



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 29, 2010

Mr. Randall K. Edington
Executive Vice President Nuclear/
Chief Nuclear Officer
Mail Station 7602
Arizona Public Service Company
P.O. Box 52034
Phoenix, AZ 85072-2034

SUBJECT: PALO VERDE NUCLEAR GENERATING STATION, UNITS 1, 2, AND 3 -
ISSUANCE OF AMENDMENTS RE: CHANGES TO TECHNICAL
SPECIFICATION (TS) 2.2, "SAFETY LIMIT VIOLATIONS," AND TS 5.2.1,
"ONSITE AND OFFSITE ORGANIZATIONS" (TAC NOS. ME3755, ME3756,
AND ME3757)

Dear Mr. Edington:

The Commission has issued the enclosed Amendment No. 183 to Facility Operating License No. NPF-41, Amendment No. 183 to Facility Operating License No. NPF-51, and Amendment No. 183 to Facility Operating License No. NPF-74 for the Palo Verde Nuclear Generating Station, Units 1, 2, and 3, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated April 8, 2010.

The amendments delete redundant reporting and operational restriction provisions from TS Section 2.2, "Safety Limit Violations," consistent with Technical Specification Task Force (TSTF) change traveler TSTF-5-A, Revision 1, "Delete Safety Limit Violation Notification Requirements," and replace plant-specific titles with generic titles in TS Section 5.2.1, "Onsite and Offsite Organizations," consistent with TSTF-65-A, Revision 1, "Use of Generic Titles for Utility Positions."

R. Edington

- 2 -

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

for 

James R. Hall, Senior Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-528, STN 50-529,
and STN 50-530

Enclosures:

1. Amendment No. 183 to NPF-41
2. Amendment No. 183 to NPF-51
3. Amendment No. 183 to NPF-74
4. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-528

PALO VERDE NUCLEAR GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 183
License No. NPF-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Arizona Public Service Company (APS or the licensee) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority dated April 8, 2010, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C(2) of Facility Operating License No. NPF-41 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 183, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Facility Operating
License No. NPF-41 and
Technical Specifications

Date of Issuance: December 29, 2010



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-529

PALO VERDE NUCLEAR GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 183
License No. NPF-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Arizona Public Service Company (APS or the licensee) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority dated April 8, 2010, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C(2) of Facility Operating License No. NPF-51 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 183, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Facility Operating
License No. NPF-51 and
Technical Specifications

Date of Issuance: December 29, 2010



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-530

PALO VERDE NUCLEAR GENERATING STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 183
License No. NPF-74

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Arizona Public Service Company (APS or the licensee) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority dated April 8, 2010, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C(2) of Facility Operating License No. NPF-74 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 183, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Facility Operating
License No. NPF-74 and
Technical Specifications

Date of Issuance: December 29, 2010

ATTACHMENT TO LICENSE AMENDMENT NOS. 183, 183, AND 183

FACILITY OPERATING LICENSE NOS. NPF-41, NPF-51, AND NPF-74

DOCKET NOS. STN 50-528, STN 50-529, AND STN 50-530

Replace the following pages of the Facility Operating Licenses Nos. NPF-41, NPF-51, and NPF-74, and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Operating Licenses

REMOVE

INSERT

Replace Page 5 of Facility Operating License No. NPF-41 with the attached Page 5.

Replace Page 6 of Facility Operating License No. NPF-51 with the attached Page 6.

Replace Page 4 of Facility Operating License No. NPF-74 with the attached Page 4.

Technical Specifications

REMOVE

INSERT

2.0-1

2.0-1

2.0-2

5.2-1

5.2-1

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 183, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

(3) Antitrust Conditions

This license is subject to the antitrust conditions delineated in Appendix C to this license.

(4) Operating Staff Experience Requirements

Deleted

(5) Post-Fuel-Loading Initial Test Program (Section 14, SER and SSER 2)*

Deleted

(6) Environmental Qualification

Deleted

(7) Fire Protection Program

APS shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility, as supplemented and amended, and as approved in the SER through Supplement 11, subject to the following provision:

APS may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

(8) Emergency Preparedness

Deleted

*The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 183, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

(3) Antitrust Conditions

This license is subject to the antitrust conditions delineated in Appendix C to this license.

