



November 11, 2010

NRC 2010-0162
10 CFR 50.73

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Units 1 and 2
Dockets 50-266 and 50-301
Renewed License Nos. DPR-24 and DPR-27

Licensee Event Report 266/2010-003-00
Potential for Residual Heat Removal Trains to be Inoperable During Mode Change

Enclosed is Licensee Event Report (LER) 266/2010-003-00 for Point Beach Nuclear Plant (PBNP), Units 1 and 2. This LER documents the condition in which both units, when in MODE 4, have exceeded the fluid temperature in the residual heat removal (RHR) pump suction header which, under loss of coolant accident conditions, could have resulted in steam voiding and potentially damaging condensation-induced water hammer. The enclosed LER is submitted in accordance with 10 CFR 50.73(a)(2)(ii)(B) for being in an unanalyzed condition, 50.73(a)(2)(v)(B) as a condition that could potentially have prevented the fulfillment of a safety function, and 50.73(a)(2)(vii) where a single cause or condition caused two independent RHR trains to become inoperable. In accordance with 10 CFR 50.73(a) requirements, this LER is submitted within 60 days following discovery.

This submittal contains no new or revised regulatory commitments.

If you have questions or require additional information, please contact Mr. James Costedio at 920/755-7427.

Very truly yours,

NextEra Energy Point Beach, LLC

A handwritten signature in black ink, appearing to read "Larry Meyer".

Larry Meyer
Site Vice President

Enclosure

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
PSCW

NRC FORM 366 (10-2010)		U.S. NUCLEAR REGULATORY COMMISSION			APPROVED BY OMB: NO. 3150-0104 EXPIRES: 10/31/2013		Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resourse@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.																
LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)																							
1. FACILITY NAME Point Beach Nuclear Plant (PBNP) – Unit 1				2. DOCKET NUMBER 05000266		3. PAGE 1 of 4																	
4. TITLE Potential for Residual Heat Removal Trains to be Inoperable During Mode Change																							
5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER													
09	13	2010	2010	003	00	11	11	10	PBNP – Unit 2	05000301													
9. OPERATING MODE Unit 1 MODE 1 Unit 2 MODE 1									11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR§: (Check all that apply)			<input type="checkbox"/> 20.2201(b)			<input type="checkbox"/> 20.2203(a)(3)(i)			<input type="checkbox"/> 50.73(a)(2)(i)(C)			<input checked="" type="checkbox"/> 50.73(a)(2)(vii)		
												<input type="checkbox"/> 20.2201(d)			<input type="checkbox"/> 20.2203(a)(3)(ii)			<input type="checkbox"/> 50.73(a)(2)(ii)(A)			<input type="checkbox"/> 50.73(a)(2)(viii)(A)		
10. POWER LEVEL Unit 1 100% Unit 2 100%									<input type="checkbox"/> 20.2203(a)(1)			<input type="checkbox"/> 20.2203(a)(4)			<input checked="" type="checkbox"/> 50.73(a)(2)(ii)(B)			<input type="checkbox"/> 50.73(a)(2)(vii)(B)					
									<input type="checkbox"/> 20.2203(a)(2)(i)			<input type="checkbox"/> 50.36(c)(1)(i)(A)			<input type="checkbox"/> 50.73(a)(2)(iii)			<input type="checkbox"/> 50.73(a)(2)(ix)(A)					
<input type="checkbox"/> 20.2203(a)(2)(ii)			<input type="checkbox"/> 50.36(c)(1)(ii)(A)			<input type="checkbox"/> 50.73(a)(2)(iv)(A)			<input type="checkbox"/> 50.73(a)(2)(x)														
<input type="checkbox"/> 20.2203(a)(2)(iii)			<input type="checkbox"/> 50.36(c)(2)			<input type="checkbox"/> 50.73(a)(2)(v)(A)			<input type="checkbox"/> 73.71(a)(4)														
<input type="checkbox"/> 20.2203(a)(2)(iv)			<input type="checkbox"/> 50.46(a)(3)(ii)			<input checked="" type="checkbox"/> 50.73(a)(2)(v)(B)			<input type="checkbox"/> 73.71(a)(5)														
<input type="checkbox"/> 20.2203(a)(2)(v)			<input type="checkbox"/> 50.73(a)(2)(i)(A)			<input type="checkbox"/> 50.73(a)(2)(v)(C)			<input type="checkbox"/> OTHER														
<input type="checkbox"/> 20.2203(a)(2)(vi)			<input type="checkbox"/> 50.73(a)(2)(i)(B)			<input type="checkbox"/> 50.73(a)(2)(v)(D)			Specify in Abstract below or in NRC Form 366A														
12. LICENSEE CONTACT FOR THIS LER																							
NAME A. Moore									TELEPHONE NUMBER (Include Area Code) 920/755-7621														
13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT																							
CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX														
14. SUPPLEMENTAL REPORT EXPECTED <input type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE)									<input checked="" type="checkbox"/> NO														
									15. EXPECTED SUBMISSION DATE		MONTH	DAY	YEAR										
ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)																							
<p>During review of NRC Information Notice 2010-0011, a concern with the operation of the emergency core cooling system (ECCS) at shutdown conditions was identified. It was determined that in MODE 4, the temperature of the fluid in the residual heat removal (RHR) pump suction header exceeded the temperature which, under postulated shutdown loss of coolant accident (LOCA) conditions, could have resulted in steam voiding and potentially damaging condensation induced water hammer. This condition could have rendered the low head safety injection (LHSI) subsystem (which includes the RHR pump and common piping) for the required single train of ECCS inoperable.</p> <p>Operations procedure changes are in progress and are being tracked in the site's corrective action program to ensure that administrative controls are in place to maintain one train of RHR isolated and cool while in MODE 4 so that it can be used for LOCA mitigation.</p> <p>This 60-day licensee event report is being submitted in accordance with the requirements of 10 CFR 50.73(a)(2)(ii)(B), 50.73(a)(2)(v)(B), and 50.73(a)(2)(vii).</p>																							

**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
Point Beach Nuclear Plant – Unit 1	05000266	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	4 of 4
		2010	- 003	- 00	

NARRATIVE

Cause:

Site procedures did not adequately support all functions of ECCS operation in MODE 4 because technical reviews of previously received operating experience information had been less than adequate.

Corrective Action:

Operations procedure changes are in progress and are being tracked in the site's corrective action program to ensure that administrative controls are in place to maintain one train of RHR isolated and cool while in MODE 4 so that it can be used for LOCA mitigation.

Previous Occurrences: There have been no similar LERs submitted within the last three (3) years.

Failed Components Identified: None