

7.0 INSTRUMENTATION AND CONTROLS

Nuclear power plant instrumentation senses various plant parameters and transmits appropriate signals to the control systems during normal operation and to the reactor trip and initiate engineered safety feature (ESF) systems during abnormal and accident conditions. The information in this chapter emphasizes those instruments and associated equipment that constitute the protection and safety systems. Every chapter in the Final Safety Analysis Report (FSAR) provides relevant information concerning the instrumentation and control (I&C) systems.

Sections 7.1 through 7.8 and Appendices 7A, 7B, and 7C of the South Texas Project (STP) Units 3 and 4 combined license (COL) FSAR incorporates by reference the corresponding sections in Chapter 7 of the certified Advanced Boiling-Water Reactor (ABWR) design control document (DCD), Revision 4, referenced in Title 10 of the *Code of Federal Regulations* (10 CFR) Part 52, Appendix A, with departures identified under each section. Tier 1 Departure STD DEP T1 3.4-1, "Safety Related I&C Architecture," requires prior U.S. Nuclear Regulatory Commission (NRC) approval and significantly impacts the content of this chapter. This departure describes the changes required to the I&C architecture and nomenclature to address obsolete data communication technology in the certified ABWR DCD. In addition, this departure addresses the changes resulting from the selection of digital I&C platforms. As allowed by the certified ABWR DCD, the applicant has elected to utilize the design acceptance criteria (DAC)/inspections, tests, analyses, and acceptance criteria (ITAAC) process for developing the detailed digital I&C design. Accordingly, the applicant has provided the I&C system design information along with the required DAC/ITAAC for developing the I&C system design details. The following site-specific COL FSAR sections supplement the I&C design information:

- Section 7.1S Site Specific Instrumentation and Control Platforms
- Section 7.6S Interlock Systems Important to Safety
- Section 7.8S Diverse Instrumentation and Control Systems
- Section 7.9S Data Communication Systems

The departures affecting the certified ABWR DCD FSAR Tier 2, Chapter 7 are evaluated under each section of this chapter. The I&C departures affecting the certified ABWR DCD FSAR Tier 1, Chapters 2 and 3 are evaluated in Safety Evaluation Report (SER) Sections 7.1.3 and 14.3S. By letter dated October 30, 2009, STP Nuclear Operating Company (STPNOC) submitted WCAP-17119-P, Revision 0, "Methodology for South Texas Project Units 3 and 4 – ABWR Technical Specification Setpoint". This report is evaluated in section 7.1.4 of this SER.

7.1 Introduction

This section of the FSAR identifies safety-related systems and related safety criteria.

The applicant incorporates by reference Section 7.1 of the certified ABWR DCD referenced in 10 CFR Part 52, Appendix A, including all subsections, tables, and figures. The applicant identifies the following departures from the certified design:

- STD DEP T1 2.3-1 Deletion of Main Steamline Isolation Valve (MSIV) Closure and Scram on High Radiation

- STD DEP T1 2.14-1 Hydrogen Recombiner Requirements Elimination (Table 7.1-1)
- STD DEP T1 3.4-1 Safety-Related I&C Architecture (Figures 7.1-1 and 7.1-2)
- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes (Table 7.1-2)
- STD DEP 7.1-1 Setpoints and Allowable Values
- STD DEP 7.1-2 Anticipated Transient Without Scram (ATWS) Design Bases (DB) for Startup Range Neutron Monitoring (SRNM)
- STD DEP 7.4-1 Alternate Rod Insertion (ARI)
- STD DEP Admin Administrative Departures (Table 7.1-1)

The acceptance criteria and regulatory basis in NUREG–0800, Chapter 7 are incorporated by reference into the DCD for the ABWR design and the FSER in NUREG–1503, “Final Safety Evaluation Report Related to the Certification of the Advanced Boiling Water Reactor.” Reviews of departures from the certified design are based on the current regulatory requirements, acceptance criteria, and guidelines in NUREG–0800 Table 7-1, Revision 5 (March 2007). In accordance with 10 CFR 50.55a(h), COL applications filed on or after May 13, 1999, which depart from the referenced certified design material (CDM), must meet the requirements for safety systems in the Institute of Electrical and Electronics Engineers (IEEE)-Standard (Std) 603–1991 and in the correction sheet dated January 30, 1995. In addition, applicable guidelines in Digital I&C (DI&C) Interim Staff Guidance (ISG) Documents DI&C-ISG-01, DI&C-ISG-02, DI&C-ISG-03, DI&C-ISG-04, and DI&C-ISG-05 are also used to evaluate the departures from the certified ABWR design.

7.1.1 Identification of Safety-Related Systems

7.1.1.1 Introduction

This FSAR section addresses I&C systems that are designated as either non-safety-related systems or safety-related systems, depending on their function. Some portions of a system may have a safety function, while other portions of the same system may be classified as non-safety related. The systems are classified into the following five categories: (1) reactor protection (trip) system (RPS), (2) ESF systems, (3) systems required for a safe shutdown, (4) safety-related display instrumentation, and (5) all other instrumentation systems required for safety.

7.1.1.2 Summary of Application

Section 7.1.1 of the STP Units 3 and 4 COL FSAR incorporates by reference Section 7.1.1 of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A. In addition, in FSAR Section 7.1.1, the applicant provides the following:

Tier 1 Departure Requiring Prior NRC Approval

- STD DEP T1 3.4-1 Safety-Related I&C Architecture
(Figure 7.1-1)

This departure describes changes to the I&C architecture and nomenclature that are needed to address obsolete data communication technology and the selection of DI&C platforms.

Tier 2 Departures Not Requiring Prior NRC Approval

- STD DEP 7.4-1 Alternate Rod Insertion

This departure provides a clear and concise understanding of the ARI function.

- STD DEP Admin

The administrative departures are defined as minor corrections, such as editorial or administrative errors in the referenced ABWR DCD (e.g., misspelled words, incorrect references, table headings, etc.). The applicant identifies one administrative departure in Subsection 7.1.1.3.9, "HVAC Emergency Cooling Water System."

7.1.1.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG-1503, "Final Safety Evaluation Report Related to the Certification of the Advanced Boiling-Water Reactor Design," (July 1994) (FSER related to the ABWR DCD).

In addition, the relevant requirements of the Commission regulations for identification of safety-related systems, and the associated acceptance criteria, are in Section 7.1 of NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants," the Standard Review Plan (SRP).

In accordance with Section VIII, "Processes for Changes and Departures," of, "Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor," the applicant identifies Tier 1 and Tier 2 departures. Tier 1 departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.A.4. Tier 2 departures not requiring prior NRC approval are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.5, which are similar to the requirements in 10 CFR 50.59.

Departures from the certified design are reviewed based on the current regulatory requirements, acceptance criteria, and guidelines in NUREG-0800 Section 7.1-T (Table 7-1), Revision 5 (March 2007).

In addition, in accordance with 10 CFR 50.55a(h), COL applications filed on or after May 13, 1999, which depart from the referenced CDM, must meet the requirements for safety systems in IEEE-Std 603-1991 and in the correction sheet dated January 30, 1995. In addition, applicable guidelines in DI&C-ISG-01, DI&C-ISG-02, DI&C-ISG-03, DI&C-ISG-04, and DI&C-ISG-05 are also used to evaluate departures from the certified ABWR design.

7.1.1.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Section 7.1.1 of the certified ABWR DCD. The staff reviewed Section 7.1.1 of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of the information in the COL FSAR and the information in the ABWR DCD appropriately represents the complete scope of information relating to this review topic.¹ The staff’s review confirmed that the information in the application and the information incorporated by reference address the required information relating to this section.

The staff reviewed the information in the COL FSAR:

Tier 1 Departure Requiring Prior NRC Approval

The following Tier 1 departure identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 1 departure.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

This departure is divided into the following five primary changes in the I&C architecture:

- Elimination of obsolete data communication technology
- Elimination of unnecessary inadvertent actuation prevention logic and equipment
- Clarifications of digital controls nomenclature and systems
- Final selection of platforms that changed the implementation architecture
- Identification of testing and surveillance changes

In COL application Part 7, “Departures Report,” Section 2.1, the applicant describes and evaluates this departure per the requirements in Section VII.A.4 of Appendix A to 10 CFR Part 52. Categories 1 and 3 of this departure are relevant to the changes in FSAR Subsection 7.1.1.1 and Figure 7.1-2. Changes in FSAR Subsection 7.1.1.1 are based on Departure STD DEP T1 3.4-1, which includes the elimination of obsolete data communication technology. As described in the Departure Report for STD DEP T1 3.4-1, proposed data communication functions are inherent in the proposed digital platforms (ESF logic and control system [ELCS], neutron monitoring system [NMS], reactor trip and isolation system [RTIS], plant information and control system [PICS], etc.) and therefore, they are separate and independent from each DI&C system and division within the systems. Also, as depicted in Figure 7C-1 “safety system logic and control (SSLC) Data Communications Paths for the ESF,” communication between the various SSLC units follows a point-to-point configuration. Whereas the certified ABWR design was based on a common data communication (multiplexer) system that was to be used by multiple DI&C systems, minimal changes made to the text (i.e., from the essential multiplexing system [EMS] to the ECF) in this section fail to clearly communicate the description of significantly different data communication features and technology. For instance, the stated description still refers to data highways for sensor input to the logic units (and other

¹ See “Finality of Referenced NRC Approvals” in SER Section 1.1.3, for a discussion on the staff’s review related to verification of the scope of information to be included in a COL application that references a design certification.

such units, etc.), which contradicts the point-to-point configuration setup of the proposed SSLC communication functions and features. With such significant changes to the data communication system, the applicant was requested in Request for Additional Information (RAI) 07.01-5 to provide a relevant and applicable description that is consistent with the proposed DI&C platforms.

In response to the NRC staff's **RAI 07.01-5** (South Texas Project Nuclear Operating Company [STPNOC] Letter U7-C-STP-NRC-090144, dated September 15, 2009), **RAI 07.01-9** (STPNOC Letter U7-C-STP-NRC-090118, dated August 27, 2009), and **RAI 07-6** (STPNOC Letter U7-C-STP-NRC-090232, dated December 30, 2009), the applicant updates FSAR Sections 7.1.1 and 7.1.1.1 and provides a new site-specific FSAR Section, 7.1S, which describes the safety-related DI&C platforms, including their data communications capabilities. The staff reviewed these responses and found that the proposed FSAR changes adequately capture the design information of the safety-related I&C platforms and associated essential communication functions (ECFs). The responses are therefore acceptable. For technical and regulatory compliance evaluations of the RTIS, ELCS, and ECF designs described in the COL application, including the new site-specific FSAR Section 7.1S, refer to Sections 7.2, 7.3, and 7.9S of this safety evaluation report (SER). Revision 3 of the STP Units 3 and 4 COL FSAR does not capture the proposed FSAR changes. Verification of these FSAR changes is being tracked as **Confirmatory Item 07.01-5**.

Based on Tier 1 Departure STD DEP T1 3.4-1, the applicant significantly modifies FSAR Figure 7.1-2, "Assignment of Interfacing Safety System Logic to SSLC Controllers," from ABWR DCD Tier 2 Figure 7.1-2. However, the applicant does not provide any explanations for the changes in the FSAR or in the Departure Report. In response to **RAI 07.01-6** (STPNOC Letter U7 C-STP-NRC-090118, dated August 27, 2009), the applicant has revised Tier 2 FSAR Figure 7.1-2, which is included in COL application Revision 3. The applicant also explains the changes in FSAR Figure 7.1-2. In addition, the applicant's response to **RAI 07-6** (STPNOC Letter U7-C-STP-NRC-090232, dated December 30, 2009) provides a supplemental design description of the SSLC platforms that corresponds to revised Figure 7.1-2. Except for editorial errors in the revised figure, the applicant's response is acceptable. The staff noted that the assignment of the interfacing safety system logic to SSLC controllers remains unchanged and is the same as in the ABWR DCD. Other than the nomenclature changes, one significant change in FSAR Figure 7.1-2 eliminates the redundant safety logic functions (SLFs) within a division. For the technical evaluation of the RTIS, ELCS (specifically, elimination of unnecessary inadvertent actuation prevention logic and equipment), and ECF see SER Sections 7.2, 7.3, and 7.9S. The confirmation of editorial corrections in FSAR Figure 7.1-2 is being tracked as **Confirmatory Item 07.01-6**. The evaluation of RAI 07.01-9 response is provided in section 7.1.2 of this SER.

Tier 2 Departures Not Requiring Prior NRC Approval

The following Tier 2 Departure/s Not Requiring Prior NRC Approval identified by the applicant in this section may also be evaluated in other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this/these departure/s.

- STD DEP 7.4-1

Alternate Rod Insertion

The applicant has revised FSAR Subsection 7.1.1.4.1, "Alternate Rod Insertion Function (ARI)," based on Tier 2 Departure STD DEP 7.4-1, which does not require prior NRC approval. This FSAR change clarifies the ARI features and functions.

The applicant's evaluation determined that this departure does not require prior NRC approval, in accordance with 10 CFR Part 52, Appendix A, Section VIII.B.5. NRC staff evaluated the changes to FSAR Subsection 7.1.1.4.1 and found that the applicant has adequately addressed this departure. Within the review scope of this section, the staff found it reasonable that this departure does not require prior NRC approval. For additional discussions of this departure, see SER Section 7.4.

- STD DEP Admin

In FSAR Subsection 7.1.1.3.9, the applicant has changed the supply of heating ventilation and air conditioning (HVAC) emergency cooling water from "diesel generator cooling coils" to "reactor building essential electrical equipment rooms," based on an administrative departure (STD DEP Admin). In the COL application Departure Report (Part 7 of COL application), administrative departures are defined as minor corrections, such as editorial or administrative errors in the referenced ABWR DCD (e.g., misspelled words, incorrect references, table headings, etc.). The applicant's evaluation determined that this departure does not require prior NRC approval in accordance with 10 CFR Part 52, Appendix A, Section VIII.B.5.

NRC staff reviewed the information and concluded that the changes to Subsection 7.1.1.3.9 are beyond administrative in nature and issued **RAI 07.01-7**. The applicant's response to **RAI 07.01-7** (STPNOC Letter U7-C-STP-NRC-090118, dated August 27, 2009) justifies this administrative change on the bases of consistency with the design basis for the HVAC emergency cooling water described in Tier 2 of FSAR Subsection 9.2.13.1.2(1) and Figure 9.4-4, "R/B Safety-Related Electrical Equipment HVAC System." Because diesel generators are located in the reactor building, this administrative departure ensures consistency with other parts of the FSAR. Therefore, the applicant's response is acceptable. Within the review scope of this section, the staff found it reasonable that this departure does not require prior NRC approval.

7.1.1.5 Post Combined License Activities

There are no post COL activities related to this section.

7.1.1.6 Conclusion

The NRC staff's finding related to information incorporated by reference is in NUREG-1503. The staff reviewed the application and checked the referenced DCD. The staff's review confirmed that the applicant has addressed the required information relating to the identification of safety-related systems. With the exception of **Confirmatory Items 7.1-5 and 7.1-6**, no outstanding information is expected to be addressed in the COL FSAR related to this section. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the identification of safety-related systems that were incorporated by reference have been resolved.

The staff's review confirmed that the applicant has adequately addressed the Tier 1 departures in accordance with Section 7.1 of NUREG-0800. The staff found it reasonable that the identified Tier 2 departures are characterized as not requiring prior NRC approval per 10 CFR Part 52, Appendix A, Section VIII.B.5. However, as a result of **Confirmatory Items 7.1-5 and 7.1-6**, the staff was unable to finalize the conclusions relating to the identification of safety-related systems, in accordance with the NRC requirements.

7.1.2 Identification of Safety Criteria

7.1.2.1 Introduction

This section of the FSAR addresses design bases and criteria for I&C equipment design based on the need for each system to perform its intended function, while meeting the requirements of applicable general design criteria, regulatory guides (RGs), industry standards, and other documents.

The safety-design basis for a safety system states, in functional terms, the unique design requirements that establish the limits within which the safety objectives shall be met. The general functional requirement portion of the safety-design basis presents those requirements determined to be sufficient for ensuring the adequacy and reliability of the system from a safety perspective. Many of these requirements have been incorporated into various codes, criteria, and regulatory requirements.

7.1.2.2 Summary of Application

Section 7.1.2 of the STP Units 3 and 4 COL FSAR incorporates by reference Section 7.1.2 of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A, with the following departures:

Tier 1 Departures

- STD DEP T1 2.3-1 Deletion of MSIV Closure and Scram on High Radiation

This departure deletes the scram and the MSIV automatic closure on high main steam line radiation.

- STD DEP T1 2.14-1 Hydrogen Recombiner Requirements Elimination (Table 7.1-1)

This departure eliminates the requirements for hydrogen control systems to mitigate a design-basis loss-of-coolant accident (LOCA) hydrogen release.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture (Figures 7.1-1, 7.1-2)

This departure describes changes to the I&C architecture and nomenclature that are needed to address obsolete data communication technology and the selection of digital I&C platforms.

Tier 2* Departure

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes (Table 7.1-2)

This departure identifies Tier 2* items in Tier 2 FSAR Tables 1.8-20 and 1.8-21, which are being updated to current revisions/editions.

Tier 2 Departures Not Requiring Prior NRC Approval

- STD DEP 7.1-1 Reference Setpoints and Allowable Values

The purpose of this departure is to clarify in the FSAR that wherever the Technical Specifications (TS) are referenced for setpoints or margins, the correct reference is to the methods for calculating setpoints and margins as described in the TS Bases.

- STD DEP 7.1-2 ATWS DB for Startup Range Neutron Monitoring

This departure clarifies the understanding of the ARI functional requirements and safety-design basis for the SRNM and the average power range monitor (APRM), which provide ATWS permissive signals to the ELCS.

- STD DEP Admin Administrative Departures (Table 7.1-1)

The administrative departures are defined as minor corrections, such as editorial or administrative errors, in the referenced ABWR DCD (e.g., misspelled words, incorrect references, table headings, etc.). The applicant identifies one administrative departure in Subsection 7.1.2.8, "Independence of Safety-Related Systems," which corrects a referenced subsection number.

7.1.2.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG-1503. In addition, the relevant requirements of the Commission regulations for the identification of safety criteria, and the associated acceptance criteria, are in Section 7.1 of NUREG-0800.

In accordance with Section VIII, "Processes for Changes and Departures," of "Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor," the applicant identifies Tier 1, Tier 2*, and Tier 2 departures. Tier 1 departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.A.4. Tier 2* departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.6. Tier 2 departures not requiring prior NRC approval are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.5, which are similar to the requirements in 10 CFR 50.59.

The design departures are satisfied based on meeting the current regulatory requirements, acceptance criteria, and guidelines in NUREG-0800 Table 7-1, Revision 5 (March 2007).

In accordance with 10 CFR 50.55a(h), COL applications filed on or after May 13, 1999, which depart from the referenced CDM, must meet the requirements for safety systems in IEEE Std 603-1991 and in the correction sheet dated January 30, 1995. In addition, applicable

guidelines in Digital I&C-ISG Documents DI&C-ISG-01, DI&C-ISG-02, DI&C-ISG-03, DI&C-ISG-04, and DI&C-ISG-05 are also used to evaluate the departures from the certified ABWR design.

7.1.2.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Section 7.1.2 of the certified ABWR DCD. The staff reviewed Section 7.1.2 of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of the information in the COL FSAR and the information in the ABWR DCD appropriately represents the complete scope of information relating to this review topic.¹ The staff’s review confirmed that the information in the application and the information incorporated by reference address the required information relating to this section.

The staff reviewed the information in the COL FSAR:

Tier 1 Departures

The following Tier 1 departures identified by the applicant in this section require prior NRC approval and the full scope of their technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these Tier 1 departures.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

This departure is divided into the following five primary changes in the I&C architecture:

- Elimination of obsolete data communication technology
- Elimination of unnecessary inadvertent actuation prevention logic and equipment
- Clarifications of digital controls nomenclature and systems
- Final selection of platforms changed the implementation architecture
- Testing and surveillance changes

The applicant has revised FSAR Subsection 7.1.2.1.6, “Protection System Inservice Testability,” based on Tier 1 Departure STD DEP T1 3.4-1. This departure requires prior NRC approval. Also per Section 7.1.1.2 of the ABWR DCD, any changes to Section 7.1.2.16 require prior NRC approval.

In the first paragraph of FSAR Subsection 7.1.2.1.6, the applicant changed “ESF” to “ELCS.” This minor change raises a number of questions (see below), and NRC staff issued

RAI 07.01-9:

- What is ELCS?
- Is it ESF Logic and Control System? (This term has not been defined in previous subsections of FSAR Section 7.1.)

¹ See “Finality of Referenced NRC Approvals” in SER Section 1.1.3, for a discussion on the staff’s review related to verification of the scope of information to be included in a COL application that references a design certification.

- What about NMS (also a part of the SSLC)? The applicant should describe the various digital I&C platforms (ELCS, NMS, RTIS, etc.) that comprise the SSLC in FSAR Section 7.1.1.1, which would allow the use of these digital I&C platforms in subsequent FSAR sections.

The applicant's response to **RAI 07.01-9** (STPNOC Letter U7-C-STP-NRC-090118, dated August 27, 2009) updates Tier 2 FSAR Section 7.1.1, which refers to Section 7.1S for terminology related to the RTIS, NMS, and ELCS. The applicant's response to RAI 07-6 (STPNOC Letter U7-C-STP-NRC-090232, dated December 30, 2009) provides a new site-specific FSAR Section 7.1S that describes the safety-related DI&C platforms. NRC staff found that these FSAR updates provide an adequate description of the SSLC platforms and related terminology. Therefore, **RAI 07.01-9** is resolved. Confirmation of these FSAR updates is being tracked as **Confirmatory Item 07.01-9**.

In FSAR Subsection 7.1.2.1.6(4), the applicant deleted a sentence that states, "The test signals are adjustable manually from the control room and also are capable of performing an automatic sequence of events." Why is the deletion of this feature acceptable? This change is based on Tier 1 Departure STD DEP T1 3.4-1. However, the "Departures Report" (Part 7) fails to provide any justification for removing this testing capability. The applicant's response to **RAI 07.01-10** (STPNOC Letter U7-C-STP-NRC-090118, dated August 27, 2009) provides an acceptable alternate method for accomplishing the calibration checks. Due to final selections of the SSLC platforms, the test signal injection and adjustments will occur at the cabinet that contains the analog to digital conversion equipment. The staff found the proposed alternate method of accomplishing the calibration checks of analog sensor inputs acceptable, because the objectives of the fourth test described in the ABWR DCD are fully met.

In Subsection 7.1.2.1.6(6)(a), "Online Continuous Testing," the applicant deleted the automatic system self-testing features that comply with the criteria for periodic surveillance testing, in accordance with IEEE-Std 338. Why is the deletion of this automatic system self-testing feature acceptable? This change is based on Tier 1 Departure STD DEP T1 3.4-1. However, the "Departures Report" (Part 7) fails to provide any justification for removing this self-testing feature. The applicant's response to **RAI 07.01-11** (STPNOC Letter U7-C-STP-NRC-090118, dated August 27, 2009) provides an adequate explanation for the changes to Subsection 7.1.2.1.6(6)(a), which resulted from the final selections of the SSLC platforms. Online continuous testing and self-diagnostic features described in the ABWR DCD are being employed in the STP 3 and 4 SSLC design, with slight variations to suit the selected SSLC platforms. The departed methods for performing online continuous testing meet the IEEE-Std 338-1987 testing requirements and the applicable acceptance criteria in Branch Technical Position (BTP) 7-17 of NUREG-0800. Therefore, NRC staff found the departed methods for performing online continuous testing acceptable. Therefore, **RAI 07.01-11** is resolved.

Subsection 7.1.2.1.6(6)(b) introduces a new term, "maintenance and test processor (MTP)," which is intended to test the functional logic of the ELCS. The proposed MTP replaces the surveillance test controller (STC) that is a part of the certified ABWR design. The STC was intended to test the SSLC functional logic that includes the NMS and RTIS platforms. This proposed change under Tier 1 Departure STD DEP T1 3.4-1 raises the following questions: Is the MTP integral to the ELCS platform? If not, is it permanently connected to the ELCS platform? Because the MTP is only intended to test the ELCS functional logic, how will the

related divisional subsystems.” The applicant’s response to **RAI 07.01-13** (STPNOC Letter U7-C-STP-NRC-090118 dated August 27, 2009) updates Tier 2 FSAR Subsection 7.1.2.6.6 to correctly identify the safety-related and nonsafety-related design bases for the containment radiation monitoring system. Hence, the design bases for the containment radiation monitoring system remain unchanged and are same as in the ABWR DCD. The changes to the design bases for the containment hydrogen and oxygen monitoring systems are consistent with those identified in STP Units 3 and 4 Tier 2 FSAR, Chapter 6. Therefore, the staff found the changes to Tier 2 FSAR Subsection 7.1.2.6.6 acceptable. Confirmation of these FSAR changes is being tracked as **Confirmatory Item 07.01-13**.

Tier 2* Departure Requiring Prior NRC Approval

The following Tier 2* departures identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 2* departure.

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

Based on Tier 2* Departure STD DEP 1.8-1, the applicant revised FSAR Subsections 7.1.2.9.1 and 7.1.2.11.1 and Table 7.1-2. Additional changes to FSAR Sections 1.8, 1.9S, and 7.1 resulting from this departure were submitted in STPNOC Letter U7-C-STP-NRC-090009, dated February 9, 2009, and in response to RAI 07.01-14 (STPNOC Letter U7-C-STP-NRC-090162, dated September 24, 2009). In general, the applicant updated the conformance to regulatory requirements, RGs, and industry standards to current revisions and editions. Specific to the safety-related I&C systems (e.g., ELCS, NMS, and RTIS), the applicant is committed to apply IEEE-Std 603–1991 as required by 10 CFR 50.55a(h), current revisions of applicable RGs, industry standards, and NUREG–0800, and the BTPs. The staff evaluated the changes to FSAR Subsections 7.1.2.9.1 and 7.1.2.11.1 and Table 7.1-2. The staff found that the applicant has adequately addressed parts of this departure that are applicable to the I&C systems. Subsequently, the applicant submitted a revised response to RAI 07.01-14, Revision 1 (STPNOC Letter U7-C-STP-NRC-100065, dated April 19, 2010), which relocated information regarding the conformance to the RGs and industry codes and standards that are applicable to I&C platform departures. Information that was previously proposed to be included in FSAR Tables 1.9S-1 and 1.9S-1a is now being appropriately included in FSAR Tables 1.8-20 and 1.8-21. In revised response to RAI 07.01-14, Revision 1, the applicant stated that for ELCS the requirements that the Common Q platform were licensed to are submitted as an alternate to current requirements based on the original NRC review and SERs. Note that the Common Q topical report was evaluated and accepted (*not licensed*) by the NRC for use in safety-related applications within the limitations stated in the SER. Also note that NRC letter (to Mr. Philip Richardson of Westinghouse, dated August 11, 2000) transmitting the Common Q SER states, “Should our criteria or regulations change so that our conclusions as to the acceptability of the report are invalidated, CE Nuclear Power and/or the applicants referencing the topical report will be expected to revise and resubmit their respective documentation, or submit justification for the continued applicability of the topical report without revision of their respective documentation.” The applicant was asked to evaluate the impact of current RGs and IEEE standards on the Common Q topical report. The applicant’s responded by submitting RAI 07.01-14, Revision 2 (STPNOC Letter U7-C-STP-NRC-100214, dated September 23, 2010), which replaces previous responses on the subject in their entirety. In this RAI response, the applicant commits to current

7.1.2.6 Conclusion

The NRC staff's finding related to information incorporated by reference is in NUREG–1503. The staff reviewed the application and checked the referenced DCD. The staff's review confirmed that the applicant has addressed the required information relating to the identification of safety criteria. With the exception of **Confirmatory Items 7.1-8, 7.1-9, 7.1-13, and 7.1-14**, no outstanding information is expected to be addressed in the COL FSAR related to this section. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the identification of safety criteria that were incorporated by reference have been resolved.

The staff's review confirmed that the applicant has adequately addressed the Tier 1 and Tier 2* departures in accordance with Section 7.1 of NUREG–0800. The staff found it reasonable that the identified Tier 2 departures are characterized as not requiring prior NRC approval per 10 CFR Part 52, Appendix A, Section VIII.B.5.

The staff compared the application to the relevant NRC regulations; acceptance criteria defined in NUREG–0800 Section 7.1; and other NRC RGs and concluded that the applicant is in compliance with NRC regulations. However, as a result of **Confirmatory Items 7.1-8, 7.1-9, 7.1-13, and 7.1-14**, the staff was unable to finalize the conclusions relating to the identification of safety-related systems, in accordance with the NRC requirements.

7.1.3 Control and Instrumentation Systems – Tier 1 Design Descriptions (Parts of STP Units 3 and 4 Tier 1 FSAR Chapters 2 and 3) (There is no related section in Regulatory Guide 1.206)

7.1.3.1 Introduction

Tier 1 FSAR Chapters 2 and 3 of the STP Units 3 and 4 COL application discuss inspections, tests, and analyses, including those applicable to controls and instrumentation systems that the applicant proposes to perform, as well as the acceptance criteria that are necessary and sufficient to provide reasonable assurance that if the proposed inspections, tests, and analyses are performed and the acceptance criteria are met, the facility has been constructed and will operate in conformance with the COL, the provisions of the Atomic Energy Act, and NRC regulations. The COL applicant provides the proposed selection methodology and criteria for establishing the ITAAC that are necessary and sufficient to provide that reasonable assurance. Each Tier 1 FSAR subsection consists of two parts, namely the “Design Description” and the “Inspections, Tests, Analyses and Acceptance Criteria.” This SER section evaluates the control and instrumentation systems related the design descriptions in the STP Units 3 and 4 COL application, Tier 1 FSAR Chapters 2 and 3. The evaluation of the ITAAC applicable to the control and instrumentation systems is in SER Section 14.3S.

7.1.3.2 Summary of Application

Chapters 2 and 3 of the STP Units 3 and 4 COL Tier 1 FSAR incorporate by reference Tier 1 Chapters 2 and 3 of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A, with the following departures applicable to the control and instrumentation systems related to the design descriptions:

Tier 1 Departures

- STD DEP T1 2.2-1 Control Systems Changes to Inputs, Tests, and Hardware

This departure modifies the ITAAC acceptance criteria based on the final rod control and information system (RCIS) design implementation, where the power supply associated with the one non-Class 1E uninterruptible power supply being tested will become inoperable, and both dual-redundant controller channels will remain operational when conducting this test.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

This departure describes changes to the I&C architecture and nomenclature that are needed to address obsolete data communication technology and the selection of DI&C platforms.

7.1.3.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG–1503. The relevant requirements of the Commission regulations for the “Control and Instrumentation Systems – Tier 1 Design Descriptions,” and the associated acceptance criteria, are in Section 14.3 of NUREG-0800.

In accordance with Section VIII, “Processes for Changes and Departures,” of, “Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor,” the applicant identifies Tier 1 departures. Tier 1 departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.A.4.

The regulatory requirements for reviewing the Tier 1 departures are in NUREG–0800, Section 14.3.

