated.

S-47 By July 31, 2001, NFS shall submit to NRC for approval the following information related to the North Site Decommissioning Plan:

(a) area factors for volumetrically-contaminated soils and the technical basis for those area factors,

(b) actual Minimum Detectable Concentrations (MDCs) for the NaI detector and the technical basis for those MDCs, and

(c) appropriate investigation levels for static and scan survey measurements that will be performed in impacted areas.

S-48 Notwithstanding the Derived Air Concentration (DAC) and Annual Limit on Intake (ALI) listed in Appendix B to 10 CFR Part 20, the licensee may use adjusted DAC values and adjusted ALI values specified in International Commission on Radiation Protection (ICRP), Publication 68 (Annals of the ICRP Volume 24, No.4).

S-49 NFS shall utilize, for setpoint determinations, conservative engineering analyses which account for safety limits, instrument and system accuracies, response times, instrument drift, manufacturer's data and operating experience. The analysis for each safety setpoint shall be a formal calculation and shall be documented for each IROFS interlock and alarm.

S-50 By February 13, 2004, NFS shall submit a revised BPF Integrated Safety Analysis Summary that incorporates changes resulting from NRC review questions documented in NFS letters dated September 3, September 5, October 31, November 5, November 7, December 5, and December 10, 2003.

S-51 The licensee shall submit a revised OCB/EPB Integrated Safety Analysis Summary that incorporates all changes to date, at least fifteen (15) days prior to the NRC's Operational Readiness Review.

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S-52 For the approval of procedures, the licensee shall ensure that concurrence with the Safety and Safeguards Review Council (SSRC) Chair's selection of the minimum designated SSRC reviewers is obtained from the entire SSRC and documented before procedures are approved. Documentation may be in the form of signature sheets, emails, memorandums or other means acceptable to the SSRC, and may include concurrence in advance by individual SSRC members for individual procedures or classes of procedures.

S-53 For existing processes (designed, installed, or in operation prior to June 30, 2008) the Configuration Management program will be applied in accordance with the following schedule: (1) apply throughout BLEU Prep Facility - September 2008: (2) apply throughout Navy Fuel Operations - June 2009; (3) apply throughout NFS - December 2010.

S-54 Introduction of UF_6 into the CD Line shall not occur until the Commission completes an operational readiness review to verify that (1) commitments in the amendment request have been fulfilled, (2) management measures for items relied on for safety have been implemented, (3) the CD Line has been constructed in accordance with design requirements, and (4) other actions necessary for safe operations are complete.

SAFEGUARDS CONDITIONS

Section-1.0 – ABRUPT LOSS DETECTION (For SSNM Only):

SG-1.1 Notwithstanding the requirement of 10 CFR 74.53(b)(1) to have a process detection capability for each unit process, the process units listed in Section 1.1.5.2 of the Plan identified in Condition SG-5.1 shall be exempt from such detection capability, and the licensee's process monitoring system shall be comprised of the control units described in Section 1.3 (and all sub-sections therein) of the above-mentioned Plan.

Section-2.0 – ITEM MONITORING (For SSNM Only):

SG-2.1 See Sensitive Conditions.

Section-3.0 – ALARM RESOLUTION:

SG-3.1 The licensee is authorized to continue material processing operations in Control Units 1, 3, 4, 5, and 15 under process monitoring alarm conditions. During the continuation of processing operations, the measures contained in Section 3.1.1 of the Plan identified in Condition SG-5.1 shall be implemented.

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Section-4.0 – QUALITY ASSURANCE (SSNM & LEU):

SG-4.1 Notwithstanding the requirements of 10 CFR 74.31(c)(2) for low-enriched uranium (LEU) and 10 CFR 74.59(d)(1) for SSNM to maintain a system of measurements to substantiate both the element and fissile isotope content of all SNM received, inventoried, shipped or discarded, SNM measured by the licensee for U-233, U-235, or Pu-239 by non-destructive assay techniques need not be measured for total element if the calculated element content is based on the measured isotope content which, in turn, is traceable to an isotopic abundance measurement at the area of generation.

SG-4.2 Notwithstanding the requirement of 10 CFR 74.59(e)(8) to establish and maintain control limits at the 0.05 and 0.001 levels of significance for all high-enriched uranium (HEU) related measurements, the licensee may use one and two scale divisions as being equivalent to the 0.05 and 0.001 control levels, respectively, for mass measurements.

