

Facility: <u>LaSalle County Station</u>		Date of Examination: 09/13/10-09/24/10
Examination Level: RO <input checked="" type="checkbox"/> SRO <input type="checkbox"/>		Operating Test Number: 2010301
Administrative Topic (see Note)	Type Code*	Describe activity to be performed
Conduct of Operations 2.1.2 4.1/4.4	N,S	Emergency Response actions of injured person per LAP-950-3, "Handling Personnel Injuries".
Conduct of Operations 2.1.5 2.9/3.9	N,S	Determine availability to assume shift work hour rules application.
Equipment Control 2.2.41 3.5/3.9	N, S	Identify isolation points for a leak in the fire protection header.
Radiation Control 2.3.7 3.5/3.6	D, S	Determine brief and protective clothing requirements of an RWP.
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.		
* Type Codes & Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) (N)ew or (M)odified from bank (≥ 1) (P)revious 2 exams (≤ 1; randomly selected)		

Facility: <u>LaSalle County Station</u>		Date of Examination: 09/13/10 – 09/24/10
Examination Level: RO <input type="checkbox"/> SRO <input checked="" type="checkbox"/>		Operating Test Number: 2010301
Administrative Topic (see Note)	Type Code*	Describe activity to be performed
Conduct of Operations 2.1.4 3.3/3.8	D,S	Reactivation of an SRO License per OP-AA-105-102, NRC Active License Maintenance
Conduct of Operations 2.1.5 2.9/3.9	N,S	Determine availability to assume shift work hour rules application.
Equipment Control 2.2.41 3.5/3.9	N, S	Identify isolation points for a leak in the fire protection header and identify any T.S. actions.
Radiation Control 2.3.7 3.5/3.6	D, S	Determine brief and protective clothing requirements of an RWP.
Emergency Procedures/Plan 2.4.29 4.4	N,S	Classify a Security Event IAW EP-AA-1005
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.		
* Type Codes & Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) (N)ew or (M)odified from bank (≥ 1) (P)revious 2 exams (≤ 1; randomly selected)		

Facility: <u>LaSalle County Station</u>		Date of Examination: 09/13/10 – 09/24/10
Exam Level: RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U <input type="checkbox"/>		Operating Test No.: 2010301
Control Room Systems [@] (8 for RO); (7 for SRO-I); (2 or 3 for SRO-U, including 1 ESF)		
System / JPM Title	Type Code*	Safety Function
a. Reactor Recirc. System / Downshift RR Pumps /Failure of one RR pump to downshift (202001, A4.01,3.7/3.7)	S,A,D	1
b. Reactor Water Level Control/ Loss of TDRFP speed feedback signal/restore to Auto (259002, A4.03, 3.8/3.6)	S,N	2
c. Emergency – High Reactor Pressure/ RPV Depressurization via RT per LGA-RT-102/ Subsequent leak (295025, A.2.2.44, 4.2/4.4)	S,A, N,L	3
d. Reactor Core Isolation Cooling/ Perform RCIC S/D following spurious init. with failure of the injection valve to close (217000, A2.01, 3.8/3.7)	S,A,D	4
e. Rod Position Indication System/RPIS data fault/ substitute rod position (214000, A4.02, 3.8/3.8)	S,N	7
f. Standby Gas Treatment/Manually initiate SGBT in response to irradiated fuel assembly damage (261000, A2.12, 3.2/3.4)	S,N,EN	9
g. Fuel Handling / Installation of Control Rod Block IAW LFP-100-1 (234000, A4.02, 3.4/3.7)	C,N,L	8
h. AC Electrical Distribution / Loss of 141Y Hard Card – Failure to Bus Tie (295003 a.1.01, 3.7/3.8)	S,A,D,L	6
In-Plant Systems [@] (3 for RO); (3 for SRO-I); (3 or 2 for SRO-U)		
i. DC Electrical System / DC Load shed after Station Blackout (263000, A4.01, 3.3/3.5)	D,L,E,R	6
j. Reactor Low Water Level / Fill SBLC tank with alternate source of water for alternate injection (295031, A1.08, 3.8/3.9)	D,E,A,R	2
k. Primary Containment System / Manually isolate a stuck open primary containment vacuum breaker (223001, A2.02, 3.9/4.1)	D,A,EN,R	5
<p>@ All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.</p>		
* Type Codes	Criteria for RO / SRO-I / SRO-U	
(A)lternate path	4-6 / 4-6 / 2-3	
(C)ontrol room		
(D)irect from bank	≤ 9 / ≤ 8 / ≤ 4	
(E)mergency or abnormal in-plant	≥ 1 / ≥ 1 / ≥ 1	
(EN)gineered safety feature	- / - / ≥ 1 (control room system)	
(L)ow-Power / Shutdown	≥ 1 / ≥ 1 / ≥ 1	
(N)ew or (M)odified from bank including 1(A)	≥ 2 / ≥ 2 / ≥ 1	
(P)revious 2 exams	≤ 3 / ≤ 3 / ≤ 2 (randomly selected)	
(R)CA	≥ 1 / ≥ 1 / ≥ 1	
(S)imulator		