7.1.3.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Tier 1 information of the certified ABWR DCD. The staff reviewed Chapters 2 and 3 of the STP Units 3 and 4 COL Tier 1 FSAR and checked the referenced ABWR DCD to ensure that the combination of the information in the COL FSAR and the information in the ABWR DCD appropriately represents the complete scope of information relating to this review topic.¹ The staff’s review confirmed that the information in the application and the information incorporated by reference address the required information relating to this section.

The staff reviewed the information in the COL FSAR:

Tier 1 Departures

The following Tier 1 departures identified by the applicant in this section require prior NRC approval and the full scope of their technical impact may be evaluated in the other sections of

¹ See “Finality of Referenced NRC Approvals” in SER Section 1.1.3, for a discussion on the staff’s review related to verification of the scope of information to be included in a COL application that references a design certification.

this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these Tier 1 departures.

- STD DEP T1 2.2-1 Control Systems Changes to Inputs, Tests, and Hardware
- STD DEP T1 3.4-1 Safety-Related I&C Architecture

NRC staff reviewed the proposed changes to the I&C systems as summarized below:

Tier 1 Section 2.2. Control and Instrument Systems

The applicant has annotated Tier 1 Section 2.2.7 with Departure STD DEP T1 2.2-1. However, NRC staff was unable to find any changes in this section resulting from this departure. The applicant's response to **RAI 07-1** (STPNOC Letter U7-C-STP-NRC-090157, dated September 22, 2009) states that STD DEP T1 2.2-1 does not change COL application Tier 1 Section 2.2.7, and the departure number referenced in Section 2.2.7 will be deleted. This action represents no change to this Tier 1 section; NRC staff found that the action incorporates by reference relevant subsections of the certified ABWR DCD. Confirmation of this FSAR change is being tracked as **Confirmatory Item 07-1**.

The applicant has revised Sections 2.2.3, 2.2.5, 2.2.7, 2.2.9, 2.2.11, and Figures 2.2.1, 2.2.5, and 2.2.7b based on Tier 1 Departure STD DEP T1 3.4-1.

As stated above, Departure STD DEP T1 3.4-1 changes the ABWR control and instrument systems architecture. The evaluation of specific changes in Tier 1 Section 2.2 is discussed below:

- (1) Section 2.2.5, Neutron Monitoring System
- (2) Section 2.2.11, Process Computer System
- (3) Figure 2.2.1, Rod Control and Information System Control Interface Diagram
- (4) Figure 2.2.5, Neutron Monitoring System
 - In the above stated subsections and figures, the applicant replaces “Process Computer System (PCS)” with “Plant Computer Functions (PCF)” and modifies related text. As described in COL application Tier 1 Section 2.2.11 in the proposed I&C architecture, all of the process computer system functions are now performed within the PICS, thereby eliminating the need for a dedicated PCS. Similar to the PCS, the PCF are classified as non-safety related. NRC staff concluded that the proposed change, which replaces ‘PCS with PCF, does not change the PCS design bases in the ABWR DCD and will not decrease the level of safety. Therefore, the staff found these changes acceptable.
 - However, the PCF design description in Tier 1 Section 2.2.11 is inconsistent with the system description in Tier 2 Section 7.7.1.5. According to 10 CFR Part 52, Appendix A, Tier 1 information such as design descriptions, interface requirements, and site parameters are derived from Tier 2. The applicant's response to **RAI 07-2** (STPNOC Letter U7-C-STP-NRC-090157, dated September 22, 2009) states that details will be added to COL application Tier 2 Section 7.7.1.5, “Plant Computer Functions (PCF) – Instrumentation and Controls,” to resolve the inconsistencies between this section and COL

application FSAR Tier 1, Section 2.2.11. The following paragraphs will be added to COL application Tier 2 Subsection 7.7.1.5(1), "System Identification":

The Plant Computer Functions (PCF) are a set of control, monitoring, and data calculation functions that are implemented on digital central processing units and associated peripheral equipment provided by the Plant Information and Control System (PICS). Redundant processors are used for functions that are important to plant operation. The PCF are classified as nonsafety related.

The PCF perform local power range monitor (LPRM) calibrations and calculations of fuel operating thermal limits data, which is provided to the automated thermal limit monitor (ATLM) function of the Rod Control & Information System (RCIS) for the purpose of updating rod block setpoints.

The PCF also include top-level controller functions that monitor the overall plant conditions, issue control commands and adjust setpoints of lower level controllers to support automation of normal plant startup, shutdown, and power range operations. In the event that abnormal conditions develop in the plant during operations in the automatic mode, these functions automatically revert to the manual mode of operation.

The above stated change to FSAR Tier 2 Subsection 7.7.1.5 addresses the staff's concerns of inconsistencies between Tier 1 and Tier 2 content. The suggested FSAR revision does not change the PCS design bases in the ABWR DCD and does not decrease the level of safety. Therefore, the staff found the proposed FSAR change acceptable. Confirmation of this FSAR change is being tracked as **Confirmatory Item 07-2**.

Tier 1 Section 2.2.3, Feedwater Control System and Tier 1 Section 2.2.9, Automatic Power Regulator System

In the sections identified above, the applicant has removed or replaced "Non Essential Multiplexing System (NEMS)" with "Non Essential Data Communication Function (NECF)" and modified related text. As described in Tier 1 Section 2.7.5 in the proposed I&C architecture, all of the non-essential communication functions are now performed through a plant-wide distributed network identified as the plant data network (PDN) system, which supports the communication functions of the non-safety-related I&C systems and other plant data and information systems. Similar to the NEMS, the equipment implementing the NECF is classified as non-safety related. NRC staff concluded that the proposed change, which replaces 'NEMS' with 'NECF,' does not decrease the level of safety. Therefore, the staff found these changes acceptable. Refer to Subsection 7.9S of this SER for an in-depth safety evaluation of the data communication system.

Tier 1 Section 2.2.7, Reactor Protection System

- In Section 2.2.7 and Figure 2.2.7b, "Reactor Protection System," the applicant has revised the reactor protection system description in accordance with the proposed I&C architecture. Most of the changes are associated with the nomenclature consistent with the proposed I&C architecture and do not adversely impact the RPS design bases in the ABWR DCD. These nomenclature changes do not decrease the level of safety. The staff issued **RAI 07-3** requesting the applicant to provide additional information in the following two areas; a description of interfaces between (1) the RPS and MSIV closure signals and (2) the RPS and suppression pool temperature trip was deleted from Tier 1 Section 2.2.7, with no justification provided in the Departure Report. The applicant's response to **RAI 07-3** (STPNOC Letter U7-C-STP-NRC-090157, dated September 22, 2009) provides the following explanation for the stated changes in the DCD:

Changes in the Design Description provided in STP Units 3 and 4 COL application Tier 1 Section 2.2.7 reflect the new Non-Rewriteable Field Programmable Gate Array (FPGA)-based platforms. The interfaces between 1) the Reactor Protection System (RPS) and Main Steam Isolation Valve (MSIV) closure signals; and 2) RPS and Suppression Pool Temperature trip were not **deleted** in STD DEP T1 3.4-1, although some text dealing with those interfaces was deleted from paragraph 4 for clarity. Both of those interfaces are shown on Figure 2.2.7a as noted in paragraph 1 and further detailed below.

The interface between the RPS and MSIV closure signals is listed as "(7) Main Steamline Isolation [NBS]" in the text.

The interface between the RPS and Suppression Pool Temperature trip is listed as "(9) High Suppression Pool Average Temperature [SPTM]" in the text.

Because these proposed changes in the COL FSAR application to the DCD do not change the RPS logic described in the ABWR DCD, the staff found them acceptable.

A new paragraph added to the design description for the RPS is inconsistent with the changes described in the Departure Report: "The OLU and LDs are implemented with non-microprocessor-based equipment. The remaining RPS functions are primarily implemented with microprocessor-based equipment configurable logic devices." The staff issued **RAI 07-4** requesting the applicant to address this inconsistency. The applicant's response to **RAI 07-4** (STPNOC Letter U7-C-STP-NRC-090157, dated September 22, 2009) provides the following corrections to FSAR Tier 1 Section 2.2.7:

The OLU and LDs are implemented with non-microprocessor-based equipment. The remaining RPS functions are primarily implemented with ~~microprocessor-based equipment~~ configurable logic devices.

NRC staff found that these FSAR corrections resolve the inconsistencies in the COL application. Therefore, these changes are acceptable. Refer to Section 7.2 of this SER for a detailed safety evaluation of the RPS platform. Confirmation of this FSAR change is being tracked as **Confirmatory Item 07-4**.

Tier 1 Section 2.7, Control Panels

The applicant has revised Section 2.7.5 based on Tier 1 Departure STD DEP T1 3.4-1.

As stated above, Departure STD DEP T1 3.4-1 addresses changes to the ABWR control and instrument systems architecture. The evaluation of specific changes in Tier 1 Section 2.7.5 is discussed below.

Based on the proposed I&C system architecture, the applicant has renamed Section 2.7.5 from “Multiplexing System” to “Data Communication.” The data communication design description was replaced in its entirety to explain the significantly different ways of essential and non-essential data communication associated with the proposed I&C architecture. The design description for the ECF states, “The ECFs are implemented through the use of divisionally dedicated networks and/or data links provided with the safety-related digital system platforms. Some of the platforms use data links only or networks only and some of the platforms use a combination of both data links and networks.” The staff issued **RAI 07-5** requesting the applicant to address the following concerns. Whereas in the subsequent paragraph, it is stated, “Data communication is provided between redundant safety related divisions to support coincident logic functions. The data communication is implemented through fiber optic based data links to ensure interdivisional isolation.” How does the interdivisional communication take place for the platforms that do not use data links? The applicant should define the terms “data link” and “network” and explain the differences.

The applicant’s response to **RAI 07-5** (STPNOC Letter U7-C-STP-NRC-090162, dated September 24, 2009) provides the following explanation for the stated changes in the DCD:

Both the RTIS and ESF Control and Logic System (ELCS) utilize data links for interdivision communication. The ELCS utilizes a network within a division, but that network does not cross division boundaries.

The RTIS utilizes only data links. A data link is defined as having a point to point communication connection between the sending unit and the receiving unit.

A network is defined as a communication method that connects multiple devices together to allow communication between the devices.

The ELCS utilizes an intra-division network to communicate between multiple processors and human-machine interfaces within a single division. The intra-division network communication is buffered from the ELCS controller by a communication module contained in the same rack that houses the ELCS controller. The intra-division communication module also performs communication diagnostics. The ELCS intra-division network is a deterministic network that utilizes a bus master. Each ELCS division includes an independent intra-division network. The intra-division network does not communicate outside the ELCS division. Each controller will send and receive periodic messages from the intra-division network communication modules. It allows communication between the control room safety displays, the Maintenance and Test Panel (MTP), and ELCS controllers for one division. This bus is used to communicate status and diagnostic data from the ELCS controllers for display on the safety

displays and MTP. It is also used to communicate test signals and data from the MTP and control room safety displays to the ELCS controllers. Each ELCS division includes an independent intra-division network. The intra-division network does not communicate outside the ELCS division.

High Speed Serial Link Communication

Each ELCS controller contains two processors. One processor is dedicated to performing the safety functions. The second processor is responsible for performing the unidirectional high speed serial link communications. The safety function processor shares a dual ported memory with the communications processor to allow data exchange. The ELCS communication processor has two independent receive communication ports and one independent transmit port.

The ELCS utilizes a high speed serial link (HSL) to communicate Class 1E safety function actuation information. The HSL is a true broadcast link that meets the communication isolation requirements of IEEE-Std-7.4.3-2. The HSL is utilized in a multi-drop communication method. In this method the transmission source is sent to multiple fiber optic modems which convert the HSL signal to utilize fiber optic communication media. The identical unidirectional signals are then connected to multiple receivers. An example of multi-drop communication is the transmission of a single division's Digital Trip Function (DTF) output actuation status signals to the other three divisions of Safety Logic Functions (SLFs).

For STP 3&4, the HSL communication is utilized for the following ELCS communication paths:

- DTF remote I/O to DTF
- DTF to SLF
- SLF safety function actuation to SLF remote I/O

Based on the above RAI response, both the RTIS and ELCS platforms utilize data links for intra-division communication. In revised response to RAI 07-5 (STPNOC Letter U7-C-STP-NRC-100027, dated January 21, 2010), the applicant clarified that none of the digital I&C platforms use networks only. ELCS utilizes a network within a division, but the network does not cross division boundaries. Accordingly, the applicant has proposed to revise FSAR Tier 1 Section 2.7.5 as the following:

“Some of the platforms use data links only ~~or networks only~~ and some of the platforms use a combination of both data links and networks.”

The staff found these proposed FSAR changes acceptable. Confirmation of these FSAR changes is being tracked as **Confirmatory Item 07-5**.

Tier 1 Section 3.4, Instrumentation and Control

The applicant has revised Section 3.4, including subsections and figures, based on Tier 1 Departure STD DEP T1 3.4-1. The applicant's response (STPNOC letter U7-C-STP-NRC-090009, dated February 9, 2009), provides additional changes to Tier 1, Section 3.4 Subsections A and C, and Figure 3.4b. A significant number of changes in this

section reflect all categories of changes included within the scope of Tier 1 Departure STD DEP T1 3.4-1. In general, the changes reflect the proposed I&C architecture, which NRC staff found acceptable with the following concerns/issues that require additional information from the applicant:

(1) Section A, Safety System Logic and Control

Based on STD DEP T1 3.4-1, the applicant has revised Tier 1 subsection A of Section 3.4 that provides design description of the proposed Safety System Logic and Control (SSLC). According to 10 CFR Part 52, Appendix A, Tier 1 information, such as, design descriptions, interface requirements, and site parameters are derived from Tier 2. The staff was unable to locate the SSLC design details in Tier 2 of the COL application that would form the basis for the SSLC design description provided in Tier 1. The staff requested the applicant (STPNOC) to resolve these inconsistencies.

The applicant's response to **RAI 07-6** (STPNOC Letter U7-C-STP-NRC-090232, dated December 30, 2009) provides a new site-specific FSAR Tier 2 Section 7.1S, "Site Specific Instrumentation and Control Platforms." This section provides an overview of the RTIS, NMS, and ELCS platforms, which is consistent with and envelops the SSLC design description in the FSAR Tier 1 Subsection A of Section 3.4. These FSAR additions resolve the inconsistencies in the COL application. Therefore, the staff finds these changes acceptable. Refer to Subsections 7.2 and 7.3 of this SER for in depth safety evaluation of the SSLC platforms. Confirmation of this FSAR change is being tracked as a **Confirmatory Item 07-6**.

Based on STD DEP T1 3.4-1, the applicant added a sentence in Tier 1 Subsection A(3) of Section 3.4 stating, "The TLF also receives input directly from the Neutron Monitoring System and manual control switches." Whereas, according to the FSAR, Figure 7.2-2, the Trip Logic Function (TLF) also receives direct input from the sensor channel(s) of corresponding division. The staff requested STPNOC to provide the design description of the sensor channel(s) with direct input to the TLF, and evaluate the impact on Figure 3.4b.

The applicant's response to **RAI 07-7** (STPNOC Letter U7-C-STP-NRC-090157, dated September 22, 2009) points to an error in FSAR Tier 2, Figure 7.2-2 that erroneously shows the TLF receiving direct input from the sensor channel. The correction in Figure 7.2-2 resolves the inconsistencies in the COL application. The staff therefore found the correction acceptable. Confirmation of this FSAR change is being tracked as **Confirmatory Item 07-7**.

Based on STD DEP T1 3.4-1, the applicant has deleted from Tier 1 Subsection A(5) of Section 3.4 the design concept of each division containing two ESFs processing channels each containing a pair of SLUs. The Departure Report fails to provide any justification for making this significant change in the design concept. NRC staff requested the applicant (STPNOC) to provide adequate justification for this change in the design concept and to demonstrate how the proposed ELCS conforms to the current regulations. The applicant's response to **RAI 07-8** (STPNOC Letter U7-C-STP-NRC-090157, dated September 22, 2009) provides the following explanation:

STP 3&4 COLA Part 7, departure STD DEP T1 3.4-1 Description, Item
(2) "Elimination of unnecessary inadvertent actuation prevention logic and

equipment” was previously addressed in the RAI 07.03-1 response. This response provided expanded bases for limiting the application of the dual redundant Safety System Logic Functions (SLFs), which replace the Safety System Logic Units (SLUs). It also provided bases for retention of two forms of dual redundancy in certain cases along with an expanded justification for item (2) of this departure. In addition, this response identified which functions did not require this logic feature and the justification bases for each function.

STP 3&4 COLA Part 2, Tier 1 Subsection 3.4.1 (changed back to Subsection 3.4.A) under “The ELCS portion of SSLC ...” Item (3) has a paragraph that captures the related Tier 1 requirements as:

“The SLF logic for ECCS functions (i.e. initiation of Reactor Core Isolation Cooling, High Pressure Core Flooder, Low Pressure Core Flooder or Automatic Depressurization) is implemented using redundant processing channels. The redundant channels receive the same input data from the DTF, manual control switch inputs and contact closures and perform the same trip decision logic. A majority of the redundant processors must agree for initiation of the function to occur, in order to assure that failure of a single electronic module will not result in inadvertent coolant injection into the core or inadvertent depressurization. The final majority vote of the system initiation signals is accomplished with non-microprocessor based equipment in the logic or with a separate actuation of system valves and pumps, where both are required to initiate coolant injection.”

Also, STP 3&4 COLA Part 2, Tier 2 will be updated with a new Section 7.1S.2 as described in STPNOC’s **RAI 07-6** response. The related and supporting Tier 2 requirements are in new Subsection 7.1S.2.1 as:

“As shown in Figure 3.4B, each of the three ESF component actuation divisions contains a minimum of two SLFs. One of the two SLFs processes initiation logic for functions that service the reactor vessel at low pressure (e.g., RHR), while a second SLF provides the same support for the vessel at high pressure (e.g., Reactor Core Isolation Cooling (RCIC) system and High Pressure Core Flooder (HPCF) system).

The SLF logic for ECCS functions (i.e., initiation of Reactor Core Isolation Cooling, High Pressure Core Flooder or Automatic Depressurization) is implemented using two redundant SLF processing channels per division. The two redundant channels receive the data from the four redundant divisional DTFs, manual control switch inputs and contact closures. The two redundant SLF processing channels perform the same ESF safety function action logic.”

And later it states:

“The two-out-of-two voting of the two SLF processing channels is performed on a component basis with non-microprocessor based equipment or with a separate actuation for a valve from one SLF

processing channel and a related pump actuation from the second SLF processing channel, where both are required to initiate coolant injection.”

The above referenced RAI responses, the related Tier 1 content, and the proposed Tier 2 additional content provide the information and resolution to this RAI. The staff therefore found the applicant’s response to **RAI 07-8** acceptable. For a detailed technical evaluation of this ESF design concept and its regulatory compliance, refer to Section 7.3 of this SER.

Based on STD DEP T1 3.4-1, the applicant has added a design description for ELCS portion of SSLC in Tier 1 Subsection A(3) of Section 3.4. Some of the added statements are vague (e.g., “A majority of redundant processors must agree...logic for some isolation functions...Other ELCS functions are implemented using redundancy, etc.). With these types of statements, inspection/verification of associate ITAAC will not be possible. NRC staff requested the applicant (STPNOC) to provide specific design details that outline the ESF functions and their need for redundant or non-redundant processing channels. Criteria for selecting the ESF functions that do not require redundant processing channels should also be provided demonstrating compliance with the current regulations. The applicant’s response to **RAI 07-9** (STPNOC Letter U7-C-STP-NRC-090157, dated September 22, 2009) provides the following clarification:

In STPNOC’s response to RAI 07-6, an addition to the COL application Part 2, Tier 2 Section 7.1S.2 was presented. In that COL application addition, the redundant SLF processing channels and the two-out-of-two voting of the channels was described. Also in response to RAI 07.03-1, a list of Functions for which there was a modification of the 2-out-of-2 voting was provided. This response also provided the Bases for limiting the application of dual redundant SLFs.

Additional information is provided below regarding design criteria for ESF functions.

SLF redundancy is categorized into the following four cases. The concept for categorization is described in COL application Part 2 Tier 1, Section 3.4.1, Item (3) under ELCS discussion as follows.

- (1) A majority of the redundant processors must agree for initiation of the function to occur, in order to assure that failure of a single electronic module will not result in inadvertent coolant injection into the core or inadvertent depressurization. The final majority vote of the system initiation signals is accomplished with non-microprocessor based equipment in the logic or with a separate actuation of system valves and pumps, where both are required to initiate coolant injection.
- (2) The SLF logic for some isolation and supporting ESF functions are also implemented using redundant channels where such implementation increases the operator response time to avoid plant operational impact following postulated failure in the control equipment. In these cases, an operator bypass that reduces the logic to a single channel may be utilized

where such logic reduces the risk of unnecessary adverse plant operational impact.

- (3) Other ELCS functions are implemented using redundancy where such logic provides overall plant operating or maintenance benefits.
- (4) Non-redundancy in processing channels.

Case 1 is applied to ECCS (i.e., RCIC, HPCF, LPFL, or ADS). This structure is described in COLA Part 2, Tier 2, Chapter 16, Figures B 3.3.1.4-2, B 3.3.1.4-3 or B 3.3.1.4-4.

Case 2 is mainly applied to equipment that is normally in operation and does not have redundancy of equipment (example: RCW isolation valve of non-safety loads). This structure is described in COLA Part 2, Tier 2, Chapter 16, Figure B 3.3.1.4-5.

Case 3 is applied when equipment malfunction does not impact plant operation. However, the logic redundancy benefits plant operation or maintenance. Applicable equipment for case (3) is determined during detailed design. COLA Part 2, Tier 2, Chapter 16, Figure B 3.3.1.4-2 or Figure B 3.3.1.4-5 is applied to the structures.

Case 4 is applied when equipment malfunction does not impact plant operation as in case 3, and failure to take action does not affect system functions. The major equipment applied is shown in the response to RAI 07.03-1. This structure is described in COLA Part 2, Tier 2, Chapter 16, Figure B 3.3.1.4-1.

The staff found the applicant's response to RAI 07-9 acceptable. For a detailed technical evaluation of this ESF design concept and its regulatory compliance, refer to Section 7.3 of this SER. Verification of the implementation of these ESF design concepts described above will be performed via Tier 1 Section 3.4 DAC/ITAAC inspections.

Based on STD DEP T1 3.4-1, the applicant has revised design description for ELCS portion of SSLC in Tier 1 Subsection A(4) of Section 3.4. The first sentence in the subsection appears to be incomplete, which simply states "local inputs." Also the applicant added a statement that states, "ELCS logic and controls are implemented through three divisions corresponding to the three divisions of controlled equipment." This statement is in contradiction with design details provided in various other parts of the COL application, which imply that there are four divisions of ELCS. The NRC staff requested the applicant to complete the missing information and resolve the stated conflicting information. In response to **RAI 07-10** (STPNOC Letter-U7-C-STP-NRC-090157, dated September 22, 2009), the applicant has proposed the following changes to the FSAR:

~~(7) For ESF functions, the trip signals in three divisions are transmitted by the Essential Multiplexing System to the RMUs local inputs, where a final 2-out-of-2 logic comparison is made prior to distribution of the control signals to the final actuators. ESF outputs do not exist in Division IV. As described above, the ELCS contains four redundant divisions of DTFs. The four divisions of DTF safety function~~

actuation status are communicated to three divisions of SLFs, which correspond to the three divisions of ESF actuated equipment. No ESF actuated equipment exists in Division IV. The final SLF actuation outputs are distributed to the final system actuated equipment control elements through the RDLC remote I/O devices. ~~ELCS logic and controls are implemented through three divisions corresponding to the three divisions of controlled equipment.~~

The staff found the proposed changes to FSAR Tier 1 Section 3.4.A consistent with the COL application and therefore acceptable. Confirmation of this FSAR change is being tracked as **Confirmatory Item 07-10**.

(2) Section B, I&C Development and Qualification Process

Based on STD DEP T1 3.4-1, the applicant had re-numbered the subsections of Tier 1 Section 3.4 (i.e., Section A to 3.4.1; B to 3.4.2; and C to 3.4.3). In STPNOC letter U7-C-STP-NRC-090009, dated February 9, 2009, the applicant reverts back the subsection numbering from 3.4.1 to A and from 3.4.3 to C. However, the applicant does not revert back to numbering Section 3.4.2 to B. Parts of the Tier 1 document still make references to Section B. NRC staff issued **RAI 07-11** requesting the applicant to resolve this conflict. The applicant's response to **RAI 07-11** (STPNOC Letter U7-C-STP-NRC-090157, dated September 22, 2009) proposes the following FSAR changes:

To be consistent with the ABWR DCD, Section 3.4 subsections were changed back to alpha designations, as shown below:

~~3.4.1~~ **A, Safety System Logic and Control**
~~3.4.2~~ **B, I & C Development and Qualification Processes**
~~3.4.3~~ **C, Diversity and Defense-in-Depth Considerations**

The staff found these proposed FSAR changes acceptable. Confirmation of these FSAR changes is being tracked as **Confirmatory Item 07-11**.

7.1.3.5 Post Combined License Activities

There are no post COL activities related to this section.

7.1.3.6 Conclusion

The NRC staff's finding related to information incorporated by reference is in NUREG-1503. The staff reviewed the application and checked the referenced DCD. The staff's review confirmed that the applicant has addressed the required information relating to "Controls and Instrumentation Systems-Tier 1 Design Description." With the exceptions of **Confirmatory Items 7-1, 7-2, 7-4 through 7-7, 7-10, and 7-11**, no outstanding information is expected to be addressed in the COL FSAR related to this section. Pursuant to 10 CFR 52.63(a)(5) and Part 52 Appendix A Section VI.B.1, all nuclear safety issues relating to the controls and instrumentation system that were incorporated by reference have been resolved.

The staff compared the application to the relevant NRC regulations; the acceptance criteria defined in NUREG-0800, Sections 7 and 14.3; and other NRC RGs. The staff concluded that

the Tier 1 design descriptions can be verified by the ITAAC and the ITAAC are necessary and sufficient. However, as a result of **Confirmatory Items 7-1, 7-2, 7-4 through 7-7, 7-10, and 7-11**, the staff was unable to finalize the conclusions relating to this section in accordance with the NRC requirements.

7.1.4 Instrument Setpoint Methodology

7.1.4.1 Introduction

In the certified ABWR DCD, development of the setpoint methodology is a part of Item 13 ITAAC listed in Tier 1, Table 3.4. As stated earlier, the applicant has elected to utilize the DAC/ITAAC process for developing the detailed digital I&C design. Likewise, the applicant's original intent was also to develop the setpoint methodology after issuance of the COL. The applicant has chosen to resolve the bracketed items in the Technical Specifications using the Setpoint Control Program (SCP). Therefore, in support of the SCP, the setpoint methodology technical report was submitted as part of the COL application. An evaluation of the SCP is in Chapter 16 of this SER. The setpoint methodology report is evaluated in this section of the SER.

7.1.4.2 Summary of Application

In a letter dated October 30, 2009, the STPNOC submitted Westinghouse commercial atomic power (WCAP)-17119-P Revision 0, "Methodology for South Texas Project Units 3 and 4 – ABWR Technical Specification Setpoint." Subsequently, in response to the NRC RAI 07.01-15 and RAI 16-65 the applicant submitted WCAP-17119-P, Revision 1 (STPNOC Letter-U7-C-STP-NRC-100072, dated March 29, 2010). This technical report is prepared by Westinghouse for STPNOC to document the instrument uncertainty calculations for the RPS and ESF functions for the ABWR plant. This document has been submitted as a part of the COL application and includes typical industry uncertainty values and assumptions that reflect the ABWR I&C design, to the extent that is required to support a COL application. This document identifies the general algorithm used as a basis for determining the overall instrument uncertainty and provides typical setpoints for each of the RPS and ESF functions. Reconciliation of this final setpoint study for the plant cannot be performed until the design for the plant is finalized. Before initial fuel loading, a reconciliation of this setpoint study against the final design for the plant will be performed, as required by the ABWR ITAAC (Section 3.4, Item 13 of Table 3.4, ABWR DCD, Revision 4).

7.1.4.3 Regulatory Basis

The objective of the WCAP-17119-P review is to confirm that the STP Units 3 and 4 setpoint methodology satisfies regulatory acceptance criteria, guidelines, and performance requirements. NRC staff evaluated the STP setpoint methodology, which is based on the guidelines prescribed in Standard Review Plan (SRP) Branch Technical Position 7-12. The following regulatory requirements and guidance documents are applicable to the staff's review of the STP setpoint methodology:

General Design Criterion (GDC) 13 of Appendix A to 10 CFR Part 50 requires, in part, that instrumentation be provided to monitor variables and systems and controls be provided to maintain these variables and systems within prescribed operating ranges.

GDC 20 of Appendix A to 10 CFR Part 50 requires, in part, that the protection system be designed to initiate automatically the operation of appropriate systems, including the reactivity control systems, to assure that specified acceptable fuel design limits are not exceeded as a result of anticipated operational occurrences.

Paragraph (c)(1)(ii)(A) of 10 CFR 50.36, "Technical Specifications," (TS), requires the TS to include limiting safety systems settings (LSSS). This paragraph specifies, in part, that "where a limiting safety system setting is specified for a variable on which a safety limit has been placed, the setting must be so chosen that automatic protective action will correct that abnormal situation before a safety limit is exceeded." Accordingly, the setpoint for instrument channels that initiate protective functions must be properly established in the setpoint methodology.

Paragraph (c)(3) of 10 CFR 50.36 states that surveillance requirements relate to tests, calibrations, or inspections to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met.

Regulation 10 CFR 50.55a(h), "Protection and Safety Systems," requires compliance with Institute of Electrical and Electronics Engineers (IEEE) Std. 603–1991 and the correction sheet dated January 30, 1995. Section 4.4 of IEEE Std. 603–1991 requires the identification of the analytical limit associated with each variable. Section 6.8.1 requires the allowances for uncertainties between the analytical limit and device setpoint to be determined using a documented methodology.

RG 1.105, Revision 3, describes a method acceptable to the staff for complying with the NRC's regulation for ensuring that setpoints for safety-related instrumentation are initially within and remain within the TS. This regulatory guide endorses Instrument Society of America (ISA-S67.4–1994, Part 1, "Setpoints for Nuclear Safety-Related Instrumentation Used in Nuclear Power Plants." Although not endorsed by the regulatory guide, ISA-S67.04–1994 Part II, "Methodology for the Determination of Setpoints for Nuclear Safety-Related Instrumentation," provides additional guidance.

7.1.4.4 Technical Evaluation

The establishment of setpoints and the relationships between trip setpoints, allowable value (AV), as-left tolerance, as-found tolerance, analytical limit, and safety limit are discussed in this report. A thorough understanding of these terms is important in order to properly utilize the method for establishing trip setpoints to protect the safety limits.

The safety limits are chosen to protect the integrity of physical barriers that guard against the uncontrolled release of radioactivity. The safety limits are typically provided in the plant safety analyses. The analytical limit is established to ensure that the safety limit is not exceeded. The analytical limit is developed from event analyses models that consider parameters such as process delays, rod insertion times, reactivity changes, and instrument response times.