SG-4.3 Notwithstanding Section 4.5.1 of the Plan identified in Condition SG-5.1, which states that a physical inventory of SSNM is conducted at an interval of at least every six calendar months with no more than 185 days elapsing between any two consecutive inventories, the licensee is granted an extension of time from April 3, 2000, to June 2, 2000, for conducting its SSNM physical inventory. This condition automatically expires on June 5, 2000.

SG-4.4 See Sensitive Conditions.

SG-4.5 Notwithstanding the requirements of 10 CFR 74.59(f)(1) and 74.59(f)(2)(viii) to measure and inventory all SSNM, the licensee may determine process exhaust ventilation system inventory quantities in accordance with Section 4.5.3.5 of the Plan identified in Condition SG-5.1.

SG-4.6 Deleted by Amendment 65, dated November 16, 2005.

SG-4.7 Deleted by Amendment 65, dated November 16, 2005.

SG-4.8 The SNM content of liquid waste discarded from collection tanks shall be analyzed and recorded at measured values. The measurement methods must have a greater sensitivity than the concentration of the sample aliquot analyzed, except when the quantity discarded does not exceed 50 grams U-235 per month from Plant I (HEU) and does not exceed 10 grams U-235 per month from MBA-4 (LEU) through those discard batches where the sample aliquot concentration is less than the sensitivity of the method.

SG-4.9 Notwithstanding the statement in Section 5.9, of the Plan identified in Condition SG-5.2, pertaining to bias corrections to inventory difference (ID) values, the licensee shall comply with Section 4.3.1 of such Plan with respect to determining any bias corrections to IDs.

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SG-4.10 Notwithstanding the requirements of 10 CFR 74.59(e)(8) relative to actions to be taken when replicate measurement data exceed a 0.001 control limit, the licensee shall comply with Section 4.4.1.7.3.4 of the Plan identified in Condition SG-5.1.

SG-4.11 Notwithstanding the requirement of 10 CFR 74.59(e)(4) that allows the pooling of data which has been shown to be not significantly different on the basis of appropriate statistical tests, the licensee may pool data from equivalent scales without testing.

SG-4.12 Notwithstanding the requirement of 10 CFR 74.59(e)(5) to evaluate all program data to establish random error variances, limits for systematic error, etc., the licensee may randomly select a partial quantity of bulk measurement program data, as described in Section 4.4.4(3) of the Plan identified in Condition SG-5.1, provided the partial data set is not statistically different from the total data population whenever the impact on the standard error of the inventory difference (SEID) is greater than 1.0 percent.

SG-4.13 Notwithstanding the requirement of 10 CFR 74.59(f)(1)(i) to calculate the SEID associated with each HEU ID value, the licensee need not determine such SEID for MBA-7 whenever its ID is less than 300 grams U-235.

SG-4.14 Notwithstanding the requirement of 10 CFR 74.31(c)(3) and of 74.59(e)(3)(i) to measure control standards for all measurement systems for the purpose of determining bias, and notwithstanding the requirement of 10 CFR 74.31(c)(4) and of 74.59(e)(8) to maintain a statistical control system to monitor such control standard measurements, the licensee need not measure nor monitor such control standards for point calibrated, bias-free, systems. To be regarded as bias-free, a measurement system must be calibrated by one or more measurements of a representative standard(s) each time process unknowns are measured, and the measurement value assigned to a given unknown is based on the associated calibration.

SG-4.15 All SNM not in transit shall be physically located within a material balance area (MBA) or item control area (ICA), except as specified in Condition SG-4.15.1.

SG-4.15.1 The requirement of Condition SG-4.15 shall not apply to HEU or LEU contained in, or precipitated from, measured liquid or gaseous waste discards.

SG-4.16 Solutions generated from the use of sinks, eye washers, safety showers, drinking fountains, etc., located within HEU material access areas (MAAs) shall be collected and measured prior to discarding.

SG-4.17 See Sensitive Conditions.

SG-4.18 Notwithstanding the requirement of 10 CFR 74.15 to include limit of error data on DOE/NRC Form-741 for all SNM shipments, the licensee is exempt from including such data on 741 Forms

associated with waste burial shipments.

SG-4.19 Whenever an SNM Material Superintendent or designated SNM Custodian is summoned to an MAA exit point to assist in resolving whether an item or container should be allowed to exit to the Protected Area, in accordance with the currently approved "Physical Safeguards Plan," the Superintendent or Custodian shall document the basis for any decision allowing the item or container to leave the area.

SG-4.20 The licensee is exempted from calculating the SEID and measurement system biases associated with LEU physical inventories provided that the calculated inventory difference does not exceed 1,000 grams U-235.