Facility: <u>LaSalle County Station</u>		Date of Examination: 09/13/10 – 09/24/10
Exam Level: RO <input type="checkbox"/> SRO-I <input checked="" type="checkbox"/> SRO-U <input type="checkbox"/>		Operating Test No.: 2010301
Control Room Systems [@] (8 for RO); (7 for SRO-I); (2 or 3 for SRO-U, including 1 ESF)		
System / JPM Title	Type Code*	Safety Function
a. Reactor Recirc. System / Downshift RR Pumps /Failure of one RR pump to downshift (202001, A4.01,3.7/3.7)	S,A,D	1
b. Reactor Water Level Control/ Loss of TDRFP speed feedback signal/restore to Auto (259002, A4.03, 3.8/3.6)	S,N	2
c. Emergency – High Reactor Pressure/ RPV Depressurization via RT per LGA-RT-102/ Subsequent leak (295025, A.2.2.44, 4.2/4.4)	S,A,N,L	3
d. Reactor Core Isolation Cooling/ Perform RCIC S/D following spurious init. with failure of the injection valve to close (217000, A2.01, 3.8/3.7	S,A,D	4
e. Rod Position Indication System/RPIS data fault/ substitute rod position (214000, A4.02, 3.8/3.8)	S,N	7
f. Standby Gas Treatment/Manually initiate SBT in response to irradiated fuel assembly damage (261000, A2.12, 3.2/3.4)	S,N,EN	9
g. Fuel Handling / Installation of Control Rod Block IAW LFP-100-1 (234000, A4.02, 3.4/3.7	C,N,L	8
h. n/a	n/a	n/a
In-Plant Systems [@] (3 for RO); (3 for SRO-I); (3 or 2 for SRO-U)		
i. DC Electrical System / DC Load shed after Station Blackout (263000, A4.01, 3.3/3.5	D,L,E,R	6
j. Reactor Low Water Level / Fill SBLC tank with alternate source of water for alternate injection (295031, A1.08, 3.8/3.9)	D,E,A,R	2
k. Primary Containment System / Manually isolate a stuck open primary containment vacuum breaker (223001, A2.02, 3.9/4.1)	D,A,EN,R	5
<p>[@] All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.</p>		
* Type Codes	Criteria for RO / SRO-I / SRO-U	
(A)lternate path	4-6 / 4-6 / 2-3	
(C)ontrol room		
(D)irect from bank	≤ 9 / ≤ 8 / ≤ 4	
(E)mergency or abnormal in-plant	≥ 1 / ≥ 1 / ≥ 1	
(EN)gineered safety feature	- / - / ≥1 (control room system)	
(L)ow-Power / Shutdown	≥ 1 / ≥ 1 / ≥ 1	
(N)ew or (M)odified from bank including 1(A)	≥ 2 / ≥ 2 / ≥ 1	
(P)revious 2 exams	≤ 3 / ≤ 3 / ≤ 2 (randomly selected)	
(R)CA	≥ 1 / ≥ 1 / ≥ 1	
(S)imulator		

Facility LaSalle County Station Date of Exam 09/13/2010 Operating Test No 2010301

A P P L I C A N T	E V E N T T Y P E	Scenarios													T O T A L	M I N I M U M (-)		
		1			2			3			4							
		C R E W P O S I T I O N			C R E W P O S I T I O N			C R E W P O S I T I O N			C R E W P O S I T I O N							
		S R O	A T C	B O P														
														R	I	U		
<input checked="" type="checkbox"/> RO <input type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U RO-1 RO-2	RX								2					1	1	1	0	
	NOR			2										1	1	1	1	
	I/C			3,5			2,3,5,6		4,6					8	4	4	2	
	MAJ			6,7			7		7,8					5	2	2	1	
	TS													0	0	2	2	
<input checked="" type="checkbox"/> RO <input type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U RO-3	RX											1		1	1	1	0	
	NOR			2										1	1	1	1	
	I/C			3,5			2,3,5,6					3,4,		8	4	4	2	
	MAJ			6,7			7					7		4	2	2	1	
	TS													0	0	2	2	
<input type="checkbox"/> RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U SRO-1 SRO-3	RX					1								1	1	1	0	
	NOR	2								1				2	1	1	1	
	I/C	1,3,4,5				4				3,5				7	4	4	2	
	MAJ	6,7				7				7,8				5	2	2	1	
	TS	1,3,4												3	0	2	2	
<input type="checkbox"/> RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U SRO-2 SRO-4	RX				1			2						2	1	1	0	
	NOR							1						1	1	1	1	
	I/C	1,4			2,3,4,5,6			3,4,5,6						11	4	4	2	
	MAJ	6,7			7			7,8						5	2	2	1	
	TS				2,3,4			2,3,4						6	0	2	2	

Instructions:

- Check the applicant level and enter the operating test number and Form ES-D-1 event numbers for each event type; TS are not applicable for RO applicants. ROs must serve in both the "at-the-controls (ATC)" and "balance-of-plant (BOP)" positions; Instant SROs must serve in both the SRO and the ATC positions, including at least two instrument or component (I/C) malfunctions and one major transient, in the ATC position. If an Instant SRO *additionally* serves in the BOP position, one I/C malfunction can be credited toward the two I/C malfunctions required for the ATC position.
- Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.5.d) but must be significant per section C.2.a of Appendix D. (-) Reactivity and normal evolutions may be replaced with additional instrument or component malfunctions on a 1-for-1 basis.
- Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements specified for the applicant's license level in the right hand columns.

