Facility:		Date	of E	xam	:					****					,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,			
Tier	Tier Group				RO K/A Category Points										SRO-Only Points			
		K 1	К 2	К 3	K 4	K 5	К 6	A 1	A 2	A 3	A 4	G *	Total		A2		G*	Total
1.	1	3	3	3				3	3			3	18		3	Ī	3	6
Emergency & Abnormal Plant	2	2	1	2		N/A			1	N/A	1	9		2		2	4	
Evolutions	Tier Totals	5	4	5				5	4	.``	<i>'</i> '^	4	27		5		5	10
	1	3	3	2	3	2	3	2	3	2	2	3	28		3		2	5
2. Plant	2	1	1	1	1	1	1	1	1	0	1	1	10		2		1	3
Systems	Tier Totals	4	4	3	4	3	4	3	4	2	3	4	38		5		3	8.
3. Generic Knowledge and Abilities							2	2	3	3	4		10	1	2	3	4	7
	ategories				(3	3	3	2	2	2			2	2	1	2	

- Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO
 and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals"
 in each K/A category shall not be less than two).
- The point total for each group and tier in the proposed outline must match that specified in the table.
 The final point total for each group and tier may deviate by ±1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.
- 3. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems that are not included on the outline should be added. Refer to ES-401, Attachment 2, for guidance regarding the elimination of inappropriate K/A statements.
- Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.
- Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.
- 6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.
- *The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system.
- 8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IRs) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above; if fuel handling equipment is sampled in other than Category A2 or G* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2 (Note # 1 does not apply). Use duplicate pages for RO and SRO-only exams.
- For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43.

ES-401, R	EV 9	T1	G1 PWR EXAMINATION OUTLINE	FORM ES-401-2		
KA	NAME / SAFETY FUNCTION:	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO SR	o			
007EG2.4.31	Reactor Trip - Stabilization - Recovery / 1	4.2 4.1		Knowledge of annunciators alarms, indications or response procedures		
008AA1.04	Pressurizer Vapor Space Accident / 3	2.8 2.5		Feedwater pumps		
009EK2.03	Small Break LOCA / 3	3 3.3		S/Gs		
011EG2.2.38						
011EG2.2.38	Large Break LOCA / 3	3.6 4.5		Knowledge of conditions and limitations in the facility license.		
015AK2.07	RCP Malfunctions / 4	2.9 2.9		RCP seals		
022AK1.01	Loss of Rx Coolant Makeup / 2	2.8 3.2		Consequences of thermal shock to RCP seals		
025AA1.19	Loss of RHR System / 4	2.6 2.4		Block orifice bypass valve controller and indicators		
027AA1.05	Pressurizer Pressure Control System	3.3 3.2		Transfer of heaters to backup power supply		
	Malfunction / 3			Mandrel of Medicine to backup power supply		
038EK1.04	Steam Gen. Tube Rupture / 3	3.1 3.3	V	Reflux boiling		
054AK1.01	Loss of Main Feedwater / 4	4.1 4.3		MFW line break depressurizes the S/G (similar to a		
				steam line break)		
055EG2.4.2	Station Blackout / 6	4.5 4.6		Knowledge of system set points, interlocks and automatic actions associated with EOP entry conditions.		

ES-401, REV 9		•	T1G1 PWR EXAMINATION OUTLINE	FORM ES-401-2		
KA	NAME / SAFETY FUNCTION:	IF	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO :	SRO			
056AK3.01	Loss of Off-site Power / 6	3.5	3.9	Order and time to initiation of power for the load sequencer		
057AA2.05	Loss of Vital AC Inst. Bus / 6	3.5	3.8	S/G pressure and level meters		
062AA2.02	Loss of Nuclear Svc Water / 4	2.9	3.6	The cause of possible SWS loss		
065AA2.03	Loss of Instrument Air / 8	2.6 2	2.9	Location and isolation of leaks		
077AK3.02	Generator Voltage and Electric Grid Disturbances / 6	3.6 3	3.9	Actions contained in abnormal operating procedures for voltage and grid disturbances		
WE04EK2.2	LOCA Outside Containment / 3	3.8 4	4.0	Facility's heat removal systems, including primary coolant, emergency coolant, the decay heat removal systems and relations between the proper operation of these systems to the operation of the facility.		
WE05EK3.2	Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4	3.7 4	4.1	Normal, abnormal and emergency operating procedures associated with (Loss of Secondary Heat Sink).		