The STP setpoint methodology combines the uncertainty components to determine the overall allowance for the functions of the safety-related systems. All appropriate and applicable uncertainties have been considered for each safety-related function. The methodology used to combine the uncertainty components for a channel is an appropriate combination of those

groups that are statistically and functionally independent. The independent and random uncertainty components are combined using the square root of the sum of the squares (SRSS) technique. The uncertainties that are not independent are conservatively treated by arithmetic summation and then systematically combined with the independent terms. The latter includes instrument (sensor and process rack) uncertainties and non-instrument-related effects (e.g., process measurement accuracy, etc.). This methodology uses the SRSS technique, which is acceptable to the NRC. Also, the American National Standard Institute (ANSI), the American Nuclear Society (ANS), and the ISA endorse the use of the same probabilistic and statistical techniques for the various standards in determining safety-related setpoints.

The STP setpoint methodology combines the uncertainty components to determine the overall Channel Statistical Allowance (CSA) for the RPS/ESF functions listed in Tables 3-1 through 79 of WCAP-17119-P, Revision 1. The CSA calculation is based on the following:

1. The sensor and rack measurement and test equipment uncertainties are treated as dependent parameters, with their respective drift and calibration accuracy allowances.
2. Although the environmental allowances are not considered statically dependent with all other parameters, the equipment testing generally results in large magnitude, non-random terms conservatively treated as limits of error that are added to the statistical summation. Westinghouse generally considers a term to be a limit of error if the term is a bias with an unknown sign. The term is added to the SRSS in the direction of conservatism.
3. Bias terms are one directional with known magnitudes that may result from several sources (e.g., drift or calibration data evaluations), which are also added to the statistical summation.
4. The calibration terms are treated in the same radical with the other terms based on the assumption that general trending data (i.e., drift and calibration data) are evaluated on a periodic and timely basis.

This STP setpoint methodology should confirm that the assumed distribution function characteristics as part of the treatment of the terms are still applicable. This approach results in a higher net reduction of the CSA magnitude than would be determined if trending was not performed. Also, in calculating the uncertainties for determining the CSA value, trip setpoints, and allowable value, the applicant uses a 95/95 tolerance limit as an acceptable criterion (i.e., a 95 percent probability and a 95 percent confidence level, which are consistent with the criterion of RG 1.105). This CSA value is compared with the total allowance (TA) for determining the instrument channel margin. The TA is established by adding the margin to the CSA. The TA defines a maximum acceptable value for the calculated instrument channel uncertainty referred to the CSA. Having determined the safety analytical limit and the TA, the nominal trip setpoint (NTS) can be calculated by subtracting (adding) from (to) the safety analysis limit (SAL), depending on the direction of the process variable change when approaching the SAL. Because there are two unknowns—the margin and the NTS—the staff was unable to identify the method used for determining the margin and/or the NTS. In response to the staff's RAI 07.01-15 (STPNOC Letter-U7-C-STP-NRC-100071, dated March 29, 2010), the applicant clarified the methods used in determining margin criteria and establishing NTS. In Section 3.3 of WCAP-17119-P, an acceptable criterion for the magnitude for margin has been defined as greater than or equal to rack calibration accuracy (RCA). In the RAI response, the applicant described three different approaches used in determining the acceptable ABWR nominal trip setpoints. Staff

finds these methods and approaches for determining the margin and NTS to be in accordance with the WCAP-17119-P setpoint methodology, and therefore acceptable.

The “As-Left Tolerance” is established by the required accuracy band (calibration accuracy) that a device or an instrument channel must be calibrated to the NTS during surveillance. The as-left condition is where the instrument channel setpoint is left within the as-left tolerance after verifying the calibration or trip setpoint. Additionally, if the as-found value is within the “As Left Tolerance,” then recalibration is not required. The AV is derived from (to) the NTS by subtracting (adding) the “As Found Tolerance” depending on the direction of the process variable change when approaching the SAL. The AV is what the instrument channel should be evaluated at for operability and to protect the safety limit when the test is performed. In response to the staff’s RAI 07.01-15 (STPNOC Letter-U7-C-STP-NRC-100071, dated March 29, 2010), the applicant clarified the definitions for “As Left Value”; “As Found Value”; “As Left Tolerance”; and “As Found Tolerance” in accordance with their applicability to sensors/transmitter or process racks. These clarified definitions, and the definition of “Allowable Value” have been included in Revision 1 of WCAP-17119-P. An allowable value column has also been added to Table 3-80 “Summary of Typical Setpoints and Allowances” of WCAP-17119-P. The staff finds that Revision 1 of WCAP-17119-P addresses all of the concerns identified in RAI 07.01-15, and therefore this RAI is considered closed.

WCAP-17119-P, Revision 1 makes a reference to WCAP-17137-P, “Westinghouse Stability Methodology for the ABWR” for determining the OPRM setpoints (Table 3-80, Note 7). Since WCAP-17137-P is a part of the fuel related topical reports that form the basis for post COL fuel amendment application, making a reference to a post COL document in the COL application is not acceptable. In response to the staff’s RAI 07.01-16 (STPNOC Letter U7-C-STP-NRC-100144, dated June 17, 2010), the applicant stated that reference to WCAP-17137-P will be removed from WCAP-17119-P, and will revise WCAP-17119-P to include typical setpoint values and associated uncertainties for the OPRM. Reference to WCAP-17137-P will also be removed from Technical Specification 5.5.2.11, “Setpoint Control Program” and from Standard Departure 16.3-100, Setpoint Control Program Implementation.” The staff finds this RAI response acceptable. Proposed changes to the FSAR and WCAP-17119-P are being tracked as **Confirmatory Item 07.01-16**.

7.1.4.5 Post Combined License Activities

There are no post COL activities related to this section.

7.1.4.6 Conclusion

As a result of **Confirmatory Item 07.01-16**, the staff is unable to finalize its conclusions relating to “Instrument Setpoint Methodology” in accordance with the NRC requirements.

7.1S Site Specific Instrumentation and Control Platforms

NRC staff issued **RAI 07-6** requesting the applicant to provide additional clarifications and descriptions. The applicant’s response to **RAI 07-6** (STPNOC Letter U7-C-STP-NRC-090232, dated December 30, 2009) provides a new site-specific supplement, FSAR Section 7.1S, which addresses the digital I&C platforms for safety-related I&C systems, including their data communications capabilities. In Section 7.1S.1, the applicant describes the RTIS, which is

based on the Non-Rewritable Field Programmable Gate Array (NRW-FPGA), and the NMS. In Section 7.1S.2, the applicant describes the implementation of the ELCS using a microprocessor-based platform. For technical and regulatory compliance evaluations of the departures related to the RTIS, NMS, and ELCS platforms and associated data communication features described in the COL application, including this new site-specific Section 7.1S, see SER Sections 7.2, 7.3, and 7.9S.

7.2 Reactor Protection (Trip) System (RPS) – Instrumentation and Controls, (Related to Regulatory Guide 1.206, Section C.I.7.2, “Reactor Trip System”)

7.2.1 Introduction

This section of the FSAR outlines the RPS or the RTIS as they are used in the COL application. The RTIS is the overall complex of instrument channels, trip logics, trip actuators, and scram logic circuitry that initiates a rapid insertion of control rods (scram) to shut down the reactor. There is also manual initiation. In addition, the RTIS establishes reactor-operating modes and provides status and control signals to other systems and annunciators.

To accomplish its overall function, the RTIS interfaces with the NMS, control rod drive system (CRDS), RCIS, reactor recirculation control system, PCF, nuclear boiler system (NBS), and other plant systems and equipment.

7.2.2 Summary of Application

Section 7.2 of the STP Units 3 and 4 COL FSAR incorporates by reference Section 7.2 of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A, with the following departures:

Tier 1 Departures

- STD DEP T1 2.3-1 Deletion of MSIV Closure and Scram on High Radiation

This departure describes the removal of the MSIV automatic closure and scram based on the high main steam line radiation monitor (MSLRM).

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

This departure describes changes to the I&C architecture and nomenclature that are needed to address obsolete data communication technology and the selection of digital I&C platforms.

Tier 2* Departure

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

This departure updates some codes, standards, and RGs to more current revisions/editions.

Tier 2 Departure Requiring Prior NRC Approval

- STD DEP 8.3-1 Plant Medium Voltage Electrical System Design

This departure identifies a change in the medium voltage distribution system that affects the certified ABWR DCD, Revision 4, for the offsite electrical power system, the onsite AC power distribution system, and safety loads. The departure specifically changes the medium-voltage electrical distribution system to a dual-voltage system consisting of 13.8 kilovolts (kV) and 4.16 kV.

Tier 2 Departure Not Requiring Prior NRC Approval

- STD DEP 7.1-1 References to Setpoints and Allowable Values

This departure clarifies in the FSAR that wherever the TS are referenced for setpoints or margins, the correct reference is to the methods for calculating the setpoints and margins, as described in the TS Bases.

- STD DEP 7.2-2 Description of Scram Actuating Relays

This departure revises the wording of the relay logic contact status from “normally closed” to “normally open” and clarifies that the tripped state is when the coil is “energized.”

- STD DEP 7.2-4 Manual Scram Monitoring

This departure deletes the statement about monitoring initiating variables because it is misplaced in the “Manual Scram” subsection.

- STD DEP 7.2-6 RPS Instrumentation Ranges (Table 7.2-1)

This departure updates RPS instrumentation ranges to reflect a range of values appropriate for optimal performance.

- STD DEP Admin

This departure describes minor corrections such as editorial or administrative errors in the certified ABWR DCD, Revision 4 (e.g., misspelled words, incorrect references, table headings, etc.).

7.2.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG–1503. In addition, the relevant requirements of the Commission regulations for the RPS I&C, and the associated acceptance criteria, are in Section 7.2 of NUREG-0800. In accordance with Section VIII, “Processes for Changes and Departures”, of, “Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor,” the applicant identifies three Tier 1 and Tier 2* departures requiring prior NRC approval. Tier 1 departures are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.A.4. Tier 2* departures are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.6. Tier 2 departures that affect TS or TS Bases require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.C.4. Tier 2 departures not requiring prior NRC

approval are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.5, which are similar to the requirements of 10 CFR 50.59.

Reviews of the departures are based on meeting the current regulatory requirements, acceptance criteria, and guidelines in NUREG–0800, Table 7-1 Revision 5 (March 2007). In accordance with 10 CFR 50.55a(h), COL applications filed on or after May 13, 1999, which depart from the certified ABWR DCD, Revision 4, must meet the requirements for safety systems in IEEE-Std 603-1991 and in the correction sheet dated January 30, 1995. In addition, applicable guidelines in Digital I&C-ISG documents are also used to evaluate the departures from the certified ABWR design.

7.2.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Section 7.2 of the certified ABWR DCD. The staff reviewed Section 7.2 of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of the information in the COL FSAR and the information in the ABWR DCD appropriately represents the complete scope of information relating to this review topic.¹ The staff's review confirmed that the information in the application and the information incorporated by reference address the required information relating to this section.

The staff reviewed the information in the COL FSAR:

Tier 1 Departures

The following Tier 1 departures identified by the applicant in this section require prior NRC approval and the full scope of their technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these Tier 1 departures.

- STD DEP T1 2.3-1 Deletion of MSIV Closure and Scram on High Radiation

This departure is classified as requiring prior NRC approval. In Part 7 of the COL application, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

NRC staff reviewed this departure and found that it deletes the MSIV automatic closure and scram based on the high MSLRM. Refer to section 7.1.2 of this SER for the discussion of this change. The staff noticed that the impacts of this departure on the RTIS I&C include the elimination of the automatic trip function and its associated initiating circuits from the high MSLRM. However, the main steam line (MSL) radiation indication and alarm is still available in the main control room. The staff found the associated I&C changes acceptable.

With respect to COL application FSAR instrument block diagram (IBD) Figure 7.2-9, Sheet 3, Revision 2 still includes the MSL radiation trip signal. The staff issued **RAI 07.01-4** requesting the applicant to delete the MSL radiation trip signal from the IBD accordingly. The applicant's

¹ See "Finality of Referenced NRC Approvals" in SER Section 1.1.3, for a discussion on the staff's review related to verification of the scope of information to be included in a COL application that references a design certification.

response to this RAI (STPNOC Letter U7-C-STP-NRC-090056) commits to revising IBD Figure 7.2-9. The staff found the applicant's response acceptable. However, because this proposed FSAR change is not captured in the current revision of the STP Units 3 and 4 COL application, the incorporation of this proposed FSAR change is being tracked as **Confirmatory Item 07.01-4**.

SER Chapter 11 documents the NRC staff's evaluation of Tier 1 Departure T1 2.3-1 for the deletion of the MSIV closure and scram on high radiation. This departure does not impact its conformance to the 10 CFR 50.55a(h) (IEEE-Std 603-1991) regulation for the RTIS safety system in the certified ABWR DCD. Hence, the staff determined that this departure is acceptable from the I&C perspective.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

NRC staff reviewed this departure and found that it includes the following five primary changes to the I&C architecture of the certified ABWR DCD, Revision 4:

1. Elimination of obsolete data communication technology
2. Elimination of unnecessary inadvertent actuation prevention logic and equipment
3. Clarification of digital controls nomenclature and systems
4. Final selection of platforms that changed the implementation architecture
5. Identification of testing and surveillance changes for the SSLC

The staff found that only items 1, 3, 4, and 5 above are changes that are relevant to the certified ABWR DCD FSAR Section 7.2, Revision 4. Item 2 is related to the ESF systems and will be addressed in Section 7.3 of this SER. The first change to the plant safety-related I&C architecture in the certified ABWR DCD replaces the EMS and the NEMS included as the communication protocol in the certified ABWR DCD, with separate and independent system level communication capabilities. It has been more than twelve years since the certification of the ABWR DCD, Revision 4. The staff agreed that the data communication standard ANSI-X3 series and fiber-distributed data interface (FDDI) used in the certified ABWR DCD have become an obsolete technology and are no longer appropriate for use with the ABWR safety-related I&C systems. The staff also found that the certified ABWR DCD used a centralized, common EMS that is subject to a common cause failure (CCF) potential of greater consequence. This CCF problem within the EMS could disable both the RTIS and the engineered safety features ELCS. The departure proposes to use different and diverse platforms for the RTIS and ELCS. The staff determined that this proposed change will allow the overall SSLC safety systems to be more resistant to CCFs; will provide a more robust communication design because a credible single failure will minimize degradation to independent communication functions; will not be subject to the CCF scenario of disabling both the RTIS and ELCS; and will provide the flexibility to utilize communication technologies that could generate benefits through simplicity of function and improved independence, such as the use of point-to-point unidirectional data links. The staff also found that the RTIS uses direct hardwired inputs to the system instead of using remote multiplexers, as described in the certified ABWR DCD, Revision 4. The staff also determined that this change will significantly reduce the complexity of data communication

requirements for this RTIS, while continuing to meet the certified ABWR DCD functional requirements.

The third change to the safety-related I&C architecture noted above in the certified ABWR DCD clarifies the use of digital controls nomenclature and systems associated with the above new data communication technology and selected platform. This change replaces all references to the previous EMS- and NEMS-related terminology and their primary components with a data communication reference accordingly.

The fourth and fifth changes are related to the platform change for the RTIS I&C safety system and its associated testing and surveillance requirements. The staff reviewed this platform change and found it necessary for incorporating currently available platforms and also for meeting both the regulatory and functional requirements. The staff reviewed the revised testing and surveillance descriptions for the SSLC (NMS, RTIS, and ELCS) and found them consistent with the characteristics of the proposed design platforms.

The staff's review of the above changes to the RTIS I&C system found that the relevant changes from this departure do not alter the certified ABWR design concepts and functional requirements for the RTIS safety system. The staff also found that this departure meets the regulatory requirements of 10 CFR 50.55a(h) (IEEE-Std 603–1991), GDC 13 and 19 to 24; and the guidelines in RG 1.152, Rev 2, DI&C-ISG-04, and other guidelines.

The proposed high-level changes for the RTIS and its associated functions in this departure are captured in the COL application FSAR, Tier 1 Section 3.4. However, there are no design details in the COL application FSAR Tier 2 Chapter 7, which can be used to form the bases for the high-level RTIS design description. The staff issued **RAI 07-6** requesting the applicant to provide enough details of the RTIS design information. The applicant's response to **RAI 07-6** (STPNOC Letter U7-C-STP-NRC-090232) commits to providing a new Tier 2 supplemental section to meet the requirements of the regulations in 10 CFR Part 52. The proposed supplemental section, "Site Specific Instrumentation and Control Platforms." includes the description of the field programmable gate array (FPGA)-based platform for the RTIS and NMS. In this supplemental section, the FPGA-based RTIS and NMS are presented in more detailed design. The staff found that this supplemental section includes adequate details of the platform design information. The staff also found that the design information for the RTIS meets the requirements of 10 CFR 50.55a(h). However, because the proposed new Tier 2 supplemental section is not captured in the current Revision of the STP Units 3 and 4 COL application, the incorporation of the proposed new Tier 2 supplemental section is being tracked as **Confirmatory Item 07-6**.

The staff found that the fourth change item noted above is not reflected in all related places of this and other sections. The staff issued **RAI 07.01-4** requesting the applicant to correct the inconsistencies. The applicant's response (STPNOC Letter U7-C-STP-NRC-090056) commits to correct all inconsistencies. The staff found the applicant's response acceptable. However, because the proposed FSAR changes are not captured in the current revision of the STP Units 3 and 4 COL application, the incorporation of the proposed FSAR changes is being tracked as **Confirmatory Item 07.01-4**.

COL application FSAR Tier 2 Subsection 7.2.1.1.6.1(3), Revision 2 refers to STD DEP T1 3.4-1, which proposes to change the time resolution of all sequences of events from 5 milliseconds

(ms) or less to 25 ms or less for the safety-related nuclear steam supply systems (NSSS). In Revision 2 of the COL FSAR, the applicant has not provided any reasons for this change or any resultant impacts. The staff issued **RAI 07.02-3** requesting the applicant to provide sufficient information and analysis to support this departure. The applicant's response to this RAI (STPNOC Letter U7-C-STP-NRC-090232) commits to reverting to the certified ABWR DCD Revision 4 value of 5 ms for the sequential events interval. The staff found this response acceptable. However, because these proposed FSAR changes are not captured in the current revision of the STP Units 3 and 4 COL application, the incorporation of these proposed FSAR changes is being tracked as **Confirmatory Item 07.02-3**.

In the safety-related RTIS, the bypass unit (BPU) with inputs and outputs to and from each division (shown in the original Figure 7.2-2 in the certified ABWR DCD, Revision 4) was deleted without any explanation in the COL FSAR. In addition, some interlocks (such as "reset permissive," "from one ACT [reset permissive]," "trips from NMS Div X," etc.) were included in the original figure but are circled as changes in this figure in the COL application FSAR. The staff issued **RAI 07.02-2** requesting the applicant to explain the above changes in Figure 7.2-2 and to clarify why the original interlocks are circled as changes. The applicant response to this RAI (STPNOC Letter U7-C-STP-NRC-090232) provides a markup of Figure 7.2-2 and the related FSAR text, which incorporates all changes and will be included in a future COL application revision. The staff's review of the markups found them consistent with the proposed changes in the departure, which the staff found acceptable. However, because the proposed FSAR changes in the applicant's response to RAI 07.02-2 are not captured in the current Revision 3 of the STP Units 3 and 4 COL application, the incorporation of the FSAR changes proposed in the applicant's RAI response is being tracked as **Confirmatory Item 07.02-2**.

Refer to Sections 7.1, 7.3, and 7.9S of this SER for detailed evaluations of this departure.

Tier 2* Departure

The following Tier 2* departure identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 2* departure.

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

As a Tier 2* departure, this departure requires prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

COL application FSAR Tier 2, Table 1.8-20 lists NRC RGs applicable to the ABWR. Table 1.8-21 lists industry codes and standards applicable to the ABWR. This departure identifies Tier 2* items in these two tables that are being updated to more current revisions/editions. The staff reviewed this departure and found that the following Tier 2* items included in this departure are related to the DI&C systems:

RG 1.75, "Physical Independence of Electric Systems," Revision 3, dated February 2005; and RG 1.153, "Criteria for Safety Systems," Revision 1, dated June 1996 are adopted to ensure that more recent industry design and

construction practices are used. The staff also found that the IEEE-Std 384–1992 “IEEE Standard Criteria for Independence of Class 1E Equipment and Circuits” was adopted. This edition of the standard is currently endorsed by the NRC. IEEE-Std 603–1991, “IEEE Standard Criteria for Safety Systems for Nuclear Power Generating Stations,” was updated to the 1991 version because this edition is required by the NRC regulation 10 CFR 50.55a(h). Because the electromagnetic interference (EMI) field has advanced considerably since the certification of the ABWR DCD, the applicant has updated the Mil-Specs for electromagnetic interference analysis and control to more current versions that comply with requirements in RG 1-180, “Guidelines for Evaluating Electromagnetic and Radio-Frequency Interference in Safety-Related Instrumentation and Control Systems.”

NRC staff found that some RGs have been updated to the current ones in force. The NRC has either approved or endorsed the revisions to the industrial codes and standards. The applicant submitted two supplemental tables (Tables 1.9S-1 and 1.9S-1a) that list the RGs, codes, and standards that are applicable to the safety-related I&C platform departures and that are considered acceptable. These updates to more current revisions or editions in the tables will increase plant reliability and performance by capturing selected advancements in engineering theory and practice since the issuance of the ABWR design certification.

From the review above, the staff concluded that this departure meets the requirements in regulations 10 CFR 50.55a(a)(1), 10 CFR 50.55a(h), and GDC 1. Hence, the staff determined that this departure is acceptable from the I&C perspective.

Refer to Section 7.1 of this SER for the detailed evaluation of this departure.

Tier 2 Departure Requiring Prior NRC Approval

The following Tier 2 departure identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 2 departure.

- STD DEP 8.3-1 Plant Medium Voltage Electrical System Design

This Tier 2 departure is classified as requiring prior NRC approval because it affects TS or TS Bases. In COL application Part 7, “Departures Report,” Section 2.2, the applicant describes and evaluates this departure per the requirements of Section VIII.C.4 of Appendix A to 10 CFR Part 52.

NRC staff reviewed this departure and found it includes the following changes:

- Medium voltage rating of the power generation (PG) buses increased to 13.8 kV
- Medium voltage rating of the plant investment protection (PIP) buses decreased to 4.16 kV
- Medium voltage rating of the class 1E buses decreased to 4.16 kV
- EDG ratings increased to 7200 kW and 4.16 kV
- Combustion turbine generator (CTG) ratings increased to 13.8 kV and at least 20 MWe

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP 7.2-4 Manual Scram Monitoring

NRC staff reviewed this departure, which deletes the statement about monitoring initiating variables, because it is misplaced in the "Manual Scram" subsection and could be misleading in light of the manual scram not being bypassed.

In COL application Part 7 "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP 7.2-6 RPS Instrumentation Ranges

NRC staff reviewed this departure and found that the change in this departure is reflected as new ranges in Table 7.2-1 for RPS instrumentation specifications, which are derived from the continuing design effort. The staff reviewed the updated ranges and found them appropriate for optimal performance.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation and determined that this departure does not require prior NRC approval. The staff conducted an audit on this Tier 2 departure and its evaluation process and found it reasonable that the departure does not require NRC approval. For additional information on this audit, see NRC Audit Report dated January 25, 2010 (ADAMS accession number ML093360537).

- STD DEP Admin

NRC staff reviewed this departure, which is administrative and makes minor corrections, such as editorial or administrative errors in the certified ABWR DCD, Revision 4 (e.g., misspelled words, incorrect references, table headings, etc.). Administrative departures do not affect the presentation of any design discussion or qualification of design margin. The changes in this departure to this section reflect clarifications of digital controls nomenclature and systems, as mentioned in Tier 1 Departure STD DEP T1 3.4-1.

In COL application Part 7, "Departures Report," Section 4.0, the applicant conducts screenings using an evaluation methodology derived from 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that administrative departures do not require prior NRC approval. The staff reviewed the Departures Report regarding this departure, and could not determine whether it is reasonable for this departure not to require prior NRC approval. The staff's review of Revision 2 of the COL FSAR, found many editorial or administrative errors. The staff issued RAI **07.01-4** requesting the applicant to correct those errors. The applicant's

response to this RAI (STPNOC Letter U7-C-STP-NRC-090056) commits to incorporating the corrections in the new revision of the COL application. The staff found the response acceptable. However, because some proposed FSAR changes are not captured in the current revision of the STP Units 3 and 4 COL application, the incorporation of these proposed FSAR changes is being tracked as **Confirmatory Item 07.01-4**.

7.2.5 Post Combined License Activities

There are no post COL activities related to this section.

7.2.6 Conclusion

The NRC staff's finding related to information incorporated by reference is in NUREG-1503. NRC staff reviewed the application and checked the referenced DCD. The staff's review confirmed that the applicant has addressed the required information relating to RPS I&C. With the exceptions of **Confirmatory Items 7-6, 7.01-4, 7.02-2, and 7.02-3**, no outstanding information is expected to be addressed in the COL FSAR related to this section. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the RPS I&C that were incorporated by reference have been resolved.

The applicant has elected to develop the detailed design for the RTIS using the ITAAC/DAC items in the certified ABWR DCD, Tier 1 Table 3.4. The staff found that the COL application provides adequate RTIS design information and acceptance criteria, so that verification of the implementation of the ITAAC/DAC items will demonstrate that the as-built RTIS conforms to the COL design.

The staff determined that the applicant has adequately addressed the identified Tier 1 and Tier 2* departures. The staff found it reasonable that the identified Tier 2 departures are characterized as not requiring prior NRC approval per 10 CFR Part 52, Appendix A, Section VIII.B.5. However, as a result of **Confirmatory Items 7-6, 7.01-4, 7.02-2, and 7.02-3**, the staff was unable to finalize the conclusions relating to the RPS I&C, in accordance with the NRC requirements.

7.3 Engineered Safety Feature Systems, Instrumentation, and Control

7.3.1 Introduction

This section of the FSAR describes the I&C for equipment in the various ESF systems. Descriptions of the ESF systems are in Chapter 6 and Chapter 9 of the certified ABWR DCD, Revision 4. The ESF systems for the ABWR utilize the ESF logic and control system (ELCS), which is a part of the SSLC system. The following ESF systems are evaluated:

- Emergency core cooling systems (ECCS)
- Leak detection and isolation system (LDS)
- Wetwell and drywell spray mode of the residual heat removal system (WDCF-RHR)
- Suppression pool cooling mode of the RHR system (SPC-RHR)
- Standby gas treatment system (SGTS)
- Emergency diesel generator support systems
- Reactor building cooling water (RCW) system and reactor service water (RSW) system

- Essential HVAC systems
- HVAC emergency cooling water (HECW) system
- High-pressure nitrogen gas supply (HPIN) system

7.3.2 Summary of Application

Section 7.3 of the STP Units 3 and 4 COL FSAR incorporates by reference Section 7.3 of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A. In addition, in FSAR Section 7.3, the applicant provides the following:

Tier 1 Departures

- STD DEP T1 2.3-1 Deletion of MSIV Closure and Scram on High Radiation

This departure describes the removal of the MSIV automatic closure and scram based on high MSLRM.

- STD DEP T1 2.4-1 RHR System and Spent Fuel Pool Cooling

This departure describes the changes that provide the ability to supply fuel pool cooling or makeup from any of the three RHR loops in the augmented fuel pool cooling or fuel pool makeup modes.

- STD DEP T1 2.4-2 Feedwater Line Break Mitigation

This departure reduces challenges to the containment pressure design value following a feedwater line break (FWLB).

- STD DEP T1 2.4-3 Reactor Core Isolation and Cooling Turbine/Pump

This departure describes an improved new design for the reactor core isolation and cooling (RCIC) turbine/pump system, which meets or exceeds all safety-related system performance criteria including start time, flow rate, and low steam pressure operation.

- STD DEP T1 2.14-1 Hydrogen Recombiner Requirements Elimination

This departure eliminates the hydrogen recombinder requirement to maintain equipment needed to mitigate a design-basis LOCA hydrogen release.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

This departure describes changes to the I&C architecture and nomenclature that are needed to address obsolete data communication technology and selection of digital I&C platforms.

Tier 2* Departure

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

This departure updates codes, standards, and RGs to more current revisions/editions.

Tier 2 Departures Requiring Prior NRC Approval

- STD DEP 7.3-12 Leak Detection and Isolation System Sump Monitoring

This departure revises alarm setpoints (nominal values) to support TS limits for reactor coolant pressure boundary leakage.

- STD DEP 7.3-17 Automatic Depressurization Subsystem (ADS) Electrical Interface

This departure provides a more complete description of the emergency core cooling systems (ECCS) compliance with this RG 1.75. This departure changes TS 3.3.1.4 Bases but does not change the intent of the generic TS.

Tier 2 Departures Not Requiring Prior NRC Approval

- STD DEP 1.1-2 Dual Units at STP 3 & 4

The certified ABWR DCD, Revision 4, is based on a single-unit site. STP Units 3 and 4 is a dual-unit project. This departure describes the common support systems shared by STP Units 3 and 4 and the existing Units 1 and 2.

- STD DEP 7.1-1 References to Setpoints and Allowable Values

This departure clarifies that, in the FSAR, wherever the TS is referenced for setpoints or margins the correct reference is to the methods for calculating the setpoints and margins, as described in the TS Bases.

- STD DEP 7.3-1 Time Intervals for Licensing Analysis

This departure deletes the specific values of the licensing analysis and provides a new reference in COL application FSAR Chapter 7 to ensure that the information is consistent.

- STD DEP 7.3-2 Automatic Depressurization System (ADS) Operator

This departure revises actuation of the automatic safety/relief valves (SRVs) as pneumatic action to clearly describe the ADS function of the SRV.

- STD DEP 7.3-4 ADS Logic

This departure provides a clearer description of the conditions under which the ADS could be initiated.

- STD DEP 7.3-5 Water Level Monitoring

This departure describes the use of standard ABWR nomenclature of various water levels for initiating signals.

- STD DEP 7.3-6 SRV Position Indication

This departure incorporates limit switches on the SRVs that provide a direct, positive indication of the SRV position and is more reliable than the original described linear variable differential transformers (LVDT).

- STD DEP 7.3-7 ADS Manual Operation

This departure changes the ADS inhibit switches from a keylock type to a normal manual switch.

- STD DEP 7.3-9 Shutdown Cooling Operation

This departure clarifies the description of the RHR shutdown cooling mode valve alignment in the certified ABWR DCD, Revision 4, after a low-pressure flooder (LPFL) actuation signal. This clarification deletes the reference to automatic closure of the RHR suction valves for the SCS mode, upon receipt of an LPFL initiation signal on water level 1.

- STD DEP 7.3-10 ESF Logic and Control System (ELCS) Mode Automation

This departure provides an expanded description of the mode switches in the main control room. In order to support the displays and reduce operator burden, the RHR has specific mode operation capability. This change eliminates the possibility of operator error and supports the display requirements.

- STD DEP 7.3-11 Leak Detection and Isolation System Valve Leakage Monitoring

This departure describes the use of one set of expanded graphite packing to seal the valve stem penetration in order to resolve the valve stem leakage issue.

- STD DEP 7.3-13 Containment Spray Logic

This departure clarifies the operation of the containment spray system, removes the manual override logic, and provides a more complete description of the operation of this mode of RHR.

- STD DEP 7.3-14 RHR Suppression Pool Cooling Logic

This departure describes changes to the logic and sequencing of the RHR suppression pool cooling mode.

- STD DEP 7.3-15 Reactor Service Water Logic Interfaces

This departure describes the modified information about the safety interfaces for the reactor cooling water system controls.

