



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

November 2, 2010

Mr. David A. Heacock  
President and Chief Nuclear Officer  
Virginia Electric and Power Company  
Innsbrook Technical Center  
5000 Dominion Boulevard  
Glen Allen, VA 23060-6711

SUBJECT: SURRY POWER STATION, UNIT NOS. 1 AND 2, CORRECTION TO  
AMENDMENTS REGARDING TECHNICAL SPECIFICATION REVISIONS  
RELATED TO THE MEASUREMENT UNCERTAINTY RECAPTURE (MUR)  
POWER UPRATE (TAC NOS. ME3293 AND ME3294)

Dear Mr. Heacock:

The U.S. Nuclear Regulatory Commission issued the Amendment No. 269 to Renewed Facility Operating License No. DPR-32 and Amendment No. 268 to Renewed Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively on September 24, 2010. The amendments changed the Technical Specifications in response to your application dated January 27, 2010<sup>1</sup>, as supplemented by letters dated February 4<sup>2</sup> and April 29<sup>3</sup>, 2010.

The amendments consisted of an approximate 1.6 percent MUR that increased the rated thermal power from 2546 megawatts thermal (MWt) to 2587 MWt.

The License Condition T, Item 16, for Surry Units 1 and 2, respectively, were incorrect. Enclosed are the corrected updated license pages for Amendment No. 269 to Renewed Facility Operating License No. DPR-32 and Amendment No. 268 to Renewed Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively.

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1 VEPCO letter to NRC, Agency Document Access Management System (ADAMS) Accession No. ML100320264  
2 VEPCO letter to NRC, ADAMS Accession No. ML100480781  
3 VEPCO letter to NRC, ADAMS Accession No. ML101200269

D. Heacock

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If you have any questions, please feel free to contact me at 301-415-1438.

Sincerely,

A handwritten signature in cursive script, appearing to read "Karen Cotton".

Karen Cotton, Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Enclosure: As stated

Docket Nos. 50-280 and 50-281

cc w/encl: Distribution via Listserv

T. (Continued)

<p>16. For the applicable UFSAR Chapter 14 events, Surry 1 will re-analyze the transient consistent with VEPCO's NRC-approved reload design methodology in VEP-FRD-42, Rev. 2.1-A.</p> <p>If NRC review is deemed necessary pursuant to the requirements of 10 CFR 50.59, the accident analyses will be submitted to the NRC for review prior to operation at the uprate power level. These commitments apply to the following Surry 1 UFSAR Chapter 14 DNBR analyses that were analyzed at 2546 MWt consistent with the Statistical DNBR Evaluation Methodology in VEP-NE-2-A:</p> <ul style="list-style-type: none"><li>• Section 14.2.7 - Excessive Heat Removal due to Feedwater System Malfunctions (Full Power Feedwater Temperature Reduction case only);</li><li>• Section 14.2.8 - Excessive Load Increase Incident;</li><li>• Section 14.2.9 - Loss of Reactor Coolant Flow; and</li><li>• Section 14.2.10 - Loss of External Electrical Load</li></ul>	<p>Prior to operating above 2546 MWt (98.4% RP).</p>
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4. This renewed license is effective as of the date of issuance and shall expire at midnight on May 25, 2032.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

Samuel J. Collins, Director  
Office of Nuclear Reactor Regulation

Attachment: Appendix A, Technical Specifications

Date of Issuance: March 20, 2003

Renewed License No. DPR-32  
Amendment No. 269

T. (Continued)

<p>16. For the applicable UFSAR Chapter 14 events, Surry 2 will re-analyze the transient consistent with VEPCO's NRC-approved reload design methodology in VEP-FRD-42, Rev. 2.1-A.</p> <p>If NRC review is deemed necessary pursuant to the requirements of 10 CFR 50.59, the accident analyses will be submitted to the NRC for review prior to operation at the uprate power level. These commitments apply to the following Surry 2 UFSAR Chapter 14 DNBR analyses that were analyzed at 2546 MWt consistent with the Statistical DNBR Evaluation Methodology in VEP-NE-2-A:</p> <ul style="list-style-type: none"><li>• Section 14.2.7 - Excessive Heat Removal due to Feedwater System Malfunctions (Full Power Feedwater Temperature Reduction case only);</li><li>• Section 14.2.8 - Excessive Load Increase Incident;</li><li>• Section 14.2.9 - Loss of Reactor Coolant Flow; and</li><li>• Section 14.2.10 - Loss of External Electrical Load</li></ul>	<p>Prior to operating above 2546 MWt (98.4% RP).</p>
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4. This renewed license is effective as of the date of issuance and shall expire at midnight on January 29, 2033.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:  
Samuel J. Collins, Director  
Office of Nuclear Reactor Regulation

Attachment: Appendix A, Technical Specifications

Date of Issuance: March 20, 2003

Renewed License No. DPR-37  
Amendment No. 268

D. Heacock

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If you have any questions, please feel free to contact me at 301-415-1438.

Sincerely,

*/RA/*

Karen Cotton, Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Enclosure: As stated

Docket Nos. 50-280 and 50-281

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