ES-401, REV 9			T10	32 PWR EXAMINATION OUTLINE	FORM ES-401-2	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
003AK3.10	Dropped Control Rod / 1	3.2	4.2		RIL and PDIL	
032AA2.07	Loss of Source Range NI / 7	2.8	3.4		Maximum allowable channel disagreement	
051AA1.04	Loss of Condenser Vacuum / 4	2.5	2.5		Rod position	
060AG2.1.25	Accidental Gaseous Radwaste Rel. / 9	3.9	4.2		Ability to interpret reference materials such as graphs, monographs and tables which contain performance data.	
061AK2.01	ARM System Alarms / 7	2.5	2.6		Detectors at each ARM system location	
068AK3.02	Control Room Evac. / 8	3.7	4.1		System response to turbine trip	
WE01EK1.2	Rediagnosis / 3	3.4	4	2	Normal, abnormal and emergency operating procedures associated with (Reactor Trip or Safety Injection / Rediagnosis).	
WE13EA1.1	Steam Generator Over-pressure / 4	3.1	3.3		Components and functions of control and safety systems, including instrumentation, signals, interlocks, failure modes and automatic and manual features.	
WE16EK1.1	High Containment Radiation / 9	2.7	3.0	Ø	Components, capacity, and function of emergency systems.	

E3-401, REV 9			T20	G1 PWR EXAMINATION OUTLINE	FORM ES-401-2		
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRC	0			
003G2.4.35	Reactor Coolant Pump	3.8	4.0		Knowledge of local auxiliary operator tasks during emergency and the resultant operational effects		
004G2.1.23	Chemical and Volume Control	4.3	4.4		Ability to perform specific system and integrated plant procedures during all modes of plant operation.		
004K6.31	Chemical and Volume Control	3.1	3.5		Seal injection system and limits on flow range		
005K2.03	Residual Heat Removal	2.7	2.8		RCS pressure boundary motor-operated valves		
005K6.03	Residual Heat Removal	2.5	2.6		RHR heat exchanger		
006K6.02	Emergency Core Cooling	3.4	3.9		Core flood tanks (accumulators)		
007K1.03	Pressurizer Relief/Quench Tank	3.0	3.2		RCS		
007K4.01	Pressurizer Relief/Quench Tank	2.6	2.9		Quench tank cooling		
008K2.02	Component Cooling Water	3.0	3.2		CCW pump, including emergency backup		
010K1.08	Pressurizer Pressure Control	3.2	3.5	V	PZR LCS		
012K1.05	Reactor Protection	3.8	3.9	Ø	ESFAS		

ES-401, REV 9			T20	G1 PWR EXAMINATION OUTLINE	FORM ES-401-:		
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRC				
012K5.01	Reactor Protection	3.3	3.8		DNB		
013K5.02	Engineered Safety Features Actuation	2.9	3.3		Safety system logic and reliability		
022A1.01	Containment Cooling	3.6	3.7		Containment temperature		
022A2.03	Containment Cooling	2.6	3.0		Fan motor thermal overload/high-speed operation		
026A3.02	Containment Spray	3.9	4.2		Verification that cooling water is supplied to the containment spray heat exchanger		
039A2.01	Main and Reheat Steam	3.1	3.2		Flow paths of steam during a LOCA		
059A3.02	Main Feedwater	2.9	3.1		Programmed levels of the S/G		
061A1.01	Auxiliary/Emergency Feedwater	3.9	4.2		S/G level		
062G2.1.27	AC Electrical Distribution	3.9	4		Knowledge of system purpose and or function.		
063K2.01	DC Electrical Distribution	2.9	3.1		Major DC loads		
063K4.02	DC Electrical Distribution	2.9	3.2		Breaker interlocks, permissives, bypasses and cross-ties.		