- STD DEP 7.3-16 Testing SRV Solenoid Valves

This departure describes improved testing capabilities for SRV solenoid valves. These improvements allow the testing of the safety/relief valve pilot solenoid valves to be performed at any pressure.

- STD DEP 7.7-2 SRV Discharge Pipe Temperature Data Recording

This departure describes a design upgrade of the SRV discharge pipe temperature data recording and alarm to replace the multipoint recorders with a historian function digital system, because there have been significant technological advances in data recording since the certified ABWR DCD, Revision 4, was written.

- STD DEP Admin

This departure describes minor corrections such as editorial or administrative errors in the certified ABWR DCD, Revision 4, (e.g., misspelled words, incorrect references, table headings, etc.).

COL License Information Item

- COL License Information Item 7.1 Cooling Temperature Profiles For Class 1E Digital Equipment

The applicant provides supplemental information to address COL License Information Item 7.1 regarding cooling temperature profiles for class 1E digital equipment.

7.3.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG–1503. In addition, the relevant requirements of the Commission regulations for the ESF systems I&C, and the associated acceptance criteria, are in Section 7.3 of NUREG-0800. In accordance with Section VIII, “Processes for Changes and Departures,” of, “Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor,” the applicant identifies Tier 1, Tier 2*, and Tier 2 departures. Tier 1 departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.A.4. Tier 2* departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.6. Tier 2 departures that affect TS or TS Bases require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.C.4. Tier 2 departures not requiring prior NRC approval are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.5, which are similar to the requirements of 10 CFR 50.59.

The regulatory basis for reviewing the COL license information is in Section 7.3 of NUREG-0800.

Reviews of the departures are based on meeting the current regulatory requirements, acceptance criteria, and guidelines in NUREG–0800, Table 7-1 Revision 5 (March 2007). In accordance with 10 CFR 50.55a(h), COL applications filed on or after May 13, 1999, which depart from the certified ABWR DCD, Revision 4, must meet the requirements for safety systems in IEEE-Std 603–1991, and in the correction sheet dated January 30, 1995. In addition, applicable guidelines in DI&C ISG documents are also used to evaluate the departures from the certified ABWR design.

7.3.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Section 7.3 of the certified ABWR DCD. The staff reviewed Section 7.3 of the STP Units 3 and 4 COL FSAR and checked

in the certified ABWR DCD. Hence, the staff determined that this departure is acceptable from the I&C perspective.

- STD DEP T1 2.4-1 RHR System and Spent Fuel Pool Cooling

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

NRC staff reviewed this departure and found that the relevant change in this departure adds the capability to allow the choice of a third loop, RHR division A, in the augmented fuel pool cooling and fuel pool makeup modes. The staff also found that this change provides the added flexibility of the capability to perform divisional outages in any order. In the certified ABWR DCD FASR Section 7.3, this departure modifies Figure 7.3-4, "RHR System IBD" accordingly. The staff found the changes from this departure acceptable from the I&C perspective since the added RHR system capability and augmented fuel pool cooling system meets regulatory requirements in 10 CFR 50.55a(h) (IEEE-Std 603-1991), 10 CFR 50.34(f)(V), GDC 13, 20-24, and 34 for the ESF system.

This departure is evaluated in Chapter 5 of this SER.

- STD DEP T1 2.4-2 Feedwater Line Break Mitigation

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

The staff reviewed this departure and found the changes included in this departure decrease the risk associated with the feedwater line break inside containment by ensuring that the containment pressure margins are maintained during the limiting containment pressurization accident. The staff found that the applicant has included the changes in FSAR Subsection 7.3.1.1.2 associated with this departure. However, the staff also found that more design information was needed to make a reasonable assurance evaluation. The staff issued **RAI 07.03-3** requesting the applicant to provide more information. In response (STPNOC letter U7-C-STP-NRC-090157), the applicant provides adequate design information. From the applicant's response, the staff found that this departure adds feedwater line differential pressure instruments to detect a line break in the piping and provide signals to the LDS logic systems. During normal plant operation, the differential pressure between the two feedwater lines is expected to be small. During a postulated break of one feedwater line inside the drywell, the differential pressure is expected to be large, because the broken line would be discharging into the low pressure drywell, and the intact line would be discharging into the high-pressure RPV. Each feedwater line differential pressure signal is compared to a set value, and its trip signal goes to a two-out-of-four voter logic. The output of this logic goes to an AND gate, into which the other input is the output of a two-out-of-four voter logic of high drywell- pressure trip signals that indicates a LOCA has occurred. This trip function is performed by the LDS. The postulated condition of a feedwater line break and a high drywell pressure will cause the operating condensate pumps to trip, which terminates feedwater flow into the drywell. The feedwater line differential pressure instrumentation and drywell pressure instrumentation are classified as safety-related for the mitigation of a postulated feedwater line break inside the drywell. The staff found that the changes in this departure will maintain the same level of plant reliability and

performance as described in the certified ABWR DCD, Revision 4. The changes will also provide a better level of plant protection and a net benefit to public health and safety. The staff's evaluation discussed above determined that this departure meets the requirements of 10 CFR 50.55a(h) (IEEE-Std 603–1991); GDC 13 and 20–24; and RG 1.97. Therefore, the staff found a reasonable assurance that this departure is acceptable from the I&C perspective.

This departure is also evaluated in Chapter 6 of this SER.

- STD DEP T1 2.4-3 RCIC Turbine/Pump

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

NRC staff found that the changes the applicant had made in FSAR Subsections 7.3.1.1.1.3 and 7.3.1.1.2 relate to this departure. The applicant also made corresponding changes to IBD 7.3-3, "Reactor Core Isolation Cooling System IBD," and 7.3-4, "Residual Heat Removal System IBD." The staff found that the changes in this departure include an improved new design for the RCIC turbine/pump system, which meets or exceeds all safety-related system performance criteria including start time, flow rate, and low steam pressure operation. In this proposed new design, the operating speed of the pump is governed by the turbine control subsystem that regulates the quantity of steam to the turbine based on discharge pressure. The main elements of the control gear are the steam stop valve, the throttle valve, and the pressure governor. In FSAR Subsections 7.3.1.1.1.3 and 7.3.1.1.2, this departure modifies the RCIC I&C description accordingly. The staff found that the related impact from this departure on the ESF I&C system is the updated I&C system for this new design of RCIC turbine/pump system. The staff found that this departure is acceptable from the I&C perspective because the staff determined that this departure meets the requirements in regulations CFR 50.55a(h) (IEEE-603-1991) and GDC 13 and 20–24.

This departure is also evaluated in Chapter 5 of this SER.

- STD DEP T1 2.14-1 Hydrogen Recombiner Requirements Elimination

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52. NRC staff found that the applicant has deleted FSAR Subsection 7.3.1.1.11 associated with the flammability control system (FCS), which is not used for the COL application. Accordingly the applicant made changes to IBD 7.3-4, "Residual Heat Removal System IBD," and IBD 7.3-5, "Leak Detection and Isolation System IBD," for the deletion of the FCS. The staff reviewed this departure and found that the change in this departure includes the elimination of the requirement to maintain equipment needed to mitigate a design-basis LOCA hydrogen release. The ABWR FCS, which consists of two redundant hydrogen recombiners, is no longer required in the response to a design basis and is eliminated. The containment hydrogen and oxygen monitoring functions of the containment monitoring system are no longer required to function for the mitigation of a design-basis LOCA. This proposed design incorporates changes to regulations that occurred after the issuance of the design certification for the ABWR.

The staff reviewed the related impacts from this departure on the ESF I&C system and found the departure acceptable from the I&C perspective, because after incorporating these design changes, the ABWR design features and requirements for controlling combustible gases still satisfy regulations in 10 CFR 50.44(c) and are consistent with the guidance in RG 1.7.

This departure is also evaluated in Chapter 6 of this SER.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

NRC staff reviewed this departure and found that this departure includes the following five primary changes to the I&C architecture of the certified ABWR DCD, Revision 4:

1. Elimination of obsolete data communication technology
2. Elimination of unnecessary inadvertent actuation prevention logic and equipment
3. Clarifications of digital controls nomenclature and systems
4. Final selection of platforms that changed the implementation architecture
5. Identification of testing and surveillance changes for the SSLC

But only above items 1, 2, 3, and 5 are changes that are relevant to the certified ABWR DCD, FSAR Section 7.3 Revision 4. Item 4 is related to the RTIS that is evaluated in Section 7.2 of this SER. The first change to the I&C architecture in the certified ABWR DCD is to replace the EMS and NEMS included as the communication protocol in the certified ABWR DCD, with separate and independent system level communication capabilities. It has been more than twelve years since the ABWR DCD, Revision 4, was certified. The staff found that the data communication standard ANSI-X3 series and the fiber-distributed data interface used in the certified ABWR DCD had become an obsolete technology and was no longer appropriate for use. The staff also found that the certified ABWR DCD used a centralized, common EMS that is subject to a common cause failure (CCF) potential of a greater consequence. This CCF within the EMS could disable both the RTIS and the ELCS. The departure proposed to use different and diverse platforms for the RTIS and ELCS. The staff determined that these proposed changes (a) allow the overall SSLC to be more resistant to a CCF; (b) provide a more robust communication design (a credible single failure will minimize degradation to independent communication functions); (c) are not subject to a single CCF disabling both the RTIS and ELCS; and (d) provide the flexibility to utilize communication technologies that have benefits in the areas of simplicity of function and improved independence, such as the use of point-to-point unidirectional data links. The staff also found that the ELCS addressed in this section will use serial, unidirectional, fiber optical-based data links to replace the fiber-distributed data interface protocol, as described in the certified ABWR DCD, Revision 4. The staff has already reviewed and approved the platform selected for the ELCS in the COL application as documented in the staff's SER dated August 11, 2000 (ADAMS Accession no. ML003740165). Therefore, staff determined that the method of communication for the ELCS meets the regulatory and functional requirements specified in the certified ABWR DCD, Revision 4.

The staff found that the purpose of the second change stated above is relevant to the ELCS I&C system and will minimize the potential for the false actuation of ESF components. The certified ABWR DCD includes a design for the ESF actuation system that is fully redundant within each

division of the ESF control system. This second change eliminates unnecessary and inadvertent actuation logic and equipment and implements the redundant actuation prevention logics only for the required ESF components, which may impact the safety or operation of the plant. But the COL application FSAR does not provide a list of logic and equipment in the ESF system selected for deletion or the bases for the deletion of each individual equipment or logic. The staff issued **RAI 07.03-1** requesting the applicant to address these concerns. The applicant's response to this RAI (STPNOC Letter U7-C-STP-NRC-0900065) provides a list of deleted logic functions and their bases for modifications. The staff reviewed the responses and found this change acceptable and therefore, RAI 07.03-1 is resolved. The changed ELCS logics still meet the safety functional requirements in the certified ABWR DCD.

The third change to the I&C architecture stated above is relevant to the ELCS in the certified ABWR DCD. This change clarifies the use of the digital controls nomenclature and systems associated with the above new data communication technology and approved platform selected for the ELCS. This change just replaces all references to the previous EMS- and non-NEMS-related terminology and primary components with a generic data communication reference.

The fifth change stated above is relevant to the ELCS and is related to the approved platform selected for the ELCS safety system and its associated testing and surveillance requirements. The staff has reviewed and approved this platform. The staff found that it meets both the regulatory and technical requirements in the certified ABWR DCD for the ELCS safety system.

After reviewing the changes to the ELCS I&C system, the staff found that the relevant changes in this departure do not depart from the concepts and functional requirements in the certified ABWR DCD for the ELCS safety system. The staff also found that this departure meets the regulatory requirements of 10 CFR 50.55a(h) (IEEE-Std 603-1991), GDC 13 and 19 to 24, and the guidelines in RG 1.152 Revision 1, DI&C-ISG-04, and other guidelines. Therefore, the staff found a reasonable assurance that this departure is acceptable from the I&C perspective with the confirmation items discussed below.

The proposed high-level changes for the ELCS and its associated functions in this departure are captured in the COL application FSAR Tier 1, Section 3.4. However, there are no design details in the COL application FSAR Tier 2, Chapter 7 that can be used to form the basis for the high-level ELCS design description. The staff issued **RAI 07-6** requesting the applicant to provide enough details of the ELCS design. The applicant's response to this RAI (STPNOC Letter U7-C-STP-NRC-090232) commits to providing a new Tier 2 supplemental section to meet the requirements of the regulations in 10 CFR Part 52. The proposed supplemental section titled "Site-Specific Instrumentation and Control Platforms" includes the description and appropriate details of the microprocessor-based platform for the ELCS, which the staff has reviewed and approved. The staff found the supplemental section acceptable. However, because it has not been captured in the current Revision 3 of the STP Units 3 and 4 COL application, the incorporation of the proposed new Tier 2 supplemental section is being tracked as **Confirmatory Item 07-6**.

The staff found that the change in item 4 stated above is not reflected in all of the related sections, including this one. The staff issued **RAI 07.01-4** requesting the applicant to correct the inconsistencies. The applicant's response to this RAI (STPNOC Letter U7-C-STP-NRC-090056) commits to correct all inconsistencies. The staff found the response acceptable. However, because the proposed FSAR changes are not captured in the

current Revision 3 of the STP Units 3 and 4 COL application, the incorporation of these proposed FSAR changes is being tracked as **Confirmatory Item 07.01-4**.

Refer to Sections 7.1, 7.2, and 7.9S of this SER for additional evaluations of this departure.

Tier 2* Departure

The following Tier 2* departure identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 2* departure.

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

NRC staff found that the applicant's change in FSAR Section 7.3, which replaces IEEE-Std 279 with IEEE-Std 603–1991, meets the requirement of 10 CFR 50.55a(h) and is therefore acceptable. Refer to Section 7.2 of this SER for an additional evaluation of the impacts from this departure on the digital I&C system.

Tier 2 Departures Requiring Prior NRC Approval

The following Tier 2 departures identified by the applicant in this section require prior NRC approval and the full scope of their technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these Tier 2 departures.

- STD DEP 7.3-12 LDI Sump Monitoring

Given that this departure affects the TS, this departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.2, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.C.4.

NRC staff found that this departure changes FSAR Subsection 7.3.1.1.2, "Leak Detection and Isolation System (LDS) – Instrumentation and Controls." The staff reviewed this departure and found that the changes include new leakage setpoint values and the addition of an increase in unidentified leakage parameters for the leak detection and isolation system. However, the staff determined that this departure does not change the intent of the generic TS. This departure therefore meets the requirements of 10 CFR 50.55a(h) (IEEE-Std 603-1991); 10 CFR 50.34(f)(2); and GDC 13, 19, and 20. The staff found reasonable assurance that this departure is acceptable from the I&C perspective.

This departure is also evaluated in Chapters 5 and 16 of this SER.

- STD DEP 7.3-17 ADS Electrical Interface

This departure affects a TS and is therefore classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.2, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.C.4.

NRC staff found that the applicant's changes in FSAR Subsection 7.3.2.1.2 relate to this departure. The staff reviewed this departure and found that the changes include a more complete description of ECCS compliance with RG 1.75. The staff found that the departure clarifies the control logic that is only in Divisions I, II, and III to conform to the three divisions of the ECCS. Sensor signals come from all four electrical divisions. Those changes are reflected in the revised FSAR Subsection 7.3.2.1.2. The staff found that this departure does not change the intent of the generic TS in the certified ABWR DCD and meets the requirements in 10 CFR 50.55a(h) and RG 1.75. Hence, the staff found this departure acceptable from the I&C perspective.

This departure is also evaluated in Chapter 16 of this SER.

Tier 2 Departures Not Requiring Prior NRC Approval

The following Tier 2 departures not requiring prior NRC approval identified by the applicant in this section may also be evaluated in other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these departures.

- STD DEP 1.1-2 Dual Units at STP Units 3& 4

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval.

NRC staff reviewed the Departures Report regarding this departure and found that this departure only clarifies that the certified ABWR DCD, Revision 4, is based on a single unit and the STP COL application is a dual-unit project. There are a few common balance-of-plant systems to be shared by STP Units 3 and 4 and the existing Units 1 and 2. The staff was unable to determine whether it is reasonable for this departure not to require NRC prior approval. The staff issued **RAI 07.07-1** requesting the applicant to address whether there is any common I&C failure that can cause multiple unit transients or shutdowns. The applicant's response to this RAI (STPNOC Letter U7-C-STP-NRC-0900056) clarifies that there are no shared I&C systems between STP Units 3 and 4 or between Units 3 and 4 and the existing Units 1 and 2 that are capable of bringing down multiple units. The safety-related ESF systems for each unit are not shared.

The staff reviewed the applicant's response to **RAI 07.07-1** and found it satisfactory; this RAI is closed. Therefore, within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections..

- STD DEP 7.1-1 References to Setpoints and Allowable Values

This departure modifies FSAR Subsections 7.3.1.1.1.1, 7.3.1.1.1.2, 7.3.1.1.3, 7.3.1.1.7, and 7.3.1.2 by referring to the bases in COL application Chapter 16 for methods of calculating setpoints and margins. NRC staff found this change acceptable.

The applicant's evaluation determined that this departure does not require prior NRC approval in accordance with 10 CFR Part 52, Appendix A, Section VIII.B.5. Within the review scope of

this section, the staff found it reasonable that this departure does not require prior NRC approval.

Refer to Section 7.2 of this SER for an additional evaluation of this departure from the I&C perspective.

- STD DEP 7.3-1 Time Intervals for Licensing Analysis

NRC staff reviewed this departure and found that the change to this section from this departure only replaces specific values with a reference to Table 6.3-1 in the ABWR DCD, thus ensuring that the information is consistent.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP 7.3-2 Automatic Depressurization System Operator

NRC staff reviewed this departure and found that it only rewords and clarifies the actuation of the automatic safety/relief valves without changing the meaning or intent of and with no adverse impact on the ADS design or function.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP 7.3-4 ADS Logic

NRC staff reviewed this departure and found that it only clarifies the initiating signals and logic for the ADS. The departure does not change the TS, setpoints for the parameters or other operational requirements. It does not change the ADS design, any plant physical features, SSCs important to safety, or fission product barriers.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. The staff conducted an audit on this Tier 2 departure and its evaluation process and found it reasonable that the departure does not require NRC approval. For additional information on this audit, see the NRC Audit Report dated January 25, 2010 (ADAMS accession number ML093360537).

- STD DEP 7.3-5

Water Level Monitoring

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval.

NRC staff reviewed the Departure Report regarding this departure and found that it includes the use of standard ABWR nomenclature for the reactor vessel level initiating instrumentation. This departure does not change the meaning or intent of and has no adverse impact on the ESF system design or function. However, the standard ABWR nomenclature for the reactor vessel level should be used in all other parts and sections of the COL application to maintain consistency. The staff was unable to determine whether it is reasonable for this departure not to require NRC prior approval. Therefore, the staff issued **RAI 07.02-5** requesting the applicant to address this concern. The applicant's response to **RAI 07.02-5** (STPNOC Letter U7-C-STP-NRC-090232) provides a markup of the impacted FSAR sections that corrects all inconsistencies. The markup will be included in a future COL application revision.

The staff found the response to this RAI acceptable. However, because the proposed FSAR changes in applicant's response are not captured in the current Revision 3 of the STP Units 3 and 4 COL application, the incorporation of these proposed FSAR changes is being tracked as **Confirmatory Item 07.02-5**.

- STD DEP 7.3-6

SRV Position Indication

NRC staff reviewed this departure and found that it includes the use of limit switches mounted on the ADS SRVs, instead of the originally described linear variable differential transformers (LVDTs), in the ABWR DCD to provide the position indication in the main control room for the SRVs. The changes in this departure are reflected in FSAR Subsection 7.3.1.1.1.2 and IBD 7.3-2. The use of the limit switch on the valves provides a direct and positive indication of the SRV position that is more reliable than that of the LVDT.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

This departure is also evaluated in Chapter 5 of this SER.

- STD DEP 7.3-7

ADS Manual Operation

NRC staff reviewed this departure and found that it replaces the ADS key-locked inhibit switch in the ABWR DCD with a normal manual switch to facilitate operator operation. This change does not affect the overall function of the ADS inhibit switches. The ADS inhibit switch is used to allow one ADS division to be taken out of service. This switch is ineffective once the ADS timers have timed out and thus cannot be used to abort and reclose the valves once they are signaled to open. The inhibit mode is continuously annunciated in the main control room. This departure only changes the type of switch and does not change the functionality of the ADS.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. The staff conducted an audit on this Tier 2 departure and its evaluation process and determined that it is reasonable that the departure does not require NRC approval. For additional information on this audit, see the NRC Audit Report dated January 25, 2010 (ADAMS accession number ML093360537).

- STD DEP 7.3-9 Shutdown Cooling Operation

NRC staff reviewed this departure, which deletes references to the automatic closure of the RHR suction valves for the shutdown cooling system mode upon receipt of an LPFL initiation signal on Level 1. These valves are already automatically closed on a Level 3 signal. This departure further clarifies that the shutdown cooling isolation valves must be closed to permit suction from the suppression pool.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. The staff conducted an audit on this Tier 2 departure and its evaluation process and found that it is reasonable that the departure does not require NRC approval. For additional information on this audit, see NRC Audit Report dated January 25, 2010 (ADAMS accession number ML093360537).

- STD DEP 7.3-10 ELCS Mode Automation

NRC staff reviewed this departure, which provides an expanded description of the mode switches in the main control room. In order to support the displays and reduce operator burden, the RHR has a specific mode operation capability that eliminates the possibility of operator error and supports the display requirements. The logic changes to the ELCS are implemented to assure that the diverse, hard-wired manual initiation function of HPCF loop "C" has priority over the normal automatic initiation logic for HPCF loop "C." These changes assure the proper implementation of the diverse, hard-wired HPCF loop "C" manual initiation capability.

For the safety-related RHR system, COL application FSAR Tier 2 Subsection 7.3.1.1.4(i), "Operational Considerations," states that "this action must occur within a limited interval." The applicant needs to provide a specific time requirement for this action and include what could happen, in addition to any mitigation that would be needed, if the operator does not act within that limited time interval.

NRC staff issued **RAI 07.03-2** requesting the applicant to address this concern. The applicant's response to this RAI (STPNOC Letter U7-C-STP-NRC-090157) provides a markup of the impacted FSAR sections that delete the specific time requirement. The applicant states that the markup will be included in a future COL application revision. The staff found this response acceptable. However, because the proposed FSAR changes in the applicant's response to **RAI 07.03-2** are not captured in the current Revision 3 of the STP Units 3 and 4 COL application, the incorporation of these proposed FSAR changes is being tracked as **Confirmatory Item 07.03-2**.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. The staff conducted an audit on this Tier 2 departure and its evaluation process and determined that it is reasonable that the departure does not require NRC approval. For additional information on this audit, see NRC Audit Report dated January 25, 2010 (ADAMS accession number ML093360537).

- STD DEP 7.3-11 Leak Detection and Isolation System Valve Leakage Monitoring

NRC staff reviewed this departure, which eliminates the reactor coolant pressure boundary isolation valve stem/gland leakage monitoring system. In the ABWR DCD, the valve stem packing rings were mostly made of asbestos material, which was prone to shrinkage during service. The shrinkage could cause voids in the packing chamber, which lead to leakage. To resolve the stem leakage issue, valves of one set of expanded graphite packing to seal the valve stem penetration were specified in the COL application FSAR. Expanded graphite has superior sealing properties, is less likely to induce corrosion and damage to the valve stem from the trace material, retains its form longer, and avoids the formation of voids that could lead to leakage. Due to the valve packing changes, the valve stem leak-off lines were eliminated. The valve gland leak-off lines were eliminated for the valves and the described instrumentation is no longer applicable.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

This departure is also evaluated in Chapter 5 of this SER.

- STD DEP 7.3-13 Containment Spray Logic

NRC staff reviewed this departure, which removes the manual override logic for the wetwell spray valves and suppression pool return valves. The annunciator status lights for these functions are removed from the table of status lights and annunciators. The departure to remove the manual override logic indicates that the spray system will continue to operate until manually terminated by the operator, or the spray system will automatically terminate and realign with the LPFL injection mode upon receipt of an RPV water level 1, because core cooling has priority.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. The staff conducted an audit on this Tier 2 departure and its evaluation process and determined that it is reasonable that the departure does not require NRC approval. For additional information on this audit, see NRC Audit Report dated January 25, 2010 (ADAMS accession number ML093360537).

- STD DEP 7.3-14 Residual Heat Removal Suppression Pool Cooling Logic

NRC staff reviewed this departure, which provides a more complete and consistent description of the suppression pool cooling (SPC) mode automatic and manual operations. The design change to an Arm and Initiate switch provides additional assurance that the operator will not inadvertently switch to the SPC mode from the LPFL mode while performing the critical operation of maintaining the water level in the RPV. This change does not impact the automatic initiation of the SPC mode on the high-suppression pool temperature. Consequently, this change will not impact the frequency or consequences of accidents or the malfunction of an SSC important to safety. There is no impact on any fission product barrier, nor is there any impact on the likelihood or consequences of an ex-vessel severe accident.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. The staff conducted an audit on this Tier 2 departure and its evaluation process and found it reasonable that the departure does not require NRC approval. For additional information on this audit, see NRC Audit Report dated January 25, 2010 (ADAMS accession number ML093360537).

- STD DEP 7.3-15 Reactor Service Water Logic Interfaces

NRC staff reviewed this departure, which provides additional and more complete information regarding RCW flow. This information is an improvement for the control room operator. This departure also changes the annunciator alarm on high differential pressure from the RCW heat exchangers to the RSW A or D strainers. The departure does not change any control room indications concerning these systems (e.g., flow rate, differential pressure, or temperature) or any SSC important to safety. Because fouling the strainers is more likely to restrict cooling flow in the RSW than would fouling the heat exchangers, differential pressure monitoring of the strainers more effectively monitors conditions that could impede flow in the RSW.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP 7.3-16 Testing Safety Relief Valve Solenoid Valves

NRC staff reviewed this departure, which improves testing capabilities of SRV solenoid valves. The staff found that these improvements allow the testing of the SRV pilot solenoid valves to be performed at any pressure instead of only when the reactor is not pressurized, thus enhancing the flexibility for testing these valves. Consequently, this departure has no adverse effects on the frequency or consequences of accidents or on functions of an SSC important to safety.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52 Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not

require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP 7.7-2 SRV Discharge Pipe Temperature Data Recording

NRC staff reviewed this departure, which changes the SRV discharge pipe temperature data recording using significant technological advances in data recording since the referenced ABWR DCD was written. The discharge temperatures of all SRVs will be shown on a historian function in the main control room. Recording SRV discharge temperature data will thus be performed in a more accurate manner that is easily retrievable. The recorded data rate meets all design criteria. The recorded data and the parameters remain the same.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure did not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

Refer to Section 7.7 of this SER for additional evaluations of this departure.

- STD DEP Admin

The applicant changed FSAR Subsections 7.3.1.1.1.1, 7.3.1.1.1.3, 7.3.1.1.1.4, 7.3.1.1.3, 7.3.1.1.10, 7.3.1.2, 7.3.2.8.2, and Figure 7.3-5. The changes are administrative and provide consistency in the COL application. The departure also clarifies descriptions and terminology.

The applicant's evaluation determined that this departure does not require prior NRC approval, in accordance with 10 CFR Part 52, Appendix A, Section VIII.B.5. Administrative departures do not affect the presentation of any design discussion or the qualification of any design margin. Therefore, NRC staff found it reasonable that this departure does not require prior NRC approval.

COL License Information Item

- COL License Information Item 7.1 Cooling Temperature Profiles for Class 1E Digital Equipment

The room cooling temperature profiles for equipment qualifications that are included in Appendix 3I of the certified ABWR DCD will be confirmed as part of the preoperational testing. ITAAC Item 14(b) in Tier 1, Table 3.4 is specifically designed with acceptance criteria to track the implementation of this COL license information item. NRC staff reviewed this ITAAC item and determined that it covers COL License Information Item 7.1, which is therefore closed because it is redundant to existing ITAAC Item 14(b) in Tier 1, Table 3.4.

7.3.5 Post Combined License Activities

There are no Post COL activities related to this section.

7.3.6 Conclusion

The NRC staff's finding related to information incorporated by reference is in NUREG–1503. The staff reviewed the application and checked the referenced DCD. The staff's review confirmed that the applicant has addressed the required information relating to the ESF systems I&C. With the exceptions of **Confirmatory Items 7-1, 7-6, 7.01-4, 7.02-5, and 7.03-2**, no outstanding information is expected to be addressed in the COL FSAR related to this section. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the ESF systems I&C that were incorporated by reference have been resolved.

The staff compared the COL application to the relevant NRC regulations; the acceptance criteria defined in NUREG–0800, Section 7.3; and other NRC RGs and concluded that the applicant is in compliance with the NRC regulations.

The applicant has elected to develop the detailed design for the ELCS using the ITAAC/DAC items in the certified ABWR DCD Tier 1 Table 3.4. The staff found that the COL application provides adequate ELCS design information and acceptance criteria; verification of the implementation of the ITAAC/DAC items will demonstrate that the as-built ELCS conforms to the COL design.

The applicant will address COL License Information Item 7.1 as part of the preoperational testing, which is covered by ITAAC item 14(b) in Tier 1, Table 3.4. The staff found that the applicant has adequately addressed the identified departures.

However, as a result of **Confirmatory Items 7-1, 7-6, 7.01-4, 7.02-5, and 7.03-2**, the staff was unable to finalize the conclusions relating to ESF systems I&C in accordance with the NRC requirements.

7.4 Systems Required for Safe Shutdown

7.4.1 Introduction

This Section 7.4 of the FSAR examines and discusses the I&C aspects of the following plant systems and functions designed to assure the safe and orderly shutdown of the ABWR:

- Alternate rod insertion (ARI) function
- Standby liquid control system (SLCS)
- Reactor shutdown cooling mode
- Remote shutdown system (RSS)

7.4.2 Summary of Application

Section 7.4 of the STP COL application FSAR incorporates by reference Section 7.4 of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A, with the following departures:

Tier 1 Departures

- STD DEP T1 2.4-1 RHR System and Spent Fuel Pool Cooling

This departure provides the ability to supply fuel pool cooling or makeup from any of the three RHR loops in the augmented fuel pool cooling or fuel pool makeup modes.

- STD DEP T1 2.14-1 Hydrogen Recombiner Requirements Elimination

This departure eliminates the hydrogen recombinder requirement to maintain equipment needed to mitigate a design-basis LOCA hydrogen release.

- STD DEP T1 3.4-1 Safety-related I&C Architecture

This departure describes changes to the I&C architecture and nomenclature that are needed to address obsolete data communication technology and the selection of digital I&C platforms.

Tier 2* Departure

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

This departure updates some codes, standards, and RGs to more current revisions/editions.

Tier 2 Departure Requiring Prior NRC Approval

- STD DEP 8.3-1 Plant Medium Voltage Electrical System Design

This departure identifies a change in the medium voltage distribution system that affects the certified ABWR DCD, Revision 4, for the offsite electrical power system, the onsite AC power distribution system, and safety loads. The departure specifically changes the medium-voltage electrical distribution system to a dual-voltage system consisting of 13.8 kV and 4.16 kV.

Tier 2 Departures Not Requiring Prior NRC Approval

- STD DEP 1.1-2 Dual Units at STP Units 3 And 4

The certified ABWR DCD, Revision 4, is based on a single-unit site; STP Units 3 and 4 is a dual-unit project. This departure describes the common supporting systems shared by STP Units 3 and 4 and the existing Units 1 and 2.