Facility		LaSalle County Station					Date of Exam			09/13/2010			Operating Test No					2010301		
A P P L I C A N T	E V E N T T Y P E	Scenarios													T O T A L	M I N I M U M (-)				
		1			2			3			4									
		C R E W P O S I T I O N			C R E W P O S I T I O N			C R E W P O S I T I O N			C R E W P O S I T I O N									
		S R O	A T C	B O P	S R O	A T C	B O P	S R O	A T C	B O P	S R O	A T C	B O P							
														R	I	U				
<input type="checkbox"/> RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U <input type="checkbox"/> SRO-5	RX					1									1	1	1	0		
	NOR	2													2	1	1	1		
	I/C	1,3,4,5				4									6	4	4	2		
	MAJ	6,7				7									4	2	2	1		
	TS	1,3,4													3	0	2	2		
<input type="checkbox"/> RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U <input type="checkbox"/> SRO-6 <input type="checkbox"/> SRO-8	RX					1									1	1	1	0		
	NOR					2,3,4,5,6									1	1	1	1		
	I/C		1,4			7									8	4	4	2		
	MAJ		2,5			2,3,4									4	2	2	1		
	TS														3	0	2	2		
<input type="checkbox"/> RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U <input type="checkbox"/> SRO-7	RX												1		1	1	0			
	NOR	2													1	1	1	1		
	I/C	1,3,4,5				2,3,5,6						3,4			10	4	4	2		
	MAJ	7				7						7			3	2	2	1		
	TS	3,4													2	0	2	2		
<input type="checkbox"/> RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U <input type="checkbox"/> SRO-9	RX					1							1		2	1	1	0		
	NOR			2									2		2	1	1	1		
	I/C			35		4							345		6	4	4	2		
	MAJ			67		7							7		4	2	2	1		
	TS												56		2	0	2	2		

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- Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.5.d) but must be significant per section C.2.a of Appendix D. (-) Reactivity and normal evolutions may be replaced with additional instrument or component malfunctions on a 1-for-1 basis.
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A P P L I C A N T	E V E N T T Y P E	Scenarios												T O T A L	M I N I M U M(-)		
		1			2			3			4						
		CREW POSITION			CREW POSITION			CREW POSITION			CREW POSITION						
		S R O	A T C	B O P													
														R	I	U	
<input type="checkbox"/> RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U SRO-10	RX							2				1		2	1	1	0
	NOR							1						1	1	1	1
	I/C							3,4, 5,6				3,4		6	4	4	2
	MAJ							7,8				7		3	2	2	1
	TS							2,3,4						3	0	2	2
<input type="checkbox"/> RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U SRO-11	RX								2		1			2	1	1	0
	NOR										2			1	1	1	1
	I/C								4,6		3,4,5			5	4	4	2
	MAJ								7,8		7			3	2	2	1
	TS										5,6			2	0	2	2
<input type="checkbox"/> RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U															1	1	0
															1	1	1
															4	4	2
															2	2	1
															0	2	2
<input type="checkbox"/> RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U															1	1	0
															1	1	1
															4	4	2
															2	2	1
															0	2	2

Instructions:

1. Check the applicant level and enter the operating test number and Form ES-D-1 event numbers for each event type; TS are not applicable for RO applicants. ROs must serve in both the "at-the-controls (ATC)" and "balance-of-plant (BOP)" positions; Instant SROs must serve in both the SRO and the ATC positions, including at least two instrument or component (I/C) malfunctions and one major transient, in the ATC position. If an Instant SRO *additionally* serves in the BOP position, one I/C malfunction can be credited toward the two I/C malfunctions required for the ATC position.
2. Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.5.d) but must be significant per section C.2.a of Appendix D. (-) Reactivity and normal evolutions may be replaced with additional instrument or component malfunctions on a 1-for-1 basis.
3. Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements specified for the applicant's license level in the right hand columns.

LaSalle Class 09-1 NRC ILT Exam
Written Examination Outline
Emergency and Abnormal Plant Evolutions – Tier 1 Group 2

Facility:		LaSalle Co. Station										Date of Exam: 09/13/2010					
Tier	Group	RO K/A Category Points											SRO-Only Points				
		K1	K2	K3	K4	K5	K6	A1	A2	A3	A4	G*	Total	A2	G*	Total	
1. Emergency & Plant Evolutions	1	3	3	3				3	4			4	20	4	3	7	
	2	1	1	1				1	2			1	7	1	2	3	
	Tier Totals	4	4	4				4	6			5	27	5	5	10	
2. Plant Systems	1	2	2	2	2	2	3	3	3	2	2	3	26	3	2	5	
	2	1	1	1	2	1	1	1	1	1	1	1	12	0	1	2	
	Tier Totals	3	3	3	4	3	4	4	4	3	3	4	38	4	4	8	
3. Generic Knowledge & Abilities Categories					1	2		3		4		10	1	2	3	4	7
					3	2		3		2			2	1	2	2	
Note :	<p>1. Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two).</p> <p>2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ±1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.</p> <p>3. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems that are not included on the outline should be added. Refer to section D.1.b of ES-401, for guidance regarding elimination of inappropriate K/A statements.</p> <p>4. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.</p> <p>5. Absent a plant specific priority, only those KAs having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.</p> <p>6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.</p> <p>7.* The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. Refer to Section D.1.b of ES-401 for the applicable K/A's</p> <p>8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IR) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above. If fuel handling equipment is sampled in other than Category A2 or G* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2 (Note #1 does not apply). Use duplicate pages for RO and SRO-only exams.</p> <p>9. For Tier 3, select topics from Section 2 of the K/A Catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10CFR55.43</p>																

LaSalle Class 09-1 NRC ILT Exam
Written Examination Outline
Emergency and Abnormal Plant Evolutions – Tier 1 Group 2