(4) Operating Staff Experience Requirements (Section 13.1.2, SSER 9)*

Deleted

(5) Initial Test Program (Section 14, SER and SSER 2)

Deleted

(6) Fire Protection Program

APS shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility, as supplemented and amended, and as approved in the SER through Supplement 11, subject to the following provision:

APS may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

(7) Inservice Inspection Program (Sections 5.2.4 and 6.6, SER and SSER 9)

Deleted

(8) Supplement No. 1 to NUREG-0737 Requirements

Deleted

*The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

(1) Maximum Power Level

Arizona Public Service Company (APS) is authorized to operate the facility at reactor core power levels not in excess of 3990 megawatts thermal (100% power), in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 183, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

(3) Antitrust Conditions

This license is subject to the antitrust conditions delineated in Appendix C to this license.

(4) Initial Test Program (Section 14, SER and SSER 2)

Deleted

(5) Additional Conditions

The Additional Conditions contained in Appendix D, as revised through Amendment No. 171, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Additional Conditions.

(6) Mitigation Strategy License Condition

APS shall develop and maintain strategies for addressing large fires and explosions and that include the following key areas:

(a) Fire fighting response strategy with the following elements:

1. Pre-defined coordinated fire response strategy and guidance.
2. Assessment of mutual aid fire fighting assets.
3. Designated staging areas for equipment and materials.
4. Command and control.
5. Training of response personnel.

2.0 SAFETY LIMITS (SLs)

2.1 SLs

2.1.1 Reactor Core SLs

- 2.1.1.1 In MODES 1 and 2, Departure from Nucleate Boiling Ratio (DNBR) shall be maintained at ≥ 1.34 .
- 2.1.1.2 In MODES 1 and 2, the peak fuel centerline temperature shall be maintained $< 5080^{\circ}\text{F}$ (decreasing by 58°F per 10,000 MWD/MTU for burnup and adjusting for burnable poisons per CENPD-382-P-A).

2.1.2 Reactor Coolant System (RCS) Pressure SL

In MODES 1, 2, 3, 4, and 5, the RCS pressure shall be maintained at ≤ 2750 psia.

2.2 SL Violations

- 2.2.1 If SL 2.1.1.1 or SL 2.1.1.2 is violated, restore compliance and be in MODE 3 within 1 hour.
 - 2.2.2 If SL 2.1.2 is violated:
 - 2.2.2.1 In MODE 1 or 2, restore compliance and be in MODE 3 within 1 hour.
 - 2.2.2.2 In MODE 3, 4, or 5, restore compliance within 5 minutes.
-

5.0 ADMINISTRATIVE CONTROLS

5.2 Organization

5.2.1 Onsite and Offsite Organizations

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be defined and established throughout highest management levels, intermediate levels, and all operating organization positions. These relationships shall be documented and updated, as appropriate, in organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements including the plant-specific titles of those personnel fulfilling the responsibilities of the positions delineated in these Technical Specifications shall be documented in the UFSAR;
- b. The plant manager shall be responsible for overall safe operation of the plant and shall have control over those onsite activities necessary for safe operation and maintenance of the plant;
- c. A specified corporate officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety; and
- d. The individuals who train the operating staff, carry out health physics, or perform quality assurance functions may report to the appropriate onsite manager; however, these individuals shall have sufficient organizational freedom to ensure their independence from operating pressures.

5.2.2 Unit Staff

The unit staff organization shall include the following:

- a. A non-licensed operator shall be assigned to each reactor containing fuel and an additional non-licensed operator

(continued)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 183 TO FACILITY OPERATING LICENSE NO. NPF-41,
AMENDMENT NO. 183 TO FACILITY OPERATING LICENSE NO. NPF-51, AND
AMENDMENT NO. 183 TO FACILITY OPERATING LICENSE NO. NPF-74
ARIZONA PUBLIC SERVICE COMPANY, ET AL.
PALO VERDE NUCLEAR GENERATING STATION, UNITS 1, 2, AND 3
DOCKET NOS. STN 50-528, STN 50-529, AND STN 50-530

1.0 INTRODUCTION

By application dated April 8, 2010 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML101110063), Arizona Public Service Company (APS, the licensee) requested changes to the Technical Specifications (TSs) for Palo Verde Nuclear Generating Station (PVNGS), Units 1, 2, and 3. The proposed changes would delete redundant reporting and operational restriction provisions from TS Section 2.2, "Safety Limit Violations," consistent with Technical Specification Task Force (TSTF) change traveler TSTF-5-A, Revision 1, "Delete Safety Limit Violation Notification Requirements," and replace plant-specific titles with generic titles in TS Section 5.2.1, "Onsite and Offsite Organizations," consistent with TSTF-65-A, Revision 1, "Use of Generic Titles for Utility Positions."