ES-401, F	REV 9	T20	31 PWR EXAMINATION OUTLINE	FORM ES-401-2	
KA	NAME / SAFETY FUNCTION:	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO SRO			
064A2.13	Emergency Diesel Generator	2.6 2.8		Consequences of opening auxiliary feeder bus (ED/G sub supply)	
073A4.02	Process Radiation Monitoring	3.7 3.7		Radiation monitoring system control panel	
073K3.01	Process Radiation Monitoring	3.6 4.2		Radioactive effluent releases	
076K3.02	Service Water	2.5 2.8		Secondary closed cooling water	
078K4.01	Instrument Air	2.7 2.9		Manual/automatic transfers of control	
103A4.03	Containment	2.7 2.7		ESF slave relays	

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ES-401, REV 9			T20	32 PWR EXAMINATION OUTLINE	FORM ES-401-2		
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G TOPIC:			
		RO	SRC				
001K2.05	Control Rod Drive	3.1	3.5	☐ ☑ ☐ ☐ ☐ ☐ M/G sets			
002A2.03	Reactor Coolant	4.1	4.3		<u> </u>		
016K1.08	Non-nuclear Instrumentation	3.4	3.4	PZR PCS			
028K5.01	Hydrogen Recombiner and Purge Control	3.4	3.9	Explosive hydrogen concentration			
033A1.02	Spent Fuel Pool Cooling	2.8	3.3	☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐			
035 G 2.2.40	Steam Generator	3.4	4.7	☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐	or a system.		
055K3.01	Condenser Air Removal	2.5	2.7	☐ ☐ ☑ ☐ ☐ ☐ ☐ ☐ Main condenser			
071K4.06	Waste Gas Disposal	2.7	3.5	Sampling and monitoring of waste gas r	elease tanks		
079A4.01	Station Air	2.7	2.7	☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐			
086K6.04	Fire Protection	2.6	2.9	Fire, smoke and heat detectors			

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ES-401, REV 9		S	RO 1	11G1 PWR EXAMINATION OUTLINE	FORM ES-401-2	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
008AA2.28	Pressurizer Vapor Space Accident / 3	3.3	3.9		Safety parameter display system indications	
011EG2.4.11	Large Break LOCA / 3	4.0	4.2		Knowledge of abnormal condition procedures.	
025AG2.4.31	Loss of RHR System / 4	4.2	4.1		Knowledge of annunciators alarms, indications or response procedures	
056AA2.73	Loss of Off-site Power / 6	3.5	3.6		PZR heater on/off	
065AG2.1.30	Loss of Instrument Air / 8	4.4	4.0		Ability to locate and operate components, including local controls.	
WE04EA2.1	LOCA Outside Containment / 3	3.4	4.3		Facility conditions and selection of appropriate procedures during abnormal and emergency operations.	

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ES-401, REV 9		s	RO T	T1G2 PWR EXAMINATION OUTLINE	FORM ES-401-2	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC			
003AG2.1.30	Dropped Control Rod / 1	4.4	4.0		Ability to locate and operate components, including local controls.	
028AG2.4.20	Pressurizer Level Malfunction / 2	3.8	4.3		Knowledge of operational implications of EOP warnings, cautions and notes,	
068AA2.03	Control Room Evac. / 8	4	4.2		T-hot, T-cold and in-core temperatures	
WE10EA2.1	Natural Circ. With Seam Void/ 4	3.2	3.9		Facility conditions and selection of appropriate procedures during abnormal and emergency operations.	