EAPE # / Name Safety Function	K1	K2	K3	A1	A2	G	K/A Topic(s)	Imp.	Q#
295021 Loss of Shutdown Cooling / 4					X		AA2.02 - Ability to determine and/or interpret the following as they apply to LOSS OF SHUTDOWN COOLING : RHR/shutdown cooling system flow	3.4	76
295023 Refueling Acc Cooling Mode / 8					X		AA2.03 - Ability to determine and/or interpret the following as they apply to REFUELING ACCIDENTS : Airborne contamination levels	3.8	77
600000 Plant Fire On-site / 8					X		AA2.06 - Ability to determine and interpret the following as they apply to PLANT FIRE ON SITE: Need for pressurizing control room (recirculating mode)	2.8	78
295038 High Off-site Release Rate / 9						X	2.1.7 - Conduct of Operations: Ability to evaluate plant performance and make operational judgements based on operating characteristics, reactor behavior, and instrument interpretation.	4.7	79
295026 Suppression Pool High Water Temp. / 5						X	2.2.44 - Equipment Control: Ability to interpret control room indications to verify the status and operation of a system, and understand how operator actions and directives effect plant and system conditions.	4.4	80
295006 SCRAM / 1						X	2.2.40 - Equipment Control: Ability to apply technical specifications for a system.	4.7	81
295031 Reactor Low Water Level / 2					X		EA2.04 - Ability to determine and/or interpret the following as they apply to REACTOR LOW WATER LEVEL : Adequate core cooling	4.8	82
295006 SCRAM / 1	X						AK1.02 - Knowledge of the operational implications of the following concepts as they apply to SCRAM : Shutdown margin	3.4	39
295021 Loss of Shutdown Cooling / 4	X						AK1.02 - Knowledge of the operational implications of the following concepts as they apply to LOSS OF SHUTDOWN COOLING : Thermal stratification	3.3	40
295037 SCRAM Conditions Present and Reactor Power Above APRM Downscale or Unknown / 1	X						EK1.04 - Knowledge of the operational implications of the following concepts as they apply to SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN: Hot shutdown boron weight: Plant-Specific	3.4	41
295005 Main Turbine Generator Trip / 3		X					AK2.07 - Knowledge of the interrelations between MAIN TURBINE GENERATOR TRIP and the following: Reactor pressure control	3.6	42
295019 Partial or Total Loss of Inst. Air / 8		X					AK2.05 - Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF INSTRUMENT AIR and the following: Main steam system	3.4	43
700000 Generator Voltage and Electric Grid Disturbances		X					AK2.07 - Knowledge of the interrelations between GENERATOR VOLTAGE AND ELECTRIC GRID DISTURBANCES and the following: Turbine/generator control.	3.6	44
600000 Plant Fire On-site / 8			X				AK3.04 - Knowledge of the reasons for the following responses as they apply	2.8	45

LaSalle Class 09-1 NRC ILT Exam
Written Examination Outline
Emergency and Abnormal Plant Evolutions – Tier 1 Group 2

EAPE # / Name Safety Function	K1	K2	K3	A1	A2	G	K/A Topic(s)	Imp.	Q#
							to PLANT FIRE ON SITE: Actions contained in the abnormal procedure for plant fire on site		
295016 Control Room Abandonment / 7			X				AK3.03 - Knowledge of the reasons for the following responses as they apply to CONTROL ROOM ABANDONMENT : Disabling control room controls	3.5	46
295025 High Reactor Pressure / 3			X				EK3.09 - Knowledge of the reasons for the following responses as they apply to HIGH REACTOR PRESSURE : Low-low set initiation: Plant-Specific	3.7	47
295004 Partial or Total Loss of DC Pwr / 6				X			AA1.02 - Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF D.C. POWER : Systems necessary to assure safe plant shutdown	3.8	48
295031 Reactor Low Water Level / 2				X			EA1.04 - Ability to operate and/or monitor the following as they apply to REACTOR LOW WATER LEVEL : High pressure core spray: Plant-Specific	4.3	49
295024 High Drywell Pressure / 5				X			EA1.14 - Ability to operate and/or monitor the following as they apply to HIGH DRYWELL PRESSURE: Drywell ventilation system	3.4	50
295030 Low Suppression Pool Water Level / 5					X		EA2.03 - Ability to determine and/or interpret the following as they apply to LOW SUPPRESSION POOL WATER LEVEL : Reactor pressure	3.7	51
295038 High Off-site Release Rate / 9					X		EA2.03 - Ability to determine and/or interpret the following as they apply to HIGH OFF-SITE RELEASE RATE : Radiation levels	3.5	52
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4					X		AA2.03 - Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION : Actual core flow	3.3	53
295023 Refueling Acc Cooling Mode / 8						X	2.1.28 - Conduct of Operations: Knowledge of the purpose and function of major system components and controls.	4.1	54
295003 Partial or Complete Loss of AC / 6						X	2.4.8 - Emergency Procedures / Plan: Knowledge of how abnormal operating procedures are used in conjunction with EOP's.	3.8	55
295026 Suppression Pool High Water Temp. / 5						X	2.4.3 - Emergency Procedures / Plan: Ability to identify post-accident instrumentation.	3.7	56
295028 High Drywell Temperature / 5					X		EA2.02 - Ability to determine and/or interpret the following as they apply to HIGH DRYWELL TEMPERATURE : Reactor pressure	3.8	57
295018 Partial or Total Loss of CCW / 8						X	2.4.11 - Emergency Procedures / Plan; Knowledge of abnormal condition procedures.	4.0	58
K/A Category Totals:	3	3	3	3	4/4	4/3	Group Point Total:		20/7

LaSalle Class 09-1 NRC ILT Exam
 Written Examination Outline
 Plant Systems – Tier 2 Group 1

