UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 20, 2010

Mr. Matthew W. Sunseri
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, KS 66839

SUBJECT: WOLF CREEK GENERATING STATION - ISSUANCE OF AMENDMENT RE: REVISE TECHNICAL SPECIFICATION 3.8.4, "DC SOURCES – OPERATING," SURVEILLANCE REQUIREMENTS 3.8.4.2 AND 3.8.4.5 (TAC NO. ME2965)

Dear Mr. Sunseri:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 192 to Renewed Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated December 16, 2009, as supplemented by letter dated August 26, 2010.

The amendment revises the battery acceptance criteria in TS 3.8.4, "DC [Direct Current] Sources – Operating," Surveillance Requirements (SRs) 3.8.4.2 and 3.8.4.5. Specifically, the amendment modifies SR 3.8.4.2 and SR 3.8.4.5 by providing limits for inter-cell, inter-tier/inter-bank/terminal, and field jumper connections for 60-cell, 59-cell, and 58-cell configurations.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

Balwant K Singal

Balwant K. Singal, Senior Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures:
1. Amendment No. 192 to NPF-42
2. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 192
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Wolf Creek Generating Station (the facility) Renewed Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated December 16, 2009, as supplemented by letter dated August 26, 2010, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-42 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 192, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and
Technical Specifications

Date of Issuance: December 20, 2010

ATTACHMENT TO LICENSE AMENDMENT NO. 192

RENEWED FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Replace the following pages of the Renewed Facility Operating License No. NPF-42 and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Renewed Facility Operating License

REMOVE	INSERT
4	4

Technical Specifications

REMOVE	INSERT
3.8-25	3.8-25
3.8-26	3.8-26

(5) The Operating Corporation, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(6) The Operating Corporation, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission, now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The Operating Corporation is authorized to operate the facility at reactor core power levels not in excess of 3565 megawatts thermal (100% power) in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 192, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Antitrust Conditions

Kansas Gas & Electric Company and Kansas City Power & Light Company shall comply with the antitrust conditions delineated in Appendix C to this license.

(4) Environmental Qualification (Section 3.11, SSER #4, Section 3.11, SSER #5)*

Deleted per Amendment No. 141.

*The parenthetical notation following the title of many license conditions denotes the section of the supporting Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE				FREQUENCY
SR 3.8.4.2	Verify no visible corrosion at battery terminals and connectors. OR Verify battery connection resistance is:			92 days
Connections	60 cells	59 cells	58 cells	
inter-cell	≤ 33 E-6 ohms	≤ 30 E-6 ohms	≤ 27 E-6 ohms	
inter-tier, inter-bank, terminal	≤ 150 E-6 ohms	≤ 150 E-6 ohms	≤ 150 E-6 ohms	
field jumper	NA	≤ 150 E-6 ohms	≤ 150 E-6 ohms	
SR 3.8.4.3	Verify battery cells, cell plates, and racks show no visual indication of physical damage or abnormal deterioration that could degrade battery performance.			18 months
SR 3.8.4.4	Remove visible terminal corrosion, verify battery cell to cell and terminal connections are clean and tight, and are coated with anti-corrosion material.			18 months
SR 3.8.4.5	Verify battery connection resistance is:			18 months
Connections	60 cells	59 cells	58 cells	
inter-cell	≤ 33 E-6 ohms	≤ 30 E-6 ohms	≤ 27 E-6 ohms	
inter-tier, inter-bank, terminal	≤ 150 E-6 ohms	≤ 150 E-6 ohms	≤ 150 E-6 ohms	
field jumper	NA	≤ 150 E-6 ohms	≤ 150 E-6 ohms	

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.8.4.6	Verify each battery charger supplies ≥ 300 amps at ≥ 128.4 V for ≥ 1 hour.	18 months
SR 3.8.4.7	----------NOTES---------- 1. The modified performance discharge test in SR 3.8.4.8 may be performed in lieu of the service test in SR 3.8.4.7. 2. This Surveillance shall not be performed in MODE 1, 2, 3, or 4. ---------- Verify battery capacity is adequate to supply, and maintain in OPERABLE status, the required emergency loads for the design duty cycle when subjected to a battery service test.	18 months

(continued)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 192 TO

RENEWED FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

1.0 INTRODUCTION

By letter dated December 16, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML093640042), as supplemented by letter dated August 26, 2010 (ADAMS Accession No. ML102450050), Wolf Creek Nuclear Operating Corporation (the licensee) requested an amendment to Renewed Facility Operating License No. NPF-42 for the Wolf Creek Generating Station (WCGS), and Appendix A, Technical Specifications (TSs), of the Renewed Facility Operating License. The proposed changes would revise the battery acceptance criteria in TS 3.8.4, "DC [Direct Current] Sources – Operating," Surveillance Requirements (SRs) 3.8.4.2 and 3.8.4.5. Specifically, the amendment would modify SR 3.8.4.2 and SR 3.8.4.5 by providing limits for inter-cell, inter-tier/inter-bank/terminal, and field jumper connections for 60-cell, 59-cell, and 58-cell configurations.