2.0 REGULATORY EVALUATION

Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36, "Technical specifications," requires TSs for nuclear reactors to include items in the following categories: (1) safety limits (SLs) and limiting safety system settings (LSSSs); (2) limiting conditions for operation (LCOs); (3) surveillance requirements; (4) design features; and (5) administrative controls. The regulation does not specify the particular requirements to be included in a plant's TSs.

On July 22, 1993, the U.S. Nuclear Regulatory Commission (NRC) published the "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (58 FR 39132). This Final Policy Statement discussed criteria for determining which items must be included in the TSs as LCOs. These criteria were subsequently incorporated into 10 CFR 50.36 (60 FR 36953; July 19, 1995).

Licensees propose revisions to their licensed nuclear reactor facilities' TSs to: (1) reflect modifications to the analyses and evaluation included in the updated safety analysis report (SAR), if required by 10 CFR 50.59, "Changes, tests, and experiments"; (2) incorporate revised NRC policy and guidance regarding the content and format of TSs; (3) clarify a specification's intent by revising ambiguous language and correcting editorial errors; and (4) enhance specifications to correct inadequacies. In determining the acceptability of TS changes, the NRC staff interprets the requirements of 10 CFR 50.36 using the guidance in the Standard Technical Specifications (STSs), contained in the model specifications, the Bases for the SLs and LCOs, and the references cited in the Bases. As discussed in the Final Policy Statement, the NRC staff reviews, on a case-by-case basis, whether enforceable regulatory controls (e.g., 10 CFR 50.59) are needed for material moved to licensee-controlled documents, such as the updated SAR, the Technical Requirements Manual, the TS Bases, the Quality Assurance Plan, and the Inservice Test Program. The NRC staff determines that plant-specific adoptions of STS format and content provide continued adequate safety when (1) the change is editorial, administrative, or provides clarification (i.e., no requirements are materially altered); (2) the change is more restrictive than the licensee's current requirement; or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards and the facility's licensing basis.

The NRC staff considered the following regulatory requirements and guidance in conducting its review of this license amendment request:

- 10 CFR 50.36(a)(1), which states, in part, that "[e]ach applicant for a license authorizing operation of a production or utilization facility shall include in his application proposed technical specifications in accordance with the requirements of this section." The regulations in 10 CFR 50.36(c)(1), "Safety limits, limiting safety system settings, and limiting control settings," state, in part, that "[i]f any safety limit is exceeded, the reactor must be shut down. The licensee shall notify the Commission, review the matter, and record the results of the review..." The regulations in 10 CFR 50.36(c)(5), "Administrative controls," state, "Administrative controls are the provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of the facility in a safe manner. Each licensee shall submit any reports to the Commission pursuant to approved technical specifications as specified in § 50.4."
- NUREG-1432, Vol. 1, Revision 3, "Standard Technical Specifications - Combustion Engineering Plants" (ADAMS Accession No. ML041830597).
- Technical Specification Task Force (TSTF) Improved Standard Technical Specifications Change Traveler TSTF-5-A, Revision 1, "Safety Limit Violation Reporting Requirements."
- Technical Specification Task Force (TSTF) Improved Standard Technical Specifications Change Traveler TSTF-65-A, Revision 1, "Use of Generic Titles for Utility Positions."

3.0 TECHNICAL EVALUATION

3.1 TSTF-5-A, Revision 1, "Safety Limit Violation Reporting Requirements"

The NRC approved this change to Revision 1 of NUREG-1432, "Standard Technical Specifications - Combustion Engineering Plants" (STS) on June 11, 1996. TSTF-5-A was later adopted into Revision 2 of NUREG-1432. This change specifically revised STS Section 2.2, "Safety Limit Violations," by deleting the following TS requirements, which must be satisfied in the event a safety limit (SL) is violated, but are redundant to other regulations:

- STS 2.2.1 (PVNGS TS 2.2.3), a 1-hour NRC reporting requirement, which is redundant to the NRC emergency reporting requirements of 10 CFR 50.72, "Immediate notification requirements for operating nuclear power reactors."
- STS 2.2.3 (PVNGS TS 2.2.4), a requirement to notify licensee management, which is redundant to the licensee's Quality Assurance (QA) program required by 10 CFR 50.54, "Conditions of licenses," and Criterion XVI, "Corrective Action," of Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants," to 10 CFR Part 50.
- STS 2.2.4 (PVNGS TS 2.2.5), a requirement to submit a Licensee Event Report (LER) within 30 days to the NRC and licensee management, pursuant to 10 CFR 50.73, "Licensee event report system." This requirement is redundant to 10 CFR 50.73 and the licensee's QA program.
- STS 2.2.5 (PVNGS TS 2.2.6), a requirement that the NRC give prior approval to resume plant operation, which is redundant to 10 CFR 50.36(c)(1)(i)(A).