EAPE # / Name Safety Function	K1	K2	K3	A1	A2	G	K/A Topic(s)	Imp.	Q#
295020 Inadvertent Cont. Isolation / 5 & 7					X		AA2.02 - Ability to determine and/or interpret the following as they apply to INADVERTENT CONTAINMENT ISOLATION : Drywell/containment temperature	3.4	83
295017 High Off-site Release Rate / 9						X	2.4.44 - - Emergency Procedures / Plan: Knowledge of emergency plan protective action recommendations.	4.4	84
295009 Low Reactor Water Level / 2						X	2.2.25 – Equipment Control: Knowledge of the bases in Technical Specifications for limiting conditions for operations and safety limits.	4.2	85
295008 High Reactor Water Level / 2	X						AK1.03 - Knowledge of the operational implications of the following concepts as they apply to HIGH REACTOR WATER LEVEL : Feed flow/steam flow mismatch	3.2	59
295022 Loss of CRD Pumps / 1		X					AK2.07 - Knowledge of the interrelations between LOSS OF CRD PUMPS and the following: Reactor pressure (SCRAM assist): Plant-Specific	3.4	60
295013 High Suppression Pool Temperature / 5			X				AK3.02 - Knowledge of the reasons for the following responses as they apply to HIGH SUPPRESSION POOL TEMPERATURE : Limiting heat additions	3.6	61
295015 Incomplete SCRAM / 1				X			AA1.08 - Ability to operate and/or monitor the following as they apply to INCOMPLETE SCRAM : Process computer/SPDS/ERIS/CRIDS/GDS: Plant-Specific	2.7	62
295032 High Secondary Containment Area Temperature / 5					X		EA2.01 - Ability to determine and/or interpret the following as they apply to HIGH SECONDARY CONTAINMENT AREA TEMPERATURE : Area temperature	3.8	63
295035 Secondary Containment High Differential Pressure / 5						X	2.4.50 - Emergency Procedures / Plan: Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.	4.2	64
295036 Secondary Containment High Sump/Area Water Level / 5					X		EA2.03 - Ability to determine and/or interpret the following as they apply to SECONDARY CONTAINMENT HIGH SUMP/AREA WATER LEVEL : Cause of the high water level	3.4	65
K/A Category Totals:	1	1	1	1	2/1	1/2	Group Point Total:		7/3

LaSalle Class 09-1 NRC ILT Exam
 Written Examination Outline
 Plant Systems – Tier 2 Group 2

System # / Name	K1	K2	K3	K4	K5	K6	A1	A2	A3	A4	G	K/A Topics	Imp	Q#
262002 UPS (AC/DC)								X				A2.01 - Ability to (a) predict the impacts of the following on the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Under voltage	2.8	86
211000 SLC								X				A2.03 - Ability to (a) predict the impacts of the following on the STANDBY LIQUID CONTROL SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: A.C. power failures	3.4	87
205000 Shutdown Cooling											X	2.4.6 - Emergency Procedures / Plan: Knowledge of EOP mitigation strategies.	4.7	88
215004 Source Range Monitor											X	2.2.36 - Equipment Control: Ability to analyze the effect of maintenance activities, such as degraded power sources, on the status of limiting conditions for operations.	4.2	89
264000 Emergency Generators								X				A2.03 - Ability to (a) predict the impacts of the following on the EMERGENCY GENERATORS (DIESEL/JET) ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Operating unloaded, lightly loaded, and highly loaded.	3.4	90
259002 Reactor Water Level Control	X											K1.03 - Knowledge of the physical connections and/or cause- effect relationships between REACTOR WATER LEVEL CONTROL SYSTEM and the following: Reactor water level	3.8	1
223002 PCIS/Nuclear Steam Supply Shutoff	X											K1.11 - Knowledge of the physical connections and/or cause- effect relationships between PRIMARY CONTAINMENT ISOLATION SYSTEM/NUCLEAR STEAM SUPPLY SHUT-OFF and the Containment atmosphere sampling	2.9	2
215003 IRM		X										K2.01 - Knowledge of electrical power supplies to the following: IRM channels/detectors	2.5	3
262001 AC Electrical Distribution		X										K2.01 - Knowledge of electrical power supplies to the following: Off-site sources of power	3.3	4

LaSalle Class 09-1 NRC ILT Exam
 Written Examination Outline
 Plant Systems – Tier 2 Group 2

261000 SGTS				X																K3.05 - Knowledge of the effect that a loss or malfunction of the STANDBY GAS TREATMENT SYSTEM will have on following: Secondary containment radiation/ contamination levels	3.2	5
264000 EDGs				X																K3.03 - Knowledge of the effect that a loss or malfunction of the EMERGENCY GENERATORS (DIESEL/JET) will have on following: Major loads powered from electrical buses fed by the emergency generator(s)	4.1	6
263000 DC Electrical Distribution					X															K4.02 - Knowledge of D.C. ELECTRICAL DISTRIBUTION design feature(s) and/or interlocks which provide for the following: Breaker interlocks, permissives, bypasses and cross ties: Plant-Specific	3.1	7
203000 RHR/LPCI: Injection Mode					X															K4.10 - Knowledge of RHR/LPCI: INJECTION MODE (PLANT SPECIFIC) design feature(s) and/or interlocks which provide for the following: Dedicated injection system during automatic system initiation (injection valve interlocks)	3.9	8
205000 Shutdown Cooling						X														K5.02 - Knowledge of the operational implications of the following concepts as they apply to SHUTDOWN COOLING SYSTEM (RHR SHUTDOWN COOLING MODE) : Valve operation	2.8	9
215005 APRM / LPRM							X													K5.05 - Knowledge of the operational implications of the following concepts as they apply to AVERAGE POWER RANGE MONITOR/LOCAL POWER RANGE MONITOR SYSTEM : Core flow effects on APRM trip setpoints	3.6	10
262002 UPS (AC/DC)								X												K6.01 - Knowledge of the effect that a loss or malfunction of the following will have on the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) : A.C. electrical power	2.7	11
212000 RPS									X											K6.03 - Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR PROTECTION SYSTEM : Nuclear boiler instrumentation	3.5	12
209001 LPCS										X										A1.07 - Ability to predict and/or monitor changes in parameters associated with operating the LOW PRESSURE CORE SPRAY SYSTEM controls including: Emergency generator loading	3.0	13

LaSalle Class 09-1 NRC ILT Exam
 Written Examination Outline
 Plant Systems – Tier 2 Group 2