Page 1 of 1

ES-401, R	EV 9	SRO	T2G1 PWR EXAMINATION OUTLINE	FORM ES-401-2		
KA	NAME / SAFETY FUNCTION:	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO SRO				
003G2.4.41	Reactor Coolant Pump	2.9 4.6		Knowledge of the emergency action level thresholds and classifications.		
007A2.03	Pressurizer Relief/Quench Tank	3.6 3.9		Overpressurization of the PZR		
010G2.1.20	Pressurizer Pressure Control	4.6 4.6		Ability to execute procedure steps.		
026A2.03	Containment Spray	4.1 4.4		Failure of ESF		
073A2.02	Process Radiation Monitoring	2.7 3.2		Detector failure		

ES-401, REV 9			RO 1	T2G2 PWR EXAMINATION OUTLINE	FORM ES-401		
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRC	0			
034G2.4.20	Fuel Handling Equipment	3.8	4.3		Knowledge of operational implications of EOP warnings, cautions and notes.		
072A2.03	Area Radiation Monitoring	2.7	2.9		Blown power-supply fuses		
079A2.01	Station Air	2.9	3.2		Cross-connection with IAS		

ES-401, REV 9		SRO T3 PWR EXAMINATION OUTLINE			FORM ES-401-2	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
G2.1.35	Conduct of operations	2.2	3.9		Knowledge of the fuel handling responsibilities of SRO's	
G2.1.40	Conduct of operations	2.8	3.9		Knowledge of refueling administrative requirements	
G2.2.19	Equipment Control	2.3	3.4		Knowledge of maintenance work order requirements.	
G2.2.35	Equipment Control	3.6	4.5		Ability to determine Technical Specification Mode of Operation	
G2.3.5	Radiation Control	2.9	2.9		Ability to use radiation monitoring systems	
G2.4.27	Emergency Procedures/Plans	3.4	3.9		Knowledge of "fire in the plant" procedures.	
G2.4.39	Emergency Procedures/Plans	3.9	3.8		Knowledge of the RO's responsibilities in emergency plan implementation.	

Facility: Plant Vogtle

Examination Level: RO

Date of Examination: 02/22/2010 Operating Test Number: 2010-301

	1	T	
Administrative Topic (see Type Note) Code*		Describe activity to be performed	
,	R, D, P	Title: Critical Safety Function Status Tree Evaluation	
Conduct of Operations		Description: Students will be provided a listing of plant parameters. This will require manually evaluating each status tree to determine the challenges to each tree and identify the highest priority challenge.	
		K/A: G2.1.7 (RO 4.4)	
		Title: Perform QPTR Calculation (QPTR)	
Conduct of Operations	R, N	Description: Perform Quadrant Power Tilt Ration (QPTR) Calculation, both the top and bottom sections will be out of spec high.	
		K/A: G2.1.37 (RO 4.3)	
		Title: Construct Tagout for Unit 2 # 4 Nuclear Service Cooling Water (NSCW) Pump	
Equipment Control	R, D	Description: Determine the appropriate hold points and required positions of components to safely isolate NSCW pump 2-1202-P4-004. Identification of K-2 links to clear control room alarms is not required.	
		K/A: G2.2.13 (RO 4.1)	
		Title: Not applicable.	
Radiation Control		Description: Not applicable.	
radiation control		K/A: Not applicable.	
Emergency Procedures/Plan	S, N	Title: Emergency Recall & Roll Call for Security Emergency	
<u> </u>	,	Description: An emergency recall and roll call is required to be performed for a security emergency. This requires the use of a different recall scenario.	
		K/A: G2.4.43 (RO 3.2)	

NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.

* Type Codes & Criteria:

(C)ontrol room, (S)imulator, or Class(R)oom

(D)irect from bank (\leq 3 for ROs; \leq 4 for SROs & RO retakes)

(N)ew or (M)odified from bank (≥ 1)

(P)revious 2 exams (≤ 1; randomly selected)