STD DEP 7.4-1 Alternate Rod Insertion Function Description

This departure provides a clear and concise description of the ARI functions. The ARI is not required for safety, nor are its components considered Class 1E.

- STD DEP 7.4-2 RHR Alarm

This departure replaces the alarm for the “RHR Logic Power Failure” with the alarm “ELCS out of Service.” The departure also clarifies that the only time the “Manual Initiation Armed” alarm is activated is when the RHR system is in the LPFL mode of operation.

- STD DEP Admin

This departure describes minor corrections in the certified ABWR DCD, Revision 4, such as editorial or administrative errors (e.g., misspelled words, incorrect references, table headings, etc.).

7.4.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG–1503. In addition, the relevant requirements of the Commission regulations for the systems required for a safe shutdown, and the associated acceptance criteria, are in Section 7.4 of NUREG-0800.

In accordance with Section VIII, “Processes for Changes and Departures,” of, “Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor,” the applicant identifies Tier 1, Tier 2*, and Tier 2 departures. Tier 1 and Tier 2* departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.A.4, and Section VIII.B.6, respectively. Tier 2 departures that affect TS or TS Bases require prior NRC approval and are subject to the requirements of 10 CFR 52 Appendix A, Section VIII.C.4. Tier 2 departures that do not require prior NRC approval are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.5, which are similar to the requirements of 10 CFR 50.59.

Reviews of the departures are based on meeting the current regulatory requirements, acceptance criteria, and guidelines in NUREG–0800 Table 7-1, Revision 5 (March 2007). In accordance with 10 CFR 50.55a(h), COL applications filed on or after May 13, 1999, which depart from the certified ABWR DCD, Revision 4, must meet the requirements for safety systems in IEEE-Std 603–1991 and in the correction sheet dated January 30, 1995. In addition, applicable guidelines in Digital I&C Interim-ISG documents are also used to evaluate the departures from the certified ABWR design.

7.4.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Section 7.4 of the certified ABWR DCD. The staff reviewed Section 7.4 of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of the information in the COL FSAR and the information in the ABWR DCD appropriately represents the complete scope of information relating to this review topic.¹ The staff's review confirmed that the information in the application and the information incorporated by reference address the required information relating to this section.

The staff reviewed the information in the COL FSAR:

Tier 1 Departures

The following Tier 1 departures identified by the applicant in this section require prior NRC approval and the full scope of their technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these Tier 1 departures.

- STD DEP T1 2.4-1 RHR System and Spent Fuel Pool Cooling

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.A.4.

NRC staff found that the applicant had changed FSAR Section 7.4.1.3. The applicant also made corresponding changes to Figure 7.4-2, "Remote Shutdown System IED," [instrument and electrical diagram] and Figure 7.4-3, "Remote Shutdown System IBD" [interlock block diagram]. The staff reviewed this departure, which adds an additional third loop for the RHR system in the augmented fuel pool cooling and fuel pool makeup modes. This change provides additional flexibility for the capability to perform divisional outages in any order. The change in this departure from Section 7.4 of the ABWR DCD includes the I&C function for this new third loop and its testability. The I&C-related changes is about testability during normal operation for all valves, except for those isolated by the reactor pressure interlock in the system.

For the DI&C system to be used for STP Units 3 and 4, the applicant should provide sufficient information on how to prevent the remote transfer devices from adversely impacting both the safety-related DI&C system and the remote shutdown system (RSS). Because no information has been provided for this transferring device, NRC staff issued **RAI 07.04-1** requesting the applicant to explain how the remote transfer devices function between the two systems.

The applicant's response to this RAI (STPNOC Letter U7-C-STP-NRC-090118) explains that the remote transfer devices are not used for DI&C communications. The remote transfer devices are mechanically reliable Class 1E switches and are utilized for selected hardwired input and output control signals. The method of using remote transfer devices assures that the remote shutdown system has the required control capability independent of the state of the

¹ See "Finality of Referenced NRC Approvals" in SER Section 1.1.3, for a discussion on the staff's review related to verification of the scope of information to be included in a COL application that references a design certification.

DI&C system. NRC staff found the applicant's response acceptable, and this **RAI 07.04-1** is hence resolved.

The staff reviewed the I&C-related changes resulting from this departure and found it acceptable from the I&C perspective.

This departure is also evaluated in Chapter 5 of this SER.

- STD DEP T1 2.14-1 Hydrogen Recombiner Requirements Elimination

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

NRC staff found that the applicant had changed FSAR Subsections 7.4.1.4.4 and 7.4.2.4.2. The applicant also made changes accordingly to Figure 7.4-2, "Remote Shutdown System IED," and Figure 7.4-3, "Remote Shutdown System IBD." The staff reviewed this departure and found that it eliminates the requirement to maintain equipment needed to mitigate a design-basis LOCA hydrogen release. The ABWR FCS, which consists of two redundant hydrogen recombiners, is no longer required in the response to a design-basis LOCA and is eliminated from this section. The containment hydrogen and oxygen monitoring functions of the containment monitoring system are no longer required to function for the mitigation of a design-basis LOCA. This proposed design incorporates changes to regulations that occurred after the issuance of the design certification for the ABWR. After incorporating the above design changes, the staff determined that the ABWR design features and requirements for controlling combustible gases satisfy regulations in 10 CFR 50.44(c) and are consistent with the guidance in RG 1.7. Hence, the staff found this departure acceptable from the I&C perspective.

This departure is also evaluated in Chapter 6 of this SER.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

NRC staff reviewed this departure and found that it includes the following five primary changes to the I&C architecture of the certified ABWR DCD, Revision 4:

1. Elimination of obsolete data communication technology
2. Elimination of unnecessary inadvertent actuation prevention logic and equipment
3. Clarifications of digital controls nomenclature and systems
4. Final selection of platforms that changed the implementation architecture
5. Identification of testing and surveillance changes for SSLC

Section 7.4 of the FSAR includes an examination and discussion of the I&C aspects for the ARI function, the SLCS, and the reactor shutdown cooling mode (RHR). The staff found that this departure impacts the interface between the SSLC and ARI functions and the RSS required for a safe shutdown in this section. So, the staff found that only the change in items 1 and 3 above are relevant to the certified ABWR DCD FSAR, Section 7.4 Revision 4. Item 2 is associated

with the ELCS and is evaluated in Section 7.3 of this SER. Items 4 and 5 are related to the SSLC systems and are reviewed in both Sections 7.2 and 7.4 of this SER.

The first change is to the I&C architecture in the certified ABWR DCD, which replaces the EMS and NEMS included as the communication protocol in the certified ABWR DCD, with separate and independent system level communication capabilities. The interface between the SSLC and systems in this section is implemented via one-way data communication with adequate isolation.

The third change noted above to the I&C architecture in the certified ABWR DCD is to clarify the use of digital controls nomenclature and systems associated with the new data communication technology and selected platform referred to above. This third change replaces all references to the previous EMS- and NEMS-related terminology and their primary components with a generic data communication reference accordingly.

The staff found that the changes from this departure to the interface between the systems in this section and the SSLC systems meet the regulatory requirements in 10 CFR 50.55a(h) (IEEE-Std 603–1991), GDC 24, and DI&C-ISG-04. The staff also found that the relevant changes in this departure do not depart from the certified ABWR design concepts and functional requirements for the systems required for a safe shutdown. This departure is therefore acceptable.

Refer to Sections 7.1, 7.2, 7.3, and 7.9S of this SER for additional evaluations of this departure.

Tier 2* Departure

The following Tier 2* departure identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 2* departure.

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

NRC staff found that the applicant had changed FSAR Sections 7.4.1 and 7.4.2 by replacing IEEE-Std 279 with IEEE-Std 603–1991, which meets the requirement of 10 CFR 50.55a(h). Therefore, the staff found the change acceptable.

Refer to Section 7.2 of this SER for the detailed evaluation of this departure from the I&C perspective.

Tier 2 Departures Requiring Prior NRC Approval

The following Tier 2 departure identified by the applicant in this section require prior NRC approval and the full scope of its/their technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 2 departure.

- STD DEP 8.3-1

Plant Medium Voltage Electrical System Design

This Tier 2 departure is classified as requiring prior NRC approval because it affects TS or TS Bases. In COL application Part 7, "Departures Report," Section 2.2, the applicant describes and evaluates this departure per the requirements of Section VIII.C.4 of Appendix A to 10 CFR Part 52.

NRC staff reviewed this departure and found that it includes the following changes:

- Medium voltage rating of the power generation (PG) buses are increased to 13.8 kV
- Medium voltage rating of the plant investment protection (PIP) buses are decreased to 4.16 kV
- Medium voltage rating of the Class 1E buses are decreased to 4.16 kV
- EDG ratings are increased to 7,200 kW and 4.16 kV
- CTG ratings are increased to 13.8 kV and to at least 20 MWe
- The time required for the CTG to start and achieve steady-state voltage and frequency is increased from two minutes to "less than 10 minutes," as required by RG 1.155 for a Station Blackout (SBO) alternate AC source.

In response to RAI 08.03.01-4, Supplement 3 (STPNOC Letter U7-C-STP-NRC-100137, dated June 17, 2010), the applicant added the following additional changes to this departure STD DEP 8.3-1:

- The capability to power the FMCRDs directly from the PIP bus and the direct transfer capability is changed to occur at the 480 volt level
- Power supplies to the FMCRD power distribution panels A-1, A-2, B-1, B-2, C-1 and C-2 and transfer switches have been re-classified as non-Class 1E
- Isolation between Class 1E bus and non-Class 1E FMCRD loads is provided by two Class 1E protective devices in Series. The Class 1E breaker trip prior to upstream breaker trip is assured by circuit protection coordination and testing of breakers. The zone selective interlock feature is removed.

The staff found that the applicant had made relevant changes to FSAR Section 7.4.1.3 and Subsections 7.4.1.4.4 and 7.4.2.4.4. The staff also found that this departure specifically impacts the power sources for the reactor shutdown and electrical power distribution system (EPDS) for the remote shutdown system. The staff found that the proposed changes will improve reliability with divisional and safety/non-safety isolation and independence increased through the use of stub buses; through greater flexibility for maintenance, surveillance, and inspection via the dual-voltage design; and through the increased availability of multiple sources of power to the various buses in the revised design and the ability to isolate the buses individually, if needed. The staff found that the changes to this section reflect the rating and other electrical changes, which do not have direct impacts on the safe shutdown I&C system. The staff reviewed the changes in this departure with the safe shutdown I&C systems and found that it meets the requirements of

Clause 8.1 of IEEE-Std 603-1991, 10 CFR 50.12(a)(2), 10 CFR 50.63, and GDC 17. Hence, the staff determines that this departure is acceptable from the I&C perspective.

This departure and applicant's response to RAI 08.03.01-4, Supplement 3 (STPNOC Letter U7-C-STP-NRC-100137, dated June 17, 2010) is evaluated in Chapter 8 of this SER. In response to RAI 08.03.01-4, Supplement 3, the applicant has proposed changes to FSAR Section 7.4.2.1.2 that removes the zone selective interlock feature associated with breaker coordination. Acceptability of this FSAR change is evaluated in Chapter 8 of the SER and is being tracked as **Confirmatory Item 8.3.1-4**.

Tier 2 Departures Not Requiring Prior NRC Approval

The following Tier 2 departures not requiring prior NRC approval identified by the applicant in this section may also be evaluated in other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these departures.

- STD DEP 1.1-2 Dual Units at STP Units 3&4

The certified ABWR design is for a single unit. The COL application is submitted for two units. However, the two units do not share any systems that are required for the safe shutdown addressed in FSAR Section 7.4. The applicant changed FSAR Subsection 7.4.2.2.2 accordingly, and the staff found the change acceptable.

Refer to Section 7.3 of this SER for the detailed evaluation of this departure from the I&C perspective.

- STD DEP 7.4-1 Alternate Rod Insertion Function Description

NRC staff reviewed this departure, which provides additional details on the functioning of the I&C for this non-safety-related ARI system. The departure adds further details for each item but does not change the basic characteristics of the system.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP 7.4-2 RHR Alarm

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval.

NRC staff reviewed the Departures Report regarding this departure, which replaces the alarm for "RHR Logic Power Failure" with the alarm for "ELCS out of Service." The second change in this departure clarifies that only the LPFL mode of the RHR system has an arming feature. However, COL application FSAR Subsections 7.3.2.3.2, 7.3.2.4.2, and 7.4.2.3.2 state that the

parent RHR system annunciates activity at the loop level, and the individual mode of the RHR system is not separately annunciated. According to Section 5.8 of IEEE-Std 603–1991, a system status indication shall be provided. Because there are a few operation modes (LPFL, wetwell and drywell spray cooling, and suppression pool cooling) for the RHR system, the display of the operation mode for the RHR system shall be provided to minimize the possibility of ambiguous indications that could be confusing to the operator. The staff was unable to determine whether it is reasonable for this departure not to require prior NRC approval. Therefore, the staff issued **RAI 07.03-4** requesting the applicant to provide additional clarifications.

The applicant's response to this RAI (STPNOC Letter U7-C-STP-NRC-090157) explains that there is no departure from the certified ABWR DCD design, Revision 4. The applicant also clarifies that the human system interface design criteria will be employed to present the optimal mode status information/indication to the control room operator via the use of modern video display units. The staff found this response acceptable, and **RAI 07.03-4** is closed. However, the incorporation of the proposed optimal mode status information/indication is being tracked as **Confirmatory Item 07.03-4**.

- STD DEP Admin

The applicant made changes to FSAR Subsection 7.4.2.1.1, Section 7.4.3, and Figures 7.4-1 and 7.4-2 that corrected a reference title and provided clearer descriptions. The applicant's evaluation determined that this departure does not require prior NRC approval in accordance with 10 CFR Part 52, Appendix A, Section VIII.B.5. Administrative departures do not affect the presentation of any design discussion or the qualification of any design margin. Within the review scope of this section, NRC staff found it reasonable that this departure does not require prior NRC approval.

7.4.5 Post Combined License Activities

There are no Post COL activities related to this section.

7.4.6 Conclusion

The NRC staff's finding related to information incorporated by reference is in NUREG–1503. NRC staff reviewed the application and checked the referenced DCD. The staff's review confirmed that the applicant has addressed the required information relating to the systems required for a safe shutdown. With the exception of **Confirmatory Items 7.03-4 and 8.3.1-4**, no outstanding information is expected to be addressed in the COL FSAR related to this section. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the systems required for a safe shutdown that were incorporated by reference have been resolved.

The staff compared the COL application to the relevant NRC regulations; the acceptance criteria defined in NUREG–0800, Section 7.4; and other NRC RGs. The staff found that the applicant has adequately addressed the identified departures. However, as a result of **Confirmatory Items 7.03-4 and 8.3.1-4**, the staff was unable to finalize the conclusions relating to the systems required for a safe shutdown, in accordance with the NRC requirements.

7.5 Information Systems Important to Safety

7.5.1 Introduction

This section of the FSAR addresses safety-related display systems that provide information for the safe operation of the plant during normal operation, anticipated operational occurrences, and accidents. The information systems important to safety include those systems that provide information for the manual initiation and control of safety systems to indicate that plant safety functions are being accomplished and to provide guidance to appropriate actions that can be taken to mitigate the consequences of anticipated operational occurrences and accidents. The safety parameter display system (SPDS), information systems associated with the emergency response facility, and the nuclear data link are information systems important to safety. The post-accident monitoring (PAM) system receives variables that are monitored. The PAM system provides information that is required to permit control room operators to take manual actions and safety systems to accomplish their safety functions, or provide operators with information.

7.5.2 Summary of Application

Section 7.5 of the STP Units 3 and 4 COL application FSAR incorporates by reference Section 7.5 of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A, with the following departures:

Tier 1 Departures

- STD DEP T1 2.3-1 Deletion of MSIV Closure and Scram on High Radiation

This departure describes the removal of the MSIV automatic closure and scram based on the high MSLRM.

- STD DEP T1 2.14-1 Hydrogen Recombiner Requirements Elimination

This departure reflects the elimination of the hydrogen recombinder requirement to maintain equipment needed to mitigate a design-basis LOCA hydrogen release.

Tier 2* Departure

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

This departure updates some codes, standards, and RGs to more current revisions/editions.

Tier 2 Departure Requiring Prior NRC Approval

- STD DEP 7.5-1 PAM – Drywell Pressure

This departure describes the updated PAM design requirements to more closely follow the guidance of RG 1.97, BTP HICB-10, and Three Mile Island (TMI)-related criteria 10 CFR 50.34.

Tier 2 Departures Not Requiring Prior NRC Approval

- STD DEP 11.5-1 Process and Effluent Radiation Monitoring and Sampling System

This departure describes several changes made to the process and effluent radiation monitoring and sampling system.

- STD DEP Admin

This departure describes minor corrections such as editorial or administrative errors in the certified ABWR DCD, Revision 4 (e.g., misspelled words, incorrect references, table headings, etc.).

7.5.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG–1503. In addition, the relevant requirements of the Commission regulations for the information systems important to safety, and the associated acceptance criteria, are in Section 7.5 of NUREG-0800.

In accordance with Section VIII, “Processes for Changes and Departures,” of, “Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor,” the applicant identifies Tier 1, Tier 2*, and Tier 2 departures. Tier 1 and Tier 2* departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.A.4 and Section VIII.B.6, respectively. Tier 2 departures that affect TS or TS Bases require prior NRC approval and are subject to the requirements of 10 CFR 52 Appendix A, Section VIII.C.4. Tier 2 departures that do not require prior NRC approval are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.5, which are similar to the requirements of 10 CFR 50.59.

The requirements of 10 CFR 52.63(a)(1)(vii) (as proposed in SECY-06-220) allow for a change to a ABWR DCD if the change contributes to an increase in standardization of the certification information. Reviews of the departures are based on meeting the current regulatory requirements, the acceptance criteria, and the guidelines in NUREG–0800, Table 7-1 Revision 5 (March 2007). In accordance with 10 CFR 50.55a(h), COL applications filed on or after May 13, 1999, which depart from the certified ABWR DCD, Revision 4 must meet the requirements for safety systems in IEEE-Std 603–1991 and in the correction sheet dated January 30, 1995. In addition, applicable guidelines in DI&C-ISG documents are also used to evaluate the departures from the certified ABWR design.

7.5.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Section 7.5 of the certified ABWR DCD. The staff reviewed Section 7.5 of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of the information in the COL FSAR and the information in the ABWR DCD appropriately represents the complete scope of information relating to this review topic.¹ The staff’s review confirmed that the information in the

¹ See “*Finality of Referenced NRC Approvals*” in SER Section 1.1.3, for a discussion on the staff’s review related to verification of the scope of information to be included in a COL application that references a design certification.

Refer to Section 7.2 of this SER for the detailed evaluation of this departure from the I&C perspective.

Tier 2 Departure Requiring Prior NRC Approval

The following Tier 2 departure identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 2 departure.

- STD DEP 7.5-1 PAM – Drywell Pressure

This departure is classified as requiring prior NRC approval due to the changes to the referenced ABWR TS, per the requirements in 10 CFR Part 52, Appendix A, Section VIII.C.4. In COL application Part 7, “Departures Report,” Section 2.2, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.C.4.

NRC staff found that the applicant had made changes to FSAR Subsection 7.5.2.1, Tables 7.5-2, Section 7.5-3, and Section 7.5-4. The staff reviewed this departure, which updates the PAM design requirements to more closely follow the guidance of RG 1.97, BTP HICB-10, and TMI-related criteria in 10 CFR 50.34. The identified changes in the departure are requirements that were exempted in the certified ABWR DCD, Revision 4, but are now redesigned to better comply with RG 1.97.

However, in Table 7.2-1 of the COL FSAR, the drywell high pressure was changed from 0 to 0.036 MpaG to -15.0 to 30.0 kPaG, but Table 7.5-2 in the COL FSAR does not show any change in the narrow range of 0.034 to 0.021 MpaG. However, the wide range was changed from 0–100 percent to 0–110 percent. Subsections 7.5.2.1(2)(b) in both the DCD and COL FSAR list the narrow range as -34.32 to +34.32 kPaG. This range does not match the range in Table 7.5-2 and makes the narrow range beyond the upper limit of the new range for the drywell high pressure.

The staff issued **RAI 07.05-1** requesting the applicant to provide sufficient information to clarify this inconsistency. The applicant’s response to this RAI (STPNOC Letter U7 C STP NRC-090118) provides a markup of the impacted FSAR sections that applicant’s response acceptable. However, because the proposed FSAR changes in the applicant’s response to **RAI 07.05-1** are not captured in the current Revision 3 of the STP Units 3 and 4 COL application, the incorporation of the proposed FSAR changes is being tracked as **Confirmatory Item 07.05-1**.

The staff’s review determined that the changes meet the requirements in the guidance of RG 1.97, BTP 7-10, and TMI-related criteria in 10 CFR 50.34. This departure is therefore acceptable.

Tier 2 Departures Not Requiring Prior NRC Approval

The following Tier 2 departures not requiring prior NRC approval identified by the applicant in this section may also be evaluated in other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these departures.

- STD DEP 11.5-1 Process and Effluent Radiation Monitoring and Sampling System

NRC staff reviewed this departure, which makes several changes in the process and effluent radiation monitoring and sampling system. The impacts from this departure on Section 7.5 of the FSAR are related to changes in Table 7.5-2. All changes in Table 7.5-2 associated with process and effluent radiation monitoring and sampling system were evaluated and found acceptable.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff determined that it is reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP Admin

This departure corrects the table numbers and typographical errors in FSAR Section 7.5.1.1, Subsection 7.5.2.1(1)(a), Tables 7.5-2, and 7.5-4. The applicant's evaluation determined that this departure does not require prior NRC approval, in accordance with 10 CFR Part 52, Appendix A, Section VIII.B.5. Administrative departures do not affect the presentation of any design discussion or the qualification of any design margin. Within the review scope of this section, NRC staff found it reasonable that this departure does not require prior NRC approval.

7.5.5 Post Combined License Activities

There are no post COL activities related to this section.

7.5.6 Conclusion

The staff's finding related to information incorporated by reference is in NUREG-1503. The staff reviewed the application and checked the referenced DCD. The staff's review confirmed that the applicant has addressed the required information relating to the information systems important to safety. With the exception of **Confirmatory Item 7.05-1**, no outstanding information is expected to be addressed in the COL FSAR related to this section. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the information systems important to safety that were incorporated by reference have been resolved.

The staff compared the application to the relevant NRC regulations; the acceptance criteria defined in NUREG-0800, Section 7.5; and other NRC RGs. The staff found that the applicant has adequately addressed the identified departures. However, as a result of **Confirmatory**

Item 7.05-1, the staff was unable to finalize the conclusions relating to the information systems important to safety, in accordance with the NRC requirements.

7.6 All Other Instrumentation Systems Required for Safety

7.6.1 Introduction

This section of the FSAR addresses those systems required for safety but not previously discussed in other sections of the certified ABWR DCD, although some aspects of these systems are included in previous sections of Chapter 7. This section examines and discusses the I&C aspects of the following plant systems:

- Neutron monitoring systems (SRNM, LPRM, and APRM)
- Process radiation monitoring system
- High-pressure/low-pressure interlock protection functions
- CAM system
- Suppression pool temperature monitoring (SPTM) system

7.6.2 Summary of Application

Section 7.6 of the STP Units 3 and 4 FSAR incorporates by reference Section 7.6 of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A, with the following departures and supplement:

Tier 1 Departures

- STD DEP T1 2.2-1 Control Systems Changes to Inputs, Tests, and Hardware

This departure changes the DCD Tier 1 ITAAC requirement for the RCIS related to the acceptance criteria associated with testing one dual-redundant, non-Class 1E uninterruptible power supply at a time.

- STD DEP T1 2.3-1 Deletion of MSIV Closure and Scram on High Radiation

This departure removes the MSIV automatic closure and scram based on the high MSLRM.

- STD DEP T1 2.14-1 Hydrogen Recombiner Requirements Elimination

This departure eliminate the hydrogen recombinder requirement to maintain equipment needed to mitigate a design-basis LOCA hydrogen release.

- STD DEP T1 3.4-1 Safety-related I&C Architecture

This departure changes I&C architecture and nomenclature that are needed to address obsolete data communication technology and the selection of DI&C platforms.

Tier 2* Departure

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

This departure updates some codes, standards, and RGs to more current revisions/editions.

Tier 2 Departure Not Requiring Prior NRC Approval

- STD DEP 1.1-2 Dual Units at STP Units 3 & 4

The certified ABWR DCD, Revision 4, is based on a single-unit site. STP Units 3 and 4 is a dual-unit project. This departure describes the common supporting systems shared by STP Units 3 and 4 and the existing Units 1 and 2.

- STD DEP 7.1-1 References to Setpoints and Allowable Values

This departure describes clarifications in the FSAR that wherever the TS is referenced for setpoints or margins, the correct reference is to the methods for calculating the setpoints and margins, as described in the TS Bases.

- STD DEP 7.1-2 ATWS DB for Startup Range Neutron Monitoring

This departure clarifies the power for the stepping motor driver modules and also adds as a general functional requirement under the safety DB that the SRNM and the APRM subsystems, respectively, will provide the ATWS permissive signals to the ELCS.

- STD DEP 7.6-1 Oscillation Power Range Monitor (OPRM) Logic

This departure clarifies that the OPRM logic system is independent from the APRM.

- STD DEP 7.6-2 SPTM Subsystem of Reactor Trip and Isolation System

This departure clarifies that the suppression pool temperature monitoring (SPTM) system is a subsystem of the RTIS.

- STD DEP 7.6-3 SPTM Sensor Arrangement

This departure provides additional clarification and detail regarding the location of the temperature sensors in the suppression pool.

- STD DEP 7.6-4 Range of Power Range Neutron Monitoring Operability

This departure clarifies that the power range neutron monitors (PRNM) provide information for monitoring the average power level of the reactor core and the local power level when the power range begins at approximately 5 percent of power.

- STD DEP 11.5-1 Process and Effluent Radiation Monitoring and Sampling System

This departure changes the process and effluent radiation monitoring and sampling system.

- STD DEP Admin

This departure describes minor corrections such as editorial or administrative errors in the certified ABWR DCD, Revision 4 (e.g., misspelled words, incorrect references, table headings, etc.).

COL License Information Item

- COL License Information Item 7.2 OPRM Oscillation Monitoring Logic

The applicant provides supplemental information to address COL License Information Item 7.2 from the certified ABWR DCD, Revision 4.

7.6.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG–1503. In addition, the relevant requirements of the Commission regulations for the all other instrumentation systems required for safety, and the associated acceptance criteria, are in Section 7.5 of NUREG-0800.

In accordance with Section VIII, “Processes for Changes and Departures,” of, “Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor,” the applicant identifies Tier 1, Tier 2*, and Tier 2 departures. Tier 1 and Tier 2* departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.A.4, and Section VIII.B.6, respectively. Tier 2 departures that do not require prior NRC approval are subject to the requirements of 10 CFR Part 52, Appendix Ai, Section VIII.B.5, which are similar to the requirements of 10 CFR 50.59.

Reviews of the departures are based on meeting the current regulatory requirements, acceptance criteria, and guidelines in NUREG–0800, Table 7-1 Revision 5 (March 2007). In accordance with 10 CFR 50.55a(h), COL applications filed on or after May 13, 1999, which depart from the certified ABWR DCD, Revision 4, must meet the requirements for safety systems in IEEE-Std 603-1991 and in the correction sheet dated January 30, 1995. In addition, applicable guidelines in DI&C-ISG documents are also used to evaluate the departures from the certified ABWR design.

7.6.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Section 7.6 of the certified ABWR DCD. The staff reviewed Section 7.6 of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of the information in the COL FSAR and the information in the ABWR DCD appropriately represents the complete scope of information relating to this review topic.¹ The staff’s review confirmed that the information in the application and the information incorporated by reference address the required information relating to this section.

The staff reviewed the information in the COL FSAR:

¹ See “*Finality of Referenced NRC Approvals*” in SER Section 1.1.3 for a discussion on the staff’s review related to verification of the scope of information to be included in a COL application that references a design certification.

Tier 1 Departures

The following Tier 1 departures identified by the applicant in this section require prior NRC approval and the full scope of their technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these Tier 1 departures.

- STD DEP T1 2.2-1 Control Systems Changes to Inputs, Tests, and Hardware

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

NRC staff reviewed this departure, which changes the DCD Tier 1 ITAAC requirement for the RCIS related to the acceptance criteria associated with one of the dual-redundant, non-Class 1E uninterruptible power supply at a time. Section 7.6 of the COL FSAR states that Table 7.6-5 and Figure 7.6-2 are impacted by Tier 1 departure STD DEP T1 2.2-1. According to the Departure Report in Part 7 of the application, this departure only impacts Tier 1 Section 2.2, and Tier 2 Section 7.7. The staff was unable to determine any correlation between this departure STD DEP T1 2.2-1 and the COL FSAR Section 7.6. The applicant was asked to resolve this discrepancy and provide the departure(s) that impact Table 7.6-5 and Figure 7.6-2. In response to RAI 07.06-3 (STPNOC Letter U7-C-STP-NRC-100104, dated May 05, 2010), the applicant clarified that the scope of Tier 1 departure STD DEP T1 2.2-1 had previously been changed and certain related COLA updates were overlooked, and the changes per this departure are now limited only to DCD Tier 1 ITAAC Table 2.2.1. Based on the current scope of Tier 1 departure STD DEP T1 2.2-1, the applicant has proposed to revise FSAR Tier 2 Section 7.6, Tables 7.6-5 and 21.0-1; Tier 1 Sections 2.2 and 2.2.7. This departure no longer applies to the FSAR Tier 2 Section 7.6, including all subsections, tables, and figures.

The staff found these proposed FSAR changes acceptable. Confirmation of these FSAR changes is being tracked as **Confirmatory Item 07.06-3**.

- STD DEP T1 2.3-1 Deletion of MSIV Closure and Scram on High Radiation

This departure modifies FSAR Section 7.6.1.2 and Figure 7.6.1.2 by downgrading the main steam line radiation monitoring subsystem as non-safety related. NRC staff found this change acceptable.

Refer to Section 7.2 of this SER for the detailed evaluation of this departure from the I&C perspective.

- STD DEP T1 2.14-1 Hydrogen Recombiner Requirements Elimination

This departure modifies FSAR Subsection 7.6.1.6 by downgrading the H₂/O₂ monitoring subsystem of the CAM system as non-safety related. NRC staff found this change acceptable.

Refer to Section 7.3 of this SER for the detailed evaluation of this departure from the I&C perspective.

- STD DEP T1 3.4-1

Safety-Related I&C Architecture

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

NRC staff reviewed this departure and found that it includes the following five primary changes to the I&C architecture of the certified ABWR DCD, Revision 4:

1. Elimination of obsolete data communication technology
2. Elimination of unnecessary inadvertent actuation prevention logic and equipment
3. Clarifications of digital controls nomenclature and systems
4. Final selection of platforms that changed the implementation architecture
5. Identification of testing and surveillance changes to the SSLC

Only changes in items 1, 3, and 4 are relevant to the certified ABWR DCD FSAR Section 7.6, Revision 4. Item 2 is related to the ELCS and is evaluated in Section 7.3 of this SER. Item 5 is related to both the RTIS and ELCS and is reviewed in both Sections 7.2 and 7.3 of this SER. The first change to the I&C architecture in the certified ABWR DCD replaces the EMS and the NEMS included as the communication protocol in the certified ABWR DCD, with separate and independent system level communication capabilities. This change impacts the interface with the NMS addressed in this section. However, the interface between the NMS and the RTIS or ELCS is via a separate one-way data communication. The unidirectional communication uses the fiber optic cables that provide electrical isolation.