The supplemental letter dated August 26, 2010, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the U.S. Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on April 6, 2010 (75 FR 17448).

2.0 REGULATORY EVALUATION

General Design Criterion (GDC) 17, "Electric power systems," of Appendix A, "General Design Criteria for Nuclear Power Plants," Part 50 to Title 10 of the *Code of Federal Regulations* (10 CFR) requires, in part, that nuclear power plants have onsite and offsite electric power systems to permit the functioning of structures, systems, and components that are important to safety. The onsite electric power supplies, including the batteries, and the onsite electric distribution system, shall have sufficient independence, redundancy, and testability to perform their safety function, assuming a single failure. The offsite power system shall be supplied by two physically independent circuits that are designed and located so as to minimize, to the extent practical, the likelihood of their simultaneous failure under operating and postulated accident and environmental conditions. In addition, this criterion requires provisions to minimize

the probability of losing electric power from any of the remaining supplies as a result of, or coincident with, the loss of power generated by the nuclear power unit, the loss of power from the transmission network, or the loss of power from the onsite power supplies.

GDC-18, "Inspection and testing of electric power systems," requires that electric power systems that are important to safety shall be designed to permit appropriate periodic inspection and testing.

The regulatory requirement in 10 CFR 50.36, "Technical specifications," provides the content required in a licensee's TSs. Specifically, the regulations in 10 CFR 50.36(c)(3), "Surveillance requirements," requires that TSs include SRs, which are "requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met."

The regulations in 10 CFR 50.63, "Loss of all alternating current power," require that "each light-water-cooled nuclear power plant licensed to operate must be able to withstand for a specified duration and recover from a station blackout..."

3.0 TECHNICAL EVALUATION

3.1 Background

In its letter dated December 16, 2009, the licensee stated that,

> The station dc (direct current) electrical power system provides the ac (alternating current) emergency power system with control power. It also provides both motive and control power to selected safety related equipment and preferred ac vital bus power (via inverters). The dc electrical power system is designed to have sufficient independence, redundancy, and testability to perform its safety functions, assuming a single failure.
>
> The 125-V dc electrical power system consists of two independent and redundant Class 1E dc electrical power subsystems (Train A and Train B). Each dc electrical subsystem consists of two 125-V dc batteries, two battery chargers, one spare battery charger, and all the associated control equipment and interconnecting cabling. The Train A batteries are NK-11 and NK-13. The Train B batteries are NK-12 and NK-14. Batteries NK-11 and NK-14 are Lucent Technologies 1600AH low specific gravity round cell batteries. Batteries NK-12 and NK-13 are Lucent Technologies 865AH low specific gravity round cell batteries.
>
> During normal operation, the 125-V dc load is powered from the battery chargers with the batteries floating on the system. In case of loss of normal power to the battery charger, the dc load is automatically powered from the station batteries.

> The Train A and Train B dc electrical power subsystems provide the control power for associated Class 1E ac power load group, 4.16 kV switchgear, and 480-V load centers. The dc electrical power subsystems also provide dc electrical power to the inverters, which in turn power the ac vital buses. Each Class 1E dc distribution circuit is capable of transmitting sufficient energy to start and operate all the required loads in that circuit. Distribution circuits to redundant equipment are independent of each other. The distribution system is monitored to the extent that it is shown to be ready to perform its intended function. Each Class 1E battery has adequate storage capacity to carry out the required load continuously for at least the 240 minute duty cycle, with margin, as discussed in the Updated Safety Analysis Report (USAR), Section 8.3.2 (Reference 6.4 [of the licensee's letter dated December 16, 2009]).
>
> The Train A and Train B batteries are formed by strings of battery cells. These strings are comprised of a series connection of the positive and negative terminal posts of adjacent cells as shown in Figure 1 [of the licensee's letter dated December 16, 2009]. The inter-cell connections, inter-rack connections, inter-bank connections, and terminal connections contribute to the total battery connection resistance, which reduces the overall battery terminal voltage. During normal operation of the battery, corrosion can occur on the battery posts, which can also increase the connection resistances and further reduce battery terminal voltage.