Describe activity to be performed orm Emergency Boration Flow Path The plant will be in Mode 4 with the SI pumps common HV-8806 tagged shut and a sustained loss of instrument This will require the student to evaluate the effect of the the boric acid flow paths in addition to the tagged SRO 4.7) rmine Active License Status the SRO will have to evaluate the status of 3 RO's to ir license status for sitting the control board position.
The plant will be in Mode 4 with the SI pumps common HV-8806 tagged shut and a sustained loss of instrument This will require the student to evaluate the effect of the the boric acid flow paths in addition to the tagged SRO 4.7) rmine Active License Status the SRO will have to evaluate the status of 3 RO's to ir license status for sitting the control board position.
HV-8806 tagged shut and a sustained loss of instrument This will require the student to evaluate the effect of the the boric acid flow paths in addition to the tagged SRO 4.7) rmine Active License Status the SRO will have to evaluate the status of 3 RO's to ir license status for sitting the control board position.
rmine Active License Status he SRO will have to evaluate the status of 3 RO's to ir license status for sitting the control board position.
he SRO will have to evaluate the status of 3 RO's to ir license status for sitting the control board position.
ir license status for sitting the control board position.
SRO 3.8)
struct Tagout for Unit 2 # 4 Nuclear Service ater (NSCW) Pump
Determine the appropriate hold points and required omponents to safely isolate NSCW pump 2-1202-P4-004.
(SRO 4.3)
rgency Exposure Limits Calculation and ion
distorical data will be provided for individuals to receive an posure during a declared event. Data will be provided culation to determine the exposure to be received, the determine the appropriate dose limit based on the exposure to be received.
SRO 3.7)
ade - Emergency Classification (From Alert a Emergency)
New data will be provided to the candidate to determine if
a declared emergency is required. Once it is determined required, the ENN Notification Form will be required to be is will be a time critical JPM.
required, the ENN Notification Form will be required to be is will be a time critical JPM. SRO 4.6)
required, the ENN Notification Form will be required to be is will be a time critical JPM.

Control Room/In-Plant Systems Outline

Form ES-301-2

Draft

Facility: Vogtle Exam Level (circle one): RO / SRO-I / SRO-U	Date of Examina Operating Te	ation: 02/22/2010 st No.: 2010-301
Control Room Systems [®] (8 for RO; 7 for SRO-I; 2 or 3 for SRO-U, incli	uding 1 ESF)	
System / JPM Title	Type Code*	Safety Function
a. Dropped Rod Recovery in Mode 1 (RO / SRO-I)	S, M, A	1
A dropped rod will be recovered IAW AOP-18003-C, Rod Control Malfunction Section A for Dropped Rods in Mode 1. During the recovery, the rod being retrieved will not move. This will require reconfiguring the rod control system back to its' previous state prior to attempting the dropped rod recovery.		
K/A: 003AA2.01 (RO 3.7 / SRO 3.9)		
b. Respond to Failure of PRZR Level Instrument (RO / SRO-I)	S, D, A	2
A PRZR level instrument will fail low requiring entry into AOP-18001 Section D. After to candidate attempts to place letdown back in service, a letdown isolation valve will NOT open requiring Excess Letdown to be placed into service.		
K/A: 028AA1.05 (RO 3.7 / SRO 3.6)		
c. Transfer ECCS Pumps to Hot Leg Recirculation. Multiple Train "A" Components Fail to Align (RO / SRO-I)	S, D, A, P	3
During alignment of ECCS to Hot Leg Recirculation, multiple Train A components will fail to align requiring the use of the RNO column.	EN, L	
K/A: 006A4.05 (RO 3.9 / SRO 3.8)		
d. Respond To RHR Pump Trip in Mode 4 (RO / SRO-I)	S, N, A, L	4P
An RHR pump will trip in Mode 4 requiring entry into AOP-18019, Section A, Loss of RHR Capability in Mode 4. The flow path through the procedure will require the candidate to start an RCP.		
K/A: 025G2.4.9 (RO 3.8 / SRO 4.2)		
e. Dilute Containment with Service Air in accordance with 19010-C, E-1, Loss of Reactor or Secondary Coolant, Step 29. (RO / SRO-I)	S, D, L	5
The student will be required to use SOP 13130-1, section 4.4.2 to align service air to containment to dilute the hydrogen.		
K/A: 028A4.01 (RO 4.0 / SRO 4.0)	١	