The third change noted above to the I&C architecture in the certified ABWR DCD clarifies the use of digital controls nomenclature and systems associated with the above new data communication technology and selected platform. This third change replaces all references to the previous EMS- and non-NEMS related terminology and primary components with a generic data communication reference accordingly.

The fourth change is related to the platform change for the NMS, which the staff reviewed. The staff found it necessary to incorporate available platforms and to also meet both the regulatory and technical requirements.

The staff found that the above changes to the NMS in this section of the certified ABWR DCD do not depart from the certified ABWR design concepts and functional requirements for the safety systems. The staff also found that the changes meet the requirements in 10 CFR 50.55a(h) (IEEE-Std 603–1991), GDC 13, and DI&C-ISG-04. Therefore, this departure is acceptable.

Refer to Sections 7.1, 7.2, 7.3, and 7.9S of this SER for additional evaluations of this departure.

Tier 2* Departures

The following Tier 2* departures identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 2* departure.

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

NRC staff found that the applicant changes FSAR Subsection 7.6.2 by replacing IEEE-Std 279 with IEEE-Std 603–1991, which meets the requirements of 10 CFR 50.55a(h). The staff found this change acceptable.

Refer to Section 7.2 of this SER for the detailed evaluation of this departure from the I&C perspective.

Tier 2 Departure Not Requiring Prior NRC Approval

The following Tier 2 departures not requiring prior NRC approval identified by the applicant in this section may also be evaluated in other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these departures.

- STD DEP 1.1-2 Dual Units at STP Units 3&4

The certified ABWR design is for a single unit; the submitted COL application is for two units that do not share any systems addressed in this FSAR section. The applicant has changed FSAR Section 7.6.2 accordingly.

The applicant's evaluation determined that this departure does not require prior NRC approval in accordance with 10 CFR Part 52, Appendix A, Section VIII.B.5. Within the review scope of this section, NRC staff found it reasonable that this departure does not require prior NRC approval. The applicant's process for evaluating departures and other changes to the certified ABWR DCD is subject to NRC inspections.

Refer to Section 7.3 of this SER for an additional evaluation of this departure.

- STD DEP 7.1-1 References to Setpoints and Allowable Values

This departure modifies FSAR Section 7.6.1.3 by referring to the bases in COL application Chapter 16 for methods of calculating setpoints and margins, for the high pressure/low pressure systems interlock protection systems.

The applicant's evaluation determined that this departure does not require prior NRC approval in accordance with 10 CFR Part 52, Appendix A, Section VIII.B.5. Within the review scope of this section, NRC staff found it reasonable that this departure does not require prior NRC approval. The applicant's process for evaluating departures and other changes to the certified ABWR DCD is subject to NRC inspections.

Refer to Section 7.3 of this SER for the additional evaluation of this departure

- STD DEP 7.1-2 ATWS DB for Startup Range Neutron Monitoring

NRC staff reviewed this departure, which clarifies the power sources for the stepping motor driver modules and also adds, as a general functional requirement under the safety DB, that the SRNM and the APRM subsystems, will provide ATWS permissive signals to the ELCS. The

impacts of this departure on DCD FSAR Tier 2 Section 7.6 are related to the changes in Table 7.6-5, which are all associated with this departure and were reviewed and found acceptable.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP 7.6-1 Oscillation Power Range Monitor (OPRM) Logic

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval.

NRC staff reviewed the Departures Report regarding this departure, which clarifies the OPRM logic system reflecting that the OPRM is independent from the APRM. In COL application FSAR Tier 1 Revision 2, Figure 2.2.5 shows that the OPRM is part of the APRM system. However, Subsection 7.6.1.1.2.2 states that the APRM is independent from OPRM. The staff was unable to determine whether it is reasonable for this departure not to require prior NRC approval. Therefore, the staff issued **RAI 07.06-1** requesting the applicant to clarify this inconsistency.

The applicant's response to this RAI (STPNOC Letter U7-C-STP-NRC-090118) provides a markup of the impacted FSAR sections and revises this departure to clarify the inconsistency. In addition, the applicant implements the proposed FSAR changes in the current Revision 3 of the STP Units 3 and 4 COL application. The staff found the applicant's response to RAI 07.06-1 acceptable, and this RAI is closed.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. The staff conducted an audit of this Tier 2 departure and its evaluation process and found it reasonable that the departure does not require NRC approval.

- STD DEP 7.6-2 SPTM Subsystem of Reactor Trip and Isolation System

NRC staff reviewed this departure, which further clarifies and defines the functional design and implementation of the digital controls platforms. The SPTM system will not be impacted by adding this clarification. This proposed change will not have any impact on Tier 1, Tier 2*, TS, TS bases, or operational requirements.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP 7.6-3 SPTM Sensor Arrangement

NRC staff reviewed this departure, which further clarifies and illustrates the location of the temperature sensors in the suppression pool in relation to the SRVs. The departure states that the SRV discharge line quenchers are in direct sight of two sets of SPTM system temperature sensors.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP 7.6-4 Range of Power Range Neutron Monitoring Operability

NRC staff reviewed this departure, which corrects the bottom of the power range for the operation of the PRNM from 15 percent to its actual value of 5 percent. Consequently, this change is more favorable to and correctly reflects the actual design, which overlaps with the SRNM for neutron flux monitoring in the range of 5 percent to 15 percent of the power. Furthermore, the departure does not change any physical features of the plant or any SSCs important to safety or fission product barriers.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval. Within the review scope of this section, the staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD is subject to NRC audits and inspections.

- STD DEP 11.5-1 Process and Effluent Radiation Monitoring and Sampling System

NRC staff reviewed this departure, which changes the instrument engineering diagram in Figure 7.6-5 (sheets 1 to 10), "Process Radiation Monitoring System."

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per requirements in Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure did not require prior NRC approval. Within the review scope of this section, the staff determines that it is reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the certified DCD are subject to NRC audits and inspections.

Refer to Chapters 9, 11, and 12 of this SER for additional evaluations of this departure.

- STD DEP Admin

The applicant corrects a referenced table number in FSAR Subsection 7.6.1.1.3 and Figure 7.6-4a that corrects the title of the figure. The applicant's evaluation determined that this departure does not require prior NRC approval in accordance with 10 CFR Part 52, Appendix A,

Section VIII.B.5. Administrative departures do not affect the presentation of any design discussion or the qualification of any design margin. Within the review scope of this section, the staff found it reasonable that this departure does not require prior NRC approval..

COL License Information Item

- COL License Information Item 7.2 APRM Oscillation Monitoring Logic

The applicant had originally proposed to provide the required information as an amendment to the FSAR at least one year prior to fuel load (COM 4.4-3). The method proposed by the applicant was not an acceptable resolution. The applicant was asked to provide a method of resolution that will allow the NRC staff to reach safety conclusion. In response to RAI 07.06-2 (STPNOC Letter U7-C-STP-NRC-100104, dated May 05, 2010), the applicant will implement the APRM oscillation monitoring logic in accordance with the ABWR DCD without any departures, and will revise FSAR Tier 2 Section 7.6.3.1. Proposed FSAR changes are shown below:

The following standard supplement addresses COL License Information Item 7.2.

There are no departures from the fuel design licensing basis that are described in the reference ABWR DCD, including the core thermal hydraulic design described in Section 4.4. The APRM oscillation logic is ~~implemented based on the information described in Subsection 4.4~~ designed in accordance with the BWR Owners Group Stability Option III and described in Subsection 7.6.1.1.2.2.

The staff found these proposed FSAR changes acceptable. Confirmation of these FSAR changes is being tracked as **Confirmatory Item 07.06-2**.

7.6.5 Post Combined License Activities

There are no post COL activities related to this section.

7.6.6 Conclusion

As a result of **Confirmatory Items 07.06-2** and **07.06-3**, the staff is unable to finalize its conclusions relating to “All Other Instrumentation Systems Required for Safety” in accordance with the NRC requirements.

7.6S Interlock Systems Important to Safety

The title of NUREG-0800, Section 7.6 is “Interlock Systems Important to Safety.” However, there is no corresponding section in the certified ABWR DCD, Revision 4, to address interlock systems important to safety. The interlock systems that are important to safety are addressed in Subsections 7.4.1.3 (Items 7 and 11), 7.6.1.3, 7.6.2.3, and Table 7.4-1. This supplemental section in the COL application was created to provide consistency with the structure of NUREG-0800, Chapter 7. The COL application FSAR makes no change to the interlock logics included in the certified ABWR DCD, Revision 4 Sections 7.4.1.3, 7.6.1.3, 7.6.2.3, and Table 7.4-1.

Refer to Sections 7.4 and 7.6 of this SER for detailed evaluations of interlock systems important to safety.

7.7 Control Systems Not Required for Safety

7.7.1 Introduction

This section of the FSAR addresses control systems that the applicant does not consider essential for the safety of the plant. These systems primarily use DI&C equipment and transmit information via communication networks. Of the 16 systems listed in ABWR DCD Chapter 7, this SER section (Section 7.7) discusses the following 9 systems:

1. Nuclear Boiler System (NBS) – reactor vessel instrumentation
2. Rod Control and Information System (RCIS)
3. Recirculation Flow Control System (RFCS)
4. Feedwater Control System (FWCS)
5. Process Computer Function (PCF) – Performance and Monitoring Control System (PMCS) and Power Generation Control System (PGCS) subsystems
6. Neutron Monitoring System – ATIP and MRBM subsystems
7. Automatic Power Regulator (APR) System
8. Steam Bypass and Pressure Control System (SB&PC)
9. Plant Data Network (PDN)

Although these systems are not directly needed for the performance of safety functions, the operation of these systems is important to the reliability of the plant. These nonsafety-related systems are designed so that their failure will not prevent the proper operation of the safety systems. The nonsafety systems are also designed to be of a high quality, so as to minimize the challenges to safety system functions.

7.7.2 Summary of Application

Section 7.7 of the STP Units 3 and 4 COL FSAR incorporates by reference Section 7.7 of the certified ABWR DCD, Revision 4, which is referenced in 10 CFR Part 52, Appendix A, with the following departures listed by the applicant as affecting Section 7.7.

Tier 1 Departures

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

Departure STD DEP T1 3.4-1 revises the descriptions of the I&C systems that are divided into five primary types of changes. Two types of change describe changes applied to the control systems: elimination of obsolete data communication technology and clarification of digital controls nomenclature and systems.

Tier 2* Departure

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

Departure STD DEP 1.8-1 identifies Tier 2* items in Tier 2 Tables 1.8-20 and 1.8-21, which will be updated to more current revisions/standards.

Tier 2 Departures Requiring Prior NRC Approval

- STD DEP 7.7-10 Control Rod Drive System Interfaces

Departure STD DEP 7.7-10 implements revisions associated with the RCIS, the PMCS, interfaces with the CRDS, and certain related TS Bases.

- STD DEP 7.7-18 RCIS Operator Information

Departure STD DEP 7.7-18 implements revisions associated with the RCIS reactor operator interface functions including annunciators, status information, and operator controls.

- STD DEP 8.3-1 Plant Medium Voltage Electrical Design

Departure STD DEP 8.3-1 changes the medium-voltage electrical distribution system from a single voltage system of 6.9 kV to a dual voltage system consisting of 13.8 kV and 4.16 kV.

- STD DEP 10.4-5 Condensate and Feedwater System

Departure STD DEP 10.4-5 changes the TS 3.3.4.2 Bases to show that there are four feedwater pumps with four feedwater pump adjustable speed drives (ASDs), while the referenced ABWR DCD specifies two feedwater pump ASDs.

Tier 2 Departures Not Requiring Prior NRC Approval

- STD DEP 5.4-1 Reactor Water Cleanup System

Departure STD DEP 5.4-1 changes to the reactor water cleanup system include increasing the flow capacity of the two pumps and two filter demineralizers and increasing the design pressure of the pumps and heat exchangers.

- STD DEP 7.7-1 RPV Water Level Instrumentation

Departure STD DEP 7.7-1 clarifies that only those RPV water level instrument lines with a condensing chamber can have entrained non-condensable gases and thus, they will be the only ones continually flushed with water supplied by the CRDS.

- STD DEP 7.7-2 SRV Discharge Pipe Temperature Data Recording

Departure STD DEP 7.7-2 implements a design upgrade to replace the multipoint recorders with a more accurate and more current technology described as a historian function digital system.

- STD DEP 7.7-3 Feedwater Turbidity

Departure STD DEP 7.7-3 removes the feedwater turbidity monitoring subsystem.

- STD DEP 7.7-4 APR/RCIS Interface

Departure STD DEP 7.7-4 clarifies that the APR system is actually the direct controlling system that interfaces with the RCIS to accomplishing the automatic rod movement mode, and the PGCS interfaces only with the APR system to initiate various reactor power change control tasks.

- STD DEP 7.7-5 RCIS Display

Departure STD DEP 7.7-5 clarifies the wording of the referenced ABWR DCD by providing more precise information about the available display at the RCIS dedicated operator interface on the main control panel.

- STD DEP 7.7-6 RCIS Commands

Departure STD DEP 7.7-6 clarifies that redundant command signals (more than a single signal) are sent from the RFCS to the RCIS for the ARI function, and redundant command signals (more than a single signal) are sent from the RFCS to the RCIS for the selected control rod run-in (SCRRI) function.

- STD DEP 7.7-7 RCIS Design Details

Departure STD DEP 7.7-7 implements changes to the RCIS descriptions in the FSAR to provide clarity, additional information, and a more complete design description.

- STD DEP 7.7-9 Selected Control Rod Run-In Function

Departure STD DEP 7.7-9 clarifies that the control rod drive (CRD) system also mitigates a loss of feedwater heating event. The departure adds information and a more complete design description showing the two functional needs for the SCRRI.

- STD DEP 7.7-11 Rod Withdrawal Sequence Restrictions

Departure STD DEP 7.7-11 significantly expands the DCD discussion of the ganged rod movement and ganged withdrawal sequence restrictions.

- STD DEP 7.7-12 RCIS Indication

Departure STD DEP 7.7-12 updates the discussion of the detailed design of the referenced rod pull sequence (RRPS).

- STD DEP 7.7-13 Optical Isolation

Departure STD DEP 7.7-13 removes the detailed description of the specific type of technology used for the optical isolation of rod block signals received by the nonsafety RCIS from Class 1E systems.

- STD DEP 7.7-14 RCIS Bypass

Departure STD DEP 7.7-14 implements and updates changes in the discussion of the RCIS bypass capabilities and provides a clearer and more complete description of the design and operation of the RCIS system.

- STD DEP 7.7-20 Recirculation Flow Control Logic

Departure STD DEP 7.7-20 provides a more complete description by stating that the 70 percent limit is for a “rod pattern where rated power accompanies 100% flow” and provides further information concerning manual and automatic operation for other rod patterns and power levels.

- STD DEP 7.7-22 ATLM Description

Departure STD DEP 7.7-22 expands the discussion and description of the ATLM setpoint and rod block action and the interface of interacting systems.

- STD DEP 7.7-23 ATIP Function

Departure STD DEP 7.7-23 discusses inputs from the “automatic fixed incore probe (AFIP)” to be used for gain adjustment factors for local power range monitoring (LPRM) and explains that this function is provided by the automated traversing incore probe (ATIP) rather than the AFIP.

- STD DEP 7.7-24 SB&PC Interfaces

Departure STD DEP 7.7-24 clarifies and corrects the description of the I&C interfaces for the SB&PC System.

- STD DEP 7.7-27 RCIS Table Deletion

Departure STD DEP 7.7-27 removes Table 7.7-1 because it is unnecessary.

- STD DEP 9.5-3 System Description – Reactor Internal Pump Motor-Generator Sets

Departure STD DEP 9.5-3 implements several changes to the technical description of the nonsafety-related motor-generator (MG) set equipment, which provides power to reactor internal pumps (RIPs) to clarify the original DCD technical descriptions or to reflect changes in the actual equipment design implementation details that have evolved since the original DCD descriptions were written.

- STD DEP Admin

Departure STD DEP Admin implements administrative departures that are defined as minor corrections, such as editorial or administrative errors in the referenced ABWR DCD (e.g., misspelled words, incorrect references, table headings, etc.).

7.7.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is documented in NUREG-1503. In addition, the relevant requirements of the Commission regulations for the control systems not required for safety, and the associated acceptance criteria, are in Section 7.7 of NUREG-0800.

In accordance with Section VIII, “Processes and Changes and Departures,” of, “Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor,” the applicant identifies Tier 1, Tier 2*, and Tier 2 departures. Tier 1 and Tier 2* departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.A.4, and Section VIII.B.6, respectively. Tier 2 departures that affect TS or TS Bases require prior

NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.C.4. Tier 2 departures that do not require prior NRC approval are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.5, which are similar to the requirements of 10 CFR 50.59.

Reviews of the departures are based on meeting the current regulatory requirements, acceptance criteria, and guidance in NUREG–800 Table 7-1, Revision 5 (March 2007).

7.7.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Section 7.7 of the DCD for the ABWR design. The staff reviewed Section 7.7 of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of the information in the COL FSAR and the information in the ABWR DCD appropriately represents the complete scope of information relating to this review topic.¹ The staff's review confirmed that the information in the application and the information incorporated by reference address the required information relating to this section.

The staff reviewed the information in the COL FSAR:

Tier 1 Departure

The following Tier 1 departure identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 1 departure.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

Departure STD DEP T1 3.4-1 is required because of five primary types of changes in the I&C architecture:

1. Elimination of obsolete data communication technology
2. Elimination of unnecessary inadvertent actuation prevention logic and equipment
3. Clarification of digital controls nomenclature and systems
4. Final selection of platforms that changed the implementation architecture
5. Testing and surveillance changes for safety system logic & control (SSLC)

Two of these changes specifically apply to departures in the FSAR Section 7.7: type (1), "Elimination of obsolete data communication technology," and type (3), "Clarifications of digital controls nomenclature and systems." This FSAR Tier 1 departure is evaluated in other sections of Chapter 7. However, the departure accounts for a number of changes in terminology affecting various sections in Section 7.7, including Sections 7.7.1.2, 7.7.1.4, 7.7.1.5, 7.7.1.7, 7.7.2.5, and 7.7.2.9. For example, a type (1) change eliminates references to the NEMS originally planned for the ABWR architecture and replaces the NEMS with independent system level data communication capabilities. A type (3) change renames the "Process Computer System" with the "Plant Computer Function."

¹ See "Finality of Referenced NRC Approvals" in SER Section 1.1.3, for a discussion on the staff's review related to verification of the scope of information to be included in a COL application that references a design certification.

NRC staff found the type (1) change in FSAR Section 7.7 acceptable, because the elimination of the multiplexer concept required all related references to be replaced with generic data communication terms that represent current and improved technology without a significant decrease in the level of safety provided by the original ABWR design. This “standard” departure is intended to be applicable to COL applicants that reference the ABWR DCD, so this departure will not result in any loss of standardization.

The staff found the type (3) change in FSAR Section 7.7 acceptable because the referenced ABWR DCD defined many functional design requirements in terms typically reserved for hardware. The terminology is corrected to refer to the requirement as a “function,” so as to eliminate confusion between purely functional requirements and physical requirements defined in the DCD. The revised terminology better defines the design and implementation with current DI&C technology and control platforms. Again, the use of these revised terms better represents the current and improved technology without a significant decrease in the level of safety the original ABWR design provides. For technical and regulatory compliance evaluations of this departure, refer to Sections 7.2, 7.3, and 7.9S of this SER.

Tier 2* Departure

The following Tier 2* departure identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 2* departure.

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

DCD Tier 2, Table 1.8-20 indicates how the referenced ABWR DCD compliance with NRC RGs, and DCD Tier 2 Table 1.8-21 lists the applicability of industry codes and standards. This departure identifies Tier 2* items in these two tables that are being updated to more current revisions/editions. IEEE-Std 603–1991, “Standard Criteria for Safety Systems for Nuclear Generating Stations,” was updated to the 1991 version. In DCD Tier 2 Subsection 7.7.2.1.2, IEEE-Std 279 was replaced with IEEE-Std 603–1991. NRC staff found this change acceptable because IEEE-Std.603–1991, as amended, is incorporated by reference in 10 CFR 50.55a(h)(2) and is the edition of the standard currently endorsed by the NRC. Hence, the staff determined that this departure is acceptable from the I&C perspective.

Tier 2 Departures Requiring Prior NRC Approval

The following Tier 2 departures identified by the applicant in this section require prior NRC approval and the full scope of their technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these Tier 2 departures.

- STD DEP 7.7-10 Control Rod Drive System Interfaces

The applicant lists this departure as requiring prior NRC approval and is requesting NRC approval of this proposed departure. Subsection 7.7.1.2.1 of the ABWR DCD provides the RCIS interfaces with the “CRD Control System for Single Rod Movement” in Subsection 7.7.1.2.1(1), for “Withdrawal Cycle” in Subsection 7.7.1.2.1(2), for “Insert Cycle” in Subsection

7.7.1.2.1(3), and for “Ganged Rod Motion” in Subsection 7.7.1.2.1(4). STD DEP 7.7-10 implements the following revisions in the rod movement subsections noted above:

Replaces the PMCS normal operational manual mode CRT display with the RCIS dedicated operator interface (DOI) on the main control room panel. DOI is simply an alternate technical means of achieving the display and control functions and interfacing with the RCIS. The DOI is part of the design utilizing recent DI&C technology.

Revises the description of allowed operator single rod movement manual commands in Subsection 7.7.1.2.1(1) and changes the subsection name from “Introduction” to “Single Rod Movement.” The ABWR DCD, identifies four rod movement commands as “SINGLE ROD,” “ROD GANG,” “STEP,” and “CONTINUOUS,” as well as commands “IN” and “OUT.”. The applicant revises this to three rod movement commands: as “STEP,” “NOTCH,” and “CONTINUOUS.” that serve as means to initiate all rod movement. In place of the “IN” and “OUT,” “insert” and “withdraw” movement commands are used by activating associated hard push button switches located adjacent to the RCIS.DOI on the main control panel.

Changes the title of Subsection 7.7.1.2(2) to “Withdrawal Cycle” and provides a more clear and complete description of the withdrawal cycle including defining nominal rod distance movement in millimeters for a “STEP,” “NOTCH,” and “CONTINUOUS” control rod movement and the full-in and full-out positions. This departure defines the “STEP” (18.3 mm) and the “NOTCH” (73.2 mm; 4 times the nominal step movement distance).

Changes the title of Subsection 7.7.1.2(3) to “Insert Cycle” and provides a description of similarities and differences from the “withdrawal” cycle.

Changes the title of Section 7.7.1.2.1(4) to “Ganged Rod Movement” and improves the description of the ganged rod movement. The improved descriptions represent a more flexible alternate but acceptable method of control rod movement.

Adds a discussion of the rod action and position information (RAPI) rod block operations. The description states that during all of the operator selections for control rod withdrawal there is continuous monitoring of the selection and movement by the Rod Action and Position Information (RAPI) function of the RCIS that enforces the rod block function. Input signals to the RAPI that would cause the RAPI to prevent undesired control movement come from both internal to the RAPI and external sources. The rod block function applies in both single and ganged rod movement.

Revises the description of ganged rod motion interface.

Revises the specific method described in TS Bases for Subsection B 3.9.4 using the synchros to verify rod full-in position. The FMCRD system provides the ABWR operator the ability to move the control rods in small increments

compared to the current BWR plants that use a hydraulic method of rod movements of 6 inch “steps.” In the fully inserted position, the synchros information provides an accurate indication of the control rod when it is fully inserted.

The applicant evaluated the proposed departure STD DEP 7.7-10 and determined that it complies with the requirements in Section VIII.C.4 of Appendix A to 10 CFR Part 52.

In this departure, the “Description” describes a change to the TS Bases for Subsection B 3.9.4. However, the “Evaluation Summary” discusses the TS Bases for B 3.4.3. NRC staff issued **RAI-7.07-9** requesting the applicant to correct this discrepancy and provide sufficient justification for NRC approval of the departure. The applicant’s response revises and corrects the departure by specifically referencing the bases of TS 3.9.4. The response adequately addresses **RAI 7.07-9**; this RAI is closed.

Further, the departure “Description” did not sufficiently identify all the changes included in Subsection 7.7.1.2.1. NRC staff issued **RAI-7.07-10** requesting the applicant to identify all the changes to Subsection 7.7.1.2.1, parts (1), (2), (3), and (4). The applicant’s response revises the departure “Description” to sufficiently identify the changes, and improves the FSAR Subsection 7.7.1.2.1, parts (1), (2), (3), and (4) with more clear and complete descriptions of the rod movement control topics and improves subsection names. The justification for the departure in the departure “Evaluation Summary” is improved to be an acceptable justification for the departure. The response adequately addresses **RAI 7.07-10**; and this RAI is **Confirmatory Item 07.07-10**.

FSAR Section 1.8 of the certified ABWR DCD, Revision 4, identifies conformance with the SRP and applicability of codes and standards. ABWR DCD FSAR Tier 2, Table 1.8-19 indicates conformance with SRP Section 7.7 of NUREG–0800 (Revision 3, February 1984); Appendix 7-A (Revision 2, July 1981); and BTP ICSB 14 (Revision 2, July 1981). FSAR Tier 2 Section 1.8 of the STP Unit 3 and 4 COL application states that the information in this section of the ABWR DCD, including all subsections and tables, is incorporated by reference with certain departures, with a standard supplement, and with a site-specific supplement. No departure is taken from Table 1.8-19. Revisions to Table 1.8-21 state that STP Unit 3 and 4 FSAR Tier 2 conforms to IEEE-Std 384–1992 and IEEE-Std 603-1991. Therefore, based on the above information and a review of the changes of this departure as listed, the NRC staff found that STD DEP 7.7-10 has adequately addressed the requirements of SRP Section 7.7 of NUREG–0800. Therefore, the staff found that the changes in departure STD DEP 7.7-10 are acceptable.

- STD DEP 7.7-18 RCIS Operator Information

The applicant lists this departure as requiring prior NRC approval and is requesting NRC approval of this proposed departure. STD DEP 7.7-18 implements the following revisions in Subsection 7.7.1.2.3 of the ABWR DCD:

Revises the list of RCIS annunciation activations at the main control panel to be consistent with the current RCIS design.

Revises the list of RCIS status information on the RCIS DOI on the main control panel is revised to be consistent with the current RCIS design.

Revises the list of RCIS operator functions allowed through the RCIS DOI panel and related RCIS displays, indications, and associated controls on the main control room panel and on the RCIS cabinets and panels to be consistent with the current RCIS design.

Revises the list of RCIS-related information displayed for the operator by main control room equipment other than the RCIS DOI to be consistent with the current RCIS design.

This departure allows the use of current and improved digital technology and related features in the design, such as the application of fiber optic cables that provide electrical isolation and help protect the I&C electronics from electromagnetic Interference.

In addition to the changes to FSAR Tier 2 Subsection 7.7.1.2.3 of the ABWR DCD described in the four revisions noted above, STD DEP 7.7-18 implements revisions to Subsections B 3.9.3, B 3.10.3, B 3.10.4, and B 3.10.5 of the TS Bases to revise the manner in which the RCIS is placed in the scram test mode. In these TS Bases, the DCD refers to the use of an RCIS "Rod Test Switch" to allow two control rods to be withdrawn for scram testing. In the current RCIS design, this "Rod Test Switch" is placed in the scram test mode through the use of the RCIS DOI panel to accomplish the same function and intent (i.e., the switch is now a touch panel button). Thus, in Bases B 3.10.3, "Background," B 3.10.4, "Background," and "Applicable Safety Analyses," and B 3.10.5, "Background," and "Applicable Safety Analyses" of Section 16.0, "Technical Specifications," of the DCD; the term "Rod Test Switch" is replaced with the term "RCIS scram test mode"; and in Subsections B 3.9.3, "Background," and "Applicable Safety Analyses," the term "Rod Test Switch" is replaced with the phrase "is placed in the scram test mode." This proposed change consists only of rewording the phrase for purposes of clarification and editorial correction, with no change to the meaning or intent of the original TS Bases.

The applicant evaluated the proposed exemption in Departure STD DEP 7.7-18 and determined that it complies with the requirements in Section VIII.C.4 of Appendix A to 10 CFR Part 52.

However, in TS Bases B 3.10.4, "LCO," "STD DEP 7.7-18," is listed as one of the departures describing the change. But "STD DEP 7.7-18" does not sufficiently identify this change. NRC staff issued **RAI 7.07-8** requesting the applicant to clarify and sufficiently justify this apparent discrepancy before the staff can accept this departure. The applicant's response, revises B 3.10.3 and replaces "rod test switch" with "RCIS scram test mode." The applicant has adequately addressed **RAI 7.07-8**, therefore, this RAI is closed.

In FSAR Section 1.8 of the certified ABWR DCD, Revision 4 identifies conformance with the SRP and applicability of codes and standards. In ABWR DCD FSAR Tier 2, Table 1.8-19 indicates conformance with SRP Section 7.7 of NUREG-0800 (Revision 3, February 1984); Appendix 7-A (Revision 2, July 1981); and BTP ICSB 14 (Revision 2 July 1981). FSAR Tier 2 Section 1.8 of the STP Unit 3 and 4 COL application states that the information in this section of the ABWR DCD, including all subsections and tables, is incorporated by reference with certain departures, a standard supplement, and a site-specific supplement. No departure is taken from Table 1.8-19. Revisions to Table 1.8-21 state that STP Unit 3 and 4 FSAR Tier 2 conforms to IEEE-Std 384-1992 and IEEE-Std 603-1991. Therefore, based on the above information and a

review of the changes in this departure as listed, the staff found that STD DEP 7.7-18 has adequately addressed the requirements of SRP Section 7.7 of NUREG-0800.

- STD DEP 8.3-1 Plant Medium Voltage Electrical Design

This departure changes the medium-voltage electrical distribution of the ABWR DCD from a single 6.9 kV system to a dual voltage consisting of 13.8 kV and 4.16 kV. The applicant states that this departure does not affect any Tier 1 or Tier 2* DCD, but the design change results in changes to TS and TS Bases. This departure requires a change in the TS and TS Bases and therefore requires NRC approval.

This departure is evaluated in Chapters 8 and Chapter 16 of this SER.

- STD DEP 10.4-5 Condensate and Feedwater System

The referenced ABWR DCD specifies two feedwater pump ASDs. TS 3.3.4.2 Bases is revised to show that there are four feedwater pumps and four ASDs. The control and logic IED and the IBD in Figures 7.7-7, 7.7-8, and 7.7-9 reflect the additional I&C for the FWCS and RFCS. NRC staff noted that the "Description" section in this departure contains a table that compares the major components of the condensate and feedwater system between the referenced ABWR DCD and the referenced ABWR DCD with departures. The staff determined that this departure is acceptable from the I&C perspective, because upgrading the design to one ASD per feedwater pump is considered an improvement. This departure is evaluated in Chapter 10 of this SER.

Tier 2 Departure Not Requiring Prior NRC Approval

The following Tier 2 departures not requiring prior NRC approval identified by the applicant in this section may also be evaluated in other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these departures.