In October 2007, the NRC staff completed a Component Design Basis Inspection at the WCGS. As documented in the NRC inspection report dated October 15, 2007 (ADAMS Accession No. ML072880678):

> The team identified a Green NCV [noncited violation] of 10 CFR, Part 50, Appendix B, Criterion III, "Design Control," for the failure of the licensee to ensure that the 125 Vdc safety-related batteries would remain operable if all the intercell and terminal connections were at the resistance value of 150 micro-ohms as allowed by Technical Specification Surveillance Requirement 3.8.4.5.

The licensee stated that it did not have a plant-specific basis for using 150 micro-ohms for connection resistance. Based on this finding, the licensee proposed a revision to the WCGS TS to provide assurance that the limits specified in the WCGS TS accurately reflect the operability limits of the WCGS Class 1E batteries.

In its letter dated December 16, 2009, the licensee stated that,

> The design function of the NK Battery connections (inter-cell, terminal, inter-tier, inter-bank, and field-jumper) is to facilitate operation of the safety-related NK batteries in order to meet their design basis function (i.e., capable of supporting their design bases loads).

3.2 Proposed Changes to TS SRs 3.8.4.2 and 3.8.4.5

Current TS SR 3.8.4.2 states,

> Verify no visible corrosion at battery terminals and connectors.
>
> OR
>
> Verify battery connection resistance is ≤150E-6 ohm for inter-cell connections and ≤150E-6 ohm for terminal connections.

The revised TS SR 3.8.4.2 would state,

> Verify no visible corrosion at battery terminals and connectors.
>
> OR
>
> Verify battery connection resistance is:

Connections	60 cells	59 cells	58 cells
inter-cell	≤33E-6 ohms	≤30E-6 ohms	≤27E-6 ohms
inter-tier, inter-bank, terminal	≤150E-6 ohms	≤150E-6 ohms	≤150E-6 ohms
field jumper	NA	≤150E-6 ohms	≤150E-6 ohms

Current TS SR 3.8.4.5 states,

> Verify battery connection resistance is ≤150E-6 ohm for inter-cell connections and ≤150E-6 ohm for terminal connections.

The revised TS SR 3.8.4.5 would state,

> Verify battery connection resistance is:

Connections	60 cells	59 cells	58 cells
inter-cell	≤33E-6 ohms	≤30E-6 ohms	≤27E-6 ohms
inter-tier, inter-bank, terminal	≤150E-6 ohms	≤150E-6 ohms	≤150E-6 ohms
field jumper	NA	≤150E-6 ohms	≤150E-6 ohms

The licensee initially proposed revising TS SR 3.8.4.2 to require verification of no visible corrosion at the battery terminals and connectors OR verification of battery connection resistance is ≤ 33 micro-ohms for inter-cell connections, ≤ 150 micro-ohms for inter-tier connections, ≤ 150 micro-ohms for inter-bank connection, and ≤ 150 micro-ohms for terminal connections on a 92-day specified Frequency. The licensee also initially proposed revising TS SR 3.8.4.5 to require verification of battery connection resistance is ≤ 33 micro-ohms for inter-cell connections, ≤ 150 micro-ohms for inter-tier connections, ≤ 150 micro-ohms for inter-bank connection, and ≤ 150 micro-ohms for terminal connections on an 18-month specified Frequency.

Similarly, the licensee proposed revising TS SR 3.8.4.5 to require verification of battery connection resistance is ≤ 33 micro-ohms for inter-cell connections for a 60-cell battery configuration, ≤ 30 micro-ohms for inter-cell connections for a 59-cell battery configuration, ≤ 27 micro-ohms for inter-cell connections for a 58-cell battery configuration, ≤ 150 micro-ohms for inter-tier connections, ≤ 150 micro-ohms for inter-bank connection, ≤ 150 micro-ohms for terminal connections, and ≤ 150 micro-ohms for a field jumper on an 18-month specified Frequency. The field jumper requirement is only applicable for 58- and 59-cell battery configurations.

The licensee stated in its letter dated December 16, 2009, that Calculation Change Notice 007 to Calculation No. NK-E-002, "Class 1E Battery Sizing," and a Basic Engineering Disposition provide the basis for the proposed resistance value limits for TS SRs 3.8.4.2 and 3.8.4.5. The NRC staff's review of these documents was limited to the information provided by the licensee in support of this license amendment request.