The remaining SL violation action requirements were renumbered and appropriate changes to the STS 2.2 Bases were made in NUREG-1432, Revision 2.

APS has proposed changes identical to those in TSTF-5-A for the PVNGS TS. In addition, APS provided conforming changes to the corresponding TS Bases, for information only.

The reporting requirements in 10 CFR 50.36(c)(7), "Initial notification," and 10 CFR 50.36(c)(8), "Written Reports," require that appropriate prompt notifications and written reports are made to the NRC. They also require that these reports be performed in accordance with the requirements of 10 CFR 50.72 and 10 CFR 50.73. Thus, if an SL is violated, appropriate reporting will be made to the NRC in accordance with the regulations. Therefore, removal of the NRC notification and reporting requirements in PVNGS TSs 2.2.3, 2.2.4, and 2.2.5 is acceptable.

In addition, the licensee proposes to remove requirements from TSs 2.2.3 and 2.2.4 to notify the PVNGS Director, Operations, and the Senior Vice President, Nuclear within 24 hours of any SL violation and to submit an LER within 30 days. These requirements are adequately addressed in the PVNGS QA program which is required by 10 CFR 50.54. Therefore, removing these licensee management notification and reporting requirements from the TSs is acceptable.

The PVNGS TS 2.2.5 requirement for submitting an LER to NRC within 30 days is inconsistent with the current regulations in 10 CFR 50.73(a)(1), which specifies 60 days. The proposed amendment will remove this inconsistency.

As noted above, the requirement for prior NRC approval for resumption of plant operation following an SL violation in TS 2.2.6 is redundant to 10 CFR 50.36. Therefore, its removal is acceptable.

The NRC staff concludes that the proposed changes are acceptable, as they do not alter existing restrictions on plant operation, but only remove the administrative burden of maintaining redundant notification, reporting, and plant startup requirements. Further, the staff concludes that the licensee has adequate administrative controls and programs for the identification and necessary reporting of safety limit violations in accordance with 10 CFR 50.36, 10 CFR 50.72, and 10 CFR 50.73, and that the proposed changes are consistent with the STSs and TSTF-5-A, Revision 1.

3.2 TSTF-65-A, Revision 1, "Use of Generic Titles for Utility Positions"

This TSTF was approved by the NRC on December 2, 1997, and allows the use of generic personnel titles as provided by American National Standards Institute/American Nuclear Society (ANSI/ANS) 3.1, "American National Standard for Selection and Training of Nuclear Power Plant Personnel," in lieu of plant-specific personnel titles. This change does not eliminate any of the qualifications, responsibilities, or requirements for these positions, since the plant-specific personnel titles are currently identified in the PVNGS Updated Final Safety Analysis Report (UFSAR). In addition, the TSTF maintains the requirements that these positions meet the qualifications acceptable to the NRC staff (i.e., ANSI/ANS-3.1-1978, as described in TS Section 5.3).

The proposed amendment modifies TS Section 5.2.1, "Onsite and Offsite Organizations," subparagraphs a, b, and c as follows:

- a. Subparagraph a. adds the phrase, "including the plant-specific titles of those personnel fulfilling the responsibilities of the positions delineated in these Technical Specifications," to the last sentence.
- b. Subparagraph b. is modified by deleting the plant-specific title, "Senior Vice President, Nuclear," and replacing it with the generic title, "plant manager."
- c. Subparagraph c. is modified by deleting the plant-specific title, "The Senior Vice President, Nuclear," and replacing it with the generic title, "A specified corporate officer."

These changes do not eliminate any of the qualifications, responsibilities, or requirements for these positions, as the plant-specific titles and position responsibilities are identified in the PVNGS UFSAR. Therefore, the NRC staff concludes that these changes are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arizona State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding published in the *Federal Register* on July 27, 2010 (75 FR 44022). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: V. Cusumano

Date: December 29, 2010

R. Edington

- 2 -

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA by N. Kalyanam for/

James R. Hall, Senior Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-528, STN 50-529,
and STN 50-530

Enclosures:

1. Amendment No. 183 to NPF-41
2. Amendment No. 183 to NPF-51
3. Amendment No. 183 to NPF-74
4. Safety Evaluation

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DATE	12/15/10	12/15/10	12/15/10	9/29/10	12/23/10	12/27/10	12/29/10

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