ES-301, Rev. 9E Control Room/In-Plant Systems Outline

Form ES-301-2

\mathcal{D}	٣	a.	J+
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f.	Synchronize Main Generator to the Grid. Field amps not indicated on one phase requires Main Turbine trip. (RO)	S, D, A, P	6
	After synchronizing the main generator the student must recognize that one phase does not show any current, requiring opening of the generator output breaker to prevent damage to the generator.		
	K/A: 062A4.07 (RO 3.1)		
g.	Respond to Fuel Handling Building High Radiation (RO/SRO-I)	S, D, A	7
	High Radiation will occur due to dropped rod in FHB but auto actuation fails, manual system alignment is required.		
	K/A: 061AA1.01 (RO 3.6 / SRO 3.6)		
h.	Transfer Steam Dumps to Steam Pressure Mode (RO / SRO-I)	S, N, A, L	4S
	The student will be required to transfer steam dumps to the steam pressure mode using step 14 of AOP 18009-C. The steam dumps will not work, the student must shift the temperature control to the SG ARVs.		
	K/A: 041A4.08 (RO 3.0 / SRO 3.1)		

In-Plant Systems [®] (3 for RO; 3 for SRO-I; 3 or 2 fo	r SRO-U)		
i. Bypass CNMT HI-1 Bistables Following Loss Bistables must be bypassed using the BTI equipment feedwater flow path to the SGs during a loss of secon also require knowledge of how / where to obtain multi 2 trains of equipment (RO / SRO-I) K/A: 012A4.03 (3.6 / 3.6)	in order to align a dary heat sink. This will	D, E, L, P	7
j. Locally Start the TDAFW Pump Following a Control Room evacuation, the TDAFW pullocally started at Shutdown Panel C. Local transfer swing manipulated, the T & T valve checked for proper status flow from the local panel is required using available flow. K/A: 068AA1.02 (RO 4.3 / SRO 4.5)	vitches must be	D, E, L	8
k. Locally Isolate RCP Seals / ACCW per 19100- During a loss of All AC power, the RCP seals are requincluded on Attachment E is opening the breakers to the preparation for local operation. K/A: 055G2.4.34 (RO 4.2 / SRO 4.1)	ired to be isolated.	M, E, L, R	6
@ All control room (and in-plant) systems must be systems and functions may overlap those test	different and serve differed in the control room.	erent safety function	ons; in-plant
* Type Codes	Criteria fo	or RO/SRO-I/SF	RO-U

ES-301, Rev. 9E

Control Room/In-Plant Systems Outline

Form ES-301-2

(A)Iternate path (C)ontrol room	4-6 / 4-6 / 2-3
(D)irect from bank (E)mergency or abnormal in-plant (EN)gineered Safety Feature (L)ow-Power / Shutdown (N)ew or (M)odified from bank including 1(A) (P)revious 2 exams (R)CA (S)imulator	$\leq 9 / \leq 8 / \leq 4$ $\geq 1 / \geq 1 / \geq 1$ - / - / > 1 (control room system) $\geq 1 / \geq 1 / \geq 1$ $\geq 2 / \geq 2 / \geq 1$ $\leq 3 / \leq 3 / \leq 2$ (randomly selected) $\geq 1 / \geq 1 / \geq 1$