- STD DEP 5.4-1 Reactor Water Cleanup System (Figures 7.7-8 and 7.7-9)

The flow capacity of the two pumps and two filter demineralizers in the reactor water cleanup system is doubled from 1 percent of the rated feedwater flow to 2 percent. This change will improve system maintainability and availability by allowing only one of the two redundant pumps to handle the full cleanup flow and filtering functional requirements. This departure is also addressed in Chapter 5 of this SER.

The applicant's evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require prior NRC approval. The applicant's process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-1 RPV Water Level Instrumentation

Subsection 7.7.1.1(6) adds a clarification that only those instrument lines with a condensing chamber are flushed and identifies the reactor vessel instrumentation with condensing pots. The concern of noncondensable gas build-up in the water column in the reactor vessel

referenced leg water level instrument lines (i.e., the reactor vessel instrument lines at the elevation near the main steam line nozzles), is addressed by continually flushing these instrument lines with water supplied by the CRDS for those instrument lines with a condensing chamber. The applicant adds that this clarification applies to items “(a) through (e)” of DCD Subsection 7.1.1.1(6). The original design intent was for flushing to only apply to lines with condensing chambers, which was not clear in the original DCD. This departure provides this clarification.

The applicant’s evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR 52 determined that this departure does not require prior NRC approval. However, DCD Subsection 7.1.1.1(6) “(e)” represents the “Reactor Well Water Level Range.” This range is used to monitor the reactor water level when the reactor vessel head is removed and the reactor system is flooded during refueling, with a calibration at 0 MPaG and 48.9 °C. The lower point uses the RPV tap below the top of the active fuel. The upper point is far above the RPV. NRC staff was unable to determine whether it is reasonable for this departure not to require prior NRC approval. The staff questioned whether this reactor well water level range instrument actually could have a condensing chamber, as stated in the departure. Therefore, the staff issued **RAI 7.07-7** requesting the applicant to verify that this water level instrument actually has a condensing chamber.

The applicant’s response to **RAI 7.07-7** (STPNOC Letter U7-C-STP-NRC-090157, dated September 22, 2009) states that the reactor well water level range instrument does not have a condensing chamber and “e” is a typographical error that should be replaced with “d.” The applicant has corrected this error in Revision 3 of the COL FSAR. The staff found this correction acceptable and **RAI 7.07-7** is closed.

The NRC staff finds it is reasonable that the departure does not require NRC approval. The applicant’s process for evaluating departures and other changes to the DCD are subject to NRC inspections.

- STD DEP 7.7-2 SRV Discharge Pipe Temperature Data Recording

In Subsections 7.7.1.1(10) and (15), the term “historian function” replaces technologically specific terms such as “multipoint recorder” or “process computer.” Often, specific terms from the ABWR DCD are too specific or too old or the instrumentation itself is obsolete. The use of the term “historian function” assures that the data recording function is still accomplished, without specifying the equipment that is now obsolete or that no longer accurately fits the description.

The applicant’s evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require prior NRC approval. The applicant’s process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-3 Feedwater Turbidity

In Subsection 7.7.1.1(15), “Reactor Operator Information,” part (g) concerns the recording of feedwater turbidity in the main control room and an associated high-alarm annunciation. This departure states that because there is no practical manner for performing this measurement,

and because measuring turbidity is not considered to have any safety significance, this (g) subsection and a related reference to the feedwater corrosion product (turbidity) monitor in Subsection 7.7.1.1(16) will be deleted.

The applicant's evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR 52 determined that this departure does not require prior NRC approval. The staff conducted an audit on this Tier 2 departure and its evaluation process and found it reasonable that the departure does not require prior NRC approval. For additional information on this audit, see NRC Audit Report dated January 25, 2010 (ADAMS access number ML093360537)

- STD DEP 7.7-4 Automatic Power Regulator/Rod Control and Information System Interface

Section 7.7.1.2 (1)(a)(ii) of the referenced ABWR DCD describes the PGCS as initiating control changes in the automatic rod movement mode. The STP Units 3 and 4 FSAR clarifies and corrects the description to state that the APR is actually the direct controlling system that interfaces with the RCIS to accomplish the automatic rod movement mode, and the PGCS interfaces only with the APR to initiate various reactor power change control tasks.

The reactor power change algorithms are implemented in the APR. Section 7.7.1.2 was updated to be consistent with this DCD description, which reflects the proper role of the APR and PGCS. This departure is a clarification only and does not affect the design or functionality of any SSC important to safety.

The applicant's evaluation in accordance with item B.5 of Section VIII of Appendix A of 10 CFR 52 determined that this departure did not require prior NRC approval. Within the review scope of this section, the NRC staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-5 Rod Control and Information System Display

In COL FSAR Subsection 7.7.1.2 (1) (b), STPNOC clarifies the wording and terminology used to identify more precisely the display information available to the operator from the RCIS through the RCIS DOI on the main control panel. This design does not affect the design or function of any SSC important to safety. This departure simply provides a more complete design description.

The applicant's evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it is reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-6 Rod Control and Information System Commands

ABWR DCD Tier 2 Subsection 7.7.1.2 (1) (f) describes the command "signal" from the RFCS to the RCIS for the ARI, and Subsection 7.7.1.2 (1) (g) describes the command "signal" from the RFCS to the SCRRRI. The description of the initiation was changed to reflect that the design involves "signals." The three channels of the RFCS (triple redundant) provide each of the two channels of the RCIS logic (dual redundant) with the ARI and SCRRRI signals. RCIS internal

logic to initiate the RCIS, ARI, and SCRRRI functions is based on two-out-of-three logic within each channel of the RCIS. Consequently, initiation of the ARI and SCRRRI functions is based on multiple signals from the RFCS.

This departure has no negative impact on the logic for the initiation of the ARI or SCRRRI from the RFCS. Nor does the departure have any negative effects on any design or function of an SSC important to safety.

The applicant's evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require prior NRC approval. The applicant's process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-7 RCIS Design Details

This departure changes the referenced ABWR DCD RCIS descriptions in FSAR Subsections 7.7.1.2(2), (3), (5), and (6) to provide clarity, additional information, and a more complete design description of the RCIS. This departure lists seven changes that apply to Subsection 7.7.1.2 and to other subsections. These changes include an expanded description of the RCIS monitoring channels, extensive information on the cabinet arrangement as to what functions are located in which cabinet, descriptions of additional RCIS-related panels and cabinets to be consistent with Figure 7.7-2 and various major RCIS subsystem functions, final remote communication cabinet (RCC) implantation details, final fine motion driver cabinet (FMDC) implementation details, and detailed descriptions of the RCIS multiplexing network information and interfaces with Class 1E systems.

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval.

NRC staff reviewed the Departures Report regarding this departure, which indicates changes to Item (5) in Subsection 7.7.1.2 of the ABWR DCD describing power sources. However, there are no changes to Item (5) shown in STP Units 3 and 4 FSAR Subsection 7.7.1.2.(5) for Subsection 7.7.1.2.(5) of the DCD. The staff was unable to determine whether it is reasonable for this departure not to require prior NRC approval. Therefore, the staff issued **RAI 7.07-6** requesting the applicant to clarify and correct this discrepancy. In one of the seven types of changes in this departure and in Item (3), "RCIC Multiplexing Network," the "multiplexing" term is used. Departure STD DEP T1 3.4-1 states in Item (1) of the five primary changes that "The elimination of the multiplexer concept required all references to the system(s) and primary components to be replaced with a generic data communication reference." The staff issued **RAI 7.07-3** requesting the applicant to clarify this apparent discrepancy.

The applicant's response to **RAI 7.07-6** (STPNOC Letter U7-C-STP-NRC-090118, dated August 27, 2009) deletes "(5)" in 7.7.1.2 from the list of subsections affected by this departure. The staff found this response acceptable and **RAI 7.07-06** is closed. In response to **RAI 7.07-3** (STPNOC Letter U7-C-STP-NRC-090118, dated August 27, 2009), the applicant clarifies that although the description of and justification for STD DEP T1 3.4-1 identifies Item (1), the elimination of the large multiplex system architecture of the "Essential Multiplexer System (EMS) and the Non-Essential Multiplexer System (NEMS)" originally envisioned in the ABWR

architecture, a dedicated RCIS multiplex communication configuration remains both a proven and functional part of the system and no changes are required. The staff found this response acceptable. The applicant adequately addresses **RAI 7.07-3**, and this RAI is therefore closed.

The NRC staff finds it is reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the DCD are subject to NRC inspections.

- STD DEP 7.7-9 Selected Control Rod Run-In Function

This departure states, "Subsection 7.7.1.2(6) of the reference ABWR DCD states that the Control Rod Drive (CRD) System provides for electromechanical insertion of selected control rods for core thermal/hydraulic stability control."

In COL application Part 7, "Departures Report," Section 3.0, the applicant describes and evaluates this departure per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. The applicant's evaluation determined that this departure does not require prior NRC approval.

NRC staff reviewed the Departures Report regarding this departure, which adds in the STP Units 3 and 4 FSAR "the CRD system also provides for mitigation of a loss of feedwater heating event showing the two functional needs for SCRRI." This function of the SCRRI is already detailed in Subsections 7.7.1.2(1)(g) and 7.7.1.2.2 (2) (b) of the DCD. However, DCD Subsection 7.7.1.2(6) is titled "(6) RCIS Scope," and discusses equipment included in the scope but does not discuss the function of the CRD system, as claimed. The staff was unable to determine whether it is reasonable for this departure not to require prior NRC approval. Therefore, the staff issued **RAI 7.07-4** requesting the applicant to clarify and correct the discrepancy between the DCD and this departure.

In response to **RAI 7.07-4** (STPNOC Letter U7-C-STP-NRC-090118, dated August 27, 2009), the applicant states that the discrepancy is an editorial error and inserts the missing heading: "(7) Integral Functional Design" in the departures report. The staff found that this response adequately addresses **RAI 7.07-4**, and this RAI is closed.

The applicant's evaluation in accordance with item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure did not require prior NRC approval. The NRC staff finds it is reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the DCD are subject to NRC inspections.

- STD DEP 7.7-11 Rod Withdrawal Sequence Restrictions

In this departure, STP Units 3 and 4 FSAR Subsection 7.7.1.2.1 (5) provides additional descriptions and clarifications that expand the DCD discussion of the ganged rod movement and ganged withdrawal sequence restrictions, including notable examples such as the following:

- Item (5)(a) provides additional details on the ganged rod mode that describe the "checker board" pattern following the withdrawal of all group 1, 2, 3, and 4 control rods.

- In Item (5)(b), the departure states that the system allows up to 26 rod gangs (for control rods in rod groups 1, 2, 3, and 4) to be withdrawn simultaneously when in the startup or run mode, under the stated restrictions.
- In Item (5)(b)(vi), the departure revises the maximum allowable difference in rod positions between the leading and trailing operable control rods.
- In Item (5)(b), the departure revises the restrictions on the withdrawal of rods in groups.
- In the "Introduction" under Item (5), the phrase indicating that the RWM of the RCIS ensures adherence to certain ganged withdrawal sequence restrictions was revised to remove the condition that the RWM restriction functions only below the low power setpoint (LPSP) and the reactor mode switch is in startup or run mode, but these are included in list of restrictions for the RCIC that follows.

The departure changes cited above provide more complete details describing the ganged rod withdrawal restrictions. The departure changes Subsections 7.7.1.2.1(5)(b)(vi)(c) and (d) so that the maximum allowable difference between operable control rods in certain groups is revised from 146 mm to 152 mm with certain restrictions. The applicant states in the evaluation that the basic functioning of the RWM of the RCIS to ensure that there are restrictions on certain ganged control rod movements is unchanged,

The applicant's evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-12 Rod Control and Information System Indication

This departure updates the discussion of the detailed design of the reference rod pull sequence (RRPS) in STP Units 3 and 4 FSAR Subsection 7.7.1.2.1 (6). This departure describes four clarifying changes that are primarily editorial in nature. For example, it clarifies that the PCF and not the PMCS is used for storing, modifying, and providing compliance verification for the RRPS.

The applicant's evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require prior NRC approval. The applicant's process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-13 Optical Isolation

This departure removes overly restrictive optical isolation information from the STP Units 3 and 4 FSAR Subsection 7.7.1.2.1(7) discussion of the rod block function. This change removes the detailed description of the specific type of technology used for optical isolation of rod block signals received by the nonsafety-related RCIS from Class 1E systems. The referenced ABWR DCD wording discusses the details of a specific technology.

The applicant's evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-14 Rod Control and Information System Bypass

This departure discusses five changes in STP Units 3 and 4 FSAR Subsection 7.7.1.2.1 concerning the RCIS bypass capabilities. For example, changes in the descriptions regarding the specific location and related operator interface, where specific bypass functions can be performed (e.g., update of control rods to be placed in the "Inoperable" status can be performed at the RCIS DOI and descriptions of the bypass permissive switch for performing certain bypass operations are added for clarity). The applicant states that these changes are a result of RCIS design evolution based on experience at operating plants and involve an enhanced discussion of RCIS bypass capabilities. The applicant also notes that "this departure is not the result of any underlying design change and functional requirements of the RCIS are unchanged."

The applicant's evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require prior NRC approval. The applicant's process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-20 Recirculation Flow Control Logic

Subsection 7.7.1.3 (1) of the referenced ABWR DCD describes the automatic operation of the RFCS as being available only at power levels above 70 percent.

STD DEP 7.7-20 implements the following revisions to Subsection 7.7.1.3(4) of the ABWR DCD:

- Provides a more complete description by stating that the 70 percent limit is for a "rod pattern where rated power accompanies 100 percent flow." Provides further information concerning manual and automatic operation for other rod patterns and power levels. Therefore, the statement "if the power level is above 70% rated" is removed from 7.7.1.3(1).
- Describes an operation below approximately 25 percent, in lieu of previous information about an operation below 70 percent.
- Enhances load follow capability to include the specific interfacing systems required for this mode of operation, in lieu of the original "main turbine regulator control."
- Changes terminology for the "main turbine pressure regulator" to "APR," and the "semi-automatic mode" becomes "core flow mode."

STD DEP 7.7-20 revises the rate limiter rate of change to 5 percent for increasing speeds and minus 5 percent for decreasing speeds in Subsection 7.7.1.3(8)(e) in the ABWR DCD to be consistent with the speed change rate described in DCD Tier 2, Subsections 15.3.2.1.1 and 15.4.5.1.1.

STD DEP 7.7-20 updates the terminology in Figure 7.7-5 and Figure 7.7-7 to be consistent with Subsection 7.7.1.3.

The applicant's evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-22 Automated Thermal Limit Monitor Description

This departure implements an expanded description of the ATLM setpoint and rod block action found in the ABWR DCD. Subsections 7.7.1.5 (7)(c) and (7)(e) are expanded in the STP Units 3 and 4 FSAR to further describe the interface of the interacting systems.

The description of the ATLM setpoint and rod block action in the ABWR DCD Subsection 7.7.1.5.1 is expanded in the STP Units 3 and 4 FSAR to further describe the interface of the interacting systems. The FSAR states that when an ATLM setpoint update is requested after calculating the power distribution within the core, the computer sends data to the ATLM of the RCIS on the calculated fuel thermal operating limits and corresponding initial LPRM values. FSAR Tier 1 of the ABWR DCD contains information on the ATLM, but the proposed change in the departure has no effect on them. This change is an expansion of the description of the interface of interacting systems and the ATLM. The setpoint calculation is performed using the NSS performance module within the PMCS. There is no underlying design change and no SSC important to safety is affected.

The applicant's evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-23 Automated Traversing Incore Probe (ATIP) Function

Subsection 7.7.1.5.1 of the referenced ABWR DCD discusses the clarification and expansion of the description and function of the "automatic fixed incore probe (AFIP)". The STP Units 3 and 4 FSAR explains that this function is provided by the ATIP rather than the AFIP in the ABWR. In Subsection 7.7.2.6.2, the departure adds that the ATIP system has isolation valves and is required to perform the automatic containment isolation function in compliance with GDC 56, by following the guidance of RG 1.11.

The applicant's evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require NRC approval. The applicant's process for evaluating departures and other changes to the DCD are subject to NRC inspections.

- STD DEP 7.7-24 Steam Bypass and Pressure Control Interfaces

Subsection 7.7.1.8(7a) of the ABWR DCD states that an external signal interface for the SB&PC system is narrow range dome pressure signals from the SB&PC system to the RFCS. STP Units 3 and 4 FSAR Subsection 7.7.1.8 (7a) states that the “narrow range dome pressure signals” are replaced by “validated dome pressure signals.” The signals are validated based on the value of the pressure and the number of signals that are in the valid range. Other items in Tier 2 Section 7.7.1.8 clarify which is the sending unit and which is the receiving unit.

Tier 2 Subsection 7.7.2.8.1 of the ABWR DCD states that the SB&PC does not interface with any engineered safeguard or safety system. STP Units 3 and 4 FSAR Tier 2 Subsection 7.7.2.8.1 states that the SB&PC system receives the reactor pressure and water level from the NBS, but only from nonsafety instrumentation. This departure clarifies that the SB&PC system does not interface with any safety instrumentation, even though the SB&PC interfaces with a safety-related system through the sense lines. The transmitters used to convert the pressure and differential pressure to electrical signals are the interface and the nonsafety instrumentation.

The applicant’s evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require prior NRC approval. The applicant’s process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 7.7-27 RCIS Table Deletion

Table 7.7-1 of the referenced ABWR DCD provides the environmental conditions for the RCIS module operation environment that consists of temperature, relative humidity, atmospheric pressure, radiation levels, and seismic acceleration. There is no reference to this table in DCD Section 7.7 or elsewhere in the DCD. DCD Subsection 7.7.1.2.5 refers the reader to references in Section 3.11, which provides the requirements for nonsafety equipment subject to adverse environments. Therefore, Table 7.7-1 was deleted because the information is repeated and is thus unnecessary.

The applicant’s evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require prior NRC approval. The applicant’s process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 9.5-3 System Description - Reactor Internal Pump Motor-Generator Sets

This departure consists of several changes to the technical description of the nonsafety-related MG set equipment that provides power to the RIPs. These changes are being made to clarify the original DCD technical descriptions or to reflect changes in the actual equipment design implementation details that have evolved since the original DCD descriptions were written. This departure is addressed in SER Chapter 9 of this report. Paragraph 7.7.1.3(7) deletes a specific power device type “gate-turn-off (GTO).” The purpose of this sentence is to describe how to implement the recirculation pump trip (RPT) function in the ASDs. The description is consistent with the ASD design. Paragraph 7.7.1.3(8)(c) also clarifies changes that replace the term

“rectifier section” with “rectifier circuitry”; and “provides the required circuitry” replaces the phrase “includes gate turn off thyristors.”

The applicant’s evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require prior NRC approval. The applicant’s process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP Admin

Throughout STP Units 3 and 4 FSAR Subsection 7.7, the applicant identifies several administrative departures. Administrative departures are defined as minor corrections, such as editorial or administrative errors, in the referenced ABWR DCD (e.g., misspelled words, incorrect references, and table headings, etc.). Administrative departures do not affect the presentation of any design discussion or the qualification of a design margin.

The applicant’s evaluation in accordance with Item B.5 of Section VIII of Appendix A of 10 CFR Part 52 determined that this departure does not require prior NRC approval. Within the review scope of this section, NRC staff found it reasonable that the departure does not require prior NRC approval. The applicant’s process for evaluating departures and other changes to the DCD is subject to NRC inspections.

7.7.5 Post Combined License Activities

There are no post COL Activities related to this section.

7.7.6 Conclusion

The NRC staff’s finding related to the information incorporated by reference is in NUREG–1503. The staff reviewed the application and checked the referenced DCD. The staff’s review confirmed that the applicant has addressed the required information relating to control systems not required for safety. With the exceptions of **Confirmatory Item 07.07-10**, no outstanding information is expected to be addressed in the COL FSAR related to this section. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the control system that are not required for safety and are incorporated by reference have been resolved.

The staff compared the application to the relevant NRC regulations; the acceptance criteria defined in NUREG–0800, Section 7.7; and other NRC RGs and concluded that the applicant is in compliance with the NRC regulations. The staff found that the applicant has adequately addressed the departures. However, as a result of **Confirmatory Item 07.07-10**, the staff was unable to finalize the conclusions relating to control systems not required for safety, in accordance with the NRC requirements.

7.8 COL License Information

7.8.1 Introduction/Overview/General

This section of the FSAR discusses COL information items.

7.8.2 Summary of Application

Section 7.8 of the STP Units 3 and 4 COL FSAR incorporates by reference Section 7.8 of the certified ABWR DCD referenced in 10 CFR Part 52, Appendix A, with no departures. The applicant provides supplemental information to address COL License Information Items 7.3, 7.4, and 7.5 from the ABWR DCD.

COL License Information Items

- COL License Information Item 7.3 Effects of Station Blackout on the HVAC

The applicant states that “During the station blackout (SBO) scenario, control room HVAC can be unavailable for up to 10 minutes while the Alternate AC source starts and connects to the safety buses.” The applicant commits (COM 7.8-1) to perform a “control room temperature rise analysis using as-procured and as-built equipment information” for the SBO scenario and will submit it to the NRC for review before fuel loading.

- COL License Information Item 7.4 Electrostatic Discharge on Exposed Equipment Components

This COL license information item addresses the effects of electrostatic discharge on exposed equipment components.

- COL License Information Item 7.5 Localized High Heat Spots in Semiconductor Materials for Computing Devices

This COL license information item addresses localized high-heat spots in semiconductor materials for computing devices. The applicant commits (COM 7.8-2) to provide an updated FSAR to reflect the results of the environmental qualification at the time that purchase orders are placed for the SSLC systems.

7.8.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG–1503.

COL License Information Items 7.3, 7.4, and 7.5 are met based on following the guidelines of NUREG–0800 Chapter 7, “Instrumentation and Controls,” 10 CFR 50.55a(h), 10 CFR Part 50, Appendix A, GDC 4, 13, and 19.’

7.8.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Section 7.8 of the certified ABWR DCD. The staff reviewed Section 7.8 of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of the information in the COL FSAR and the information in the ABWR DCD appropriately represents the complete scope of information relating to this review topic.¹ The staff’s review confirmed that the information in the

¹ See “*Finality of Referenced NRC Approvals*” in SER Section 1.1.3, for a discussion on the staff’s review related to verification of the scope of information to be included in a COL application that references a design certification.

application and the information incorporated by reference address the required information relating to this section..

The staff's review of this application is limited to COL Information Items 7.3, 7.4, and 7.5.

COL License Information Items

- COL License Information Item 7.3 Effects of Station Blackout on the HVAC

Specific information provided by the applicant to address COL License Information Item 7.3 includes Commitment (COM), 7.8-1, which commits the applicant to perform a control room temperature-rise analysis before fuel loading and using as-procured and as-built equipment. The analysis will demonstrate that the available equipment used during an SBO will not generate sufficient heat to raise the environmental temperature above the qualified limits of the operating equipment. **Commitment Number COM 7.8-1** will track the applicant's commitment to perform an analysis of the control room temperature heat rise for the SBO scenario based on plant-specific environmental parameters.

- COL License Information Item 7.4 Electrostatic Discharge on Exposed Equipment

Specific information provided by the applicant to address COL License Information Item 7.4 includes implementation of General Electric (GE) recommendations (response to Question 420.90 in Chapter 20 of the ABWR DCD) for limiting the effects of electrostatic discharge (ESD) at keyboards, keyed switches, and other exposed equipment. The applicant also provides assurance that the grounding and shielding techniques employed are consistent with the GE recommendations. The applicant has committed to verify the lack of susceptibility of ABWR control equipment to electrostatic discharges using the test procedures included in IEC Publication 801-2, "Electromagnetic Compatibility for Industrial Process Measurement and Control Equipment," and Part 2, "Electrostatic Discharge Requirements," which meet the following acceptance criteria:

No change in trip output status shall be observed during the test.

Equipment shall perform its intended functions after the test.

The applicant will use ITAAC Table 3.4, Item 12, to document these test results.

Based on the staff's evaluation, IEC Publication 801-2 is no longer active. IEC 61000-4-2, "Electromagnetic Compatibility," (EMC) Part 4-2, "Testing and Measurement Techniques - Electrostatic Discharge Immunity Test," replaces IEC 801-2. This international standard relates to the immunity requirements and test methods for electrical and electronic equipment subjected to static electricity discharges directly from operators and to adjacent objects. The standard also defines ranges of test levels that relate to different environmental and installation conditions and establishes test procedures. NRC staff issued **RAI 07.08-1**. The applicant's response (STPNOC Letter U7-C-STP-NRC-090118, dated August 27, 2009) states that IEC 61000-4-2 will be used to qualify electrical and electronic equipment subjected to electrostatic discharges. Accordingly, the applicant proposes an update to FSAR Sections 7.8.2 and 7A.3 and Table 1.8-21. RG 1.180, Revision 1 endorses the IEC 61000 series standards and ensures compliance with the stated regulations. Therefore, the staff found that the applicant has adequately addressed COL License Information Item 7.4. The applicant's

Letter U7-C-STP-NRC-090009 dated February 9, 2009, commits to conformance with RG 1.180 and proposes an update to FSAR Table 1.9S-1 accordingly. These proposed FSAR changes are not captured in Revision 3 of the STP Units 3 and 4 COL application. Confirmation of these proposed FSAR changes is being tracked as **Confirmatory Item 07.08-1**.

- COL License Information Item 7.5 Localized High Heat Spots in Semiconductor Materials for Computing Devices

The applicant provides specific information to address COL License Information Item 7.5 that includes a commitment (COM 7.8-2) to update the FSAR with the results of environmental qualifications of the SSLC systems. The applicant provides an acceptable alternative to the thermal analysis performed at the circuit board, instrument, and panel design stages that are called for in the ABWR DCD. The applicant commits to include the provisions in the purchase specifications for the SSLC systems, which will ensure that the supplied equipment is designed to perform its intended safety functions in the installed normal, abnormal, and accident environments. A local temperature sensor and diagnostic alarm will be provided in each cabinet. Temperature rise internal to the cabinet will be verified during the environmental qualification and during factory acceptance testing for each cabinet. Commitment number **COM 7.8-2** will track whether the applicant performs the above stated actions and will document the environmental qualification results in the FSAR.

7.8.5 Post Combined License Activities

The applicant identifies the following commitments:

- Commitment (**COM 7.8-1**) – Perform a control room temperature rise analysis using as-procured and as-built equipment information for the SBO scenario, before fuel loading.
- Commitment (**COM 7.8-2**) – Provide an updated FSAR to reflect the results of the environmental qualification at the time that purchase orders are placed for the SSLC systems.

7.8.6 Conclusion

NRC staff found that this area is addressed in the ABWR DCD and in the related NRC FSER in NUREG–1503.

The staff's finding related to information incorporated by reference is in NUREG–1503. The staff reviewed the application and checked the referenced DCD. The staff's review confirmed that the applicant has addressed the required information relating to the COL license information. With the exception of **Confirmatory Item 07.08-1**, no outstanding information is expected to be addressed in the COL FSAR related to this section. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the COL license information that were incorporated by reference have been resolved.

The staff compared the application to the relevant NRC regulations; the acceptance criteria defined in NUREG–0800, Chapter 7; and other NRC RGs. The applicant has adequately addressed COL License Information Items 7.3, 7.4, and 7.5. However, as a result of **Confirmatory Item 07.08-1**, the staff was unable to finalize the conclusions relating to the COL license information in accordance with the NRC requirements.

7.8S Diverse Instrumentation and Control Systems

This section of the FSAR refers to Section 7C.5 for the discussion on the diversity of the I&C systems. NRC staff's evaluation of the diverse I&C systems is in Section 7C of this SER.

7.9S Data Communications Systems

7.9S.1 Introduction

This supplemental section of the FSAR addresses the essential (safety-related) and nonessential (nonsafety-related) data communication functions (DCFs) that are part of or that support the systems described in COL application FSAR Sections 7.1S and 7.2 through 7.7. The DCF reviewed in this section includes communication between systems and communication between computers within a system. This supplemental section addresses both safety and nonsafety communication systems. Internal computer buses are specifically excluded from the definition of DCF used in this section.

7.9S.2 Summary of Application

This supplemental section includes an overview of the ABWR data communication system for STP Units 3 and 4. In this supplemental section, the applicant addresses both the safety-related and nonsafety-related data communication functions, which are part of or support the I&C systems described in COL application FSAR Sections 7.1S and 7.2 through 7.7. Specifically, this section includes data communications between systems and between divisions within a system. Communication within a system is an integral part of that system. The DCFs of the RTIS, NMS, and ELCS are required to support the safety-related functions of these systems. The DCFs associated with the SSLC systems perform data collection and distribution using both local and remote data acquisition and control units, which are connected by dedicated data links and/or networks for the above three safety-related systems.

The safety-related data communication systems also provide alarm and status data from safety-related plant sensors located in the SSLC systems to the nonsafety-related PICS for main control room indication and computer logging, through isolated gateway interfaces and the nonsafety-related PDN. It also provides selected safety-related plant process data to the nonsafety-related control systems through isolated gateway interfaces. The interconnection of Class 1E communication to non-Class 1E devices is achieved using fiber optic cable, which provides the necessary electrical isolation. Communication from safety-related to non-safety-related systems is controlled by the safety device to assure that no communication task will interfere with the safety system performing intended functions. The majority of the non-safety-related data communications are performed through a plant-wide PDN. The PDN distributes process and other data required to support the nonsafety-related operational functions.

7.9S.3 Regulatory Basis

Reviews of this supplemental section are based on meeting the current regulatory requirements, the acceptance criteria, and guidelines in NUREG-0800 Table 7-1, Revision 5 (March 2007). In accordance with 10 CFR 50.55a(h), COL applications filed on or after May 13, 1999, which depart from the certified ABWR DCD, Revision 4, must meet the requirements for safety systems in IEEE-Std 603-1991 and in the correction sheet dated January 30, 1995. In addition, applicable guidelines in DI&C-ISG documents are also used to evaluate the departures from the certified ABWR design.

7.9S.4 Technical Evaluation

NRC staff reviewed the applicant's supplemental changes to the certified ABWR DCD, Revision 4, using the review procedures described in NUREG-0800 Section 7.9, "Data Communication Systems." The staff's evaluation of this supplemental section affecting the certified ABWR DCD, Revision 4, is as follows.

The DCFs are an integral part of the RTIS, NMS, and ELCS safety platforms. The staff found that the RTIS, NMS, and ELCS platforms meet the applicable quality requirements in 10 CFR Part 50, Appendix B, 10 CFR 50.55a(h) (IEEE-Std 603-1991), and IEEE Std 7-4.3.2-2002. Based on this review, the staff concluded that the DCFs also meet the requirements of 10 CFR 50 Appendix B, 10 CFR 50.55a(h), and IEEE-Std 7-4.3.2-2002. In addition, the DCF software quality as a part of the overall software quality for the DI&C safety-related systems will be inspected as a post COL activity through the ITAAC process.

The staff found in the COL application that the communication between divisions for the SSLC safety systems preserves divisional independence, so that a failure in one division does not affect other divisions. Therefore, the staff found that the DCFs of the SSLC systems conform to the guidelines for the application of the single-failure criterion in IEEE-Std 379-1992, "IEEE Standard Application of the Single-Failure Criterion to Nuclear Power Generating Station Safety Systems," as supplemented by RG 1.53, "Application of the Single-Failure Criterion to Nuclear Power Plant Protection Systems".