3.3 NRC Staff Evaluation

After observing that the proposed TS limits on battery resistance do not bound the "provisional" configurations that are identified in the licensee's Basic Engineering Disposition, the NRC staff requested in a request for additional information (RAI) letter dated July 30, 2010 (ADAMS Accession No. ML102080003), that the licensee explain how the proposed TS limits address permissible battery configurations. In its letter dated August 26, 2010, in response to the staff's RAI, the licensee proposed revising its original license amendment request to incorporate acceptance criteria for permissible battery configurations (i.e., 58-, 59-, and 60-cell battery configurations). The licensee specifically proposed revising TS SR 3.8.4.2 to require verification of no visible corrosion at the battery terminals and connectors OR verification of battery connection resistance is ≤ 33 micro-ohms for inter-cell connections for a 60-cell battery configuration, ≤ 30 micro-ohms for inter-cell connections for a 59-cell battery configuration, ≤ 27 micro-ohms for inter-cell connections for a 58-cell battery configuration, ≤ 150 micro-ohms for inter-tier connections, ≤ 150 micro-ohms for inter-bank connection, ≤ 150 micro-ohms for terminal connections, and ≤ 150 micro-ohms for a field jumper on a 92-day specified Frequency. The field jumper requirement is only applicable for 58- and 59-cell battery configurations.

In its letter dated July 30, 2010, the NRC staff also requested that the licensee provide a summary of the calculation that demonstrates that the proposed TS changes will result in the battery being able to provide the minimum required voltages to the downstream loads during the

worst-case scenario. In its letter dated August 26, 2010, the licensee noted that it utilized the results of its voltage drop calculation (NK-E-001-002, "Class 1E DC Voltage Drop") when preparing Calculation NK-E-002-004, "Class 1E Battery Sizing." The licensee stated that calculation NK-E-001-002 established that the design bases loads powered from the Class 1E DC Buses, which are fed from the safety-related batteries, have adequate/sufficient terminal voltages to operate under worst-case minimum voltage conditions associated with a station blackout event or a loss-of-offsite power concurrent with a loss-of-coolant accident event. The NRC staff did not review Calculation NK-E-001-002; however, in its letters dated December 16,2009, and August 26, 2010, the licensee stated that WCGS successfully passed the service test and modified performance discharge tests during the prior three refueling outages. Since the licensee considered the voltage drop calculation in the sizing of the Class 1E batteries, the staff concludes that successful completion of these tests demonstrates that the Class 1E batteries have sufficient capacity to perform their specified safety function which includes satisfying the minimum voltage requirements.

Based on the above, the NRC staff concluded that the WCGS Class 1E batteries will perform their intended design functions when connections are ≤ 33 micro-ohms for inter-cell connections for a 60-cell battery configuration, ≤ 30 micro-ohms for inter-cell connections for a 59-cell battery configuration, ≤ 27 micro-ohms for inter-cell connections for a 58-cell battery configuration, ≤ 150 micro-ohms for inter-tier connections, ≤ 150 micro-ohms for inter-bank connection, ≤ 150 micro-ohms for terminal connections, and ≤ 150 micro-ohms for a field jumper. The staff, therefore, concludes that the proposed changes are acceptable.

The NRC staff evaluated the licensee's request to modify TS SRs 3.8.4.2 and SR 3.8.4.5 by requiring verification of no visible corrosion at the battery terminals and connectors OR verification of battery connection resistance is ≤ 33 micro-ohms for inter-cell connections for a 60-cell battery configuration, ≤ 30 micro-ohms for inter-cell connections for a 59-cell battery configuration, ≤ 27 micro-ohms for inter-cell connections for a 58-cell battery configuration, ≤ 150 micro-ohms for inter-tier connections, ≤ 150 micro-ohms for inter-bank connection, ≤ 150 micro-ohms for terminal connections, and ≤ 150 micro-ohms for a field jumper. Based on the above, the NRC staff concludes that the proposed changes to the WCGS TS provides reasonable assurance of the continued availability of the required electrical power to shut down the reactor and to maintain the reactor in a safe condition after an anticipated operational occurrence or a postulated design-basis accident and, therefore, are acceptable. Furthermore, the NRC staff concludes that the proposed TS changes are in accordance with 10 CFR 50.36 and 10 CFR 50.63 and meet the intent of GDCs 17 and 18.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no

significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding published in the *Federal Register* on April 6, 2010 (75 FR 17448). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Matthew McConnell

Date: December 20, 2010

December 20, 2010

Mr. Matthew W. Sunseri
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, KS 66839

SUBJECT: WOLF CREEK GENERATING STATION - ISSUANCE OF AMENDMENT RE: REVISE TECHNICAL SPECIFICATION 3.8.4, "DC SOURCES – OPERATING," SURVEILLANCE REQUIREMENTS 3.8.4.2 AND 3.8.4.5 (TAC NO. ME2965)

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A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

Balwant K. Singal, Senior Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-482

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