SG-4.21 Notwithstanding Section 7.1 of the Plan identified in Condition SG-5.2, which states that "confirmatory measurements of scrap receipts are performed after the scrap is dissolved," the term *"scrap receipts"* shall not apply to receipt materials whose SNM content can be determined on the as-received-material by weighing, sampling and analyses with a measurement uncertainty (at the 95% confidence level [C.L.]) of less than 2.00 percent (based on a single sample).

SG-4.22 Notwithstanding the heading "Typical MC&A Procedures" for Table 3.5 of the Plan identified in Condition SG-5.2, all procedures listed in Table 3.5 shall be officially designated as "Critical MC&A Procedures," and any revisions to these procedures shall be subject to the same review and approval requirements (as specified in Section 3.5 of the Plan) that applied to the original procedures.

SG-4.23 Notwithstanding statements contained in Section 4.2.4 of the Plan identified in Condition SG-5.2, if the normal minimum number of control standard measurements per week, day, or shift of system use (depending on type of measurement system) does not generate at least 25 control standard measurements for a given LEU measurement system during any inventory period in which the active inventory is greater than 9,000 grams U-235, the licensee shall nevertheless generate at least 16 control standard measurements for each key measurement system utilized during the inventory period.

SG-4.24 Deleted by Amendment 3, March 2000. This Condition expired May 15, 1999.

SG-4.25 Deleted by Amendment 16, January 2001. This Condition expired July 8, 2000.

SG-4.26 Deleted by Amendment 21, March 2001. This Condition expired February 11, 2001.

SG-4.27 Deleted by Amendment 28, June 2001. This Condition expired April 14, 2001.

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SG-4.28 Notwithstanding the commitments of Section 4.5.1 of the Fundamental Nuclear Material Control (FNMC) Plan identified in Condition SG-5.1 to submit a completed Strategic Special Nuclear Material Physical Inventory Summary Report on NRC Form 327 not later than 45 days from the start of the physical inventory, the licensee is exempted from the above stated requirements and shall have 21 additional days to complete the May 2002 physical inventory report. This condition automatically expires on July 23, 2002.

SG-4.29 Notwithstanding the commitments in Section 4.7 of the FNMC Plan identified in Condition SG-5.1 to perform receipt verification measurements within 30 days, the licensee shall have until August 31, 2003, to fulfill the above-stated commitment relative to the shipment of highly-enriched uranium material identified in the July 23, 2003, request letter.

SG-4.30 Deleted by Amendment 48, February 2004. This condition expired October 2003.

SG-4.31 Deleted by Amendment 48, February 2004. This condition expired November 2003.

SG-4.32 Notwithstanding the commitments in Section 4.7.2.1 of the FNMC Plan identified in Condition SG-5.1 to perform material receipt measurements from each material lot, the licensee may use the original receipt values with a limited confirmatory measurement relative to the received material identified in the October 1, 2004, request letter. This condition shall automatically expire on completion of the final shipment of the subject oxide material.

SG-4.33 Notwithstanding the commitments in Section 4.5.3.7 of the FNMC Plan identified in Condition SG-5.1 to perform material measurements for physical inventories, the licensee may use a material inventory measurement modification with regard to a quantity of partially processed scrap material identified in the October 20, 2004, request letter. This condition shall automatically expire on completion of the final processing of the subject scrap material.

SG-4.34 Notwithstanding the commitment in Section 4.7.1 of the FNMC Plan identified in Condition SG-5.1 to perform certain receipt verification measurements of strategic special nuclear material, the licensee shall have five (5) additional days to fulfill the above stated commitment relative to the shipment of high-enriched uranium identified in the November 20, 2009, request letter. This condition automatically expires on April 30, 2011.

SG-4.35 See Sensitive Conditions.

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SG-4.36 Notwithstanding the requirements of Section 7.4.2.1 of the facility's FNMC Plan, NFS may use the shipper's quantities to resolve the shipper-receiver differences and for material accounting purposes for batches FZF-KAS-096, FZF-KAS-077, and JXI-KAS-001 for the Blended Low-Enriched Uranium Preparation Facility.

SG-4.37 Notwithstanding the requirements of 10 CFR 74.59(f)(1) and Section 4.5.1 of the facility's FNMCP identified in Condition SG-5.1 to perform a physical inventory of strategic SNM, the licensee shall perform a physical inventory within 90 days of the date of the letter informing the licensee that the Commission has no objection to restarting the BLEU Preparation Facility Uranium Oxide process line and supporting operations addressed in Confirmatory Action Letter No. 2-2010-001 dated January 7, 2010.