211000 SLC										X											A1.01 - Ability to predict and/or monitor changes in parameters associated with operating the STANDBY LIQUID CONTROL SYSTEM controls including: Tank level	3.6	14																					
215004 Source Range Monitor																					X											A2.02 - Ability to (a) predict the impacts of the following on the SOURCE RANGE MONITOR (SRM) SYSTEM ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: SRM inop condition	3.4	15										
239002 SRVs																					X											A2.04 - Ability to (a) predict the impacts of the following on the RELIEF/SAFETY VALVES ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: ADS actuation	4.1	16										
400000 Component Cooling Water																															X						A3.01 - Ability to monitor automatic operations of the CCWS including: Setpoints on instrument signal levels for normal operations, warnings, and trips that are applicable to the CCWS	3.0	17					
209002 HPCS																															X						A3.06 - Ability to monitor automatic operations of the HIGH PRESSURE CORE SPRAY SYSTEM (HPCS) including: Lights and alarms: BWR-5,6	2.8	18					
218000 ADS																																				X						K/A A4.02 - Ability to manually operate and/or monitor in the control room: ADS logic initiation	4.2	19
217000 RCIC																																				X						A4.05 - Ability to manually operate and/or monitor in the control room: Reactor water level	4.1	20
300000 Instrument Air																																				X						2.2.4 Equipment Control: (multiple-unit license) Ability to explain the variations in control board/control room layouts, systems, instrumentation, and procedural actions between units at a facility.”	3.6	21
212000 RPS																																				X						2.4.45 - Emergency Procedures / Plan: Ability to prioritize and interpret the significance of each annunciator or alarm.	4.1	22
259002 Reactor Water Level Control																															X						A1.02 - Ability to predict and/or monitor changes in parameters associated with operating the REACTOR WATER LEVEL CONTROL SYSTEM controls including: Reactor feedwater flow	3.6	23					
263000 DC Electrical Distribution																																				X						2.2.3 - Equipment Control: (multi-unit license) Knowledge of the design, procedural, and operational differences between units.	3.8	24

LaSalle Class 09-1 NRC ILT Exam
 Written Examination Outline
 Plant Systems – Tier 2 Group 2

262001 AC Electrical Distribution								X				A2.01 - Ability to (a) predict the impacts of the following on the A.C. ELECTRICAL DISTRIBUTION ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Turbine/generator trip	3.4	25
203000 RHR/LPCI: Injection Mode						X						K6.09 - Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: INJECTION MODE (PLANT SPECIFIC) : Nuclear boiler instrumentation	3.4	26
K/A Category Totals:	2	2	2	2	2	3	3	3/3	2	2	3/2	Group Point Total:	26/5	

LaSalle Class 09-1 NRC ILT Exam
 Written Examination Outline
 Plant Systems – Tier 2 Group 2

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topics	Imp	Q#
201002 RMCS								X				A2.02 - Ability to (a) predict the impacts of the following on the REACTOR MANUAL CONTROL SYSTEM ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Rod drift alarm	3.3	91
219000 RHR/LPCI: Torus/Pool Cooling Mode											X	2.4.20 – Equipment Control: Knowledge of the operational implications of EOP warnings, cautions, and notes.	4.3	92
241000 Reactor/Turbine Pressure Regulator											X	2.2.38 - Equipment Control: Knowledge of conditions and limitations in the facility license.	4.5	93
256000 Reactor Condensate	X											K1.05 - Knowledge of the physical connections and/or cause- effect relationships between REACTOR CONDENSATE SYSTEM and the following: CRD hydraulics system	3.1	27
239001 Main and Reheat Steam		X										K2.01 - Knowledge of electrical power supplies to the following: Main steam isolation valve solenoids	3.2	28
234000 Fuel Handling Equipment			X									K3.03 - Knowledge of the effect that a loss or malfunction of the FUEL HANDLING EQUIPMENT will have on following: Fuel handling operations	3.1	29
202001 Recirculation				X								K4.02 - Knowledge of RECIRCULATION System design feature(s) and/or interlocks which provide for the following: Adequate recirculation pump NPSH	3.1	30
290002 Reactor Vessel Internals					X							K5.02 - Knowledge of the operational implications of the following concepts as they apply to REACTOR VESSEL INTERNALS : Fission product poisons	2.9	31
286000 Fire Protection						X						K6.02 - Knowledge of the effect that a loss or malfunction of the following will have on the FIRE PROTECTION SYSTEM D. C . electrical distribution	2.8	32
272000 Radiation Monitoring							X					A1.01 - Ability to predict and/or monitor changes in parameters associated with operating the RADIATION MONITORING SYSTEM controls including: Lights, alarms, and indications associated with normal operations	3.2	33

LaSalle Class 09-1 NRC ILT Exam
 Written Examination Outline
 Plant Systems – Tier 2 Group 2

202002 Recirculation Flow Control										X								A2.02 - Ability to (a) predict the impacts of the following on the RECIRCULATION FLOW CONTROL SYSTEM ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Loss of A.C.	2.9	34
215001 Traversing In-core Probe											X							A3.03 - Ability to monitor automatic operations of the TRAVERSING IN-CORE PROBE including: Valve operation: Not-BWR1	2.5	35
245000 Main Turbine Generator and Aux. Systems												X						A4.01 - Ability to manually operate and/or monitor in the control room: Turbine lube oil pumps	2.7	36
288000 Plant Ventilation													X					2.4.4 - Emergency Procedures / Plan: Ability to recognize abnormal indications for system operating parameters which are entry-level conditions for emergency and abnormal operating procedures.	4.5	37
214000 RPIS				X														K4.02 - Knowledge of the effect that a loss or malfunction of the ROD POSITION INFORMATION SYSTEM will have on following: Thermocouple	2.5	38
K/A Category Totals:	1	1	1	2	1	1	1	1	1	1	1	1	1	1	1	1	1	Group Point Total:		12/3