The safety-related DCFs are integral functions of the SSLC systems. The SSLC testing features and surveillances that are covered in Subsection 7.1.2.1.6 of the COL application also include the DCF function testing. After reviewing the above relevant section of the COL application, the staff found that the DCFs of the SSLC system satisfy the requirements in the guidelines for periodic testing in RG 1.22, "Periodic Testing of Protection System Actuation Functions," and RG 1.118, "Periodic Testing of Electric Power and Protection Systems". See Section 7.1 of this SER for additional evaluations of the SSLC testing features and surveillances.

SSLC bypass and inoperable status indications (BISI) that encompass those related to the DCFs provide information to meet the requirements in RG 1.47, "Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems". So after reviewing the BISI description in the application, the staff determined that the bypass and inoperable status indication for the DCF functions conforms to the guidelines of RG 1.47. Based on this review, the staff concluded that the DCFs satisfy the requirement of IEEE-Std 603-1991, with regard to the system bypass and inoperable status indications.

In the letter dated February 9, 2009 (STPNOC Letter U7-C-STP-NRC-090009), and the applicant's responses to the RAIs (STPNOC Letter U7-C-STP-NRC-090232), the applicant indicates that RG 1.152 Revision 2 will be followed for the RTIS and NMS, because the new field configurable logic device-based platform will be used for these two systems. But the ELCS will be in conformance with RG 1.152 Revision 1 because the applicant states that the Common-Q platform approved by NRC will be used for this ELCS. The applicant will address any new changes to the approved Common-Q platform and associated DI&C security concern in a technical report under the Tier 1, Table 3.4 ITAAC/DAC as a post-COL application activity. The applicant will address all new DI&C security requirements through the ITAAC closure

process, as post COL activities according to any new DI&C security guidelines. The applicant submitted the programmatic cyber security plan to meet 10 CFR 73.54 regulations. The certified ABWR DCD Tier 1 Table 3.4, Revision 4, covers ITAAC items for the SSLC systems that include software development plans and the inspections of the computer development process and design outputs. The inspections of the above ITAAC items for software lifecycle development plans are included as post-COL activities. The safety-related DCFs are integral functions of the SSLC systems. Therefore, the staff concluded that the DCF systems conform to the guidance of RG 1.152, and the special characteristics of DCF systems have been adequately addressed.

The DCF functions were included in the staff's review of the diversity and defense-in-depth (D3) analysis for the RTIS and ELCS. The diversity and D3 system is covered in COL application FSAR Sections 7.8S and Appendix 7C. Refer to Sections 7.8S and Appendix 7C of this SER for detailed evaluations of the diverse D3 system.

The staff found that SSLC divisions are physically separated and electrically isolated from one another. The divisions have separate power sources, and the transmission of logic signals between divisions is through qualified isolation devices. For the fiber optic cables used for the DCF data communication network, its protective covering is flame retardant. The cables are passed through physical safety class barriers, where necessary, for separating Class 1E circuits and equipment from other Class 1E equipment or from non-Class 1E equipment. The DCF equipment is kept physically separate to minimize the effects of design-basis events. The staff also found that during operations, the functionality of the DCFs for the ELCS, NMS, and RTIS is independent of nonsafety-related systems. So the staff determined that DCF functions conform to the guidelines in RG 1.75, "Criteria for Digital Computers in Safety Systems of Nuclear Power Plants," for protection system independence. The staff also concluded that the DCF functions satisfy the requirements for independence of IEEE-Std 384-1992 and IEEE-Std 603-1991.

DCD FSAR Tier 1, Section 2.7.5 states that "data cannot be transmitted from the non-safety-related side to EMS" safety-related equipment. However, COL application FSAR Tier 2 Section 7.9S.2.2 allows manual data transmission from the nonsafety-related PICS to the safety-related NMS. The staff issued **RAI 07.09-8** requesting the applicant to make necessary changes for purposes of consistency, to provide sufficient information on how the manual data transmission will be implemented, and to illustrate how to prevent any adverse impact on the safety-related NMS from the manual operation. In response to this RAI (STPNOC Letter U7-C-STP-NRC-090232), the applicant states that data cannot be transmitted from the non-safety-related equipment to online safety-related equipment. A separate offline method is used to transfer LPRM calibration data from PICS to NMS, which places the NMS receiving division in an inoperative status and requires a key lock switch to be enabled at the NMS to allow the transfer. Manual verification is required before the NMS division is placed back in service. The staff determined that the explanation in the RAI response is acceptable, and RAI 07.09-8 is therefore resolved.

The DCFs are integral functions of the SSLC systems. The staff found that for the SSLC systems, divisions are physically separated and electrically isolated from one another. The divisions have separate power supplies. The data communication of logic signals between divisions is via qualified isolation devices. There is no online data communication from the non-safety-related control systems to the safety-related systems. The data communication from the safety-related control systems to the nonsafety-related systems is through fiber optic cables,

which also provides the electrical isolation between the safety- and non-safety-related systems. Based on the review of system independence and separation, the staff concluded that the DCF satisfies the requirement of IEEE-Std 603–1991 with regard to systems independence. Therefore, the staff found that the DCFs also meet the requirements of GDC 22.

The RTIS and NMS systems are designed to fail into a safe state upon the loss of communication. The ELCS fails as-is during communication failure. That is, system controllers continue to operate based on the last command. The staff determined that the failure mode for RTIS, NMS, and ELCS is acceptable. The DCF failure modes are accounted for in the failure modes and effects analysis (FMEA) for the RTIS and ELCS. Based on the staff's review, the staff concluded that the SSLC protection systems, which include the DCFs, satisfy the requirements of GDC 23.

The DCFs are integral functions of the RTIS, ELCS, and NMS safety systems. There are one-way data communications from the RTIS or ELCS to plant operating control systems. This unidirectional data communication is based on the fiber optic cables. During the normal operation, there is also only one-way data communication from the safety-related NMS to plant operating control systems via the fiber optic cables. During normal operation, there is manual data transmission from the non-safety-related plant control system to the safety-related NMS. This manual data transmission from PICS to only one division of NMS at a time occurs when it's placed in the inoperable mode. The staff found this manual data communication function acceptable. During the normal operation, the control system that is usually used to control the plant power does not have any adverse impact on the safety systems. The SSLC protection systems are separate from the plant operating control system. Any failure from the plant control system component will not impair the safety systems. Based on the review of the interfaces between the DCFs and plant operating control systems, the staff concluded that the RTIS, ELCS, and NMS safety systems satisfy the requirements of IEEE-Std 603-1991, with regard to the control and protection system interactions. Therefore, the staff also found that the DCFs satisfy the requirements of GDC 24.

The DCFs are an integral part of the RTIS, NMS, and ELCS to support their safety-related functions. The DCFs also support the functions of the information systems important to safety, such as the PAM system. The DCFs are integral functions of the safety interlock system. The staff found that DCFs used in information systems that are important to safety, and in interlock systems important to safety, transmit the variables and commands necessary to maintain the fission process; the integrity of the reactor core; the reactor coolant pressure boundary; and the containment and its associated systems within prescribed operating ranges during a plant shutdown. Therefore, the staff determined that the DCFs employed by the information and interlock systems important to safety satisfy the requirements of GDC 13.

The staff found that the DCF functions in the COL application detect errors using self-diagnostic tests such as checksum, parity check, or the reception of a keep-alive signal. If the data are not available, the safety logic takes predetermined actions based on the specific data involved. The staff found that the DCF functions of the SSLC safety systems have the simplicity of the communication design, which is of a high reliability. Hence, the staff determined that the DCF functions in the COL application support the reliability requirements in 10 CFR 50.55a(h) (IEEE-Std 603-1991).

The DCF systems that are integral parts of the SSLC systems support instruments and controls within the control room to allow actions to be taken to maintain the nuclear power unit in a safe condition during a shutdown, including a shutdown following an accident. The staff also found in the COL application that equipment at appropriate locations outside the control room provide support to achieve (1) a prompt, hot shutdown of the reactor; and (2) a subsequent, cold shutdown of the reactor. Therefore, the staff concluded that the DCFs employed by the safe shutdown system satisfy the requirements of GDC 19.

The staff's review included the identification of those systems and components for the DCF systems that are designed to survive the effects of earthquakes, other natural phenomena, abnormal environments, and missiles. COL FSAR Section 7.9S.2.5 states that all of the equipment implementing the DCFs of the SSLC, which is located in Seismic Category I structures, meets the requirements of RG 1.100 and IEEE-Std 344–1997. Based on the review, the staff concluded that the applicant has identified DCF systems and components consistent with the design bases for those DCF systems. The staff also reviewed EMI/RFI susceptibility and seismically exposed portions of the DCFs. Sections 3.10 and 3.11 of this SER address the qualification programs to demonstrate the capability of these systems and components to survive these events. Refer to Sections 3.10 and 3.11 of this SER for additional evaluations. Therefore, the staff found that the identification of the DCF systems and components satisfy the requirements of GDC 2 and 4.

Setpoint analyses accounting for measurement inaccuracies attributable to the DCF will be conducted in accordance with the guidance of RG 1.105, Revision 3 (Ref. 7.10-XX). The applicant has submitted a technical report on "Methodology for STP Units 3 and 4 ABWR Technical Specifications Setpoints". For the technical and regulatory compliance evaluations, refer to Section 7.1 of this SER. Also, as an ITAAC item in the certified ABWR DCD Tier 1 Table 3.4, the setpoints for initiating safety-related functions will be determined, documented, and maintained for NRC's inspection, which is included as a post COL application activity. The staff concluded that the DCF systems adequately support the RTIS and ELCS safety functions as necessary, to sense accident conditions and anticipated operational occurrences, in order to initiate protective actions consistent with the accident analysis presented in Chapter 15 of the COL application FSAR. Therefore, the staff determined that the DCFs appropriately support RTIS and ELCS compliance with the requirements of GDC 20.

The staff found that each communication interface in the DCF systems operates independently and also asynchronously. The maximum time delay from input to output is deterministic. The data transmission for the safety-related systems is purely unidirectional, without acknowledgement from the other end. The data transmission cycle time is fixed and the communication is of the deterministic type. So based on this review, the staff determined that the DCF performance meets the requirements and guidelines in the BTP 7-21, DI&C-ISG-04, and 10 CFR 50.55a(h) (IEEE-Std 603–1991).

The staff found that the COL application specifies that only the one-way data transfer is allowed from the safety-related to the nonsafety-related systems for the DCF functions. The COL application states that there is no unprotected electronic paths in the DCF functions by which unauthorized personnel can change plant software or display erroneous status information. There are security-protected external interfaces but no remote access to the safety systems. Based on this review, the staff determined that the DCF functions meet the requirements in 10 CFR 50.55a(h) (IEEE-Std 603-1991), IEEE-Std 7-4.3.2-2002, RG 1.152, and DI&C-ISG-04.

The safety-related DCFs are integral functions of the SSLC systems, which include the safety related ELCS, NMS, and RTIS. For the SSLC systems, the staff issued **RAI 07.09-1** and **RAI 07.09-2** requesting the applicant to provide sufficient information detailing how the communication isolation and interface are implemented for the gateways among multiple digital platforms, especially the ones to be used between the safety-related NMS and the nonsafety-related PICS, because manually conducted communication is needed from the PICS to the NMS. In response to these RAIs (STPNOC Letter U7-C-STP-NRC-090157), the applicant commits to revising this supplemental section in a future revision of COL application. The staff found the responses acceptable. However, because these proposed FSAR changes are not captured in the current Revision 3 of the STP Units 3 and 4 COL application, the incorporation of the proposed FSAR changes is being tracked as **Confirmatory Item 07.09-1**.

Both the fiber optic cable and its components such as terminations, field splices, and connectors are critical to providing electrical and communication isolation for the fiber optic-based and safety-related systems. The staff issued **RAI 07.09-3** requesting the applicant to provide sufficient qualification information in the COL application for the fiber optic cable components intended to prevent the fiber optic cable-related common cause failures. In response to this RAI (STPNOC Letter U7-C-STP-NRC-090157), the applicant states that the qualification of all safety-related I&C equipment is within the scope of the ITAAC. The inspection of the ITAAC for all I&C equipment qualification is included as post-COL application activities.

COL application FSAR Section 7.9S.3.1 states that network gateways are safety related, but Figure 7.9S-1 shows that the gateways are not safety related. The staff issued **RAI 07.09-4** requesting the applicant to clarify this inconsistency. In response to this RAI (STPNOC Letter U7-C-STP-NRC-090157), the applicant confirms that the network gateways are non-safety related. The applicant commits to revise the related FSAR section. The staff found the applicant's response acceptable. However, the incorporation of the proposed FSAR changes is being tracked as **Confirmatory Item 07.09-4**.

According to SRP Section 7.9 and BTP 7-21, the staff requested the applicant to provide the system design analysis because the DI&C system proposed in the COL application is different from the one included in the certified ABWR DCD, Revision 4. In addition, the certified ABWR DCD, Revision 4, requires a safety and hazards analysis, a sneak circuit analysis, and a timing analysis for the DI&C systems. However, those analyses are not included in the applicant's COL application. Therefore, the staff issued **RAI 07.09-5** requesting the applicant to provide sufficient information addressing those analysis requirements. In response to this RAI (STPNOC Letter U7-C-STP-NRC-090157), the applicant states that the verification of the I&C system design and analysis will be accomplished during the ITAAC phase. The staff has verified the adequacy of the related ITAAC and found it to be acceptable, such that when the ITAAC is performed and the acceptance criteria are met the facility would have been constructed and will operate in conformance with the COL and the NRC regulations.

Because the technical reports for the field configurable logic device based NMS and RTIS will not be included as part of the COL application, the staff issued **RAI 07.09-6** requesting the applicant to provide sufficient information in the COL application on how the configurable logic device related technical issues are addressed in the NMS and RTIS design to meet the safety and reliability requirements such as timing, delay, race conditions, gate skew, power dissipation, partitioning, maintainability, testability, tool usage and qualification, and environmental and fabrication issues related to the underlying technologies to be used. In response to this RAI

(STPNOC Letter U7-C-STP-NRC-090157), the applicant states that all of the issues mentioned in the RAI will be addressed in a technical report in support of ITAAC/DAC activities. The staff has verified the adequacy of the related ITAAC and found it to be acceptable, such that when the ITAAC is performed and the acceptance criteria are met the facility would have been constructed and will operate in conformance with the COL and the NRC regulations.

7.9S.5 Post Combined License Activities

There are no post COL activities related to this section.

7.9S.6 Conclusion

NRC staff reviewed this supplemental section of DCF functions for conformance to the requirements and guidelines in the regulations, RGs, and industry codes and standards applicable to the DCF system. The staff concluded that the applicant has adequately identified the regulations and guidelines applicable to the DCF system. Based on the review, the staff found reasonable assurance that the DCF system generally conforms to the regulations and guidelines applicable to the DCF system, which is subject to the incorporation of changes as the applicant's responses to RAIs commit to in the future revision of the COL application FSAR. These commitments are being tracked as confirmatory items. In addition, the inspection and verification of the associated technical reports and ITAAC items of the proposed new data communication technology and platforms is also being tracked as a post COL activity.

However, as a result of **Confirmatory Items 7.9-1, and 7.9-4**, the staff was unable to finalize the conclusions relating to the data communications systems, in accordance with the NRC requirements.

7A Design Response to Appendix B, ABWR LRB Instrumentation and Controls

7A.1 Introduction

This FSAR appendix addresses corresponding changes to the original design responses to the ABWR licensing review board (LRB) in the certified ABWR DCD Appendix 7A, Revision 4, which were caused by some of the departures in the COL application, Part 7, "Departures Reports."

7A.2 Summary of Application

Appendix 7A of the STP Units 3 and 4 COL FSAR incorporates by reference Appendix 7A of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A, with the following departures:

Tier 1 Departures

- STD DEP T1 2.14-1 Hydrogen Recombiner Requirements Elimination

This departure reflects the elimination of the hydrogen recombinder requirement to maintain the equipment needed to mitigate a design-basis LOCA hydrogen release.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

This departure describes changes to the I&C architecture and nomenclature that are needed to address obsolete data communication technology and the selection of digital I&C platforms.

Tier 2* Departure

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

This departure updates some codes, standards, and RGs to more current revisions/editions.

Tier 2 Departures Not Requiring Prior NRC Approval

- STD DEP 7.1-1 References to Setpoints and Allowable Values

This departure clarifies that wherever the TS is referenced in the FSAR for setpoints or margins, the correct reference is to the methods for calculating the setpoints and margins, as described in the TS Bases.

- STD DEP Admin

This departure describes minor corrections, such as editorial or administrative errors in the certified ABWR DCD, Revision 4 (e.g., misspelled words, incorrect references, table headings, etc.).

7A.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG–1503. In addition, the relevant requirements of the Commission regulations for the design response to Appendix B, ABWR LRB I&Cs, and the associated acceptance criteria, are in Sections 7.2, 7.3, 7.4, 7.5, 7.6, 7.8, and 7.9 of NUREG-0800.

In accordance with Section VIII, “Processes for Changes and Departures,” of, “Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor,” the applicant identifies three Tier 1, Tier 2*, and Tier 2 departures. Tier 1 and Tier 2* departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.A.4, and Section VIII.B.6, respectively. Tier 2 departures that do not require prior NRC approval are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.5, which are similar to the requirements of 10 CFR 50.59.

The requirements of 10 CFR 52.63(a)(1)(vii) (as proposed in SECY-06-220) allow for a change to a ABWR DCD if the change contributes to the increased standardization of the certification information. Reviews of the departures are based on meeting the current regulatory requirements, acceptance criteria, and guidelines in NUREG–0800 Table 7-1, Revision 5 (March 2007). In accordance with 10 CFR 50.55a(h), COL applications filed on or after May 13, 1999, which depart from the certified ABWR DCD, Revision 4, must meet the requirements for safety systems in IEEE-Std 603–1991 and in the correction sheet dated January 30, 1995. In addition, applicable guidelines in DI&C ISG documents are also used to evaluate the departures from the certified ABWR design.

7A.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Appendix 7A of the certified ABWR DCD. The staff reviewed Appendix A of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of the information in the COL FSAR and the information in the ABWR DCD appropriately represents the complete scope of information relating to this review topic.¹ The staff's review confirmed that the information in the application and the information incorporated by reference address the required information relating to this appendix.

The staff reviewed the information in the COL FSAR:

Tier 1 Departures

The following Tier 1 departures identified by the applicant in this section require prior NRC approval and the full scope of their technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these Tier 1 departures.

- STD DEP T1 2.14-1 Hydrogen Recombiner Requirements Elimination

The departure modifies FSAR Table 7A-1 by deleting the FCS, which is not used in the COL application. NRC staff found the departure acceptable.

Refer to Section 7.3 of this SER for the detailed evaluation of this departure from the I&C perspective.

- STD DEP T1 3.4-1 Safety-related I&C Architecture

This departure is classified as requiring prior NRC approval. In COL application Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VIII.A.4 of Appendix A to 10 CFR Part 52.

NRC staff reviewed this departure and found that it includes the following five primary changes to the I&C architecture of the certified ABWR DCD, Revision 4:

1. Elimination of obsolete data communication technology
2. Elimination of unnecessary inadvertent actuation prevention logic and equipment
3. Clarifications of digital controls nomenclature and systems
4. Final selection of platforms that changed the implementation architecture
5. Identification of testing and surveillance changes for the SSLC

The staff found that the relevant impacts from this departure on the certified ABWR DCD Appendix 7A, Revision 4, are related to the obsolete multiplexing systems and associated technical issues. All changes made to Section 7A, Table 7A-1 and Figure 7A-1 are associated with the above change items 1, 3, and 4. Due to this departure, the applicant revised the responses to NRC requests related to multiplexing systems, electrical isolators, computer

¹ See "Finality of Referenced NRC Approvals" in SER Section 1.1.3 for a discussion on the staff's review related to verification of the scope of information to be included in a COL application that references a design certification.

hardware, and computer software. These revised responses to the NRC requests are in accordance with the departed digital I&C architecture described in the FSAR Sections 7.1S and 7.9S. This change does not depart from the certified ABWR design concepts and functional requirements.

In response to the NRC staff's **RAI 07.02-4** (STPNOC Letter U7-C-STP-NRC-090162, dated September 24, 2009), the applicant replaced FSAR Figure 7A-1, "Safety System Logic and Control (SSLC)," with new FSAR Figure 7.9S-1, "Data Communications Interfaces," which illustrates, at a block diagram level, the entire plant digital I&C architecture including the safety-related DI&C platforms and their data communications configurations. The staff reviewed this response and found that the proposed FSAR change adequately capture the design information of the proposed I&C platforms and associated data communication functions. This RAI response is therefore acceptable. Proposed FSAR change has not been captured in the STP 3 and 4 COLA, Rev. 3. Verification of this FSAR change is being tracked as **Confirmatory Item 07.02-4**.

Refer to Sections 7.1, 7.1S, 7.2, 7.3, and 7.9S of this SER for additional evaluations of this departure.

Tier 2* Departures

The following Tier 2* departure identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 2* departure.

- STD DEP 1.8-1 Tier 2* Codes, Standards, and Regulatory Guide Edition Changes

This departure modifies FSAR Subsection 7A.2 by replacing IEEE-Std 279 with IEEE-Std 603-1991, which meets the requirements of 10 CFR 50.55a(h). This departure also updates the MIL Electromagnetic Compatibility (EMC) standards to the current version. This update complies with the guidelines in RG 1.180.

Refer to Section 7.2 of this SER for the detailed evaluation of this departure from the I&C perspective.

Tier 2 Departures Not Requiring Prior NRC Approval

The following Tier 2 departures not requiring prior NRC approval identified by the applicant in this section may also be evaluated in other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by these departures.

- STD DEP 7.1-1 References to Setpoints and Allowable Values

This departure modifies FSAR Subsection 7A.7 by referring to the bases in COL application Chapter 16 for methods of calculating setpoints and margins.

The applicant's evaluation determined that this departure does not require prior NRC approval, in accordance with 10 CFR Part 52, Appendix A, Section VIII.B.5. Within the review scope of

this section, NRC staff found it reasonable that the departure does not require prior NRC approval. The applicant's process for evaluating departures and other changes to the certified ABWR DCD is subject to NRC inspections.

- STD DEP Admin

The applicant changed FSAR Subsection 7A.2 to clarify and specificity associated grammatical corrections.

The applicant's evaluation determined that this departure does not require prior NRC approval, in accordance with 10 CFR Part 52, Appendix A, Section VIII.B.5. Within the review scope of this section, NRC staff found it reasonable that this departure does not require prior NRC approval.

7A.5 Post Combined License Activities

There are no post COL activities related to this appendix.

7A.6 Conclusion

The NRC staff's finding related to information incorporated by reference is in NUREG–1503. The staff reviewed the application and checked the referenced DCD. The staff's review confirmed that the applicant has addressed the required information, and no outstanding information is expected to be addressed in the COL FSAR related to this section. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the design response to Appendix B, ABWR LRB I&Cs that were incorporated by reference have been resolved.

The staff reviewed departures in this appendix for conformance to the current regulations and applicable guidelines. The staff concluded that the applicant has adequately identified the regulations and guidelines applicable to these systems. Based on this review, the staff found with reasonable assurance that the departures in this appendix generally conform to the guidelines applicable to these systems.

The staff compared the COL application to the relevant NRC regulations; the acceptance criteria in Chapter 7 of NUREG–0800; and other NRC RGs and concluded that the applicant is in compliance with the NRC regulations. The applicant has adequately addressed the identified departures, which are therefore considered resolved.

However, as a result of **Confirmatory Item 7.02-4**, the staff was unable to finalize the conclusions relating to Section 7A, in accordance with the NRC requirements.

7B Implementation Requirements for Hardware/Software Development

Appendix 7B of the STP Units 3 and 4 COL FSAR incorporates by reference Appendix 7B, "Implementation Requirements for Hardware/Software Development," of the ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A, with no departures or supplements. NRC staff reviewed the application and checked the referenced DCD to ensure that no issue

relating to this section remains for review¹. The staff's review confirmed that there is no outstanding issue related to this section. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the Implementation Requirements for Hardware/Software Development have been resolved.

7C Defense Against Common-Mode Failure in Safety-Related, Software-Based I&C Systems

7.C.1 Introduction

This FSAR appendix addresses the defense against common-mode failure and implementation of diversity in the ABWR protection system.

7.C.2 Summary of Application

Appendix 7C of the STP Units 3 and 4 COL FSAR incorporates by reference Appendix 7C of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A, with the following departures:

Tier 1 Departure Requiring Prior NRC Approval

- STD DEP T1 3.4-1 Safety-Related I&C Architecture (Figure 7C-1)

This departure changes I&C architecture and nomenclature that are needed to address obsolete data communication technology and the selection of DI&C platforms.

Tier 2 Departure Not Requiring Prior NRC Approval

- STD DEP Admin Administrative Departures

The administrative departures are defined as minor corrections, such as editorial or administrative errors in the reference ABWR DCD (e.g., misspelled words, incorrect references, table headings, etc.). The applicant identifies one administrative departure in Section 7C.1, "Introduction," which removes an incorrect reference to Appendix 7A.

7.C.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG-1503. In addition, the relevant requirements of the Commission regulations for the "Defense Against Common-Mode Failure in Safety-Related, Software-Based I&C Systems", and the associated acceptance criteria, are in Section 7.1 of NUREG-0800.

In accordance with Section VIII, "Processes for Changes and Departures," of, "Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor," the applicant identifies Tier 1 and Tier 2 departures. Tier 1 departures require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.A.4. Tier 2 departures

¹ See "Finality of Referenced NRC Approvals" in SER Section 1.1.3 for a discussion on the staff's review related to verification of the scope of information to be included in a COL application that references a design certification.

not requiring prior NRC approval are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.5, which are similar to the requirements in 10 CFR 50.59.

The design departures are satisfied based on meeting the requirements of the current regulatory requirements, acceptance criteria, and guidelines in NUREG-0800 Table 7-1, Revision 5 (March 2007). In accordance with 10 CFR 50.55a(h), applications filed on or after May 13, 1999, that depart from the referenced CDM must meet the requirements for safety systems in IEEE-Std 603-1991 and in the correction sheet dated January 30, 1995. In addition, applicable guidelines in DI&C ISG Documents DI&C-ISG-01, DI&C-ISG-02, DI&C-ISG-03, DI&C-ISG-04, and DI&C-ISG-05 are also used to evaluate the departures from the certified ABWR design.

Regarding SRP acceptance criteria for plants with a digital RTS or ESFAS, the NRC position on D3 is in Item II.Q, "Defense Against Common-Mode Failures in Digital Instrument and Control Systems," of the Staff Requirements Memorandum (SRM) on SECY-93-087, "Policy, Technical, and Licensing Issues Pertaining to Evolutionary and Advanced Light-Water Reactor (ALWR) Designs." SRM requirements applicable to diverse I&C functions are as follows:

"If a postulated common-mode failure could disable a safety function, then a diverse means, with a documented basis that the diverse means is unlikely to be subject to the same common-mode failure [as the safety system], shall be required to perform either the same function [as the safety system function that is vulnerable to common mode failure] or a different function [that provides adequate protection]. The diverse or different function may be performed by a non-safety system if the system is of sufficient quality to perform the necessary functions under the associated event conditions."

"A set of displays and controls located in the main control room shall be provided for manual system-level actuation of critical safety functions and monitoring of parameters that support the safety functions. The displays and controls shall be independent and diverse from the safety computer system[s] ..."

7.C.4 Technical Evaluation

As documented in NUREG-1503, NRC staff reviewed and approved Appendix 7C of the certified ABWR DCD. The staff reviewed Appendix 7C of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of the information in the COL FSAR and the information in the ABWR DCD appropriately represents the complete scope of information relating to this review topic.¹ The staff's review confirmed that the information in the application and the information incorporated by reference address the required information relating to this appendix.

The staff reviewed the information in the COL FSAR:

¹ See "Finality of Referenced NRC Approvals" in SER Section 1.1.3, for a discussion on the staff's review related to verification of the scope of information to be included in a COL application that references a design certification.

Tier 1 Departure

The following Tier 1 departure identified by the applicant in this section require prior NRC approval and the full scope of its technical impact may be evaluated in the other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this Tier 1 departure.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

Tier 1 Departure STD DEP T1 3.4-1 is divided into the following five primary changes in the I&C architecture:

1. Elimination of obsolete data communication technology
2. Elimination of unnecessary inadvertent actuation prevention logic and equipment
3. Clarifications of digital controls nomenclature and systems
4. Final selection of platforms that changed the implementation architecture
5. Identification of testing and surveillance changes

This departure is classified as requiring prior NRC approval. In Part 7, "Departures Report," Section 2.1, the applicant describes and evaluates this departure per the requirements of Section VII.A.4 of Appendix A to 10 CFR Part 52. Categories 1, 3, and 4 of this departure are relevant to the changes in FSAR Sections 7C.1, 7C.3, 7C.4, and 7C.5, and Figure 7C-1. All of the changes in this subsection are limited to the proposed SSLC platform configurations, related data communication architectures, and nomenclature. These changes do not depart from the certified ABWR design concepts and processes employed to achieve defense against common-mode failure in safety-related, software-based I&C systems and diversity implemented in the ABWR protection system. Existing Tier 1, Table 3.4 ITAAC Item 16 continues to apply to the STP Units 3 and 4 design, which verifies that diversity is provided, as described in Tier 1 Section 3.4C in the form of a hardwired backup of a reactor trip, diverse display of important process parameters, D3 arrangement of equipment, and equipment diversity. NRC staff compared the departed design descriptions in Tier 2 FSAR Section 7C to the relevant NRC regulations; the acceptance criteria in NUREG-0800, Section 7.8 and BTP 7-19; D&IC-ISG-02; and NRC Policy SECY-93-087 and concluded that the applicant is in compliance with the NRC regulations.

Tier 2 Departure Not Requiring Prior NRC Approval

The following Tier 2 departure not requiring prior NRC approval identified by the applicant in this section may also be evaluated in other sections of this SER accordingly. For more information, please refer to COLA Part 07, Section 5.0 for a listing of all FSAR sections affected by this departure.

- STD DEP Admin Administrative Departures

The applicant made one administrative correction in FSAR Section 7C.1, which removes an incorrect reference to Appendix 7A. Administrative departures do not affect the presentation of any design discussion or the qualification of any design margin.

The applicant's evaluation determined that this departure does not require prior NRC approval in accordance with 10 CFR Part 52, Appendix A, Section VIII.B.5. Within the review scope of

this section, NRC staff found it reasonable that this departure does not require prior NRC approval.

7.C.5 Post Combined License Activities

There are no post COL activities related to this appendix.

7.C.6 Conclusion

The staff's finding related to information incorporated by reference is in NUREG-1503. The staff reviewed the application and checked the referenced DCD. The staff's review confirmed that the applicant has addressed the required information, and no outstanding information is expected to be addressed in the COL FSAR related to this appendix. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the "Defense Against Common-Mode Failure in Safety-Related, Software-Based I&C Systems," that were incorporated by reference have been resolved.

The staff compared the application to the relevant NRC regulations; acceptance criteria defined in NUREG-0800, Section 7.8 and BTPs 7-19; D&IC-ISG-02; and NRC Policy SECY-93-087 and concluded that the applicant is in compliance with the NRC regulations. The staff found that the applicant has adequately addressed the identified departures.