Section-5.0 – FNMC PLANS AND SPECIAL ISSUES IN PLAN APPENDICES:

SG-5.1 In order to achieve the performance objectives of 10 CFR 74.51(a) and maintain the system capabilities identified in 10 CFR 74.51(b), the licensee shall follow its "Fundamental Nuclear Material Control Plan" (Plan) with respect to all activities involving strategic SNM, except as noted in License Condition SG-5.5. The Plan, as currently revised and approved, consists of:

General Discussion ----------	Rev. 8 (dated April 2005)
Section 1 – Process Monitoring ---------	Rev. 24 (dated November 2008)
Section 2 – Item Monitoring ----------	Rev. 8 (dated August 2006)
Section 3 – Alarm Resolution ----------	Rev. 9 (dated November 2008)
Section 4 – QA & Accounting ----------	Rev. 17 (dated June 2007)
Annex A ----------	Rev. 6 (dated June 2007)
Annex B ----------	Rev. 1 (dated August 1998)
Annex C ----------	Rev. 1 (dated August 1998)
Annex D ----------	Rev. 3 (dated June 2007)

Revisions to this Plan shall be made only in accordance with, and pursuant to, either 10 CFR 70.32(c) or 70.34.

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SG-5.2 In order to achieve the performance objectives of 10 CFR 74.31(a) and maintain the system capabilities identified in 10 CFR 74.31(c), the licensee shall follow its "Fundamental Nuclear Material Control Plan (Plan) for SNM of Low Enriched Uranium" with respect to all activities involving SNM of low strategic significance. The Plan, as currently revised and approved, consists of:

Section 1 ---------- Rev. 6 (dated January 2008)
Section 2 --- ---------- Rev. 4 (dated August 2004)
Section 3 ---------- Rev. 8 (dated January 2008)
Section 4 ---------- Rev. 6 (dated August 2006)
Section 5 ---------- Rev. 4 (dated January 2005)
Section 6 ---------- Rev. 5 (dated August 2006)
Section 7 ---------- Rev. 2 (dated January 2002)
Section 8 ---------- Rev. 3 (dated January 2005)
Section 9 ---------- Rev. 1 (dated February 1993)
Annex ---------- Rev. 6 (dated January 2008)

Revisions to this Plan shall be made only in accordance with, and pursuant to, either 10 CFR 70.32(c) or 70.34.

SG-5.3 Notwithstanding the requirement of 10 CFR 74.59(f)(1)(i) to estimate the standard error associated with SSNM inventory difference values, and notwithstanding the requirements of 10 CFR 74.59(e)(3) through (e)(8), the licensee may, in lieu of said requirements, follow Appendix G of the Plan identified in SG-Condition 5.1 with respect to plutonium measurements and measurement control associated with the plutonium decommissioning project.

SG-5.3.1 With regard to the plutonium decommissioning project (described in Appendix G of the Plan identified in Condition SG-5.1), the licensee shall comply with the following:

(a) For plutonium accountability measurements, the maximum measurement uncertainty (at the 95% C.L.) of measurement values equal to or greater than 100 grams Pu shall not exceed plus or minus 10.0%. For measurement values less than 100 grams Pu, but equal to or greater than 25 grams Pu, the maximum measurement uncertainty shall not exceed plus or minus 20.0% (at the 95% C.L.).

(b) For net weight measurements utilized for establishing "nanocuries Pu per gram waste" values (which in turn are used for establishing the category of waste), the maximum measurement uncertainty (at the 95% C.L.) shall not exceed plus or minus 2.00 percent.

(c) Sufficient control measurements shall be generated and documented so as to demonstrate compliance with 5.3.1(a) and (b) above.

(d) For each inventory period during which plutonium decommissioning activities are conducted, the measurement uncertainty associated with the total quantity of plutonium in item form generated and measured during the period shall be derived from all relevant measurement control data generated during that inventory period.

(e) For each inventory period during which plutonium decommissioning activities are conducted, plutonium "additions to" and "removals from material in process" (ATP and RFP) shall be calculated. Any measured Pu quantity, in item form, which is generated from existing residual holdup shall be regarded as an ATP at the time of its generation. Any measured Pu quantity, in item form, which is tamper-safe sealed and which will not undergo any additional processing (such as washing, compaction, etc.) prior to shipment offsite shall be regarded as an RFP upon obtaining such status. The limit for total plutonium measurement uncertainty for each inventory period shall be the larger of (1) 250 grams plutonium or (2) 10.0 percent of the larger of ATP or RFP.