Facility:		LaSalle County Station		Exam Date:		09/13/2010	
Category	K/A #	Topic	RO		SRO-Only		
			IR	Q#	IR	Q#	
1. Conduct of Operations	2.1.4	Knowledge of individual licensed operator responsibilities related to shift staffing, such as medical requirements, "no-solo" operation, maintenance of active license status, 10CFR55, etc.			3.8	94	
	2.1.20	Ability to interpret and execute procedure steps.			4.6	98	
	2.1.36	Knowledge of procedures and limitations involved in core alterations.	3.0	66			
	2.1.32	Ability to explain and apply all system limits and precautions.	3.8	67			
	2.1.44	Knowledge of RO duties in the control room during fuel handling such as responding to alarms from the fuel handling area, communication with fuel storage facility, systems operated from the control room in support of fueling operations, and supporting instrumentation.	3.9	74			
	Subtotal			3		2	
2. Equipment Control	2.2.5	Knowledge of the process for making design or operating changes to the facility.			3.2	95	
	2.2.42	Ability to recognize system parameters that are entry-level conditions for Technical Specifications.	3.9	68			
	2.2.15	Ability to determine the expected plant configuration using design and configuration control documentation, such as drawings, line-ups, tag-outs, etc.	3.9	69			
	Subtotal			2		1	
3. Radiation Control	2.3.11	Ability to control radiation releases.			4.3	96	
	2.3.13	Knowledge of Radiological Safety Procedures pertaining to licensed operator duties, such as response to radiation monitor alarms, containment entry requirements, fuel handling responsibilities, access to locked high radiation areas, aligning filters, etc.			3.8	100	
	2.3.15	Knowledge of radiation monitoring systems, such as fixed radiation monitors and alarms, portable survey instruments, personnel monitoring equipment, etc.	2.9	70			
	2.3.4	Knowledge of radiation exposure limits under normal or emergency conditions.	3.2	71			
	2.3.14	Knowledge of radiation or containment hazards that may arise during normal, abnormal, or emergency conditions or activities.	3.4	75			
	Subtotal			3		2	

4. Emergency Procedures / Plan	2.4.38	Ability to take actions called for in the facility emergency plan, including supporting or acting as emergency coordinator if required.			4.4	97
	2.4.35	Knowledge of local auxiliary operator tasks during emergency and the resultant operational effects.			4.0	99
	2.4.34	Knowledge of RO tasks performed outside the main control room during an emergency and the resultant operational effects.	4.2	72		
	2.4.41	Knowledge of the emergency action level thresholds and classifications.	2.9	73		
	Subtotal				2	2
Tier 3 Point Total				10		7

Note: Western Technical Services BWR Outline software was used to randomly select K/As.

Question #	Tier / Group	Randomly Selected K/A	Reason for Rejection
21	Tier 2 / Gr. 1 RO	300000 2.2.36	2.2.36 - Equipment Control: Ability to analyze the effect of maintenance activities, such as degraded power sources, on the status of limiting conditions for operations" was also selected for SRO question #89. 2.2.4 Equipment Control: (multiple-unit license) Ability to explain the variations in control board/control room layouts, systems, instrumentation, and procedural actions between units at a facility." was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology to replace the original K/A to prevent duplication of the K/A on the exam.
56	Tier 1 / Gr. 1 RO	295026 2.4.6	2.4.6 – Emergency Procedures / Plan: "Knowledge of EOP mitigation strategies" was also selected for SRO question #88. K/A 2.4.3 "Emergency Procedures / Plan "Ability to identify post-accident instrumentation" was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology to replace the original K/A to prevent duplication of the K/A on the exam.
84	Tier 1 / Gr. 2 SRO	295022	EAP 295022 was over-sampled, so a new EAP was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology. The new EAP is 295017, High Off-site release rate. The original Generic K/A is retained, which is 2.2.22.
85	Tier 1 / Gr. 2 SRO	295013 2.2.3	EAP 295013 was over-sampled, so a new EAP was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology. The new EAP is 295009, Low Reactor Water Level. Generic 2.2.3 Equipment Control: (multi-unit license) Knowledge of the design, procedural, and operational differences between units, is not applicable to SRO questions. Replaced with Generic K/A 2.2.25 "Equipment Control: Knowledge of the bases in Technical Specifications for limiting conditions for operations and safety limits", which was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology.
87	Tier 2 / Gr. 1 SRO	212000	System 212000 was over-sampled, so a new system was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology. The new system is 211000, Standby Liquid Control. The original K/A is retained, which is A2.03.
90	Tier 2 / Gr. 1 SRO	262001 A2.11	System 262001 was over-sampled, so a new system was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology. The new system is 264000, Emergency Generators (Diesel/Jet). The originally selected K/A A2.11 does not exist under System 264000, so K/A 2.03 Ability to (a) predict the impacts of the following on the EMERGENCY GENERATORS (DIESEL/JET) ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Operating unloaded, lightly loaded, and highly loaded, was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology.