Dratt

Facility: Vogtle Exam Level (circle one): RO / SRO-I / SRO-U		ation: 02/22/2010 st No.: 2010-301
Control Room Systems [@] (8 for RO; 7 for SRO-I; 2 or 3 for SRO-U, inclu	uding 1 ESF)	
System / JPM Title	Type Code*	Safety Function
a. Dropped Rod Recovery in Mode 1 (RO / SRO-I)	S, M, A	1
A dropped rod will be recovered IAW AOP-18003-C, Rod Control Malfunction Section A for Dropped Rods in Mode 1. During the recovery, the rod being retrieved will not move. This will require reconfiguring the rod control system back to its' previous state prior to attempting the dropped rod recovery.		
K/A: 003AA2.01 (RO 3.7 / SRO 3.9)		
b. Respond to Failure of PRZR Level Instrument (RO / SRO-I)	S, D, A	2
A PRZR level instrument will fail low requiring entry into AOP-18001 Section D. After to candidate attempts to place letdown back in service, a letdown isolation valve will NOT open requiring Excess Letdown to be placed into service.		
K/A: 028AA1.05 (RO 3.7 / SRO 3.6)		
c. Transfer ECCS Pumps to Hot Leg Recirculation. Multiple Train "A" Components Fail to Align (RO / SRO-I)	S, D, A, P	3
During alignment of ECCS to Hot Leg Recirculation, multiple Train A components will fail to align requiring the use of the RNO column.	EN, L	
K/A: 006A4.05 (RO 3.9 / SRO 3.8)		
d. Respond To RHR Pump Trip in Mode 4 (RO / SRO-I)	S, N, A, L	4P
An RHR pump will trip in Mode 4 requiring entry into AOP-18019, Section A, Loss of RHR Capability in Mode 4. The flow path through the procedure will require the candidate to start an RCP.		
K/A: 025G2.4.9 (RO 3.8 / SRO 4.2)		
e. Dilute Containment with Service Air in accordance with 19010-C, E-1, Loss of Reactor or Secondary Coolant, Step 29. (RO / SRO-I)	S, D, L	5
The student will be required to use SOP 13130-1, section 4.4.2 to align service air to containment to dilute the hydrogen.		
K/A: 028A4.01 (RO 4.0 / SRO 4.0)		
f. N/A	N/A	N/A

Control Room/In-Plant Systems Outline Form ES-301-2

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g.	Respond to Fuel Handling Building High Radiation (RO/SRO-I)	S, D, A	7
	High Radiation will occur due to dropped rod in FHB but auto actuation fails, manual system alignment is required.		
	K/A: 061AA1.01 (RO 3.6 / SRO 3.6)		
h.	Transfer Steam Dumps to Steam Pressure Mode (RO / SRO-I)	S, N, A, L	4S
	The student will be required to transfer steam dumps to the steam pressure mode using step 14 of AOP 18009-C. The steam dumps will not work, the student must shift the temperature control to the SG ARVs.		
	K/A: 041A4.08 (RO 3.0 / SRO 3.1)		

In-Plant Systems [®] (3 for RO; 3 for SRO-I; 3 or 2 for SRO-U)		
i. Bypass CNMT HI-1 Bistables Following Loss of Heat Sink Bistables must be bypassed using the BTI equipment in order to align a feedwater flow path to the SGs during a loss of secondary heat sink. This will also require knowledge of how / where to obtain multiple BTI keys to bypass 2 trains of equipment (RO / SRO-I) K/A: 012A4.03 (3.6 / 3.6)	D, E, L, P	7
j. Locally Start the TDAFW Pump Following a Control Room evacuation, the TDAFW pump is required to be locally started at Shutdown Panel C. Local transfer switches must be manipulated, the T & T valve checked for proper status, and throttling of AFW flow from the local panel is required using available flow indications. K/A: 068AA1.02 (RO 4.3 / SRO 4.5)	D, E, L	8
k. Locally Isolate RCP Seals / ACCW per 19100-C, Attachment E During a loss of All AC power, the RCP seals are required to be isolated. Included on Attachment E is opening the breakers to the SG ARVs in preparation for local operation. K/A: 055G2.4.34 (RO 4.2 / SRO 4.1)	M, E, L, R	6

@ All control room (and in-plant) systems must be different and serve different safety functions; in-plant systems and functions may overlap those tested in the control room.

* Type Codes	Criteria for RO / SRO-I / SRO-U		
(A)Iternate path (C)ontrol room (D)irect from bank (E)mergency or abnormal in-plant (EN)gineered Safety Feature (L)ow-Power / Shutdown (N)ew or (M)odified from bank including 1(A) (P)revious 2 exams (R)CA	$4-6 / 4-6 / 2-3$ $\leq 9 / \leq 8 / \leq 4$ $\geq 1 / \geq 1 / \geq 1$ $- / - / > 1$ (control room system) $\geq 1 / \geq 1 / \geq 1$ $\geq 2 / \geq 2 / \geq 1$ $\leq 3 / \leq 3 / \leq 2$ (randomly selected) $\geq 1 / > 1 / > 1 / > 1$		