(f) The licensee shall investigate any non-zero inventory difference, since a non-zero ID will be (for this operation) indicative of an item(s) discrepancy.

SG-5.3.2 Storage of plutonium items generated during plutonium decommissioning activities shall be in accordance with the commitments contained in the licensee's Plan identified in Condition SG-6.1.

SG-5.4 Operations involving SNM which are not described in the appropriate Plan identified by either Condition SG-5.1 or SG-5.2 shall not be initiated until an appropriate safeguards plan (describing all new and/or modified security and MC&A measures to be implemented) has been approved by the appropriate NRC safeguards licensing authority.

SG-5.5 Notwithstanding the requirements of 10 CFR 74.51(b) and (d), 74.53, and 74.59(d)(3), during periods of curtailed SSNM activities limited to (1) use of less than five (5.000) formula kilograms of SSNM contained in encapsulated or tamper-safe sealed standards; (2) use of less than five (5.000) formula kilograms of SSNM contained in materials associated with research and development activities and/or laboratory services; (3) vault storage of HEU oxides in item form except for samples utilized for independent receipt measurement; (4) storage of low-level waste materials destined for offsite disposal; and (5) decontamination and decommissioning operations involving residual holdup and site remediation; the licensee is exempt from the above-mentioned regulations and shall, in lieu of these regulations, follow Sections 1.0 through 4.0 of its "Fundamental Nuclear Material Control Plan Applicable for Periods of Limited HEU Processing Activities." This Plan, as currently revised and approved, consists of:

General Discussion --- Revision 1 (dated October 1994)
Section 1 ---------- Revision 1 (dated October 1994)
Section 2 ---------- Revision 1 (dated October 1994)
Section 3 ---------- Revision 1 (dated October 1994)
Section 4 ---------- Revision 0 (dated February 1994)

During such periods of limited HEU processing, the licensee need not follow the Plan identified in Condition SG-5.1. Whenever the possession and use limitations defined above in this condition are not applicable, the Plan identified herein shall be regarded as null and void, and the SG-5.1 Plan shall be in full force.

Section-6.0 – PHYSICAL PROTECTION REQUIREMENTS FOR STRATEGIC SPECIAL NUCLEAR MATERIAL:

SG-6.1 The licensee shall follow the physical protection plan entitled “NFS Physical Protection Plan for Protection Of Category 1, High Enriched-Uranium (Strategic Special Nuclear Material), Revision 7,” and as the Plan may be further revised in accordance with the provisions of 10 CFR 70.32(e).

SG-6.2 The licensee shall follow the safeguards contingency plan titled "NFS Safeguards Contingency Response Plan, Revision 1," dated March 31, 2006; and as may be further revised in accordance with the provisions of 10 CFR 70.32(g).

SG-6.3 The licensee shall follow the guard training and qualification plan titled "NFS Site Security Training and Qualification Plan, Revision 2," dated November 25, 2008; and as may be further revised in accordance with the provisions of 10 CFR 70.32(e).

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SG-6.4 The licensee shall comply with the provisions of the physical protection plan titled "Protection of Special Nuclear Material of Moderate Strategic Significance, Revision 0," submitted by letter dated November 2, 2007, and supplemented by letter dated May 23, 2008, and as it may be further revised in accordance with the provisions of 10 CFR 70.32, as follows:

a. The licensee shall comply with the transportation security provisions (Chapters 7-13) for Category II quantities of SNM, and

b. The licensee may only implement the fixed site security provisions (Chapters 1-6) after notifying the NRC at least 90 days before implementation begins. The notice shall be made in writing to the Director, Division of Fuel Cycle Safety and Safeguards, NRC Headquarters, with a copy to the Director, Division of Fuel Facility Inspection, NRC Region II.

SG-6.5 The licensee shall comply with the provisions of the physical protection plan titled, "Protection of Special Nuclear Material of Low Strategic Significance, Revision 2," submitted by letter dated May 26, 2004, and as it may be further revised in accordance with the provisions of 10 CFR 70.32.

TRANSPORTATION CONDITIONS

Section-1.0 – TRANSPORTATION SECURITY MEASURES:

TR-1.1 The licensee shall follow the measures described in the physical security plan titled "Physical Security Plan for the Protection of Special Nuclear Material of Moderate Strategic Significance, Revision 4," dated October 1991 (letter dated December 20, 1991), and as it may be further revised in accordance with the provisions of 10 CFR 70.32 (e).

TR-1.2 See Sensitive Conditions.