Question #	Tier / Group	Randomly Selected K/A	Reason for Rejection
92	Tier 2 / Gr. 2 SRO	219000 2.4.2	2.4.2 - Emergency Procedures / Plan: "Knowledge of system set points, interlocks and automatic actions associated with EOP entry conditions" is not applicable to SRO questions. Replaced with Generic K/A 2.4.20 "Knowledge of the operational implications of EOP warnings, cautions, and notes", which was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology.
93	Tier 2 / Gr. 2 SRO	241000 2.2.12	2.2.12 - Equipment Control: "Knowledge of surveillance procedures", is not applicable to SRO questions. Replaced with Generic K/A 2.2.38 "Knowledge of conditions and limitations in the facility license", which was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology.
Changes after initial submittal			
15	Tier 2 / Gr. 1 RO	239002 A2.02	A2.02 - Ability to (a) predict the impacts of the following on the RELIEF/SAFETY VALVES ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Leaky SRV, was originally selected however it was not possible to create a question that had specific procedural direction provided for a "Leaky SRV". Replaced with K/A A2.04 - Ability to (a) predict the impacts of the following on the RELIEF/SAFETY VALVES ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: ADS actuation, which was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology.
19	Tier 2 / Gr. 1 RO	218000 A4.04	A4.04 – Ability to manually operate and/or monitor in the control room: ADS inhibit: Plant Specific. This K/A was rejected due to overlap with the operating portion of the exam. Replaced with K/A A4.02 - Ability to manually operate and/or monitor in the control room: ADS logic initiation, which was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology.
25	Tier 2 / Gr. 1 RO	262001 A2.08	A2.08 - Ability to (a) predict the impacts of the following on the A.C. ELECTRICAL DISTRIBUTION ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Opening a disconnect under load was originally selected however it was not possible to create a question that had specific procedural direction provided for "Opening a disconnect under load". Replaced with A2.01 - Ability to (a) predict the impacts of the following on the A.C. ELECTRICAL DISTRIBUTION ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Turbine/generator trip, which was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology.
36	Tier 2 / Gr. 2 RO	204000 A4.01	A4.01 - Ability to manually operate and/or monitor in the control room: System pumps. Original K/A overlapped with a simulator JPM and required replacement. Replaced with System 245000, Main Turbine Generator and Auxiliary Systems, which was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology. Original K/A number was retained, and

			is basically the same, A4.01 - Ability to manually operate and/or monitor in the control room: Turbine lube oil pumps
58	Tier 1 / Gr. 1 RO	295018 2.4.30	2.4.30 - Emergency Procedures / Plan; Knowledge of events related to system operation / status that must be reported to internal organizations or external agencies, such as the state, the NRC, or the transmission system operator was originally selected however it was not possible to create a question on the RO level and tied to a CCW system. Replaced with 2.4.11, Knowledge of abnormal condition procedures which was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology.
78	Tier 1 / Gr.1 SRO	600000 AA2.11	AA2.11 - Ability to determine and interpret the following as they apply to PLANT FIRE ON SITE: "Time limit for use of respirators", was originally selected however it was not possible to create a question which could be supported with specific time limits for respirator use. Time limits specified in support documents were general in nature, using language such as "short term", "approximately", etc. Replaced with K/A AA2.06 Ability to determine and interpret the following as they apply to PLANT FIRE ON SITE: Need for pressurizing control room (recirculating mode), which was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology.
84	Tier 1 / Gr. 2 SRO	295017 2.2.22	2.2.22 – Equipment Control: "Knowledge of limiting conditions for operations and safety limits" was originally selected however it was not possible to create a question which tied an LCO or safety limit to 295017, High Off-site release rate. Replaced with K/A 2.4.44 "Knowledge of emergency plan protective action recommendations" which was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology.
91	Tier 2 / Gr. 2 SRO	201002 A2.01	A2.01 - Ability to (a) predict the impacts of the following on the REACTOR MANUAL CONTROL SYSTEM ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Rod movement sequence timer malfunctions was originally selected, however it was not possible to create a question because the electronic circuitry used in RCMS has integrated back-up checks to deal with timer issues. Replaced with A2.02, - Ability to (a) predict the impacts of the following on the REACTOR MANUAL CONTROL SYSTEM ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Rod drift alarm, which was randomly selected using ES-401 Attachment 1 Systematic Sampling Methodology.

Facility: LaSalle County Station Scenario No.: 3 Test No.: 2010301

Examiners: _____ Operators: _____

Initial Conditions: 100% Power

Turnover:

- S/D "C" Condensate/ Booster Pump IAW LOP-CD-03

Event No.	Malf. No.	Event Type*		Event Description
1	N/A	N	BOP	Swap CD pumps
2	MMS047	R	RO	Main Turbine #5 Bypass valve fails open (TS)
3	CAE	I	BOP	RCIC Drain Pot Drain Fails Closed (TS)
4	MNI095	I	RO	LPRM fails downscale (TS)
5	CAE	C	BOP	Steam Seal Evaporator PCV fails closed
6	CAE MMS056	C	RO	Turbine Lube Oil TCV fails open / Main Turbine Shaft Bow / Scram
7	MRD277,278	M	All	ATWS
8	MNB106, 109, 113	M	All	"A" MSL steam leak / Fails to isolate
* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor				

Facility: LaSalle County Station Scenario No.: 4 Test No.: 2010301

Examiners: _____ Operators: _____

Initial Conditions: Ramping to full power

Turnover:

- Unit 1 at 85% power ramping to full power.
- LOS-LP-Q1 needs completed from step A.6.
- "B" RHR OOS
- "A" IN Compressor OOS

Event No.	Malf. No.	Event Type*		Event Description
1	N/A	R	RO	Ramp to full power
2	N/A	N	BOP	Complete LOS-LP-Q1
3	MRW001	C	RO	"A" RT Pump trips
4	CAE	C	RO	"A" TDRFP lube oil leak
5	MRM009	I	BOP	"A" OG Post treat rad high (TS)
6	R1165 R0517	I	SRO	Primary Containment CAM 1PL75J and 1PL15J INOP (TS)
7	MNB104	M	ALL	Main Steam Line Leak in DW before restrictor
8	MMS048-052	C	BOP	Main Turbine Bypass Valves Fail Closed
9	MRC033	C	BOP	Primary Containment Bypass Path
10	VMRH16A	C	BOP	RHR Drywell Spray 16A fails closed.

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor