## U.S. Nuclear Regulatory Commission Surry Power Station

# SR10301 Job Performance Measure APE022.AA1.01

Applicant	Start Time
Examiner	
Date	Stop Time
<u>Title</u>	

ISOLATE CHARGING CROSSTIE IN ACCORDANCE WITH 2-ECA-0.2.

K/A: APE022.AA1.01: Ability to operate and / or monitor the following as they apply to the Loss of Reactor Coolant Makeup: CVCS letdown and Charging.

Applicability Estimated Time Actual Time

RO/SRO(I) 15 Minutes

#### **Initial Conditions**

- Task is to be SIMULATED in the plant.
- A Loss of Offsite Power has occurred on Unit 2 and power has subsequently been restored to Unit 2. The
  team has completed 2-ECA-0.0 (Loss of All A/C Power) and transitioned to 2-ECA-0.2 (Loss of All AC Power
  Recovery with SI Required) and are currently performing Step 15 (Check if Charging Crosstie Can Be
  Isolated) of 2-ECA-0.2.
- It has been determined that Charging Crosstie can be isolated. You are to complete Step 15 of 2-ECA-0.2.

#### **Standards**

 Candidate correctly manipulates valves as required by 2-ECA-0.2, step 15, to allow for isolation of Charging Crosstie.

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Shift Manager. Unit 2 had experienced a loss of offsite power and charging crosstie from Unit 1 to Unit 2 was utilized for RCS makeup.
- Following the restoration of power to Unit 2, the team transitioned to 2-ECA-0.2, Loss of All AC Power Recovery with SI Required.
- The team has reached step 15 of 2-ECA-0.2, Loss of All AC Power Recovery with SI Required.
- You are to perform step 15 sub-steps d and e.
- When you finish the actions necessary to accomplish this, please inform me.

#### **Terminating Cues**

• Completion of 2-ECA-0.2, Loss of All AC Power Recovery with SI Required, step 15.

#### **Procedures**

• 2-ECA-0.2, Loss of All AC Power Recovery with SI Required (Revision 17).

#### **Tools and Equipment**

## **Safety Considerations**

None

- Standard Personal Safety Equipment
- Bump hazards in Auxiliary Building Basement
- ALARA

#### Notes to the Evaluator.

- Task briefing should occur in the pre-determined location.
- This task is to be SIMULATED. Do NOT allow the operator to manipulate controls, operate switches or reposition valves.
- Task critical elements are bolded and noted by the words "Critical Step" at the end of the step.

#### **Initiating Cues**

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Nuclear Shift Manager. Unit 2 had experienced a loss of offsite power and charging crosstie from Unit 1 to Unit 2 was utilized for RCS makeup.
- Following the restoration of power to Unit 2, the team transitioned to 2-ECA-0.2, Loss of All AC Power Recovery with SI Required.
- The team has reached step 15 of 2-ECA-0.2, Loss of All AC Power Recovery with SI Required.
- You are to perform steps 15 sub-steps d and e.
- When you finish the actions necessary to accomplish this, please inform me.

#### **Notes**

Expected RWP to be used for 08.2 License Class is -

- RWP 0001
- Dose ALARM 5 mr
- Dose RATE ALARM- 20 mr/hr

# PERFORMANCE CHECKLIST

# **Notes to the Evaluator**

• Task critical elements are bolded and noted by the words "Critical Step" at the end of the step.

• START TIME:

STEP 1:  Reviews the NOTE prior to Step 15 and notes that Step 15 sub-steps 'a' through 'c' are signed off.	SAT
STANDARD:	
<ul> <li>Reads note regarding the fact that FRs may now be implemented.</li> <li>Notes that Step 15 sub-steps 'a' through 'c' are complete.</li> </ul>	
EVALUATOR'S NOTE:	
This item is not related to the task directed to be performed.	
COMMENTS:	

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STEP 2:	
	SAT
STEP 15d- Locally close CHG crosstie valves:	

**UNSAT** 

# • 1-CH-728

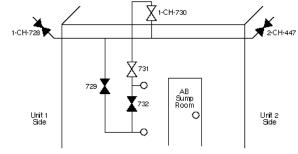
#### STANDARD:

- (a) Proceeds to Auxiliary Building Basement Unit 1 side of Aux Bldg.
- (b) Locates 1-CH-728 (see location sketch).
- (c) Places the ratchet selector in the DOWN position (rotates selector lever clockwise until bottom stop reached). This is a Critical Step.
- (d) Closes 1-CH-728 by "ratcheting" the valve handwheel to turn the valve handwheel in the clockwise direction. This is a Critical Step.

#### **EVALUATOR'S NOTE:**

- **If asked:** The valve ratchet selector is in the UP position and the valve is currently open (valve stem is extended).
- **Tell operator:** If operator attempts to reposition valve with ratchet selector in the UP position, as he/she pushes up on handwheel, it will not move since valve is open. If the operator pushes DOWN on handwheel, it will "click".
- **Tell operator:** AFTER moving ratchet selector to DOWN position, handwheel "clicks" as it is moved up & valve moves clockwise & begins closing as handwheel is pushed down.
- **Tell operator:** When 1-CH-728 properly operated, tell operator that flow noise is no longer heard through the line.
- When properly operated: Valve stem retracts in, operator ratchets the valve until valve no longer travels and flow noise is no longer heard.

#### Area location sketch:



Graphter No.: LESS.03

In order to close 1-CH-728, the ratchet selector must be in the DOWN position. The ratchet selector in the UP direction opens 1-CH-728.

#### **COMMENTS:**

SIEP 3:			

\_\_\_ SAT

STEP 15d- Locally close CHG crosstie valves (continued):

UNSAT

• 2-CH-447

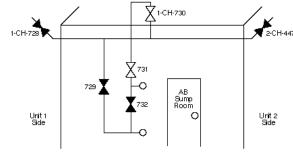
#### STANDARD:

- (a) Proceeds to Auxiliary Building Basement Unit 2 side.
- (b) Locates 2-CH-447 (see location sketch).
- (c) Closes 2-CH-447 (by turning valve handwheel in the clockwise direction). This is a Critical Step.

#### **EVALUATOR'S NOTE:**

- If asked: The valve is currently open (valve stem is extended).
- When correctly operated: Handwheel rotates, stem retracts in and then the handwheel stops.

Area location sketch:



**COMMENTS:** 

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STEP 4:	SAT
STEP 15e- Locally open CHG line valve:	
• 2-CH-304	UNSAT
STANDARD:  (a) Proceeds to Auxiliary Building Basement Unit 2 side.  (b) Locates 2-CH-304 (located near 2-CH-LCV-2122)  (c) Opens 2-CH-304 (by turning valve handwheel in the clockwise direction).  This is a Critical Step.	
<ul> <li>EVALUATOR'S NOTE:</li> <li>If asked: The valve is currently open (valve stem is extended).</li> <li>When correctly operated: Handwheel rotates, stem retracts in and then the handwheel stops.</li> </ul>	
COMMENTS:	

STEP 5:  Reports completion of the Step to the Shift Manager.  STANDARD:  • Verbal status report that charging crosstie from Unit 1 to Unit 2 has been locally secured in accordance with 2-ECA-0.2.	SAT
<ul><li>EVALUATOR'S NOTE:</li><li>Acknowledge the completion of the task.</li></ul>	
COMMENTS:	
STOP TIME:	

# Operator Directions Handout (TO BE READ TO APPLICANT BY EXAMINER)

#### **Task**

- Task is to be SIMULATED in the plant.
- Secure Charging Crosstie in accordance with 2-ECA-0.2, Loss of All AC Power Recovery with SI Required, Step 15.

#### **Directions**

The evaluator will explain the initial conditions of the task to be performed and will provide the initiating cue. Ensure you indicate to the evaluator when you understand your assigned task.

#### **Initial Conditions**

A Loss of Offsite Power has occurred on Unit 2 and power has subsequently been restored to Unit 2. The team
has completed 2-ECA-0.0 (Loss of All A/C Power) and transitioned to 2-ECA-0.2 (Loss of All AC Power Recovery
with SI Required) and are currently performing Step 15 (Check if Charging Crosstie Can Be Isolated) of 2-ECA0.2.

#### **Initiating Cues**

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Shift Manager. Unit 2 had experienced a loss of offsite power and charging crosstie from Unit 1 to Unit 2 was utilized for RCS makeup.
- Following the restoration of power to Unit 2, the team transitioned to 2-ECA-0.2, Loss of All AC Power Recovery with SI Required.
- The team has reached step 15 of 2-ECA-0.2, Loss of All AC Power Recovery with SI Required.
- You are to perform step 15 sub-steps d and e.
- When you finish the actions necessary to accomplish this, please inform me.

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# Operator Directions Handout (TO BE GIVEN TO APPLICANT)

#### **Initial Conditions**

A Loss of Offsite Power has occurred on Unit 2 and power has subsequently been restored to Unit 2. The team
has completed 2-ECA-0.0 (Loss of All A/C Power) and transitioned to 2-ECA-0.2 (Loss of All AC Power Recovery
with SI Required) and are currently performing Step 15 (Check if Charging Crosstie Can Be Isolated) of 2-ECA0.2.

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Shift Manager. Unit 2 had experienced a loss of offsite power and charging crosstie from Unit 1 to Unit 2 was utilized for RCS makeup.
- Following the restoration of power to Unit 2, the team transitioned to 2-ECA-0.2, Loss of All AC Power Recovery with SI Required.
- The team has reached step 15 of 2-ECA-0.2, Loss of All AC Power Recovery with SI Required.
- You are to perform step 15 sub-steps d and e.
- When you finish the actions necessary to accomplish this, please inform me.

NUMBER	PROCEDURE TITLE  LOSS OF ALL AC POWER RECOVERY WITH SI REQUIRED	REVISION 17
2-ECA-0.2	LOSS OF ALL AC POWER RECOVERY WITH SI REQUIRED	PAGE 10 of 10

OTED	ACTION/EXPECTED RESPONSE	l	RESPONSE NOT OBTAINED
STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
NOTE	: FRs may now be implemented as necess	sary.	
15	CHECK IF CHG CROSSTIE CAN BE ISOLATED:		
□	a) Check Unit 2 CHG pump - RUNNING		<ul> <li>a) GO TO Step 16. WHEN Unit 2 CHG pump running, THEN do Steps 15b through 15e.</li> </ul>
□	b) CHG crosstie - IN SERVICE		b) GO TO Step 16.
	c) Close CHG line isolation MOV(s):		
☑ ☑	<ul><li>2-CH-MOV-2289A</li><li>2-CH-MOV-2289B</li></ul>		
	d) Locally close CHG crosstie valves:		
	• 1-CH-728		
	• 2-CH-447		
	e) Locally open CHG line valve:		
	• 2-CH-304		
16	GO TO 2-E-1, LOSS OF REACTOR OR SECONDARY COOLANT		
	-	END -	

# U.S. Nuclear Regulatory Commission Surry Power Station

# SR10301 Job Performance Measure EPE029EA1.12 Alternate Path

Applicant	Start Tin	ne
Examiner		
Date	Stop Tin	ne
<u>Title</u>		
Locally Trip Unit 1 Reactor.		
K/A: EPE029EA1.12: Ability to e supply and reactor trip breakers	operate and monitor the following as they a s.	ipply to a ATWS: M/G set power
<u>Applicability</u>	Estimated Time	Actual Time
RO/SRO(I)	10 Minutes	

#### **Initial Conditions**

- Task is to be SIMULATED in the plant.
- An Anticipated Transient Without Scram (ATWS) is in progress on Unit 1 with a coincident failure of both bench board reactor trip push buttons. The operating team has entered FR-S.1, Response to Nuclear Power Generation/ATWS.
- A local trip of the reactor is required. You have been directed to open Unit 1 reactor trip breakers locally in accordance with FR-S.1 Step 8.a RNO.

#### **Standards**

Candidate opens Unit 1 Motor Generator Breakers and de-energizes power to the Rod Control System.

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Shift Manager. UNIT 1 is currently experiencing an ATWS and the control room reactor trip push buttons won't work.
- I need you to locally trip the UNIT 1 reactor trip breakers in accordance with FR-S.1 Step 8.a RNO.
- When you finish the actions necessary to accomplish this, please inform me.

# **Terminating Cues**

• Unit 1 Motor Generator Breakers are open.

#### **Procedures**

• FR-S.1, Response to Nuclear Power Generation/ATWS (Revision 26).

## **Tools and Equipment**

# **Safety Considerations**

None

Standard Personal Safety Equipment

#### Notes to the Evaluator.

- Task briefing should occur in the pre-determined location.
- This task is to be SIMULATED. Do NOT allow the operator to manipulate controls, operate switches or reposition valves.
- Task critical elements are bolded and noted by the words "Critical Step" at the end of the step.

#### **Initiating Cues**

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Shift Manager. UNIT 1 is currently experiencing an ATWS and the control room reactor trip push buttons won't work.
- I need you to locally trip the UNIT 1 reactor trip breakers in accordance with FR-S.1 Step 8.a RNO.
- When you finish the actions necessary to accomplish this, please inform me.

#### **Notes**

When the candidate attempts to open the Unit 1 Reactor Trip Breakers, the candidate will find that the reactor trip breakers cannot be manipulated. Upon report of this condition to the Control Room / Shift Manager, the evaluator will provide direction to open the MG set output breakers in accordance with FR-S.1 Step 8.a RNO.

# PERFORMANCE CHECKLIST

# **Notes to the Evaluator**

• Task critical elements are bolded and noted by the words "Critical Step" at the end of the step.

• START TIME:

STEP 1:	SAT
Step 8.a RNO – Send operator to locally trip Reactor:  Open reactor trip and bypass breakers OR Open the MG set output breakers. GEN NO. 1 GEN NO. 2	UNSAT
STANDARD:  a) Locates Unit 1 Reactor Trip and Bypass Breakers.	
b) Attempts to open Unit 1 Reactor Trip Breaker A (RTB A) by depressing the TRIP push button on the breaker.	
EVALUATOR'S NOTE:	
<ul> <li>a) Proceeds to Unit 1 Cable Spreading Area (between the Pzr Acc monitor cabinet and the ASC/RMP panels) and enters the Reactor Trip Breaker cubicle.</li> </ul>	
<ul> <li>b) If asked:</li> <li>Prior to breaker manipulation, the CLOSED indication is showing (i.e., the breaker is "as you see it").</li> <li>Following the breaker manipulation, no breaker movement is seen or heard and the CLOSED indication is showing (i.e., the breaker is "as you see it").</li> <li>The 'A' Reactor Trip Bypass Breaker has the OPEN indication showing (i.e., the breaker is "as you see it").</li> </ul>	
COMMENTS:	

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STEP 2:	SAT
Step 8.a RNO – (continued) Send operator to locally trip Reactor:  Open reactor trip and bypass breakers OR	UNSAT
Open the MG set output breakers.  GEN NO. 1  GEN NO. 2	
STANDARD:	
a) Attempts to open Unit 1 Reactor Trip Breaker B (RTB B) by depressing the TRIP push button on the breaker.	
b) Acknowledges that both of the Reactor Trip Breakers will NOT open locally.	
EVALUATOR'S NOTE:	
<ul> <li>a) If asked:</li> <li>Prior to breaker manipulation, the CLOSED indication is showing (i.e., the breaker is "as you see it").</li> <li>Following the breaker manipulation, no breaker movement is seen or heard and the CLOSED indication is showing (i.e., the breaker is "as you see it").</li> <li>The 'B' Reactor Trip Bypass Breaker has the OPEN indication showing (i.e., the breaker is "as you see it").</li> </ul>	
ADDITIONAL DIRECTION TO BE PROVIDED TO CANDIDATE:	
<ul> <li>b) Upon identification of the failure of the reactor trip breakers to be opened, provide the following direction to the candidate, as necessary.</li> <li>I need you to locally trip the UNIT 1 MG SET OUTPUT BREAKERS in accordance with FR-S.1 Step 8.a RNO.</li> <li>When you finish the actions necessary to accomplish this, please inform me.</li> </ul>	
COMMENTS:	

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Step 8.a RNO – (continued) Send operator to locally trip Reactor:  Open reactor trip and bypass breakers OR  Open the MG set output breakers. GEN NO. 1 GEN NO. 2	SATUNSAT
standard:  a) Locates Unit 1 MG Set Output Breakers.  b) Trips one of the Unit 1 MG Set Output Breakers by either of the following:  • Operating the breaker mounted trip mechanism (pushes the TRIP push button)  • Rotates the Generator Control pistol-grip switch to the TRIP position. This is a Critical Step.	
EVALUATOR'S NOTE:	
<ul> <li>a) Proceeds to the MG Set Control Panel (center of Unit 1 4160V Switchgear Room) and locates the MG Set Output Breakers (at bottom of panel).</li> </ul>	
<ul> <li>b) If asked:</li> <li>Both MG Sets are operating.</li> <li>Prior to breaker manipulation, the CLOSED indication is showing (i.e., the breaker is "as you see it").</li> <li>When properly operated, the MG Set Output Breaker indication changes from CLOSED to OPEN and breaker opening noise is heard.</li> </ul>	
COMMENTS:	

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STEP 4:	
SIEP 4.	SAT
	5A1
Step 8.a RNO – (continued)	UNSAT
Send operator to locally trip Reactor:	
Open reactor trip and bypass breakers	
OR	
Open the MG set output breakers.	
o GEN NO. 1	
o GEN NO. 2	
STANDARD:	
<ul> <li>a) Trips the other Unit 1 MG Set Output Breakers by either of the following:</li> <li>Operating the breaker mounted trip mechanism (pushes the TRIP push button)</li> </ul>	
Rotates the Generator Control pistol-grip switch to the TRIP position.	
This is a Critical Step.	
This is a Official Ocep.	
EVALUATOR'S NOTE:	
a) If asked:	
<ul> <li>Prior to breaker manipulation, the CLOSED indication is showing (i.e., the</li> </ul>	
breaker is "as you see it").	
<ul> <li>When properly operated, the MG Set Output Breaker indication changes from CLOSED to OPEN and breaker opening noise is heard.</li> </ul>	
COMMENTS:	

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Reports completion of the Step to the Shift Manager.  STANDARD:  • Verbal status report that Unit 1 MG Breakers have been opened.  EVALUATOR'S NOTE:  • Acknowledge the completion of the task.  COMMENTS:	SAT
STOP TIME:	

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# Operator Directions Handout (TO BE READ TO APPLICANT BY EXAMINER)

#### **Task**

- Task is to be SIMULATED in the plant.
- A local trip of the reactor is required. The Candidate has been directed to open Unit 1 reactor trip breakers locally in accordance with FR-S.1 Step 8.a RNO.

#### **Directions**

The evaluator will explain the initial conditions of the task to be performed and will provide the initiating cue. Ensure you indicate to the evaluator when you understand your assigned task.

#### **Initial Conditions**

 An Anticipated Transient Without Scram (ATWS) is in progress on Unit 1 with a coincident failure of both bench board reactor trip push buttons. The operating team has entered FR-S.1, Response to Nuclear Power Generation/ATWS.

#### **Initiating Cues**

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Shift Manager. UNIT 1 is currently experiencing an ATWS and the control room reactor trip push buttons won't work.
- I need you to locally trip the UNIT 1 reactor trip breakers in accordance with FR-S.1 Step 8.a RNO.
- When you finish the actions necessary to accomplish this, please inform me.

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# Operator Directions Handout (TO BE GIVEN TO APPLICANT)

#### **Initial Conditions**

 An Anticipated Transient Without Scram (ATWS) is in progress on Unit 1 with a coincident failure of both bench board reactor trip push buttons. The operating team has entered FR-S.1, Response to Nuclear Power Generation/ATWS.

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Shift Manager. UNIT 1 is currently experiencing an ATWS and the control room reactor trip push buttons won't work.
- I need you to locally trip the UNIT 1 reactor trip breakers in accordance with FR-S.1 Step 8.a RNO.
- When you finish the actions necessary to accomplish this, please inform me.

NUMBER	PROCEDURE TITLE RESPONSE TO NUCLEAR POWER GENERATION/ATWS	REVISION 26
1-FR-S.1	RESPONSE TO NUCLEAR POWER GENERATION/ATWS	PAGE 4 of 9

STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
7,611611/2/11/2012/11/2013/11/2018/11/2	NEG. GNOZ NOT GETAINED
8 CHECK IF THE FOLLOWING TRIPS HAVE OCCURRED:	<b>:</b>
☐ a) Reactor trip	a) Send operator to locally trip Reactor:
	<ul> <li>Open reactor trip and bypass breakers.</li> <li>OR</li> </ul>
	Open the MG set output breakers.
	□ • GEN NO. 1 □ • GEN NO. 2
	GEN NO. 2
□ b) Turbine trip	<ul> <li>b) Send operator to locally trip turbine using trip lever.</li> </ul>
	ceeded, the Gamma-Metrics Excore Neutron Monitor d be used to monitor neutron flux for the duration of the
*9 CHECK REACTOR - SUBCRITICAL	
<ul> <li>□ a) Check power range channels - LESS THAN 5% [Gamma-Metrics Wide Range Power - LESS THAN 5%]</li> </ul>	□ a) GO TO Step 10.
<ul> <li>b) Check Intermediate range channels - NEGATIVE STARTUP RATE</li> <li>[Gamma-Metrics Wide Range Power - DECREASING]</li> </ul>	□ b) GO TO Step 10.
□ c) GO TO Step 18	

## U.S. Nuclear Regulatory Commission Surry Power Station

## SR10301 Job Performance Measure 008.A2.02

Applicant	Start Time
Examiner	
Date	Stop Time
Title	

Fill the CC Head Tank using 1-BC-P-2.

K/A: 008.A2.02: Ability to (a) predict the impacts of the following malfunctions or operations on the CCWS, and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: High/low surge tank level.

Applicability Estimated Time Actual Time

RO/SRO(I) 15 Minutes

#### **Initial Conditions**

- Task is to be SIMULATED in the plant.
- An adverse trend was identified on the CC Head Tank and a CC leak was identified and subsequently isolated. The CC Head Tank level is low and needs to be filled; current level is 30%.
- The Water Treatment (WT) makeup header is not available to supply water to the CC Head Tank.
- It is desired to fill the CC Head Tank to 50% using 1-BC-P-2 in accordance with 0-OP-BC-001.

#### **Standards**

 Candidate correctly manipulates valves, starts 1-BC-P-2 in accordance with 0-OP-BC-001 and fills the CC Head Tank.

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Shift Manager. An adverse trend was identified on the CC Head Tank and a CC leak was identified and subsequently isolated.
- The CC Head Tank level is low and needs to be filled; current level is 30%.
- Water Treatment (WT) makeup header is not available to supply water to the CC Head Tank.
- You are to fill the CC Head Tank to 50% using 1-BC-P-2 in accordance with 0-OP-BC-001, Operation of 1-BC-P-2.
- When you finish the actions necessary to accomplish this, please inform me.

## **Terminating Cues**

The CC Head Tank has been filled to the desired level (50%).

#### **Procedures**

• 0-OP-BC-001, Operation of 1-BC-P-2 (Revision 7), Subsections 5.1, 5.2, and 5.4.

## **Tools and Equipment**

# **Safety Considerations**

None

Standard Personal Safety Equipment

# Notes to the Evaluator.

- Task briefing should occur in the pre-determined location.
- This task is to be SIMULATED. Do NOT allow the operator to manipulate controls, operate switches or reposition valves.
- Task critical elements are bolded and noted by the words "Critical Step" at the end of the step.

#### **Initiating Cues**

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Nuclear Shift Manager. An adverse trend was identified on the CC Head Tank and a CC leak was identified and subsequently isolated.
- The CC Head Tank level is low and needs to be filled; current level is 30%.
- Water Treatment (WT) makeup header is not available to supply water to the CC Head Tank.
- You are to fill the CC Head Tank to 50% using 1-BC-P-2 in accordance with 0-OP-BC-001, Operation of 1-BC-P-2.
- When you finish the actions necessary to accomplish this, please inform me.

#### **Notes**

# PERFORMANCE CHECKLIST

# **Notes to the Evaluator**

• Task critical elements are bolded and noted by the words "Critical Step" at the end of the step.

• START TIME:

STEP 1:	SAT
Reviews procedure Section 3.0, Initial Conditions, and Section 4.0, Precautions and Limitations.	UNSAT
3.0 Initial Conditions	
None	
4.0 Precautions and Limitations	
4.1 Push button station 1-BS-CS-P2, BC M/U Pump Control Switch, is mounted on the vertical beam near the pump.	
4.2 Because the LCVs for 1-CD-TK-1A, 1-CD-TK-1B, and 2-CD-TK-1, Chilled Water Surge Tanks, are not reliable, makeup is done using the LCV Bypass.	
4.3 The BC flow path is normally aligned from 1-BC-P-2 to Unit 1 BC System and to Unit 2 BC System. Starting 1-BC-P-2 will fill the head tanks if required.	
<ul> <li>4.4 The following equipment will be required if performing Subsection 5.9:</li> <li>Red rubber hose (100 ft) with Chicago fittings on both ends</li> <li>Five gallon bucket</li> </ul>	
STANDARD:	
<ul> <li>Reviews procedure Section 3.0, Initial Conditions, and Section 4.0, Precautions and Limitations.</li> </ul>	
EVALUATOR'S NOTE:	
<ul> <li>The candidate will only utilize Subsection 5.1, 5.2, and 5.4.</li> <li>If the candidate looks at the local CC Head Tank Level indication (by ESGR entry doors) indicate level at 30%.</li> </ul>	
COMMENTS:	

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STEP 2:	SAT
Section 5.4 – Filling the CC Surge Tank	
<b>Step 5.4.1-</b> Verify started or start 1-BC-P-2 IAW Subsection 5.1. Enter N/A if filling from the WT makeup header.	UNSAT
STANDARD:  a) Proceeds to procedure Subsection 5.4 and performs Step 5.4.1 which directs performance of Section 5.1, if not already performed.	
EVALUATOR'S NOTE:  a) If asked: The WT makeup header is not available for makeup to the CC Head Tank, use 1-BC-P-2.	
Based on the directions, it is acceptable for the candidate to commence with Subsection 5.1.	
COMMENTS:	

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STEP 3:	SAT
Section 5.1 – Starting 1-BC-P-2, BC Alternate M/U Pump	
Step 5.1.1- Verify that 1-BC-253, BC Alt M/U Pump Discharge Hdr Drain, is closed.	UNSAT
STANDARD:  a) Proceeds to Unit 1 Turbine Building Basement. b) Verifies that 1-BC-253 is closed.	
EVALUATOR'S NOTE:  a) Proceeds to the west side of Unit 1 Turbine Building Basement between the outlet waterboxes and the Condensate Pumps b) If asked: 1-BC-253 is "as you see it".	
COMMENTS:	
STEP 4: Section 5.1 – Starting 1-BC-P-2, BC Alternate M/U Pump	SAT
<b>Step 5.1.2</b> - Open 1-BC-177, BC Alt M/U Pump Suct.	UNSAT
STANDARD:  a) Opens 1-BC-177 by rotating the handwheel in the counter-clockwise direction. This is a Critical Step	
EVALUATOR'S NOTE:  a) If asked:  • Prior to valve manipulation 1-BC-177 is "as you see it" (i.e., closed).  • Following valve manipulation indicate that 1-BC-177 is OPEN (i.e., valve stem exposed).	
COMMENTS:	

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STEP 5:  Section 5.1 – Starting 1-BC-P-2, BC Alternate M/U Pump  Step 5.1.3- Open 1-BC-180, BC Alt M/U Pump Recirc Line Isol.  STANDARD:  a) Opens 1-BC-180 by rotating the handwheel in the counter-clockwise direction. This is a Critical Step  EVALUATOR'S NOTE:	SAT
a) If asked:  • Prior to valve manipulation 1-BC-180 is "as you see it" (i.e., closed).  • Following valve manipulation indicate that 1-BC-180 is OPEN.  COMMENTS:	
STEP 6:  Caution prior to Step 5.1.4: Valve 1-BC-179 may require throttling to prevent cavitation when condensate temperature is high.	SAT
STANDARD:  a) Acknowledges the Caution.	
evaluator's Note:  a) If asked:  • Throttle 1-BC-179 2 turns open.  The operator may decide to throttle 1-BC-179 open until cavitation occurs and then throttle closed on 1-BC-179.	
COMMENTS:	

STEP 7:		
Section 5.1 – S	Starting 1-BC-P-2, BC Alternate M/U Pump	SAT UNSAT
<b>Step 5.1.4</b> - Th	rottle open or open 1-BC-179, BC Alt M/U Pump Disch.	
STANDARD: a)	Throttles open or opens 1-BC-179 by rotating the handwheel in the counter-clockwise direction. This is a Critical Step	
EVALUATOR'S a)  COMMENTS:	<ul> <li>If asked:</li> <li>Prior to valve manipulation 1-BC-179 is "as you see it" (i.e., closed).</li> <li>Following valve manipulation indicate that 1-BC-179 is OPEN or THROTTLED (stem fully or partially exposed).</li> <li>Throttle 1-BC-179 2 turns open.</li> <li>The operator may decide to throttle 1-BC-179 open until cavitation occurs and then throttle closed on 1-BC-179.</li> <li>As long as 1-BC-179 was opened to allow flow forward, it is not critical that 1-BC-179 be fully open (i.e., it is acceptable for this valve to be throttled)</li> </ul>	
STEP 8:		
	Starting 1-BC-P-2, BC Alternate M/U Pump	SAT
<b>Section 5.1</b> – S <b>Step 5.1.5</b> - Sta	art 1-BC-P-2 and verify discharge pressure is stable at approximately 70 to 5 psig as indicated on 1-BC-PI-117, BC Alternate M/U Pump Discharge Press	SAT UNSAT
Section 5.1 – S Step 5.1.5- Sta	art 1-BC-P-2 and verify discharge pressure is stable at approximately 70 to 5 psig as indicated on 1-BC-PI-117, BC Alternate M/U Pump Discharge Press	
Section 5.1 – Step 5.1.5- Stall Inc. STANDARD:	art 1-BC-P-2 and verify discharge pressure is stable at approximately 70 to 5 psig as indicated on 1-BC-PI-117, BC Alternate M/U Pump Discharge Press I.  Starts 1-BC-P-2 by depressing the start push button.	
Section 5.1 – Stap 5.1.5- Stap 11: Inc. STANDARD:	art 1-BC-P-2 and verify discharge pressure is stable at approximately 70 to 5 psig as indicated on 1-BC-PI-117, BC Alternate M/U Pump Discharge Press I.	
Section 5.1 – Stap 5.1.5- Stap 5.1.5- Stap Inc. STANDARD: a) b)	art 1-BC-P-2 and verify discharge pressure is stable at approximately 70 to 5 psig as indicated on 1-BC-PI-117, BC Alternate M/U Pump Discharge Press d.  Starts 1-BC-P-2 by depressing the start push button.  This is a Critical Step.  Determines discharge pressure by reading 1-BC-PI-117 to be approximately 95 psig.	

STEP 9:	SAT
Section 5.4 – Filling the CC Surge Tank	UNSAT
Step 5.4.1- Verify started or start 1-BC-P-2 IAW Subsection 5.1. Enter N/A if filling from the WT makeup header.	
STANDARD:  a) Notes that 1-BC-P-2 is running.	
EVALUATOR'S NOTE:  a) If asked: The WT makeup header is not available for makeup to the CC Head Tank, use 1-BC-P-2.	
COMMENTS:	
STEP 10:	SAT
Section 5.4 – Filling the CC Surge Tank	UNSAT
<b>Step 5.4.2</b> - Notify an Operator in the Main Control Room to monitor CC Surge Tank Level Indicator.	ono/
STANDARD:  a) Contacts the Main Control Room and notifies the MCR to monitor CC Surge Tank level.	
EVALUATOR'S NOTE:  a) If asked/contacted:  • Will acknowledge the need to monitor CC Head Tank Level.  • Will contact the operator when level approaches 50%.	
COMMENTS:	

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STEP 10:	SAT
Section 5.4 – Filling the CC Surge Tank	UNSAT
Step 5.4.3- Slowly open 1-CN-800, CN M/U to CC Surge Tk Isol.	UNSAT
STANDARD:  a) Slowly opens 1-CN-800 by rotating the valve in the counter-clockwise direction. It is not necessary to fully open 1-CN-800.  This is a Critical Step.	
<ul> <li>EVALUATOR'S NOTE: <ul> <li>a) If asked/contacted:</li> <li>Prior to valve manipulation 1-CN-800 is "as you see it" (i.e., closed).</li> <li>Following valve manipulation indicate that 1-CN-800 is OPEN or THROTTLED (stem fully or partially exposed).</li> <li>The MCR will note that CC Head Tank Level is increasing.</li> <li>If after 1-CN-800 is opened and the candidate looks at the local CC Head Tank Level indication (by ESGR entry doors) indicate level increasing.</li> </ul> </li></ul>	
<ul> <li>If 1-CN-800 is in a contaminated area, the candidate will NOT enter the contaminated area, but simulate the actions required and valve manipulations.</li> <li>Contact HP or Supervisor</li> <li>Verify proper dosimetry requirements with HP or Supervisor</li> <li>Don the required anti-contamination clothing as directed by HP.</li> </ul>	
COMMENTS:	

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STEP 11:  Section 5.4 – Filling the CC Surge Tank  Step 5.4.4- When the CC Surge Tank is at the desired level, then close 1-CN-800.  STANDARD:  a) Closes 1-CN-800 by rotating the valve in the clockwise direction. This is a Critical Step.	SATUNSAT
EVALUATOR'S NOTE:  a) If asked/contacted:  • Contact the operator to indicate CC Head Tank Level is approximately 50%.  • Following valve manipulation indicate that 1-CN-800 is CLOSED or "as you see it" (valve stem retracted).  • The MCR will note that CC Head Tank Level is stable following the closure of 1-CN-800.  • If after 1-CN-800 is closed and the candidate looks at the local CC Head Tank Level indication (by ESGR entry doors) indicate level stable at approximately 50%.  If 1-CN-800 is in a contaminated are, the candidate will NOT enter the contaminated area, but simulate the actions required and valve manipulations.  • Remove the required anti-contamination clothing as directed by HP.  • Perform a frisk, as necessary.  COMMENTS:	
STEP 12:  Section 5.4 – Filling the CC Surge Tank  Step 5.4.5- Stop 1-BC-P-2 IAW Subsection 5.2. Enter N/A if filled from WT makeup header.  STANDARD:  a) Proceeds to procedure Subsection 5.2 to secure 1-BC-P-2.  EVALUATOR'S NOTE: N/A  COMMENTS:	SATUNSAT

STEP 13:  Section 5.2 – Stopping 1-BC-P-2, BC Alternate M/U Pump  Step 5.2.1- After pump run is done, then stop 1-BC-P-2.  STANDARD:  a) Stops 1-BC-P-2 by depressing the stop push button.  EVALUATOR'S NOTE:  a) If asked: Discharge pressure decreases to 30 psig after the pump is stopped.  COMMENTS:	SATUNSAT
STEP 14:  Section 5.2 – Stopping 1-BC-P-2, BC Alternate M/U Pump  Step 5.2.2- Close 1-BC-179, BC Alt M/U Pump Disch.  STANDARD:  a) Closes 1-BC-179 by rotating the handwheel in the clockwise direction.  EVALUATOR'S NOTE:  a) If asked:  Prior to valve manipulation 1-BC-179 is OPEN.  Following valve manipulation indicate that 1-BC-179 is CLOSED or "as you see it" (valve stem retracted).  COMMENTS:	SATUNSAT
STEP 15:  Section 5.2 – Stopping 1-BC-P-2, BC Alternate M/U Pump  Step 5.2.3- Close 1-BC-180, BC Alt M/U Pump Recirc Line Isol.  STANDARD:  a) Closes 1-BC-180 by rotating the handwheel in the clockwise direction.  EVALUATOR'S NOTE:  a) If asked:  • Prior to valve manipulation 1-BC-180 is OPEN.  • Following valve manipulation indicate that 1-BC-180 is CLOSED or "as you see it".	SATUNSAT

SIEP 16:	SAT
Section 5.2 – Stopping 1-BC-P-2, BC Alternate M/U Pump	UNSAT
Step 5.2.3- Close 1-BC-177, BC Alt M/U Pump Suct.	ONOAT
STANDARD:  a) Closes 1-BC-177 by rotating the handwheel in the clockwise direction.	
EVALUATOR'S NOTE:  a) If asked: • Prior to valve manipulation 1-BC-177 is OPEN. • Following valve manipulation indicate that 1-BC-177 is CLOSED or "as you see it" (valve stem retracted).  COMMENTS:	
STEP 17:	SAT
Reports completion of the Step to the Shift Manager.	UNSAT
STANDARD:  • Verbal status report that the CC Head Tank was filled with 1-BC-P-2.	
EVALUATOR'S NOTE:  • Acknowledge the completion of the task.	
COMMENTS:	
STOP TIME:	

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# Operator Directions Handout (TO BE READ TO APPLICANT BY EXAMINER)

#### **Task**

- Task is to be SIMULATED in the plant.
- Fill the CC Head Tank to 50% using 1-BC-P-2 in accordance with 0-OP-BC-001.

#### **Directions**

The evaluator will explain the initial conditions of the task to be performed and will provide the initiating cue. Ensure you indicate to the evaluator when you understand your assigned task.

#### **Initial Conditions**

- An adverse trend was identified on the CC Head Tank and a CC leak was identified and subsequently isolated.
   The CC Head Tank level is low and needs to be filled; current level is 30%.
- The Water Treatment (WT) makeup header is not available to supply water to the CC Head Tank.

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Shift Manager. An adverse trend was identified on the CC Head Tank and a CC leak was identified and subsequently isolated.
- The CC Head Tank level is low and needs to be filled; current level is 30%.
- Water Treatment (WT) makeup header is not available to supply water to the CC Head Tank.
- You are to fill the CC Head Tank to 50% using 1-BC-P-2 in accordance with 0-OP-BC-001, Operation of 1-BC-P-2.
- When you finish the actions necessary to accomplish this, please inform me.

## Operator Directions Handout (TO BE GIVEN TO APPLICANT)

#### **Initial Conditions**

- An adverse trend was identified on the CC Head Tank and a CC leak was identified and subsequently isolated.
   The CC Head Tank level is low and needs to be filled; current level is 30%.
- The Water Treatment (WT) makeup header is not available to supply water to the CC Head Tank.

#### **Initiating Cues**

- This task is to be SIMULATED. Do NOT turn switches, manipulate controls or reposition valves.
- I am the Shift Manager. An adverse trend was identified on the CC Head Tank and a CC leak was identified and subsequently isolated.
- The CC Head Tank level is low and needs to be filled; current level is 30%.
- Water Treatment (WT) makeup header is not available to supply water to the CC Head Tank.
- You are to fill the CC Head Tank to 50% using 1-BC-P-2 in accordance with 0-OP-BC-001, Operation of 1-BC-P-2.
- When you finish the actions necessary to accomplish this, please inform me.

Dom	inion SURB	Y POWER S	STATION		PROCEDURE NO 0-OP-	D: BC-001
PROCEDURE TY	PE:	RATING PROC			UNIT NO:	<i>,</i> & 2
PROCEDURE TIT	LE:	OPEF	RATION OF 1-E	3C-P-2		
REVISION SUMM	ARY:					
Revised in resp	onse to CR 3	17052 (OP FB 09-	0140):			
Added Subset	ection 5.9 for f	flushing piping tur	nnel sump pump	discharge using	1-BC-P-2.	
PROCEDURE US	ED:	Entirely	Partially	Note: If used p	artially, note reasor	ns in remarks.
PROBLEMS ENC	OUNTERED:	□ NO □	YES	Note: If YES, n	ote problems in rer	marks.
REMARKS:						
					(Use back for a	additional remarks.)

CONTINUOUS USE

DATE:

SHIFT SUPERVISION:

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#### 1.0 PURPOSE

1.1 To provide instructions for the operation of 1-BC-P-2, BC Alternate Makeup Pump. (Ref. 2.4.1)

#### 2.0 REFERENCES

#### 2.1 Source Documents

- 2.1.1 UFSAR Section 10.3.9, Bearing Cooling Water System
- 2.2 Technical Specifications Surry Power Station Units 1 and 2

None

#### 2.3 Technical Documents

- 2.3.1 11448-FM-73A, Bearing Cooling Water System (Sheet 1 of 2)
- 2.3.2 11548-FM-73A, Bearing Cooling Water System (Sheet 1 of 1)
- 2.3.3 11448-FM-067A, Condensate System (Sheet 1 of 2)
- 2.3.4 11448-FM-072D, Component Cooling Water System (Sheet 1 of 5)
- 2.3.5 11448-FM-072H, Component Cooling Water System (Sheet 1 of 1)
- 2.3.6 11548-FM-072D, Component Cooling Water System (Sheet 1 of 1)

#### 2.4 Commitment Documents

2.4.1 PI S-2001-0080, Develop Procedure for the Operation of 1-BC-P-2

#### Init Verif

#### 3.0 INITIAL CONDITIONS

None

#### 4.0 PRECAUTIONS AND LIMITATIONS

- 4.1 Push button station 1-BC-CS-P2, BC M/U Pump Control Switch, is mounted on the vertical beam <u>near</u> the pump.
- 4.2 Because the LCVs for 1-CD-TK-1A, 1-CD-TK-1B, and 2-CD-TK-1, Chilled Water Surge Tanks, are <u>not</u> reliable, makeup is done using the LCV Bypass.
- 4.3 The BC flow path is normally aligned from 1- BC-P-2 to Unit 1 BC System <u>and</u> to Unit 2 BC System. Starting 1-BC-P-2 will fill the head tanks if required.
- 4.4 The following equipment will be required for performing Subsection 5.9:
  - Red rubber hose (100 ft) with Chicago fittings on both ends
  - · Five gallon bucket

Init	Verif						
		5.0	INST	RUCTIONS			
		5.1	Starti	ng 1-BC-P-2, BC Alterna	te M/U Pump		
			5.1.1	Verify that 1-BC-253, BC	Alt M/U Pump D	isch Hdr Drain, is c	losed.
			5.1.2	Open 1-BC-177, BC Alt M	I/U Pump Suct.		
			5.1.3	Open 1-BC-180, BC Alt M	I/U Pump Recirc	Line Isol.	
				CAUTION	1		
Valve	1-BC-179	may re	equire th	rottling to prevent cavitation	n when condensat	e temperature is <u>hi</u> g	<u>th</u> .
			5.1.4	Throttle open or open 1-BO	C-179, BC Alt M/	U Pump Disch.	
			5.1.5	Start 1-BC-P-2 and verify	<i>C</i> 1		
				approximately 70 to 115 ps		1-BC-PI-117,	
				BC Alternate M/U Pump I	olsch Press ma.		
		Perf	ormed b	y:			
				Signature	Initial	Print	Date
				Signature	Initial	Print	Date

5.2	Stoppi	ing 1-BC-P-2, BC Altern	ate M/U Pump		
	5.2.1	AFTER pump run is done,	THEN stop 1-BC	C-P-2.	
	5.2.2	Close 1-BC-179, BC Alt M	1/U Pump Disch.		
	5.2.3	Close 1-BC-180, BC Alt M	1/U Pump Recirc	Line Isol.	
	5.2.4	Close 1-BC-177, BC Alt M	1/U Pump Suct.		
Pe	rformed b	y:Signature	Initial	Print	 
		0.9.1			2410
		Signature	Initial	Print	Date

5.3	-	tion Of 1-BC-P-2 For Was onents	hing Unit 2	Turbine Building	
	5.3.1	Get a suitable hose <u>and</u> attach 2 BC Hdr Vent. (Consider len be washed.)		•	
	5.3.2	Coordinate with Maintenance washed.	personnel and	run hose to componen	t to be
	5.3.3	Start 1-BC-P-2, BC Alternate	M/U Pump, L	AW Subsection 5.1.	
	5.3.4	Coordinate with Maintenance washing.	Personnel and	l open 2-BC-436 to beg	in
	5.3.5	WHEN washing is complete,	THEN do the	following:	
		a. Stop 1-BC-P-2 IAW Subs	ection 5.2.		
		b. Close 2-BC-436.			
	5.3.6	IF additional components will Otherwise, enter N/A.	l be washed, <u>T</u>	<u>HEN</u> RETURN TO Ste	p 5.3.2.
	5.3.7	Remove hose from 2-BC-436			
Perfo	rmed b	y: Signature	 Initial	Print	Date
		Signature	 Initial	Print	 Date

5.4 Filling	g the CC Surge Tank			
 5.4.1	Verify started or start 1-B from WT makeup header.		etion 5.1. Enter N/	A if filling
 5.4.2	Notify an Operator in the level indicator.	Main Control Rooi	m to monitor the C	C Surge Tank
 5.4.3	Slowly open 1-CN-800, C	EN M/U to CC Surg	ge Tk Isol.	
 5.4.4	WHEN the CC Surge Tan	k is at the desired	level, <u>THEN</u> close	1-CN-800.
 5.4.5	Stop 1-BC-P-2 IAW Subsheader.	ection 5.2. Enter N	I/A if filled from V	VT makeup
Performed b				
	Signature	Initial	Print	Date
	Signature	 Initial	Print	Date

5.5 Filli	ing 2-CD-TK-1, Chilled	Water Surge Tank	•	
 5.5.	1 Verify started or start 1-	BC-P-2 IAW Subsec	ction 5.1.	
 5.5.	2 Monitor the level and op	pen 2-CD-248, CD S	urge TK-1 LCV-20	)4 Byp.
 5.5.	3 <u>WHEN</u> the CD Surge T	ank is at the desired	level, <u>THEN</u> close	2-CD-248.
 5.5.	4 Stop 1-BC-P-2 IAW Su	bsection 5.2.		
Performed	I by:Signature		Print	 
	oignataro			Date
	Signature	Initial	Print	Date

5.6 Fillin	g 1-CD-TK-1A, Chilled V	Water Surge Ta	ak	
 5.6.1	Verify started or start 1-B0	C-P-2 IAW Subsec	etion 5.1.	
 5.6.2	Monitor the level <u>and</u> oper Byp.	n 1-CD-372, CD S	urge TK-1A M/U LO	CV-104A
 5.6.3	WHEN the CD Surge Tank	k is at the desired	level, <u>THEN</u> close 1	-CD-372.
 5.6.4	Stop 1-BC-P-2 IAW Subse	ection 5.2.		
Performed l	oy:Signature	Initial	Print	 Date
	Signature	— ——— — Initial	Print	Date

5.7 <b>F</b>	illing	1-CD-TK-1B, Chilled Wat	ter Surge Ta	nk	
 5	.7.1	Verify started or start 1-BC-P	-2 IAW Subse	ction 5.1.	
 5	.7.2	Monitor the level <u>and</u> open 1-Byp.	CD-398, CD S	Surge TK-1B M/U LCV	7-104B
 5	.7.3	WHEN the CD Surge Tank is	at the desired	level, <u>THEN</u> close 1-C	D-398.
 5	.7.4	Stop 1-BC-P-2 IAW Subsection	on 5.2.		
Perform	ned by	Signature	 Initial	Print	Date
		Signature	Initial	Print	Date

5.8	Using	1-BC-P-2 With the Hose Connection
	5.8.1	Get a suitable hose <u>and</u> attach to one of the following:
		1-BC-454, BC Alt M/U Pump Discharge Drain
		1-BC-455, BC Alt M/U Pump Discharge Drain
	5.8.2	Open 1-BC-177, BC Alt M/U Pump Suct.
	5.8.3	Open 1-BC-180, BC Alt M/U Pump Recirc Line Isol.
	5.8.4	Start 1-BC-P-2 <u>and</u> verify discharge pressure is stable at <u>approximately</u> 70 to 115 psig as indicated on 1-BC-PI-117, BC Alternate M/U Pump Disch Press Ind.
	5.8.5	Open 1-BC-253, BC Alt M/U Pump Discharge Drain.
		CAUTION
Valve 1-BC-454 or 1-BC <u>high</u> .	C-455 m	nay require throttling to prevent cavitation when condensate temperature is
	5.8.6	Throttle open or open the valve selected in Step 5.8.1.
		Throttle open or open the valve selected in Step 5.8.1.
		Throttle open or open the valve selected in Step 5.8.1.  1-BC-454, BC Alt M/U Pump Discharge Drain
	5.8.6	Throttle open or open the valve selected in Step 5.8.1.  1-BC-454, BC Alt M/U Pump Discharge Drain  1-BC-455, BC Alt M/U Pump Discharge Drain  WHEN hose connection use is no longer required, THEN close or verify
	5.8.6	Throttle open or open the valve selected in Step 5.8.1.  1-BC-454, BC Alt M/U Pump Discharge Drain  1-BC-455, BC Alt M/U Pump Discharge Drain  WHEN hose connection use is no longer required, THEN close or verify closed both of the following valves.
	5.8.6	Throttle open or open the valve selected in Step 5.8.1.  1-BC-454, BC Alt M/U Pump Discharge Drain  1-BC-455, BC Alt M/U Pump Discharge Drain  WHEN hose connection use is no longer required, THEN close or verify closed both of the following valves.  1-BC-454, BC Alt M/U Pump Discharge Drain

DOMINION Surry Power Station					0-OP-BC-001 Revision 7 Page 13 of 15
	5.8.10	Close 1-BC-180, BC Alt M	1/U Pump Recirc	Line Isol.	
	5.8.11	Close 1-BC-177, BC Alt M	1/U Pump Suct.		
	5.8.12	IF hose use is no longer red Otherwise, enter N/A.	quired, <u>THEN</u> ren	nove and properly	y store hose.
1	Performed b	y:Signature	Initial	Print	Date
		 Signature	 Initial	Print	Date

5.9	Flushi	ng of Piping Tunnel Sump Pump Discharge Using 1-BC-P-2
	5.9.1	Place red rubber hose under CC heat exchangers from 1-BC-P-2 to the sump pump pit.
	5.9.2	Unplug sump pump and uncouple the sump pump discharge line.
	5.9.3	Connect red rubber hose to the pump and the discharge line for the sump that goes to the floor drain.
	NOTE:	Pushbutton station 1-BC-CS-P2, BC M/U Pump Control Switch, is mounted on vertical beam near the pump.
	5.9.4	Verify open or open 1-BC-253, BC Alt M/U Pump Disch Hdr Drain.
	5.9.5	Open 1-BC-177, BC Alt M/U Pump Suct.
	5.9.6	Open 1-BC-180, BC Alt M/U Pump Recirc Line Isol.
	5.9.7	Throttle open 1-BC-455, BC Alt M/U Pump Discharge Drain.
	5.9.8	Start 1-BC-P-2.
	NOTE:	<ul> <li>Max flush time is <u>two minutes</u> to protect integrity of discharge PVC piping.</li> </ul>
		• Discharge line is in drain by #1 Demin.
	5.9.9	Open 1-BC-455. Check volume of water being discharged to verify line is clear.
	5.9.10	After approximately two minutes, secure pump.
	5.9.11	Carefully remove discharge line and allow water to drain into sump pump pit.
	5.9.12	Reconnect red rubber hose to discharge of the sump pump line going into the pit.

Signature

Signature

Print

Print

Date

Date

Initial

Initial

Performed by:

### U.S. Nuclear Regulatory Commission Surry Power Station

# SR10301 Simulator Job Performance Measure APE003.AA1.02 (3.6/3.4)

Applicant	_ Start Time_	
Examiner		
Date	_ Stop Time_	
<u>Title</u>		
RECOVER A DROPPED ROD.		
K/A: APE003.AA1.02 Ability to operate and/or Rod: Controls and components necessary to		oply to the Dropped Control
Applicability	Estimated Time	Actual Time

#### **Conditions**

RO/SRO(I)

Surry

- Task is to be PERFORMED in the simulator.
- A single dropped RCCA (P-10) has occurred. Reactor power has been reduced to approximately 65-70% power and stabilized. AP-1.00 has been performed through step 23 and transition made to AP-1.01, step 5 where it has been performed through step 14.

22 Minutes

#### **Standards**

• Rod P-10 fully withdrawn IAW 0-AP-1.01, Control Rod Misalignment.

#### **Initiating Cues**

- 0-AP-1.01, Control Rod Misalignment, step 15
- Shift Manager direction.

#### **Terminating Cues**

• 0-AP-1.01, Control Rod Misalignment, step 28 completed.

#### **Procedures**

0-AP-1.01, Control Rod Misalignment

#### **Tools and Equipment**

#### **Safety Considerations**

None

None

#### **Simulator Setup**

- Recall IC-322 OR
- Call up 70% power IC and initialize.
- Enter malfunctions for dropped RCCA P-10 (RD1224), annuciator overrides (0ff) GC4 (UPPER ION CHAMBER DEVIATION OR AUTO DEFEAT < 50%), GD4 (LOWER ION CHAMBER DEVIATION OR AUTO DEFEAT < 50%) and GE4 (NIS PWER RGE CH1 AVG FLUX DEVIATION). Perform 0-AP-1.00 through step 23 and transition to 0-AP-1.01 step 5, perform through step 13 and stabilize plant.</li>
- Sign off copy of 0-AP-1.01, Control Rod Misalignment, from step 5 through step 14.
- Verify Tave/Tref recorder set on proper scale for current Tave/Tref.
- REMOVE MALFUNCTION (RD1224) & freeze simulator until ready to perform JPM.
- Place an Orange Magnet next to the Control Bank 'A' Rod Position Indicator for P-10.

#### **Initiating Cues**

- 0-AP-1.01, Control Rod Misalignment, step 15
- Shift Manager direction.

#### **Directions to the Applicant**

- I am the Shift Manager. Control Rod P-10 dropped about 1 hour ago due to a blown fuse. The unit has been stabilized and the Instrument Techs have repaired the fuse. Tech Spec 3.12.E has been reviewed by the Shift Manager and STA. A pre-job brief has been held and we are now ready to withdraw the rod. I&C is standing by for your instructions.
- Here's a copy of AP-1.01, I want you to recover the dropped control rod IAW steps 15 through and including step 28.
- When you finish the actions necessary to accomplish this and return the rods to MANUAL, please inform me.

#### **Notes**

## PERFORMANCE CHECKLIST

## Notes to the Evaluator

- Task critical elements are bolded and noted at the end of the step as CRITICAL STEP.
- An additional instructor may be needed to silence and acknowledge alarms for the examinee.
- START TIME:

STEP 1:  CAUTIONS PRIOR TO STEP 15		SAT
CAUTION:	This procedure is NOT valid for realignment of a control rod if Reactor is subcritical.  Realignment SHALL be performed with Reactor power held less than or equal to 75%.	UNSAT
STANDARD:		
<ul> <li>Acknow</li> </ul>	ledges cautions.	
EVALUATOR'S NO	TE: N/A	
COMMENTS:		
<b>STEP 2: STEP 15</b> - TRANS	SFER ROD CONT MODE SEL SWITCH TO AFFECTED BANK	SAT
STANDARD:		
Places ROI This is a Cr	D CONT MODE SEL SWITCH to CBA (Control Bank 'A') position. itical Task.	
EVALUATOR'S NO	TE: N/A	
COMMENTS:		

STEP 3:		
Step 16 –	RECORD AFFECTED ROD AND ROD BANK:	SAT
Ctop 10		UNSAT
	• Rod:	
	Rod Bank:	
STANDARI	D:	
• Re	cords rod P-10 and CBA.	
EVALUATO	DR'S NOTE: N/A	
COMMENT	'S:	
STEP 4:		SAT
Step 17 -	ALIGN LIFT COIL DISCONNECT SWITCHES FOR AFFECTED BANK	
	Place all disconnect switches to DISCONNECTED position	UNSAT
	Place affected rod disconnect switch to CONNECTED position	
	Have alignment switches independently verified.	
STANDARI	D:	
•	Proceeds behind Vertical Board 1-2 to Lift Coil Disconnect Switch Panel and opens panel door. <b>NOTES SIGN REQUIRING REMOVAL OF JEWELRY PRIOR TO ENTRY.</b>	
•	Places all 8 disconnect switches for Control Bank 'A' in the DISCONNECT position. This is a Critical Task.	
•	Places disconnect switch for rod P-10 in the CONNECT position. This is a Critical Task.	
•	Asks for an independent verification of disconnect switches position.	
EVALUATO	<b>DR'S NOTE:</b> If asked, Lift Coil Disconnect Switches have been independently verified.	
COMMENT	rs:	

STEP 5:		SAT
Step 18 - RECO	RD BANK POSITION OF AFFECTED ROD:	UNSAT
• Gr	oup 1 step counter:	
• Gr	oup 2 step counter:	
STANDARD:		
• Record	s group 1 & 2 step counters for CBA at 228 steps.	
EVALUATOR'S NO	TE: N/A	
COMMENTS:		
STEP 6:		SAT
Cautions prior to S	Step 19	UNSAT
CAUTION:	The affected withdrawal rate during realignment is limited to 2/P (P=fraction of Core Power where 100% power is equal to 1.0) steps per hour (if not a whole number, round down to the whole number) if affected rod remains misaligned for more than 12 hours or the duration of misalignment can NOT be determined.	
	The withdrawal rate limitation may be relaxed with the authorization from the Reactor Engineer or Nuclear Analysis and Fuels.	
STANDARD:		
Acknow	vledges cautions.	
EVALUATOR'S NO	<b>PTE</b> : If asked, Reactor Engineering has authorized that the 2/P rod speed limit be relaxed. No limit on rod speed is required.	
COMMENTS:		
STEP 7:		SAT
Step 19 - RECOR	RD THE FOLLOWING	UNSAT
Reactor Po	wer:	UNSAT
<ul> <li>Withdrawal</li> </ul>	Rate:	

STANDARD:	
Records current reactor power level (will be approximately 65-70%).	
Records withdrawal rate at 48 spm.	
<b>EVALUATOR'S NOTE:</b> If asked, Reactor Engineering has authorized that the 2/P rod speed limit be relaxed. No limit on rod speed is required.	
If asked, Do not exceed 70% power, a 1 dpm SUR or temperature >565°F during dropped rod recovery.	
COMMENTS:	
STEP 8:	SAT
Note prior to step 20	UNSAT
NOTE: Refer to Attachment 2 before resetting Group Step Counter.	
STANDARD:	
Acknowledges note.	
<ul> <li>Refers to attachment 2 (trainee may wait and refer to attachment 2 before resetting group step counters).</li> </ul>	
EVALUATOR'S NOTE: N/A	
COMMENTS:	
STEP 9:	SAT
Step 20 - CHECK AFFECTED ROD - ON BOTTOM	UNSAT
STANDARD:	
Checks rod P-10 indicating zero steps using digital and light indication.	
EVALUATOR'S NOTE: N/A	
COMMENTS:	

STEP 10:           Step 21 –         REFER TO TECH SPEC 3.12.E	SAT
STANDARD:  • Trainee recalls from directions that Tech Spec 3.12.E has been reviewed.	UNSAT
EVALUATOR'S NOTE: If asked, Tech Spec 3.12.E has been reviewed.	
COMMENTS:	
STEP 11:  Step 22 - RESET AFFECTED GROUP STEP COUNTER TO 0.	SAT
STANDARD:	
Trainee refers to attachment 2 to reset group step counter to zero steps.	
<ul> <li>Trainee resets group step counter CONT BANK A GP 2 by pressing reset button OR by pulsing the group step counter until it reads 0 steps. This is a Critical Task.</li> </ul>	
EVALUATOR'S NOTE: If asked, it is preferred to use the reset button.	
COMMENTS:	
STEP 12:	0.4.T
Step 23 – HAVE I&C RESET AFFECTED BANK DEMAND TO 000.	SAT
STANDARD:	
<ul> <li>Trainee calls I&amp;C and requests they reset affected bank CONT BANK A demand to 000.</li> </ul>	
EVALUATOR'S NOTE: N/A	
BOOTH OPERATOR'S NOTE: When requested to reset bank demand, inform the trainee that a time compression has occurred and CONT BANK A has been reset to 000 by I&C.	

COMMENTS:		
STEP 13:		SAT
Note prior to step	24	UNSAT
NOTE:	Annunciator 1G-A6, ROD CONT SYS URGENT FAILURE, will alarm when the affected rod is withdrawn indicating that the lift coils of the remaining rods in the bank are deenergized.	UNSAT
STANDARD:		
Acknowledge	ges note.	
EVALUATOR'S NO	DTE: N/A	
COMMENTS:		
STEP 14:		SAT
Step 24 - REALIG	GN AFFECTED ROD TO ITS BANK POSITION RECORDED IN STEP 18.	UNSAT
STANDARD:		
Places SH     Critical Tas	UTDN AND CONT ROD CONT SWITCH to the OUT position. This is a sk.	
<ul> <li>Verifies out</li> </ul>	ward rod motion indicated by observing affected rod CERPI.	
Acknowledge	ges annunciator 1G-A6 (ROD CONT SYS URGENT FAILURE).	
• Withdraws	affected rod to 228 steps. This is a Critical Task.	
	sly monitors SUR, PR NI's, IR NI's, $\Delta T$ , Tave, group step counters, CERPI, out indication light and TR-1-409A.	
EVALUATOR'S NO	DTE: N/A	
COMMENTS:		

STEP 15:		SAT
Step 25 –	PLACE AFFECTED BANK LIFT COIL DISCONNECT SWITCHES TO THE CONNECT POSITION.	UNSAT
STANDARD:		
	ds behind Vert Bd 1-2 to Lift Coil Disconnect Switch Panel. NOTES SIGN RING REMOVAL OF JEWELRY PRIOR TO ENTRY.	
	disconnect switches for affected bank (F-2, B-10, K-14, P-6, K-2, B-6 & F-nis is a Critical Task.	
EVALUATOR'S	S NOTE: N/A	
COMMENTS:		
STEP 16:		SAT
Step 26 –	HAVE ALIGNMENT OF DISCONNECT SWITCHES INDEPENDENTLY VERIFIED	UNSAT
STANDARD:		
<ul> <li>Trainee</li> </ul>	recalls from directions that Tech Spec 3.12.E has been reviewed.	
EVALUATOR'S	NOTE: If asked, Lift Coil Disconnect Switches have been independently verified.	
COMMENTS:		
STEP 17:		SAT
Step 27 –	RESET ROD CONTROL URGENT FAILURE	UNSAT
• Depres	ss ROD CONT SYS INTERNAL ALARM RESET pushbutton	
STANDARD:		
	sses ROD CONT SYS INTERNAL ALARM RESET pushbutton. a Critical Task.	

EVALUATOR'S NOTE: N/A	
COMMENTS:	
STEP 18:	SAT
Step 28 – TRANSFER ROD CONT MODE SEL SWITCH TO MANUAL	UNSAT
STANDARD:	
Moves rod control mode selector switch from CBA to Manual.	
EVALUATOR'S NOTE: N/A	
COMMENTS:	
STEP 11:	CAT
REPORT TO SHIFT SUPERVISOR (EVALUATOR).	SAT
STANDARD:	UNSAT
Verbal status report that control rod P-10 has been realigned with its control bank and steps 15-28 of AP-1.01 have been completed.	
EVALUATOR'S NOTE: N/A	
COMMENTS:	
STOP TIME:	

## Operator Directions Handout (TO BE READ TO APPLICANT BY EXAMINER)

#### **Task**

- Task is to be performed in the simulator.
- Perform Steps 15 28 of 0-AP-1.01, CONTROL ROD MISALIGNMENT, to align control rod P-10 with its control bank.

#### **Directions**

The evaluator will explain the initial conditions of the task to be performed and will provide the initiating cue. Ensure you indicate to the evaluator when you understand your assigned task

#### **Initial Conditions**

 A single dropped RCCA (P-10) has occurred. Reactor power has been reduced to approximately 65-70% power and stabilized. AP-1.00 has been performed through step 23 and transition made to AP-1.01, step 5 where it has been performed through step 14.

#### **Initiating Cues**

- I am the Shift Manager. Control Rod P-10 dropped about 1 hour ago due to a blown fuse. The unit has been stabilized and the Instrument Techs have repaired the fuse. Tech Spec 3.12.E has been reviewed by the Shift Manager and STA. A pre-job brief has been held and we are now ready to withdraw the rod. I&C is standing by for your instructions.
- Here's a copy of AP-1.01, I want you to recover the dropped control rod IAW steps 15 through and including step 28.
- When you finish the actions necessary to accomplish this and return the rods to MANUAL, please inform me.

## Operator Directions Handout (TO BE GIVEN TO APPLICANT)

#### **Initial Conditions**

 A single dropped RCCA (P-10) has occurred. Reactor power has been reduced to approximately 65-70% power and stabilized. AP-1.00 has been performed through step 23 and transition made to AP-1.01, step 5 where it has been performed through step 14.

#### **Initiating Cues**

- I am the Shift Manager. Control Rod P-10 dropped about 1 hour ago due to a blown fuse. The unit has been stabilized and the Instrument Techs have repaired the fuse. Tech Spec 3.12.E has been reviewed by the Shift Manager and STA. A pre-job brief has been held and we are now ready to withdraw the rod. I&C is standing by for your instructions.
- Here's a copy of AP-1.01, I want you to recover the dropped control rod IAW steps 15 through and including step 28.
- When you finish the actions necessary to accomplish this and return the rods to MANUAL, please inform me.

NUMBER	PROCEDURE TITLE	REVISION 20
0-AP-1.01	CONTROL ROD MISALIGNMENT	PAGE 6 of 9

OTED ACTION/EXPECTED DECREASES
STEP ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED
*************
CAUTION: An infrequently conducted or complex test requires a Senior Operations Manager presence.
***************
14. 8/ DIRECT SENIOR OPERATIONS MANAGER
TO CONDUCT PRE-JOB BRIEF IAW
ATTACHMENT 1
* * * * * * * * * * * * * * * * * * * *
CAUTION: • This procedure is NOT valid for realignment of a control rod if Reactor is subcritical.
Backgroupert CHALL has nowforward with Bacatan payor hold loss than an arrival to 750/
<ul> <li>Realignment SHALL be performed with Reactor power held less than or equal to 75%.</li> </ul>
* * * * * * * * * * * * * * * * * * * *
45 TRANSFER BOD CONT MODE CEL
15 TRANSFER ROD CONT MODE SEL SWITCH TO AFFECTED BANK
40 DECORD AFFECTED DOD AND DOD DANK
16 RECORD AFFECTED ROD AND ROD BANK:
□ • Rod:
□ • Rod Bank:
17 ALICALIET COIL DISCONNECT SWITCHES
17 ALIGN LIFT COIL DISCONNECT SWITCHES  FOR AFFECTED BANK:
☐ a) Place all disconnect switches to
DISCONNECTED position
☐ b) Place affected rod disconnect switch to
CONNECTED position
□ c) Have alignment of disconnect switches
independently verified

NUMBER	PROCEDURE TITLE  CONTROL ROD MISALIGNMENT	REVISION 20
0-AP-1.01	CONTROL ROD IVIISALIGNIVIENT	PAGE 7 of 9

STEP ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED
Netion Extremises ones in the second of the
18 RECORD BANK POSITION OF AFFECTED ROD:
□ • Group 1 Step Counter:
□ • Group 2 Step Counter:
************
CAUTION: • The affected withdrawal rate during realignment is limited to 2/P (P=fraction of Core Power where 100% power is equal to 1.0) steps per hour (if not a whole number, round down to the whole number) if affected rod remains misaligned for more than 12 hours or the duration of misalignment can NOT be determined.
<ul> <li>The withdrawal rate limitation may be relaxed with authorization from the Reactor Engineer or Nuclear Analysis and Fuels.</li> </ul>
***********
19 RECORD THE FOLLOWING:
<del></del>
□ • Withdrawal rate:

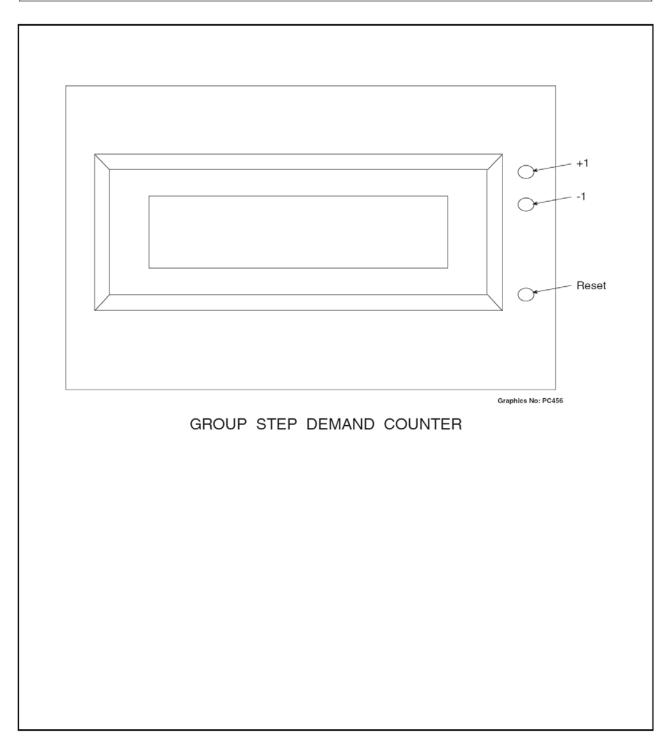
- 1			
	NUMBER	PROCEDURE TITLE	REVISION
	0-AP-1.01	CONTROL ROD MISALIGNMENT	20
	0-Al -1.01		PAGE
			8 of 9
- 1			

_				
$\dashv$	STEP	ACTION/EXPECTED RESPONSE	<u> </u>	RESPONSE NOT OBTAINED
			-	
	NOT	E: Refer to Attachment 2 before resetting G	Froun Sten C	ounter
	NOT	E. Reier to Attachment 2 before resetting of	oroup Step C	ounter.
	20	_ CHECK AFFECTED ROD - ON BOTTOM		Do the following:
				a) Refer to Tech Spec 3.12.E.
				<ul> <li>Reset affected Group Step Counter to IRPI of misaligned rod.</li> </ul>
				c) Have I&C reset affected Bank Demand to IRPI of misaligned rod.
				d) Withdraw the rod until affected Group Step Counter is at 242.
				e) Reset affected Group Step Counter to 230.
				f) GO TO Step 24.
	21	REFER TO TECH SPEC 3.12.E		
	22	RESET AFFECTED GROUP STEP COUNTER TO 0		
	23	HAVE I&C RESET AFFECTED BANK DEMAND TO 000		
	NOTE: Annunciator ( )G-A6, ROD CONT SYS URGENT FAILURE, will alarm when the affected rod is withdrawn indicating that the lift coils of the remaining rods in the bank are deenergized.			
	24	REALIGN AFFECTED ROD TO ITS BANK POSITION RECORDED IN STEP 18	K 🗆	Have Engineering determine hot channel factors within limits IAW 0-NPT-RX-010 within 72 hours.
	25	PLACE AFFECTED BANK LIFT COIL DISCONNECT SWITCHES TO THE CONNECTED POSITION		

NUMBER	PROCEDURE TITLE	REVISION 20
0-AP-1.01	CONTROL ROD MISALIGNMENT	PAGE 9 of 9

STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
26	HAVE ALIGNMENT OF DISCONNECT SWITCHES INDEPENDENTLY VERIFIED		
27	RESET ROD CONTROL URGENT FAILUR	E	
	Depress ROD CONT SYS INTERNAL ALARM RESET pushbutton		
28	TRANSFER ROD CONT MODE SEL SWITCH TO MANUAL		
29	VERIFY REACTOR AND TURBINE POWER MATCHED AND STABLE	R- □	Adjust Rod Control or Turbine Controls to match and stabilize power.
30	VERIFY DELTA FLUX - IN BAND		Borate or dilute as necessary to restore delta flux to band.
31	DIRECT INSTRUMENT DEPARTMENT TO CHECK THE FOLLOWING FOR PROPER SETTINGS:		
	Master Cycler		
	Bank Demand		
	Bank Overlap Counter		
32	POSITION ROD CONT MODE SEL SWITC IAW SHIFT SUPERVISION AND REACTOR ENGINEERING DIRECTION		
33	CHECK INSTRUMENTATION SETPOINTS CHANGED DUE TO TECH SPEC REQUIREMENTS - RETURNED TO NORMAL		Direct Instrumentation Department to reset setpoints.
34	NOTIFY SHIFT SUPERVISION		
	- E	ND -	

NUMBER 0-AP-1.01		ATTACHMENT 2
REVISION 20	GROUP STEP DEMAND COUNTER	PAGE 1 of 1



## U.S. Nuclear Regulatory Commission Surry Power Station

# SR10301 Simulator Job Performance Measure 011.A2.11 (3.4/3.6)

Applicant	Start Time
Examiner	
Date	Stop Time

#### **Title**

#### RESPOND TO A FAILED LOW PRESSURIZER LEVEL CHANNEL

K/A: 011.A2.11 Ability to (a) predict the impacts of the following malfunctions or operations on the PZR LCS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Failure of PZR level instrument - low.

Applicability Estimated Time Actual Time

RO/SRO(I) 15 minutes

#### **Conditions**

- Task is to be PERFORMED in the simulator.
- Unit is operating at 100% power with all systems normal and in automatic.

#### **Standards**

• 0-AP-53.00 complete up and not including step 14 (Failed channel defeated, pressurizer heaters energized as required, charging in Auto).

#### **Initiating Cues**

- AP-53.00 Immediate Actions
- Shift Manager direction.

#### **Terminating Cues**

0-AP-53.00, Loss of Vital Instrumentation/Controls completed up to step 14.

#### **Procedures**

0-AP-53.00, Loss of Vital Instrumentation/Controls

#### **Tools and Equipment**

#### **Safety Considerations**

None

None

#### **Simulator Setup**

- Recall IC-321 OR
- Call up 100% power IC and initialize place in RUN, select position III/II on the Pressurizer level control selector switch.
- Place pressurizer level recorder on Channel 3, 1-RC-LI-1461.
- Program malfunction for low failure of Pressurizer Level Channel III (RC-LT-1461, RC4903), trigger 1.
- Implement malfunctions to fail letdown auto isolation (CH62, CH63 to true)
- After evaluation begins, activate trigger 1.

#### **Initiating Cues**

- AP-53.00 Immediate Actions
- Shift Manager direction.

#### **Directions to the Applicant**

- You are the Unit 1 RO and I am the Shift Manager. The unit is operating at 100% power with all systems in automatic.
- You are to respond to a plant transient.
- When you finish the actions necessary to stabilize the unit at pre-event conditions, please inform me.

#### **Notes**

## PERFORMANCE CHECKLIST

## Notes to the Evaluator

- Task critical elements are bolded and noted at the end of the step as CRITICAL STEP.
- An additional instructor may be needed to silence and acknowledge alarms for the examinee.
- START TIME:

Ackno	IDENTIFIES CHANNEL III PRESSURIZER LEVEL FAILED LOW.  es 1-RC-LI-1461 failed low.  wledges Annunciator E-H-6, PRZR LO LVL CH-3.  0-AP-53.00, Loss of Vital Instrumentation/Controls.  S NOTE: N/A	SATUNSAT
	VERIFY REDUNDANT INSTRUMENT CHANNEL(S) INDICATION - NORMAL  S RC-LT-1459 and 1460 normal.  Wedges annunciator C-E-8, PRZR LO LVL HTRS OFF & LETDOWN ISOL.  S NOTE: N/A	SATUNSAT

STEP 3:		CAT
Step 2 –	PLACE AFFECTED CONTROL(S)/COMPONENT(S) IN MANUAL CONTROL AND STABILIZE PARAMETER USING REDUNDANT	SAT UNSAT
	INDICATION	
STANDARD:		
	charging flow control in manual and reduces to pre-event flow (80-90 This is a Critical Task.	
EVALUATOR'S	S NOTE: N/A	
COMMENTS:		
STEP 4:		SAT
	IMMEDIATE ACTIONS OF 0-AP-53.00 COMPLETE.	UNSAT
STANDARD:		
• Op	erator reports the immediate actions of AP-53.00 are complete.	
EVALUATOR'S	S NOTE: Once the operator has reported the immediate actions of 0-AP-53.00 are complete, acknowledge the completion of the immediate action steps and inform the operator that you would like them to continue on and perform 0-AP-53.00 up to and not including step 14 and hand them a copy of the procedure.	
COMMENTS:		
STEP 5:		SAT
Step 3 -	VERIFY REACTOR POWER – LESS THAN OR EQUAL TO 100% (continuous action step).	UNSAT
CTANDADD.	(continuous action step).	ONSAT
STANDARD:		
	rifies Reactor Power indicating less than or equal to 100% using PCS 10 minute d/or instantaneous power indications.	
EVALUATOR'S	S NOTE: N/A	
COMMENTS:		

STEP 6:  NOTES PRIOR	TO STEP 4	SAT
NOTE:	<ul> <li>Step 4 failures are listed in order of performance priority. Only the failed instrument/control and associated step number should be read aloud.</li> <li>When the affected instrument/controller malfunction(s) has been addressed by this procedure, recovery actions should continue at step 14.</li> </ul>	UNSAT
STANDARD:		
<ul> <li>Acknow</li> </ul>	vledges notes	
EVALUATOR'S	S NOTE: N/A	
COMMENTS:		
STEP 7:		SAT
Step 4 -	DETERMINE THE FAILED INSTRUMENT/CONTROL AND GO TO APPROPRIATE STEP.	UNSAT
STANDARD:		
<ul> <li>Determ</li> </ul>	ines PRZR Level Control to be the failed instrument and goes to step 12.	
EVALUATOR'S	NOTE: N/A	
COMMENTS:		
STEP 8:		
Step 12a –	CHECK PRZR LEVEL CONTROL CHANNELS - NORMAL	SAT
		UNSAT
STANDARD:		
<ul> <li>Determ</li> </ul>	ines that all pressurizer level channels are not normal and goes to 12aRNO.	
EVALUATOR'S	CUE: N/A	
COMMENTS:		

STEP 9:	SAT
Step 12aRNO –	
a) Do the following, as necessary:	UNSAT
<ul> <li>1) Place either of the following in MANUAL:         <ul> <li>1-CH-FC-1122C, CHG FLOW CNTRL</li> <li>OR</li> <li>1-CH-LC-1459G, PRZR LEVEL CNTRL</li> </ul> </li> </ul>	
2) Control PRZR level at program level.	
3) Move PRZR LVL - CH SEL switch to defeat the failed channel.	
<ol> <li>Verify or place recorder 1-RC-LR-1459 on an operable channel.</li> </ol>	
<ol> <li>Refer to Tech Spec 3.1.A.5 (if Pressurizer heaters deenergized), Table 3.7-1, Item 9 and Table 3.7-6, Item 13.</li> </ol>	
6) Refer to Attachment 3.	
STANDARD:	
<ul> <li>Places either 1-CH-FCV-1122C or 1-CH-LC-1459G in manual to control pressurizer level at program level. Note: One of these may already have been placed in manual during the immediate action steps.</li> <li>Moves PRZR LVL – CH SEL switch to defeat channel III. The channel should be placed in postion 1 which selects channel I as the upper and channel II as the lower. This is a Critical Task.</li> <li>Places 1-RC-LR-1459 on an operable channel.</li> <li>Notifies SM to review Tech Specs.</li> <li>Refers to attachment 3 in 0-AP-53.00.</li> </ul>	
<b>EVALUATOR'S NOTE:</b> If asked, Tech Specs will be reviewed by Unit Supervisor / Shift Manager.	
COMMENTS:	

STEP 10:  Step 12b— Verify Pressurizer Heaters - ENERGIZED  STANDARD:  • Trainee determines that not all pressurizer heaters are energized as required and that "C" Group pressurizer proportional heaters will need to be reset. Goes to RNO.  EVALUATOR'S NOTE: N/A  COMMENTS:	SATUNSAT
STEP 11:  Step 12bRNO- Reset Pressurizer Heaters.  STANDARD:  Resets Group "C" pressurizer proportional heaters and verifies all heaters are energized or in Auto as required.  EVALUATOR'S NOTE: N/A  COMMENTS:	SATUNSAT
STEP 12:  Step 12c- Check letdown - IN SERVICE.  STANDARD:  • Verifies letdown is in service by observing normal letdown flow indicated. Note: The trainee may point out that auto letdown isolation failed when 1-RC-LI-1461 failed low.  EVALUATOR'S NOTE: N/A  COMMENTS:	SATUNSAT

STEP 13:  Step 12d— Check PRZR level control – IN AUTOMATIC  STANDARD:  • Determines pressurizer level control is in manual by observing 1-CH-FCV-1122C or 1-CH-LC-1459G in manual. Goes to RNO.  EVALUATOR'S NOTE: N/A  COMMENTS:	SATUNSAT
STEP 14:  Step 12dRNO- Check PRZR level control – IN AUTOMATIC	SAT
d) Do the following as required:	
1) Verify PRZR level restored to program.	
2) Unsaturate 1-CH-LC-1459G, PRZR LEVEL CNTRL, as required.	
<ol> <li>Return 1-CH-FCV-1122 to AUTOMATIC by verifying or placing the following in AUTOMATIC:</li> </ol>	
<ul><li>1-CH-FC-1122C, CHG FLOW CNTRL</li><li>1-CH-LC-1459G, PRZR LEVEL CNTRL</li></ul>	
STANDARD:	
Checks pressurizer level at program level.	
Unsaturates 1-CH-LC-1459G if necessary.	
Returns 1-CH-FCV-1122C and/or 1-CH-LC-1459G to Auto.	
EVALUATOR'S NOTE: N/A	
COMMENTS:	

IAW the Note prior to step 4, recovery actions should continue at step 14.  STANDARD:  • Transitions to step 14.  EVALUATOR'S NOTE: N/A  COMMENTS:	SATUNSAT
STEP 16:  REPORT TO SHIFT SUPERVISOR (EVALUATOR).  STANDARD:  Verbal status report that 0-AP-53.00 is complete up to but not including step 14.  EVALUATOR'S NOTE: N/A  STOP TIME:  COMMENTS:	SATUNSAT
STOP TIME:	

### Operator Directions Handout (TO BE READ TO APPLICANT BY EXAMINER)

#### **Task**

- Task is to be performed in the simulator.
- Perform Steps immediate action steps of AP-53.00, Loss of Vital Instrumentation/Controls, and then perform AP-53.00 up to but not including step 14.

#### **Directions**

The evaluator will explain the initial conditions of the task to be performed and will provide the initiating cue. Ensure you indicate to the evaluator when you understand your assigned task

#### **Initial Conditions**

• Unit is at 100% power and stable.

#### **Initiating Cues**

- You are the Unit 1 RO and I am the Shift Manager. The unit is operating at 100% power with all systems in automatic.
- You are to respond to a plant transient.
- When you finish the actions necessary to stabilize the unit at pre-event conditions, please inform me.

# Operator Directions Handout (TO BE GIVEN TO APPLICANT)

#### **Initial Conditions**

Unit is at 100% power and stable.

#### **Initiating Cues**

- You are the Unit 1 RO and I am the Shift Manager. The unit is operating at 100% power with all systems in automatic.
- You are to respond to a plant transient.
- When you finish the actions necessary to stabilize the unit at pre-event conditions, please inform me.

# NUMBER 0-AP-53.00 PROCEDURE TITLE REVISION 15 CONTROLS (WITH 8 ATTACHMENTS) PAGE 1 of 14

#### **PURPOSE**

To provide guidance for malfunctions of vital instrumentation or controls.

#### **ENTRY CONDITIONS**

- 1. A faulty indication occurs on a vital instrumentation channel.
- 2. Failure of a Main Control Room controller
- 3. Unit Supervisor direction.

# CONTINUOUS USE

- 1			
	NUMBER	PROCEDURE TITLE	REVISION
	0-AP-53.00	LOSS OF VITAL INSTRUMENTATION / CONTROLS	15
			PAGE
			2 of 14
- 1	1		

STEP	ACTION/EXPECTED RESPONSE	 RESPONSE NOT OBTAINED
[1]	VERIFY REDUNDANT INSTRUMENT CHANNEL(S) INDICATION - NORMAL	IF unable to determine if the Reactor is in a
[2]	PLACE AFFECTED CONTROL(S)/	safe condition, <u>THEN</u> GO TO ( )-E-0, REACTOR TRIP OR SAFETY INJECTION. Do the following:
	COMPONENT(S) IN MANUAL CONTROL AND STABILIZE PARAMETER USING REDUNDANT INDICATION	a) IF no control function affected, THEN GO TO Step 4.  b) IF Foodwater related feilure, THEN.
		b) IF Feedwater-related failure, THEN control SG level with Feedwater Isolation MOVs.
		c) <u>IF PRZR PORV-related failure, THEN</u> close the PRZR PORV Block Valve.
		<ul> <li>d) <u>IF</u> manual control is ineffective or unavailable, <u>THEN</u> evaluate transition to ( )-E-0, REACTOR TRIP OR SAFETY INJECTION.</li> </ul>
*3	VERIFY REACTOR POWER - LESS THAN OR EQUAL TO 100%	Initiate Attachment 7.

NUMBER	PROCEDURE TITLE	REVISION 15
0-AP-53.00	LOSS OF VITAL INSTRUMENTATION / CONTROLS	PAGE 3 of 14

STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
NOTE:	<ul> <li>Step 4 failures are listed in order of performal associated step number should be read aloud</li> </ul>		priority. Only the failed instrument/control and
	<ul> <li>When the affected instrument/controller malfurecovery actions should continue at Step 14.</li> </ul>	unctio	on(s) has been addressed by this procedure,
· · ·	DETERMINE THE FAILED NSTRUMENT / CONTROL AND GO TO APPROPRIATE STEP:		
	PRZR Pressure Control, Step 5		PRZR Level Control, Step 12
	NI Malfunction, Step 5		Reactor Coolant Flow, Step 13a
	SG Feed Flow, Step 8		PRZR Pressure Protection, Step 13b
	SG NR Level, Step 8		RCS Wide Range Pressure, Step 13f
	SG Pressure, Step 8		VCT Level, Step 13e
	SG Steam Flow, Step 8		CTMT Pressure, Step 13c
	Turbine First Stage Pressure, Step 9		RWST Level, Step 13d
	Median Tave, Step 10		<ul> <li>Underground FO Storage Tk, Step 13g</li> </ul>
	Loop Tave, Step 10		CAT Level, Step 13h
	Loop ΔT, Step 10		Emergency Condensate Makeup Tank
	Steam Dumps / SG PORVs, Step 11		Level, Step 13i
	Turbine Controls, Step 11		<ul> <li>FP/Domestic Water Level, Step 13j</li> </ul>
	Turbine Valve Position, Step 11		
	Turbine Monitoring Lights, Step 11		

NUMBER	PROCEDURE TITLE	REVISION 15
0-AP-53.00	LOSS OF VITAL INSTRUMENTATION / CONTROLS	PAGE 4 of 14

	1	
STEP ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
5 CHECK THE FOLLOWING INSTRUMENT CONTROLS - AFFECTED	TS/	GO TO Step 8.
<ul><li>Nuclear Instrumentation</li><li>Pressurizer Pressure control</li></ul>		
NOTE: RCS pressure decrease will cause a slig the moderator pressure coefficient.	ght decrease	in RCS Tave due to negative reactivity from
6 CHECK PRZR SPRAY VALVE CONTROLLERS - NORMAL		Do the following:
CONTROLLERS - NORWAL		a) Place failed controller in Manual.
		<ul> <li>Restore RCS pressure to normal and stabilize.</li> </ul>
		c) Verify or stabilize Turbine load.
		d) <u>IF</u> manual control is ineffective or unavailable, <u>THEN</u> GO TO ( )-AP-31.00, Increasing or Decreasing RCS Pressure.
		e) <u>IF</u> RCS pressure returns to normal, <u>THEN</u> do the following:
		<ol> <li>Maintain stable Turbine load until pressure control system is returned to normal.</li> </ol>
		<ol> <li>IF no other instrumentation failure exists, <u>THEN</u> GO TO Step 14. Otherwise, GO TO Step 8.</li> </ol>

NUMBER	PROCEDURE TITLE	REVISION 15
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ACTION/EXPECTED DECIDING	DESPONSE NOT OBTAINED
STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
**********	* * * * * * * * * * * * * * * * * * * *
CAUTION: Delta flux must be monitored and maintained within k	pand if rods have moved.
* * * * * * * * * * * * * * * * * * * *	
7 GO TO THE APPROPRIATE ABNORMAL PROCEDURE	
□ • ()-AP-4.00, Nuclear Instrumentation	
Malfunction	
()-AP-31.00, Increasing or Decreasing     RCS Pressure	
I	

NUMBER PROCEDURE TITLE		REVISION 15
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STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	$\neg$
		_
* * * * * * * * * * * * * * * * * * * *	* * * * * * * * * * * * * * * * * * * *	* * * *
CAUTION: When CALCALC is based on Feedwater Reactor power must be monitored when	changes in feed flow will affect calorimetric povadjusting feed flow.	/er.
* * * * * * * * * * * * * * * * * * * *	* * * * * * * * * * * * * * * * * * * *	* * * *
	1 <del>-</del> 1	
8 CHECK STEAM GENERATOR LEVEL CONTROL INSTRUMENTS - NORMAL	IF the selected steam flow, steam por feed flow input to the SG Water	Level
□ • Steam Pressure	Control system has failed, <u>THEN</u> d following:	o the
□ • Steam Flow	_	Valva in
□ • Feed Flow	<ul> <li>a) Place the associated Feed Reg MANUAL.</li> </ul>	valve III
□ • Steam Generator Level	□ b) Control SG level at program level	∍l.
	<ul><li>c) Select the redundant channel for SG(s)</li></ul>	affected
	d) WHEN SG level returned to nor THEN place the Feed Reg Valve AUTOMATIC.	nal, ∍ in
	IF SG Level Channel III has failed, the following:	<u>ΓΗΕΝ</u> do
	<ul> <li>a) Place the associated Feed Reg MANUAL.</li> </ul>	Valve in
	b) <u>IF</u> manual control of Feedwater inoperable, <u>THEN</u> do the followi	
	1) Control SG level with Feedw Isolation MOVs.	ater
	<ul> <li>2) Consult with the Shift Manag concerning the need to place MFRV on the jack.</li> </ul>	
	☐ c) Control SG level at program leve	el.
	d) Main Feed Reg Valve Bypass V may be used for fine control of S	
(STEP 8 CONTINUED ON NEXT PAGE)		

NUMBER		REVISION 15
0-AP-53.00	LOSS OF VITAL INSTRUMENTATION / CONTROLS	PAGE 7 of 14

		,	
STEP	ACTION/EXPECTED RESPONSE	<u> </u>	RESPONSE NOT OBTAINED
8.	CHECK STEAM GENERATOR LEVEL CONTROL INSTRUMENTS - NORMAL (Continued)		
			Perform follow-up actions:
			<ul> <li>a) Consult with Shift Manager on need to initiate ( )-OP-RP-001, ALIGNING CONTROL SYSTEM FOR PERFORMANCE OF CHANNEL I, II, III, AND IV PROCESS AND PROTECTION TESTING.</li> </ul>
			b) Refer to the following Tech Spec 3.7 items:
			<ul> <li>Table 3.7-1, 12 and 17</li> <li>Table 3.7-2, 1.c, 1.e, and 3.a</li> <li>Table 3.7-3, 2.a, and 3.a</li> <li>Table 3.7-6, 15 and 16</li> </ul>
			c) Refer to Attachment 1.
			d) <u>IF</u> no other instrumentation failure exists, <u>THEN</u> GO TO Step 14.
9	CHECK TURBINE FIRST STAGE PRESSURE CHANNELS - NORMAL		GO TO Attachment 2.

NUMBER	PROCEDURE TITLE  LOSS OF VITAL INSTRUMENTATION / CONTROLS	REVISION 15
0-AP-53.00	LOSS OF VITAL INSTRUMENTATION / CONTROLS	PAGE 8 of 14

STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED			
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* * * * * * * * * * * * * * * * * * *				
*****	*******			
<b>NOTE:</b> Depending on the instrumentation failure, the T <sub>AVE</sub> input to the Steam Dumps may be invalid, causing the Steam Dumps to stay open longer or never open at all.				
10 CHECK LOOP/MEDIAN ΔT/T <sub>AVE</sub> - NORM	<i>I</i> IAL			
☐ a) Median T <sub>AVE</sub> - NORMAL	a) Do the following:			
	<ul> <li>1) Place ROD CONT MODE SEL switch in MANUAL, as required.</li> </ul>			
	<ul> <li>2) Take manual control of charging, as necessary to maintain PRZR level at program.</li> </ul>			
	□ 3) Refer to Attachment 4.			
	4) <u>IF</u> no other instrumentation failure exists, <u>THEN</u> GO TO Step 14.			
□ b) Loop T <sub>AVE</sub> / ΔT - NORMAL	b) Do the following:			
	□ 1) Verify ∆T Recorder, ( )/TR-( )-412 is selected to an operable channel.			
	2) Refer to the following Tech Spec 3.7 items:			
	<ul> <li>Table 3.7-1, 5 and 6</li> <li>Table 3.7-2, 1.e, and 6</li> <li>Table 3.7-3, 2a</li> </ul>			
	□ 3) Refer to Attachment 4.			
	4) <u>IF</u> no other instrumentation failure exists, <u>THEN</u> GO TO Step 14.			

NUMBER	PROCEDURE TITLE  LOSS OF VITAL INSTRUMENTATION / CONTROLS	REVISION 15
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STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
* * * * * * * * * * * * * * * * * * * *	* * * * * * * * * * * * * * * * * * * *
CAUTION: If Reactor power has been affected by a to control Tave.	secondary transient, Turbine adjustment may be needed
* * * * * * * * * * * * * * * * * * * *	* * * * * * * * * * * * * * * * * * * *
11 CHECK STEAM SYSTEM CONTROLS - NORMAL	☐ Initiate ( )-AP-38.00, MAIN STEAM SYSTEM CONTROL MALFUNCTION.
□ • Steam Dumps / PORVs	
□ • Turbine Controls	
□ • Turbine Valve Positions	
☐ • Turbine Monitoring Lights	

NUMBER	PROCEDURE TITLE  LOSS OF VITAL INSTRUMENTATION / CONTROLS	REVISION 15
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STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
12 CHECK PRZR LEVEL CONTROL CHANNELS - NORMAL	
OTANNELO - NONMAL	
a) Check PRZR LVL Instrumentation-	a) Do the following, as necessary:
NORMAL	<ol> <li>Place either of the following in MANUAL:</li> </ol>
	• ( )-CH-FC-( )122C, CHG FLOW CNTRL
	OR
	• ( )-CH-LC-( )459G, PRZR LEVEL CNTRL
	☐ 2) Control PRZR level at program level.
	<ul> <li>3) Move PRZR LVL - CH SEL switch to defeat the failed channel.</li> </ul>
	<ul> <li>4) Verify or place recorder         <ul> <li>()-RC-LR-()459 on an operable channel.</li> </ul> </li> </ul>
	<ul> <li>5) Refer to Tech Spec 3.1.A.5         <ul> <li>(if Pressurizer heaters deenergized),</li> <li>Table 3.7-1, Item 9 and Table 3.7-6,</li> <li>Item 13.</li> </ul> </li> </ul>
	☐ 6) Refer to Attachment 3.
□ b) Verify Pressurizer Heaters - ENERGIZED	□ b) Reset Pressurizer Heaters.
(STEP 12 CONTINUED ON NEXT PAGE)	

NUMBER PROCEDURE TITLE  O AD 53 00 LOSS OF VITAL INSTRUMENTATION / CONTROLS	REVISION 15	
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
9,2,	, to 11010/ EAR 20125 (1201 0110E	ALEX GIVE HE FOR SELF MILES
12.	CHECK PRZR LEVEL CONTROL CHANNELS - NORMAL (Continued)	
	c) Check letdown - IN SERVICE	c) Restore normal letdown IAW ( )-OP-CH-020, PLACING LETDOWN IN SERVICE FOLLOWING AN AUTOMATIC OR MANUAL ISOLATION.
		IF normal letdown can NOT be restored, THEN place Excess Letdown in service IAW ( )-OP-CH-006, SHIFTING OR INCREASING/DECREASING LETDOWN FLOW.
	d) Check PRZR level control - IN	d) Do the following as required:
	AUTOMATIC	<ol> <li>Verify PRZR level restored to program.</li> </ol>
		<ol> <li>Unsaturate ( )-CH-LC-( )459G, PRZR LEVEL CNTRL, as required.</li> </ol>
		<ol> <li>Return ()-CH-FCV-()122 to AUTOMATIC by verifying or placing the following in AUTOMATIC:</li> </ol>
		<ul> <li>( )-CH-FC-( )122C, CHG FLOW CNTRL</li> </ul>
		( )-CH-LC-( )459G, PRZR LEVEL CNTRL

NUMBER	PROCEDURE TITLE  LOSS OF VITAL INSTRUMENTATION / CONTROLS	REVISION 15
0-AP-53.00	EGGG OF VITAL INSTRUMENTATION / CONTROLS	PAGE 12 of 14

STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
13	CHECK PROPER OPERATION OF THE FOLLOWING INSTRUMENTS:		
	<ul> <li>a) Reactor Coolant Flow instrumentation - NORMAL</li> </ul>		a) Refer to Tech Spec 3.7, Table 3.7-1, Item 10.
	b) PRZR Pressure Protection instrumentat - NORMAL	tion	b) Refer to the following Tech Spec 3.7 items:
			<ul> <li>Table 3.7-1, 5, 7, and 8</li> <li>Table 3.7-2, 1.d and 6.a (Refer to ( )-OPT-RP-001)</li> </ul>
	c) CTMT pressure instrumentation - NORMAL		c) Refer to the following Tech Spec 3.7 items:
			1) Protection Channels:
			<ul><li>Table 3.7-2, 1.b and 2.b</li><li>Table 3.7-3, 1.b, 1.c, and 2.b</li></ul>
			Accident Monitoring (CTMT Wide Range Pressure)
			<ul><li>Table 3.7-6, Item 3</li><li>Tech Spec 3.7.E</li></ul>
	d) RWST level instrumentation - NORMAL	. 0	d) Refer to Tech Spec 3.7, Table 3.7-2, Items 7 and 8.
	e) VCT level instrumentation - NORMAL		e) Refer to Attachment 6.
(STEP 13	CONTINUED ON NEXT PAGE)		

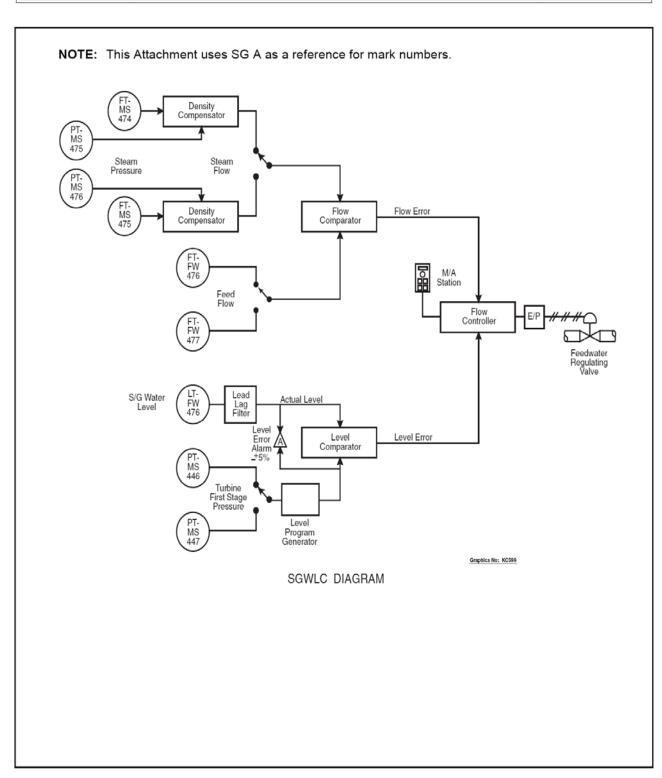
NUMBER	PROCEDURE TITLE  LOSS OF VITAL INSTRUMENTATION / CONTROLS	REVISION 15
0-AP-53.00	LOSS OF VITAL INSTRUMENTATION / CONTROLS	PAGE 13 of 14

$\dashv$	STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED	-
	13.	CHECK PROPER OPERATION OF THE FOLLOWING INSTRUMENTS: (Continued)  f) RCS Wide Range Pressure instrumentation - NORMAL  • PT-402  • PT-402-1  • PT-403  • PT-458		<ul><li>f) Refer to the following:</li><li>• Tech Spec 3.7.E, Table 3.7-6</li><li>• TRM 3.3.2, Table 3.3.2-1</li></ul>	
		g) Underground Fuel Oil Storage Tank leve instrumentation - NORMAL		<ul><li>g) Do the following:</li><li>1) Review Tech Spec Table 4.1-2A, Item 11.</li><li>2) Initiate Attachment 5.</li></ul>	
		h) Chemical Addition Tank level instrumentation - NORMAL	_	<ul><li>h) Do the following:</li><li>1) Review Tech Spec 3.4.A.4.</li><li>2) Initiate Attachment 5.</li></ul>	
		i) Emergency Condensate Makeup Tank level instrumentation - NORMAL	_ _	<ul><li>i) Do the following:</li><li>1) Review TRM Section 3.7.17.</li><li>2) Initiate Attachment 5.</li></ul>	
		j) Fire Protection and Domestic Water Tan level instrumentation - NORMAL	nk 🗆	j) Initiate Attachment 5.	
	14	REVIEW ( )-OPT-RX-001, REACTOR POWER CALORIMETRIC USING PCS COMPUTER PROGRAM, TO DETERMINE OPERABILITY OF CALORIMETRIC AND NEED TO PERFORM ( )-OPT-RX-007, SHI AVERAGE POWER CALCULATION			
	15	CHECK FAILED INSTRUMENT - IS A REGULATORY GUIDE 1.97 VARIABLE		GO TO Step 17.	

NUMBER	LOGG OF WITH INCTRUMENTATION (CONTROLS	REVISION 15
0-AP-53.00	LOSS OF VITAL INSTRUMENTATION / CONTROLS	PAGE 14 of 14

OTED	ACTION/FYDECTED DESDONSE	l	DESPONSE NOT OPTAINED	1
STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED	
16	REVIEW CEP 99-0029, REG GUIDE 1.97 OPERABILITY, REPORTABILITY, AND ALTERNATE INDICATIONS			
17	REVIEW THE FOLLOWING:			
	Tech Spec 3.7			
	VPAP-2802, NOTIFICATIONS AND REPORTS			
	TRM SECTION 3.3, INSTRUMENTATION			
18	CHECK ADDITIONAL INSTRUMENT / CONTROLLER MALFUNCTION - EXISTS		GO TO Step 20.	
19	RETURN TO STEP 4			
20	PROVIDE NOTIFICATIONS AS NECESSARY:			
	<ul><li>Shift Supervision</li><li>OMOC</li><li>STA (PRA determination)</li><li>I&amp;C</li></ul>			
	-	END -		

NUMBER 0-AP-53.00	ATTACHMENT TITLE	ATTACHMENT 1
REVISION 15	SG WATER LEVEL CONTROL SYSTEM	PAGE 1 of 1



NUMBER 0-AP-53.00	ATTACHMENT TITLE	ATTACHMENT 2
REVISION 15	TURBINE FIRST STAGE PRESSURE	PAGE 1 of 4

* * * * * * * * * * * * * * * * * * * *
<b>CAUTION:</b> Delta flux must be monitored and maintained within band if rods have moved.
* * * * * * * * * * * * * * * * * * * *
NOTE: Page 4 of this Attachment shows a one-line diagram of Turbine First Stage Pressure.
1. <u>IF</u> the selected Turbine First Stage Pressure channel has failed, <u>THEN</u> do the following:
a. Verify or place the ROD CONT MODE SEL switch in MANUAL.
b. Select redundant Turbine First Stage Pressure channel IAW the following:
1. Verify or place STM DUMP CNTRL switch in OFF-RESET.
2. Verify or place ( )-CP-43-( )CP( )01, SUDDEN LOSS LOAD DEFEAT switch in DEFEAT position.
3. Verify or place Main Feed Reg Valves in MANUAL:
• ( )-FW-FCV-( )478, SG A FEED REG
□ • ()-FW-FCV-()488, SG B FEED REG
□ • ()-FW-FCV-()498, SG C FEED REG
4. Verify or place TURB FIRST STAGE PRESS CH SEL switch to redundant channel.
* * * * * * * * * * * * * * * * * * * *
<b>CAUTION:</b> When CALCALC is based on Feedwater, changes in feed flow will affect calorimetric power. Reactor power must be monitored when adjusting feed flow.
* * * * * * * * * * * * * * * * * * * *
2. Return affected components to normal position IAW Shift Supervision direction:
a. WHEN SG level returned to normal, THEN place Main Feed Reg Valves in AUTO:
• ( )-FW-FCV-( )478, SG A FEED REG
• ()-FW-FCV-()488, SG B FEED REG
□ • ( )-FW-FCV-( )498, SG C FEED REG

NUMBER 0-AP-53.00	ATTACHMENT TITLE	ATTACHMENT 2
REVISION 15	TURBINE FIRST STAGE PRESSURE	PAGE 2 of 4

## **NOTE:** • Unless there is evidence that a Turbine runback is imminent, the preferred alignment of the Steam Dump control system is ON in the Tave Mode.

- If ( )-MS-PT-( )446 fails low, Steam Dump Demand will go to 100%, but steam dumps will
  not open, since an arming signal (load reject or trip) is not present. If the steam dumps are
  not in Steam Pressure Mode following ( )-MS-PT-( )446 failing low and a load reject occurs,
  then all the steam dumps will open. AMSAC will not function six minutes after the failure.
- If ()-MS-PT-()446 fails high, Steam Dump Demand will remain at 0%, and if a load reject signal is present, the dumps will not open (Steam Dump Tref is greater than Tave).
   However, if a trip signal does occur, the steam dumps will open as they now compare Tave to Tnoload.
- If ( )-MS-PT-( )447 fails low, a load reject signal will develop, but the steam dumps will not open since there is no steam dump demand signal. However, the CP Bldg will be bypassed. The CP Bldg should be returned to service expeditiously to maintain secondary chemistry. AMSAC will not function six minutes after the failure.
- If ( )-MS-PT-( )447 fails high, the load reject arming signal is effectively defeated.

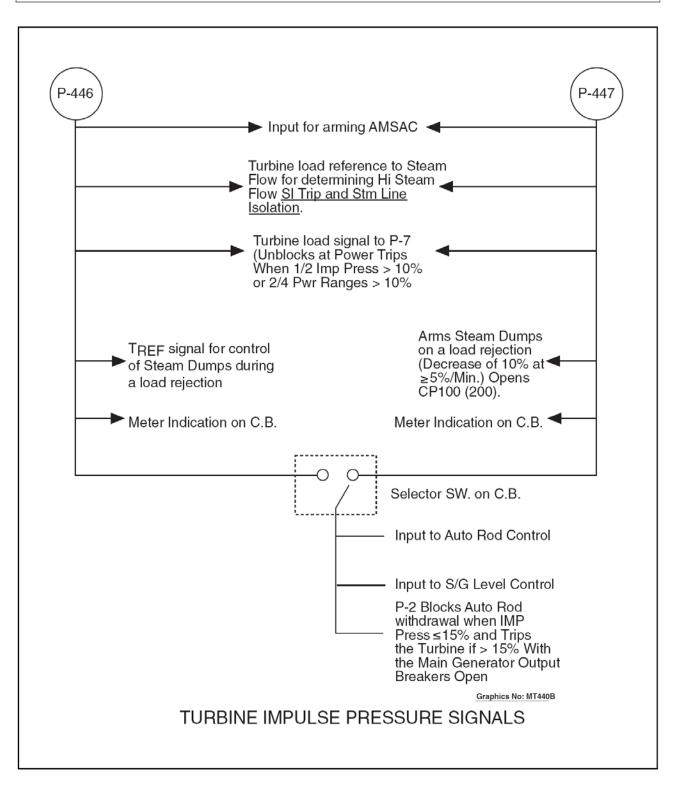
b. Return Steam Dumps to service IAW Shift Supervision direction.

1. Momentarily place STM DUMP MODE SEL switch in RESET.
2. Place STM DUMP CNTRL switch in ON.
3. Place ( )-CP-43-( )CP( )01, SUDDEN LOSS LOAD DEFEAT switch in NORMAL position.
4. <u>IF</u> ( )-MS-PT-( )447 has failed, <u>THEN</u> restore the CP Bldg as required.
□ a. Have the CP Bldg Operator open ( )-CP-AOV-( )22 in MANUAL.
□ b. Close ( )-CP-MOV-( )00.
□ c. Have the CP Bldg Operator slowly close ( )-CP-AOV-( )22 in MANUAL.
d. Place ( )-CP-AOV-( )22 in AUTOMATIC and adjust setpoint to 62 psid.
5. <u>IF</u> ( )-MS-PT-( )446 has failed low, <u>THEN</u> consult with Shift Manager for the need to place the STM DUMP MODE SEL switch in STEAM PRESSURE mode.

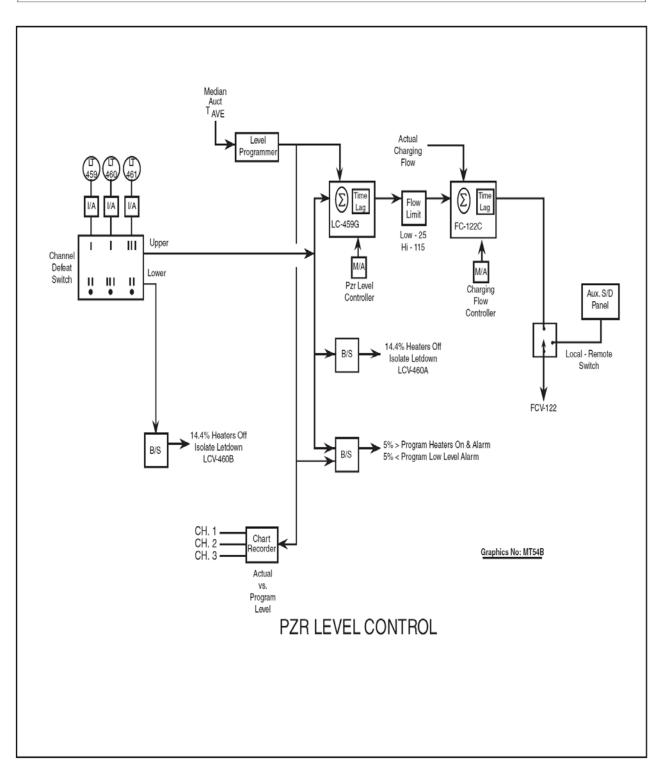
NUMBER 0-AP-53.00	ATTACHMENT TITLE	ATTACHMENT 2
REVISION 15	TURBINE FIRST STAGE PRESSURE	PAGE 3 of 4

c. Return Rod Control to Auto IAW Shift Supervision direction.
1. Review appropriate ARP(s) to verify correct rod position. (for example, G-F-8)
2. Place the ROD CONT MODE SEL switch in AUTO.
Perform follow-up actions:
a. Consult with Shift Manager on need to initiate ( )-OP-RP-001, ALIGNING CONTROL SYSTEM FOR PERFORMANCE OF CHANNEL I, II, III, AND IV PROCESS AND PROTECTION TESTING.
b. Refer to the following Tech Spec 3.7 items.
□ • Table 3.7-1, 20 (Refer to ( )-OPT-RP-001 - one hour clock) and Table 3.7-2, 1.e
c. Refer to VPAP-2802 for AMSAC inoperability.
d. <u>IF</u> no other instrumentation failure exists, <u>THEN</u> GO TO Step 14.

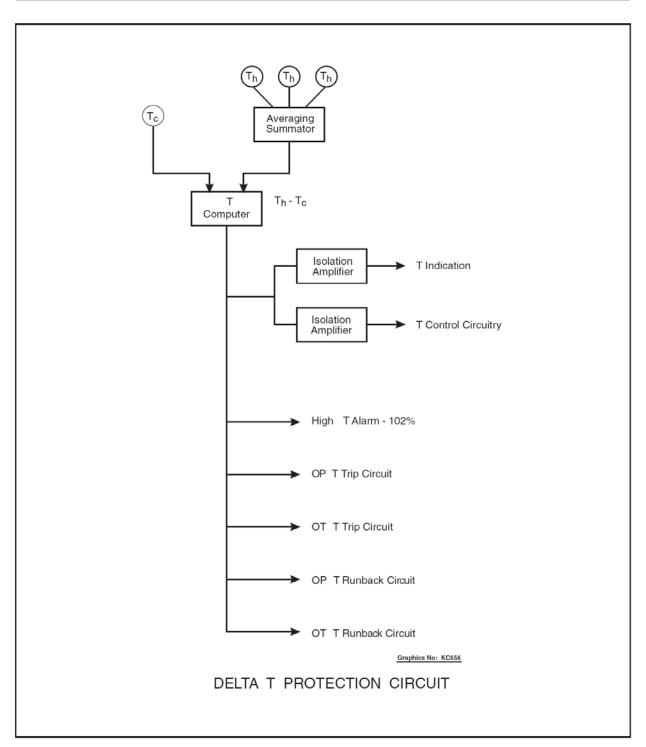
NUMBER 0-AP-53.00	ATTACHMENT TITLE	ATTACHMENT 2
REVISION 15	TURBINE FIRST STAGE PRESSURE	PAGE 4 of 4



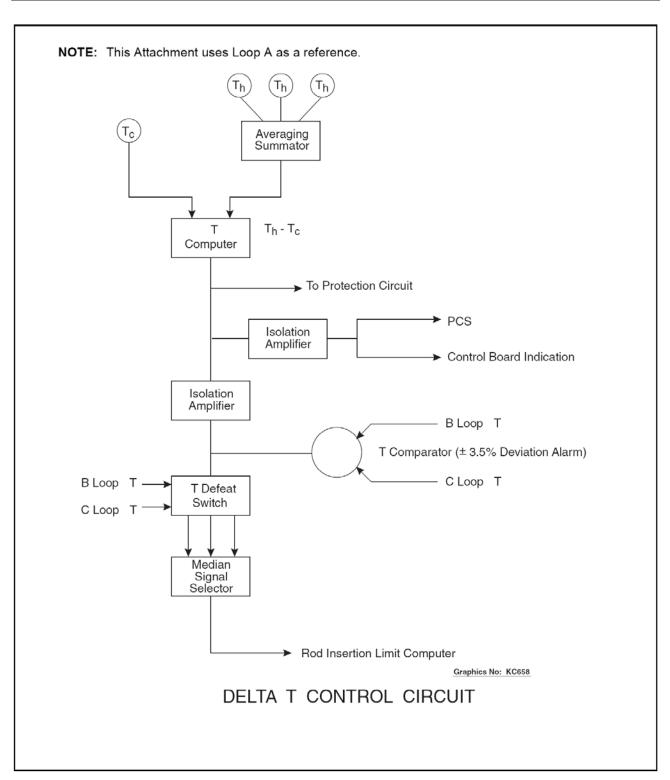
NUMBER 0-AP-53.00	ATTACHMENT TITLE	ATTACHMENT 3
REVISION 15	PRESSURIZER LEVEL CONTROL	PAGE 1 of 1



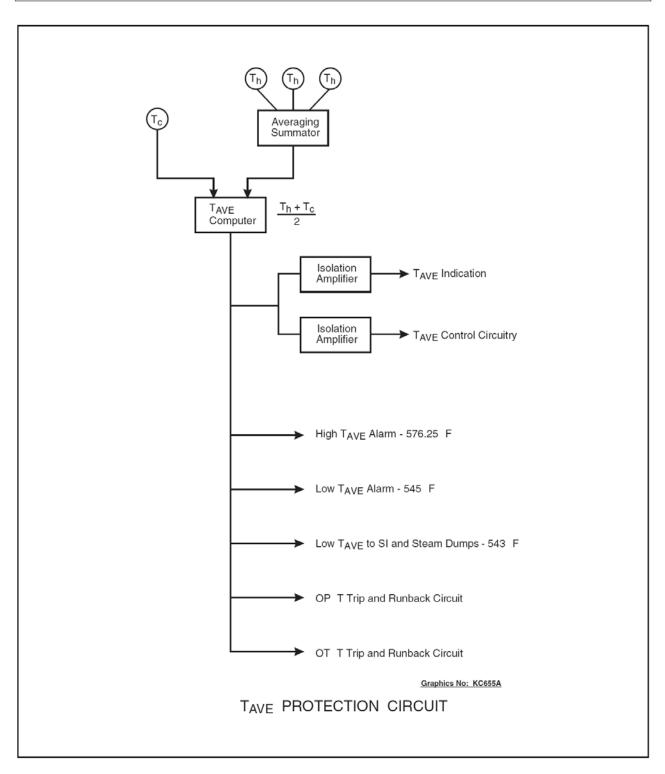
NUMBER 0-AP-53.00	ATTACHMENT TITLE	ATTACHMENT 4
REVISION 15	ΔI/TAVE	PAGE 1 of 4



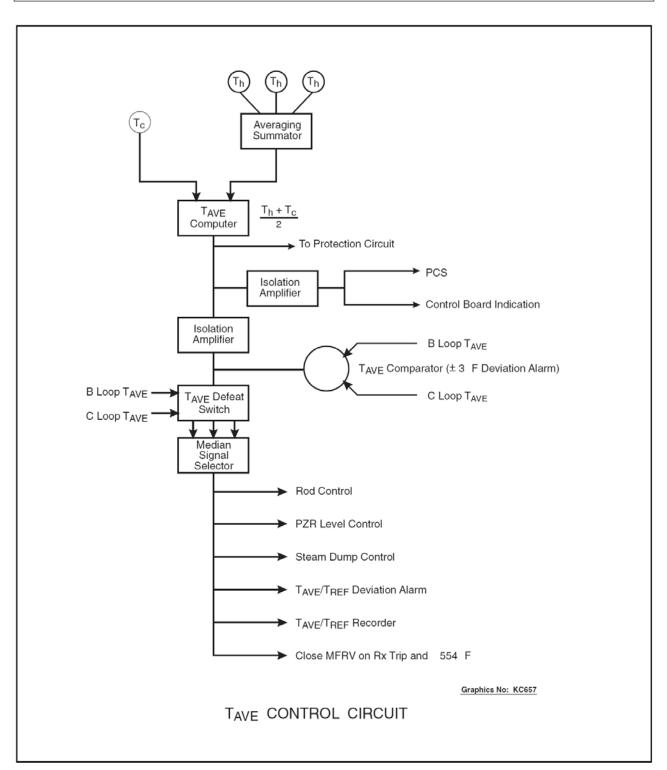
NUMBER 0-AP-53.00	53.00	ATTACHMENT 4
REVISION 15	ΔT/TAVE	PAGE 2 of 4



NUMBER 0-AP-53.00	0-AP-53.00	ATTACHMENT 4
REVISION 15	ΔT/TAVE	PAGE 3 of 4



NUMBER 0-AP-53.00	ATTACHMENT TITLE ΔT/TAVE	ATTACHMENT 4
REVISION 15		PAGE 4 of 4



NUMBER 0-AP-53.00	ATTACHMENT TITLE	ATTACHMENT 5
REVISION 15	FAILED TANK LEVEL INSTRUMENTATION (TANKS WITH SINGLE INDICATIONS) - CONTINGENCY ACTIONS	PAGE 1 of 2

<u>1.</u>	1-EE-TK-2A/2B, UNDERGROUND FUEL OIL STORAGE TANK
,	Immediately verify (and every eight hours thereafter) that there is no visible leakage at the tank or in the Underground Fuel Oil Pump House. The last known level indication should be verified to have been in specification. If the last level indication was in specification, and no leakage is observed, the tank may remain operable.
	Immediate actions should be taken to restore the indicator to service. A clock should be started to verify level in the tank using the normal indication or an alternate indication, such that the Tech Spec Table 4.1-2A, Item 11 surveillance will be met. One type of alternate indication could be to lift the missile shield and verify level by hand.
2.	1/2-CS-TK-2, CHEMICAL ADDITION TANK
	Immediately verify (and every eight hours thereafter) that there is no visible leakage at the CAT or in Safeguards. The last known level indication should be verified to have been specification. If the last level indication was in specification, and no leakage is observed, the tank may remain operable. The likelihood of internal leakage is very small, as long as the standby condition of the CAT valves is not disturbed.
	Immediate actions should be taken to restore the indicator to service. The indication should be restored within 24 hours, or an alternate means used to check level.
,	One means to check level is to have I&C connect a Heise Gauge (calibrated in inches of water) to the drain connection at 1/2-CS-LT-101/201. The connection can be isolated using the existing ICVs. Pressure in inches of water column can be converted directly into level using the data in 1-CAL-005 or 2-CAL-010.
3.	1/2-CN-TK-3, EMERGENCY CONDENSATE MAKEUP TANK
	Immediately verify (and every eight hours thereafter) that there is no visible leakage at the Makeup Tank or in Safeguards. The last known level indication should be verified to have been in specification. If the last level indication was in specification, and no leakage is observed, the tank may remain operable.
	Tank level should be checked within 24 hours either by verifying that the tank is filled to overflowing, measuring actual level, or by checking pressure at the Booster Pumps and converting to height in feet. The mechanical indicators are not calibrated.

NUMBER 0-AP-53.00  REVISION 15  ATTACHMENT TITLE  ATTACHMENT TITLE  FAILED TANK LEVEL INSTRUMENTATION (TANKS WITH SINGLE INDICATIONS) - CONTINGENCY ACTIONS	ATTACHMENT 5	
		,

4	. 1-FP-TK-1A/1B, FIRE PROTECTION AND DOMESTIC WATER TANK
_·	If the tank level indication fails on one tank, and the tanks are cross-connected (normal operation), then verify level at the opposite tank. If the opposite tank level is stable and within Operations Logs specifications, and no leakage is seen, then the tank with the failed indication may remain operable. Otherwise, declare the tank with the failed indication inoperable.
_·	If the tank level indication fails on one tank, and the tanks are NOT cross-connected, then verify immediately (and every eight hours thereafter) that no leakage exists at the tank and in the Fire Pump House. If the last known reading was within Operations Logs specifications, and no leakage is seen, then the tank with the failed indicator may remain operable. Otherwise, declare the tank with the failed indication inoperable.

NUMBER	ATTACHMENT TITLE	ATTACHMENT
0-AP-53.00	VOLUME CONTROL TANK LEVEL FAILURE INFORMATION	6
REVISION	VOLUME CONTROL TANK LEVEL FAILURE INFORMATION	PAGE
15		1 of 1

()-CH-LT-()112
□ • If ( )-CH-LT-( )112 fails high, ( )-CH-LCV-( )115A will open. Automatic swap-over of the CHG pump suction from the VCT to the RWST will not function.
<ul> <li>If ( )-CH-LT-( )112 fails low, ( )-CH-LCV-( )115A will only automatically open from a high level signal from ( )-CH-LT-( )115. Automatic swap-over of the CHG pump suction from the VCT to the RWST will still function.</li> </ul>
( )-CH-LT-( )112 provides input to the following:
<ul> <li>a. Controller ( )-CH-LC-( )115 for modulating ( )-CH-LCV-( )115A open at the setpoint set on the controller</li> </ul>
□ b. CHG pump suction swap over to the RWST at 13% (2/2)
( )-CH-LT-( )115
<ul> <li>If ( )-CH-LT-( )115 fails high, ( )-CH-LCV-( )115A will open. Manual control of the blender will be required. Automatic swap-over of the CHG pump suction from the VCT to the RWST will not function.</li> </ul>
<ul> <li>If ( )-CH-LT-( )115 fails low, ( )-CH-LCV-( )115A will only automatically open from a high level signal from ( )-CH-LT-( )112. Automatic swap-over of the CHG pump suction from the VCT to the RWST will still function.</li> </ul>
( )-CH-LT-( )115 provides input to the following:
□ a. VCT High level divert of ( )-CH-LCV-( )115A to PDT at 85%
□ b. VCT High level alarm at 82%
□ c. Auto makeup to VCT stop at 34%
□ d. Auto makeup to VCT start at 27%
□ e. VCT Low level alarm at 24%
f. CHG pump suction swap over to the RWST at 13% (2/2)

NUMBER 0-AP-53.00	ATTACHMENT TITLE	ATTACHMENT 7
REVISION 15	PROMPT ACTIONS TO REDUCE REACTOR POWER	PAGE 1 of 2

1 <u>IF</u> increase in Reactor Power caused by secondary transient, <u>THEN</u> GO TO Step 2. <u>IF</u> increase in Reactor Power caused by primary transient, <u>THEN</u> GO TO Step 6.
<b>NOTE:</b> During secondary transient conditions, the most reliable indications of Reactor Power are Core ΔT and Power Range NI. These indications should be used as the basis for Reactor Power until stable conditions have been reestablished.
<ol> <li>Mitigate increases in Reactor Power caused by secondary transients by decreasing steam demand or Turbine load as follows:</li> </ol>
☐ a) Reduce the Turbine Setter by an appropriate amount.
□ b) Place the Load Rate Selector thumbwheel to 1% / min.
□ c) Depress the GO pushbutton.
3 <u>IF</u> Turbine load can <u>NOT</u> be reduced using Operator Auto, <u>THEN</u> reduce Turbine load using the Valve Position Limiter.
4 IF Turbine load still can NOT be reduced, THEN do the following.
☐ a) Push the Turbine Manual pushbutton.
□ b) Verify Turbine controls in Manual.
□ c) Depress the GV down pushbutton to reduce GV position.
NOTE: The CALCALC 10 minute Average Power will lag actual Reactor Power following a transient, and may continue to increase even after core thermal power has been reduced to less than 100%.
5 Verify Reactor Power decreases to less than 100%.
□ • Power Range NI power.
□ • Core ∆T power
□ • CALCALC Instantaneous power (if reliable)

NUMBER 0-AP-53.00	ATTACHMENT TITLE PROMPT ACTIONS TO REDUCE REACTOR POWER	ATTACHMENT 7		
REVISION 15	PROMPT ACTIONS TO REDUCE REACTOR FOWER	PAGE 2 of 2		

6	Mitigate increases in Reactor Power caused by primary transients as follows. Enter N/A for Steps 6 through 9 if <u>NOT</u> applicable.
	a) Place the Rod Control Selector switch in Manual.
	b) Insert control rods.
NOTE	The CALCALC 10 minute Average Power will lag actual Reactor Power following a transier and may continue to increase even after core thermal power has been reduced to less than 100%.
7	Verify Reactor Power decreases to less than 100%.
	Power Range NI power.
	• Core ΔT power
	CALCALC Instantaneous power (if reliable)
8	$\underline{IF}$ necessary, $\underline{THEN}$ initiate a normal boration to control $\DeltaFlux$ .
9	WHEN Tave and Tref are within 1°F, THEN place control rods in Auto.

NUMBER 0-AP-53.00	ATTACHMENT TITLE PROBABLE CAUSES AND REFERENCES	ATTACHMENT 8
REVISION 15	PROBABLE CAUSES AND REFERENCES	PAGE 1 of 1

## I. PROBABLE CAUSES

1. Instrumentation or controller failure

## II. REFERENCES

- 1. Technical Specifications 3.1 and 3.7
- 2. VPAP-2802, Notifications and Reports
- 3. Technical Requirements Manual
- 4. ET CEP 99-0029, Reg Guide 1.97 Operability, Reportability, and Alternate Indications
- 5. ( )-OPT-RP-001, Check of Permissive Status Lights P-6, P-7, P-8, and P-10
- 6. ( )-OP-RP-001, Aligning Control System for Performance of Channel I, II, III, and IV Process and Protection Testing
- 7. 0-AP-1.00, Rod Control System Malfunction
- 8. ( )H-G5, STM GEN ( )A LVL ERROR
- 9. DCP 01-008, Instrument and Controls Upgrade Project, Unit 1
- 10. DCP 01-011, Plant Computer Replacement, Surry/Unit 2
- 11. PI S-2004-0548, Tanks with one Means of Level Indication
- 12. DCP 02-063, EHC Power Supply Replacement
- 13. DCP 04-073, Lube Oil Pressure Switch Redundancy
- 14. PI S-2005-0929, Failed Spray Valve (Step 6)
- DCP 99-093, FC9, Replacement of Control Room Chart Recorders with Paperless Recorders

## U.S. Nuclear Regulatory Commission Surry Power Station

# SR10301 Simulator Job Performance Measure 005.A4.01 (3.6/3.4)

Applicant	Start Time
Examiner	
Date	Stop Time
T:41a	

## <u>Title</u>

RESPOND TO A LOSS OF THE OPERATING RHR PUMP.

K/A: 005.A4.01 Ability to manually operate and/or monitor in the control room: Controls and indication for RHR pumps.

 Applicability
 Estimated Time
 Actual Time

 RO/SRO(I)/SRO(U)
 12 minutes

## **Conditions**

- Task is to be PERFORMED in the simulator.
- 1-RH-P-1A was in service on "A" RHR HX. We just received annunciator B-G-6, RHR HX LO FLOW, and 1-RH-P-1A has tripped.

## **Standards**

 1-RH-P-1B started and RHR flow restored in accordance with 1-AP-27.00, Loss of Decay Heat Removal Capability.

## **Initiating Cues**

- Trip of the operating RHR pump.
- Shift Manager direction.

## **Terminating Cues**

• 1-AP-27.00, Loss of Decay Heat Removal Capability, step 14 completed.

## **Procedures**

1-AP-27.00, Loss of Decay Heat Removal Capability.

## **Tools and Equipment**

## **Safety Considerations**

None

None

## **Simulator Setup**

- Recall IC-323 OR
- Call up RHR IC and initialize. Place simulator in RUN.
- Verify "A" RHR pump running and "B" in AUTO.
- Verify 1-RH-FCV-1605 in auto with flowrate set at 3400 gpm.
- Implement malf for over-current trip of 1-RH-P-1A and allow annunciators B-G-6 & B-G-7 to alarm.
- Place simulator in FREEZE until ready to perform JPM.
- Simulator Operator Note: Place RED magnets on 1-RH-MOV-1700, 1701 and 1720A red bulbs. Place green
  magnets on SI accumulator green bulbs and verify magnets are correct for SI system for CSD. Place a white
  magnet and green arrow on the Pressurizer level cold cal channel 1-RC-LI-1460 and make sure the trend
  recorder is set for this channel.

## **Initiating Cues**

- Trip of the operating RHR pump.
- Shift Manager direction.

## **Directions to the Applicant**

- I am the Shift Manager and you are the Unit RO. The unit has been operating on RHR. 1-RH-P-1A, "A" RHR pump was in service on "A" RHR heat exchanger at 3400 gpm.
- We just received annunciator B-G-6, RHR HX LO FLOW, and 1-RH-P-1A has tripped. Here is a copy of 1-AP27.00, Loss of Decay Heat Removal Capability. I need you to perform the necessary steps to address this
  condition and restore RHR flow. No RCS draindown evolutions are in progress and inventory is stable, not
  decreasing.
- When you complete 1-AP-27.00, please inform me.

#### **Notes**

## PERFORMANCE CHECKLIST

## Notes to the Evaluator

- Task critical elements are bolded and noted at the end of the step as CRITICAL STEP.
- An additional instructor may be needed to silence and acknowledge alarms for the examinee.
- START TIME:

CAUTIONS and NOTE PRIOR TO STEP 1	SAT
CAUTION:	UNSAT
<ul> <li>Loss of RHR due to a total loss of IA is addressed by 0-AP-40.00, NON-RECOVERABLE LOSS OF IA.</li> <li>Loss of RHR may cause CTMT radiological and heat stress conditions to degrade. Local actions in CTMT should be coordinated with HP.</li> <li>During solid plant operation, inadvertent actuation of the OPMS may occur if letdown is isolated.</li> <li>If RCS boiling occurs, non-essential personnel should be evacuated from CTMT.</li> </ul> NOTE: EPIPs may be applicable.	
STANDARD:  • Acknowledges note and acknowledges cautions and recognizes that a total loss of IA is not occuring.	
COMMENTS:	
STEP 1 - CHECKS RCS INVENTORY - DECREASING.  PRZR level - DECREASING Standpipe level - DECREASING Reactor cavity level - DECREASING RCS Narrow Range level - DECREASING CTMT sump level - INCREASING Makeup rate - INCREASING PRT level, pressure, or temperature - INCREASING PDTT level - INCREASING RWST level - INCREASING	SATUNSAT

Surry 2010-301 Loss of RHR Pump

STANDARD:								
•	Notes that there are no draindown evolutions in progress and inventory is stable based on the directions given.  Checks Containment Sump level (1-DA-LI-100) is stable and not increasing.  Checks PRT conditions (level, LI-1-470; pressure, PI-1-472; and temperature, TI-1-471) are stable and not increasing.  Checks PDTT level (1-DG-LI-107) is stable and not increasing.  Checks RWST level stable.  Determines that RCS inventory is NOT decreasing and performs RNO to transition to procedure STEP 4.							
EVALUATOR'S	EVALUATOR'S NOTE: If asked, all indications are as you see them. If asked, no personnel are in Containment. If asked, cavity is not flooded up.							
COMMENTS:								
<b>Step 4</b> – V	SAT							
STANDARD:								
	s that no RHR pumps are running by observing zero amps indicated and 1-RH-as red and amber lights lit, 1-RH-P-1B has green light lit. Goes to RNO.							
EVALUATOR'S	S NOTE: N/A							
COMMENTS:								
STEP 4:		SAT						
Step 4RNO -	a) Manually close RH control valves:  • 1-RH-FCV-1605  • 1-RH-HCV-1758  b) Start one RHR pump. c) Adjust RH control valves to return flow to pre-event rate:	UNSAT						

Surry 2010-301 Loss of RHR Pump

STANDARD:		
<ul> <li>HCV-1758 using ten turn pot.</li> <li>Starts 1-RH-P-1B by taking c amp are indicated. This is a 0</li> <li>Opens 1-RH-FCV-1605 until fin AUTO. This is a Critical Tax</li> </ul>	ontrol switch to the start position and verifying Critical Task.  low is approximately 3400 gpm and then places	
EVALUATOR'S NOTE: N/A		
COMMENTS:		
STEP 5:		0.17
Step 5 - VERIFY RHR FLOW - IND	DICATED ON RHR SYS FLOW	SAT
• 1-RH-FI-1605		UNSAT
STANDARD:		
Checks RHR flow at approxima	ately 3400 gpm on 1-RH-FI-1605.	
EVALUATOR'S NOTE: N/A		
COMMENTS:		
STEP 6:		SAT
Step 6 - CHECK RHR PUMP - VOI	RTEXING	UNSAT
<ul><li>Flow indication on 1-RH-l</li><li>Amperage indication - OS</li></ul>		ONOAT
STANDARD:		
<ul><li>Checks flow steady on 1-RH-FI</li><li>Goes to step 6 RNO.</li></ul>	-1605 and amps steady for 1-RH-P-1B.	
EVALUATOR'S NOTE: N/A		
COMMENTS:		

Surry 2010-301 Loss of RHR Pump

STEP 7:		
Step 12 -	SAT	
	a) Flow on 1-RH-FI-1605 - NORMAL	UNSAT
	b) CC to RHR HX	
	1) RHR HX CC Outlet HDR Flow - NORMAL • 1-CC-FI-110A OR • 1-CC-FI-110B	
	2) RHR HX CC Outlet HDR TEMP - NORMAL • 1-CC-TI-109A OR • 1-CC-TI-109B	
STANDARD:		
<ul> <li>Checks</li> </ul>	s flow on 1-RH-FI-1605 indicating normal about 3400 gpm. s CC to RHR HX on 1-CC-FI-110A normal. s RHR HX CC Outlet Hdr Temp on 1-CC-TI-109A normal.	
EVALUATOR'S	NOTE: If asked, pre-event flow on 1-CC-FI-110A was 4700 and temperature on 1-CC-TI-109A was approximately 88 degrees.	
COMMENTS:		
STEP 8:		SAT
Step 13-	VERIFY RCS TEMPERATURE - STABLE OR DECREASING	UNSAT
STANDARD:		UNSAT
• Checks	RCS temperature and/or RHR temperature stable or decreasing.	
EVALUATOR'S	S CUE: N/A	
COMMENTS:		

STEP 9:  Step 14 - RETURN TO PROCEDURE IN EFFECT  STANDARD:  Operator Exits 1-AP-27.00.  EVALUATOR'S NOTE: N/A  COMMENTS:	SATUNSAT
STEP 10:  REPORT TO SHIFT SUPERVISOR (EVALUATOR).  STANDARD:  Verbal status report that 1-AP-27.00, Loss of Decay Heat Removal Capability is complete and RHR flow has been restored.  EVALUATOR'S NOTE: N/A  STOP TIME:  COMMENTS:	SATUNSAT
STOP TIME:	

## Operator Directions Handout (TO BE READ TO APPLICANT BY EXAMINER)

## **Task**

- Task is to be performed in the simulator.
- Perform 0-AP-27.00, LOSS OF DECAY HEAT REMOVAL CAPABILITY, to restore RHR flow to the core by starting 1-RH-P-1B.

## **Directions**

The evaluator will explain the initial conditions of the task to be performed and will provide the initiating cue. Ensure you indicate to the evaluator when you understand your assigned task

## **Initial Conditions**

1-RH-P-1A was in service on "A" RHR HX. We just received annunciator B-G-6, RHR HX LO FLOW, and 1-RH-P-1A has tripped.

## **Initiating Cues**

- I am the Shift Manager and you are the Unit RO. The unit has been operating on RHR. 1-RH-P-1A, "A" RHR pump was in service on "A" RHR heat exchanger at 3400 gpm.
- We just received annunciator B-G-6, RHR HX LO FLOW, and 1-RH-P-1A has tripped. Here is a copy of 1-AP-27.00, Loss of Decay Heat Removal Capability. I need you to perform the necessary steps to address this condition and restore RHR flow. No RCS draindown evolutions are in progress and inventory is stable, not decreasing.
- When you complete 1-AP-27.00, please inform me.

## Operator Directions Handout (TO BE GIVEN TO APPLICANT)

## **Initial Conditions**

• 1-RH-P-1A was in service on "A" RHR HX. We just received annunciator B-G-6, RHR HX LO FLOW, and 1-RH-P-1A has tripped.

## **Initiating Cues**

- I am the Shift Manager and you are the Unit RO. The unit has been operating on RHR. 1-RH-P-1A, "A" RHR pump was in service on "A" RHR heat exchanger at 3400 gpm.
- We just received annunciator B-G-6, RHR HX LO FLOW, and 1-RH-P-1A has tripped. Here is a copy of 1-AP-27.00, Loss of Decay Heat Removal Capability. I need you to perform the necessary steps to address this condition and restore RHR flow. No RCS draindown evolutions are in progress and inventory is stable, not decreasing.
- When you complete 1-AP-27.00, please inform me.



## SURRY POWER STATION

#### ABNORMAL PROCEDURE

NUMBER 1-AP-27.00	PROCEDURE TITLE  LOSS OF DECAY HEAT REMOVAL CAPABILITY	REVISION 20
1-Al21.00	(WITH 11 ATTACHMENTS)	PAGE 1 of 18

## **PURPOSE**

To provide guidance when the RHR System fails to remove decay heat.

## **ENTRY CONDITIONS**

- 1) No RHR pumps running due to failure or loss of power.
- 2) Air-binding of the operating RHR pump as indicated by any of the following:
  - · Motor amperage oscillations
  - · Flow oscillations
  - · Excessive pump noise
  - RHR HX LO FLOW annunciator, 1B-G6
- 3) Failure of the RHR system to control RCS temperature due to loss of Component Cooling or valve failure.
- 4) Loss of RCS inventory while on RHR as indicated by any of the following:
  - · Decreasing Pressurizer Level
  - Increasing PRT level, pressure, or temperature
  - · Local observation of RCS inventory loss
  - · CTMT SUMP HI LVL annunciator, 1B-A3
  - SHUTDOWN COOLING LO LVL annunciator, 1B-G8
  - Decreasing trend on 1-RC-LR-105, COLD SHUTDOWN RCS LEVEL NARROW RANGE
- 5) Transition from 1-FR-C.3, RESPONSE TO SATURATED CORE COOLING.

# CONTINUOUS USE

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NUMBER	PROCEDURE TITLE	REVISION 20
1-AP-27.00	LOSS OF DECAY HEAT REMOVAL CAPABILITY	PAGE 2 of 18

_								_			_
Н	STEP	$\vdash$	A	CTION/E	EXPECT	ED RES	PONSE	<u> </u>		RESPONSE NOT OBTAINED	
											•
*	* * * *	* *	* * *	* * * *	* * *	* * * *	* * * * *	* * * *	* *	* * * * * * * * * * * * * * * * *	* * *
۱	CAUTIO	N:	• Los OF		R due to	o a total	loss of IA i	s addres	sed	by 0-AP-40.00, NON-RECOVERABLE	LOSS
							TMT radiolonated with l	-	nd he	at stress conditions to degrade. Local a	ections
			• Dur	ing solid	l plant o	peratio	n, inadvert	ent actua	ation	of the OPMS may occur if letdown is is	solated
			• If R	CS boili	ng occi	ırs, non-	essential p	ersonne	el sho	ould be evacuated from CTMT.	
*	* * * *	* *	* * *	* * * *	* * *	* * * *	* * * * *	* * * *	* *	* * * * * * * * * * * * * * * * * *	* * *
	NOT	E:	EPIPs	may be	applic	able.					
	1	_ c	CHECK	RCS IN	VENT	ORY - D	ECREASII	٧G		GO TO Step 4.	
						EASING					
						CREAS - DECR	EASING				
			RCS	Narrow	Range						
			DEC	REASIN	G						
						NCREA					
						EASING	erature -				
		•		EASIN		or temp	erature -				
		•	PDTT	level -	INCRE	ASING					
		•	RWS	T level -	INCRE	ASING					
l											

NUMBER	PROCEDURE TITLE  LOSS OF DECAY HEAT REMOVAL CAPABILITY	REVISION 20
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COTED	ACTION/SYSTEM RESPONSE	_	DESCRIVE NOT OBTAINED	
STEP	ACTION/EXPECTED RESPONSE —	L	RESPONSE NOT OBTAINED	
2	ATTEMPT TO IDENTIFY AND STOP INVENTORY LOSS:			
	a) Stop any known draining evolution			
	b) Close RHR LETDOWN FLOW valve	b)	Close 1-CH-PCV-1145.	
	• 1-RH-HCV-1142			
	c) Close or verify closed RCS loop drains			
	• 1-RC-HCV-1557A			
	• 1-RC-HCV-1557B			
	• 1-RC-HCV-1557C			
	d) Increase RCS makeup			
	e) Terminate any activities that could cause leakage			
	<ul> <li>Valve alignments</li> </ul>			
	Periodic testing			
	Maintenance			
	f) Coordinate local walkdowns with HP to identify and isolate RCS leakage			
	g) Check RCS level - STABLE OR INCREASING	g)	IF RCS temperature greater than 2 THEN GO TO 1-AP-16.01, SHUTD LOCA.	
			IF RCS temperature less than 200° THEN align any available SI flowpa maintain stable or increasing RCS	th to
3	GO TO STEP 15			
Ÿ. <u> </u>	33 13 3121 13			

NUMBER	PROCEDURE TITLE	REVISION 20
1-AP-27.00	LOSS OF DECAY HEAT REMOVAL CAPABILITY	PAGE 4 of 18

STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
4 VERIFY RHR PUMP - ONE RUNNING	<u>IF</u> Emergency Bus power is available, <u>THEN</u> do the following:
	a) Manually close RH control valves:
	<ul><li>□ • 1-RH-FCV-1605</li><li>□ • 1-RH-HCV-1758</li></ul>
	□ b) Start one RHR pump.
	c) Adjust RH control valves to return flow to pre-event rate:
	<ul><li>□ • 1-RH-FCV-1605</li><li>□ • 1-RH-HCV-1758</li></ul>
	□ d) <u>IF an RHR pump can NOT</u> be started, <u>THEN</u> GO TO Step 16.
	<u>IF</u> RHR pump <u>NOT</u> running due to loss of Emergency Bus power, <u>THEN</u> do the following:
	<ul> <li>□ a) Verify initiated or initiate 1-AP-10.07, LOSS OF UNIT 1 POWER.</li> </ul>
	□ b) GO TO Step 16.
5 VERIFY RHR FLOW - INDICATED ON RHR	Verify opened or open the following valves:
SYS FLOW	□ • 1-RH-MOV-1700
□ • 1-RH-FI-1605	□ • 1-RH-MOV-1701
	□ • 1-RH-MOV-1720A
	□ • 1-RH-MOV-1720B
6 CHECK RHR PUMP - VORTEXING	☐ GO TO Step 12.
<ul> <li>Flow indication on 1-RH-FI-1605 - OSCILLATING</li> </ul>	
☐ • Amperage indication - OSCILLATING	

NUMBER	PROCEDURE TITLE	REVISION
1-AP-27.00	LOSS OF DECAY HEAT REMOVAL CAPABILITY	20 PAGE
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STEP ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
7,01,01,02,120,120,120,101	l	NEST STOP TO WINED
***********************		
CAUTION: RCS temperature may increase if RHR fl shutdown. (Attachment 1)	low rate is le	ess than required based on time after
* * * * * * * * * * * * * * * * * * * *	* * * * * *	* * * * * * * * * * * * * * * * * * * *
7 REDUCE RHR FLOW TO STOP VORTEX	ING	
□ • Use 1-RH-FCV-1605 in MANUAL		
<u>OR</u>		
□ • Use 1-RH-HCV-1758		
8 CHECK RHR PUMP - STILL VORTEXING	;	GO TO Step 12.
9 CHECK RCS LEVEL - WITHIN ACCEPTAE REGION	BLE 🗆	Restore RCS level to Acceptable Region of Attachment 2 or 3.
□ • 1-RC-LI-100A (Attachment 2)		
OR		
□ • 1-RC-LR-105 (Attachment 3)		

NUMBER	PROCEDURE TITLE	REVISION 20
1-AP-27.00	LOSS OF DECAY HEAT REMOVAL CAPABILITY	PAGE 6 of 18

			_	
Г	STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
	10	VERIFY RHR PUMPS - BOTH AVAILABLE	E	Restore RHR pump:
				a) Stop pump.
				b) Verify RHR flow - NONE INDICATED.
				c) Vent pump.
				<ul><li>1-RH-P-1A, 1-RH-9</li><li>1-RH-P-1B, 1-RH-3</li></ul>
				d) Restart pump.
				e) <u>IF</u> RHR pump can <u>NOT</u> be restored, <u>THEN</u> GO TO Step 16.
				f) <u>IF</u> RHR pump is restored, <u>THEN</u> GO TO Step 12.
	11	RESTORE RHR PUMPS:		
		a) Stop vortexing pump		
		b) Verify RHR flow - NONE INDICATED		
		c) Manually close 1-RH-FCV-1605 and 1-RH-HCV-1758		
		d) Start other RHR pump		
		e) Adjust RH control valves to return flow pre-event rate:	to 🗆	e) GO TO Step 16.
		<ul><li>1-RH-FCV-1605</li><li>1-RH-HCV-1758</li></ul>		

NUMBER	PROCEDURE TITLE	REVISION 20
1-AP-27.00	LOSS OF DECAY HEAT REMOVAL CAPABILITY	PAGE 7 of 18

STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
	ACTION EXI ECTED RESPONSE		NEST ONSE NOT OBTAINED
10	CHECK BUB HEAT CINIC		
12	CHECK RHR HEAT SINK:		
	a) Flow on 1-RH-FI-1605 - NORMAL		a) Adjust 1-RH-HCV-1758 and 1-RH-FCV-1605 to control flow.
	b) CC to RHR HX		
	<ol> <li>RHR HX CC Outlet HDR Flow - NORMAL</li> </ol>		<ol> <li>Verify opened or open 1-CC-TV-109A or 1-CC-TV-109B.</li> </ol>
	• 1-CC-FI-110A		$\underline{IF}\ TV\ can\ \underline{NOT}\ be\ opened\ due\ to\ a$
	OR		localized loss of IA, <u>THEN</u> locally open IAW 0-FCA-16.00, LOCAL
	• 1-CC-FI-110B		OPERATION OF AIR OPERATED VALVES.
			IF the in-service RHR HX TV can NOT be opened, THEN place the other RHR HX in service IAW 1-OP-RH-001, RHR OPERATIONS.
			<u>IF</u> CC flow can <u>NOT</u> be established to either RHR HX, <u>THEN</u> do the following:
			<ul> <li>Evaluate initiating 1-AP-15.00, LOSS OF COMPONENT COOLING.</li> </ul>
			b. GO TO Step 16.
	RHR HX CC Outlet HDR TEMP -     NORMAL		2) Do the following:
			a. Verify open or open
	• 1-CC-TI-109A OR	_	1-SW-MOV-102A or B.
	<del></del>		b. Adjust SW flow to CC HXs.
	• 1-CC-TI-109B		<u>IF</u> temperature can <u>NOT</u> be stabilized, <u>THEN</u> GO TO Step 16.

NUMBER	PROCEDURE TITLE  LOSS OF DECAY HEAT REMOVAL CAPABILITY	REVISION 20
1-AP-27.00	LOGS OF BEOM FRENIOVAL CAPABILITY	PAGE 8 of 18

Н	STEP	ACTION/EXPECTED RESP	ONSE		RESPONSE NOT OBTAINED	
				'		
	12	VEDIEV BOS TEMBERATURE	STABLE OR		Adiabat 4 DILLIGN/ 4759 to control	
	13	VERIFY RCS TEMPERATURE	- STABLE OR	ш	Adjust 1-RH-HCV-1758 to control	
		DECREASING			temperature.	
					IF towns and we are NOT be atabilized	
				ш	IF temperature can NOT be stabilized	,
					THEN GO TO Step 16.	
	14	RETURN TO PROCEDURE IN	EFFECT			

NUMBER	PROCEDURE TITLE  LOSS OF DECAY HEAT REMOVAL CAPABILITY	REVISION 20
1-AP-27.00	LOSS OF DECAT HEAT REMOVAL CAPABILITY	PAGE 9 of 18

STEP	ACTION/EXPECTED RESPONSE		_	RESPONSE NOT OBTAINED
			Ь	
****	*******		- 4	
CAUTION	<ol> <li>RCS temperature may increase if RHR flow rate shutdown. (Attachment 1)</li> </ol>	) IS 11	955	than required based on time after
* * * * *	* * * * * * * * * * * * * * * * * * * *	* * *	* * :	* * * * * * * * * * * * * * * * * * * *
NOTE	<ul> <li>Changes in RCS pressure can result in vesse vessel level indicator.</li> </ul>	el wa	iter l	level changes not shown by the RCS
	Any dilution of the RCS should be stopped un	ntil R	.HR	flow has been reestablished.
15	CHECK IF RHR PUMPS SHOULD BE STOPPED:			
	a) RHR Pumps - ANY RUNNING		a)	GO TO Step 16.
	b) RCS level - WITHIN ACCEPTABLE REGION		b)	Do the following:
				Restore RCS level to Acceptable  Region of Attachment 2 or 2
	1-RC-LI-100A (Attachment 2)			Region of Attachment 2 or 3
	OR			OR
	1-RC-LR-105 (Attachment 3)			<ul> <li>Reduce RHR flow to Acceptable Region of Attachment 2 or 3 using 1-RH-FCV-1605 or 1-RH-HCV-1758</li> </ul>
	c) RHR pumps - VORTEXING		c)	RETURN TO appropriate plant
	Flow indication on 1-RH-FI-1605 - OSCILLATING			procedure.
	Amperage indication -     OSCILLATING			
	d) Stop RHR pumps			

NUMBER	PROCEDURE TITLE	REVISION 20
1-AP-27.00	LOSS OF DECAY HEAT REMOVAL CAPABILITY	PAGE 10 of 18

STEP	ACTION/EXPECTED RESPONSE	<u> </u>	RESPONSE NOT OBTAINED
	ACTION EXILECTED NEOF CITIES	J	NEST STISE ITST SETTING
****	* * * * * * * * * * * * * * * * * * * *	* * * * * *	* * * * * * * * * * * * * * * * * * * *
CAUTION:	Securing CTMT purge and closing the P	Personnel Ha	tch door(s) must be closely coordinated.
*****	* * * * * * * * * * * * * * * * * * * *	* * * * *	* * * * * * * * * * * * * * * * * * * *
	NITIATE ACTIONS TO PROTECT PERSONNEL WORKING IN CTMT:		
	a) Notify HP		
	c) Evacuate non-essential personnel in CTMT		
	c) Periodically monitor CTMT radiation conditions		
	d) Verify CTMT purge - SECURED		d) Secure CTMT purge.

NUMBER	PROCEDURE TITLE	REVISION 20
1-AP-27.00	LOSS OF DECAY HEAT REMOVAL CAPABILITY	PAGE 11 of 18

ı	OTED	ACTION/EXPECTED DECDONOR			DEODONOE NOT OPTAINED	
Ц	STEP	ACTION/EXPECTED RESPONSE		<u> </u>	RESPONSE NOT OBTAINED	
	17	INITIATE ACTIONS TO ESTABLISH CTMT CLOSURE WHILE CONTINUING WITH PROCEDURE				
		<ul> <li>a) If initially at Reduced Inventory, establish CTMT closure IAW 1-OSP-CT-214, Containment Closure for Reduced or Potentially Reduced Inventory Conditions (Refer to Attachments 6 and 7 1-OSP-CT-214)</li> </ul>				
		b) If initially at Decreased Inventory, establish CTMT closure IAW 1-OP-CT-002, Containment Penetration Breach Log	n			
		<ul> <li>c) If initially not at Reduced or Decreased Inventory, establish CTMT closure IAW Shift Supervision direction</li> </ul>				
		d) Check CTMT Closure Team -		d) [	Do the following:	
		ESTABLISHED		1	<ol> <li>Direct Mechanical Foreman to initiate 0-MCM-1202-06, Emergency Closure of the Equipment Hatch, as necessary.</li> </ol>	
				2	2) GO TO Step 17f.	
		e) Direct CTMT Closure Team to initiate 0-MCM-1202-06, Emergency Closure of the Equipment Hatch, as necessary				
		f) Verify closed or close at least one door of the Personnel Hatch				
	18	START AVAILABLE CTMT AIR RECIRC FANS				

NUMBER	PROCEDURE TITLE	REVISION 20
1-AP-27.00	LOSS OF DECAY HEAT REMOVAL CAPABILITY	PAGE 12 of 18

STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
NOTE: . Stand 10 through 22 actablish on alternate model	a of dance has transport
NOTE: • Steps 19 through 22 establish an alternate mode	
<ul> <li>Attachment 10 may be used for cooling the RCS</li> </ul>	S with the SFP and RWST coolers.
19 CHECK THE FOLLOWING - AVAILABLE TO E SUPPORT NATURAL CIRCULATION COOLING:	GO TO Step 21.
<ul> <li>1-OSP-ZZ-003, Attachment 2 equipment - AVAILABLE</li> </ul>	
OR	
<ul> <li>1-OSP-ZZ-004, Attachment 2 equipment - AVAILABLE</li> </ul>	
20 GO TO ATTACHMENT 4	
1	

NUMBER	PROCEDURE TITLE	REVISION 20
1-AP-27.00	LOSS OF DECAY HEAT REMOVAL CAPABILITY	PAGE 13 of 18

STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
21. CHECK THE FOLLOWING - AVAILABLE TO SUPPORT REFLUX COOLING:  1-OSP-ZZ-003, Attachment 3 OR 1-OSP-ZZ-004, Attachment 4 - EQUIPMENT AVABILABLE	☐ IF CHG and LHSI Pumps NOT available due to electrical or other reasons, THEN GO TO Attachment 8 to align gravity feed.  IF forced flow available, THEN do the following:
□ • RCS inventory - NOT DECREASING	a) IF PRZR available, <u>THEN</u> do the following:
	<ol> <li>Increase PRZR level to between 40 and 60% using any of the following:</li> </ol>
	<ul> <li>Normal Charging</li> <li>LHSI Pump</li> <li>CHG Pump HHSI</li> <li>Charging Crosstie</li> </ul>
	<ul> <li>2) WHEN RHR system available, THEN GO TO Step 23.</li> </ul>
	<ul> <li>3) <u>IF</u> RCS approaches saturation, <u>THEN</u></li> <li>GO TO Attachment 6.</li> </ul>
	b) <u>IF PRZR NOT</u> available, <u>THEN</u> do the following:
	<ul> <li>1) <u>WHEN</u> RHR system available, <u>THEN</u></li> <li>GO TO Step 23.</li> </ul>
	<ul> <li>2) <u>IF</u> RCS approaches saturation, <u>THEN</u></li> <li>GO TO Attachment 6.</li> </ul>
22 GO TO ATTACHMENT 5	

NUMBER	PROCEDURE TITLE	REVISION 20
1-AP-27.00	LOSS OF DECAY HEAT REMOVAL CAPABILITY	PAGE 14 of 18

STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
********	* * * * * * * * * * * * * * * * * * * *
CAUTION: • RCS standpipe level indication, 1-RC-LI-100A, w of 12.1 FT.	ill not indicate below actual RCS level
Personnel working in CTMT should be warned be contamination of personnel working near RCS or	
Only borated water should be added to the RCS	to maintain adequate shutdown margin.
********	* * * * * * * * * * * * * * * * * * * *
23 CHECK RCS LEVEL	GO TO Step 25.
☐ • Greater than 12.1 FT on 1-RC-LI-100A	
<u>OR</u>	
☐ • Greater than 12 FT 1 IN on 1-RC-LR-105	
24 GO TO STEP 26	

NUMBER	PROCEDURE TITLE  LOSS OF DECAY HEAT REMOVAL CAPABILITY	REVISION 20
1-AP-27.00	LOSS OF DECAT REMOVAL CAPABILITY	PAGE 15 of 18

STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
		_	
25	REFILL THE RCS:		
	Align and start at least one CHG pump for cold leg injection	□ a)	) Align and start one LHSI pump for cold leg injection.
			<u>IF</u> a cold leg flow path is <u>NOT</u> available, <u>THEN</u> make up to the RCS using <u>one</u> of the following:
			1) CHG pump to hot leg.
			2) LHSI pump to hot leg.
			<ol> <li>RWST gravity feed or VCT overpressure feed.</li> </ol>
			4) Any other CHG flowpath.
	b) Refill the RCS until level is greater than required:		
	• 12.1 FT on 1-RC-LI-100A		
	OR		
	• 12 FT 1 IN on 1-RC-LR-105		
NOTE	Before additional actions are taken to obtain a RCS should be considered when deciding hor		
	<ul> <li>If adequate time to completely vent the RHR s RHR lines by filling the RCS to 13.5 FT, (off-s subcooling, and running an RHR pump at a fl</li> </ul>	cale hig	gh on 1-RC-LR-105) verifying 10°F RCS
26	VENT RHR SYSTEM AS NECESSARY:		
	Maintain RCS level while venting RHR system		
	b) Locally vent from 1-RH-42		

NUMBER	PROCEDURE TITLE  LOSS OF DECAY HEAT REMOVAL CAPABILITY	REVISION 20
1-AP-27.00	EOOS OF BEOAT HEAT REMOVAE OAF ABIETT	PAGE 16 of 18

STEP	ACTION/EXPECTED RESPONSE	<b></b>	RESPONSE NOT OBTAINED
		J	
* * * * * *	* * * * * * * * * * * * * * * * * * * *	* * * * * *	* * * * * * * * * * * * * * * * * * * *
CAUTION: E	Extended operation at low RHR flowrate	s may cause	e cavitation concerns in 1-RH-FCV-1605.
* * * * * *	* * * * * * * * * * * * * * * * * * *	* * * * * *	* * * * * * * * * * * * * * * * * * * *
NOTE: •	<ul> <li>Starting an RHR pump may result in a</li> </ul>	decrease in	RCS level due to shrink or void collapse.
•	<ul> <li>The RCS level necessary to operate F provide guidance for determining the r</li> </ul>		is a function of RHR flow. Attachment 2 or 3 S level.
27 RE	ESTORE RHR FLOW:		
a)	Close RH control valves		
	• 1-RH-HCV-1758		
	• 1-RH-FCV-1605		
□ b)	Start one RHR pump		
□ c)	Maintain RCS level IAW Attachment 2	or 3	
□ d)	Increase RHR bypass flow to the desir flowrate	ed	
(STEP 27 CO	NTINUED ON NEXT PAGE)		

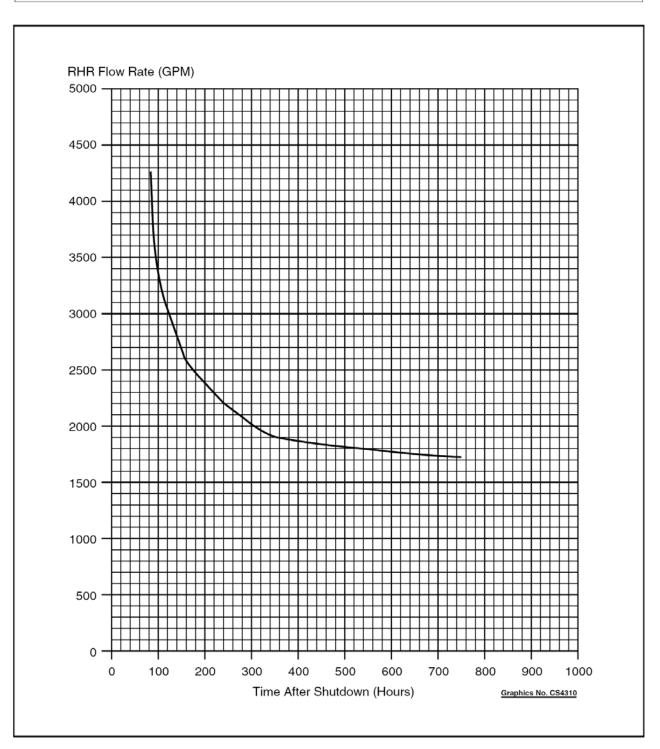
NUMBER	PROCEDURE TITLE  LOSS OF DECAY HEAT REMOVAL CAPABILITY	REVISION 20
1-AP-27.00	LOSS OF DECAT HEAT REMOVAL CAPABILITY	PAGE 17 of 18

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
O I E I	AGTION/EXI EGTES REGI GROE	KEGI GROE HOT GETAINEE
27.	RESTORE RHR FLOW: (Continued)	
	e) Check RHR flow - RESTORED	e) Do the following:
		<ol> <li>Continue to monitor RCS temperature.</li> </ol>
		<ol> <li>Consult with TSC or plant staff and GO TO the appropriate Attachment for alternate means of decay heat removal:</li> </ol>
		<ul> <li>Attachment 4, Natural Circulation Cooling</li> </ul>
		<ul> <li>Attachment 5, Reflux Boiling Heat Removal</li> </ul>
		Attachment 6, Forced Feed Cooling
		Attachment 8, Gravity Feed Cooling
	f) Terminate alternate mode of decay heat removal	
	g) Control RCS cooldown rate at less than 50°F/hr	
28	CHECK IF RCS MAKEUP SHOULD BE REDUCED:	
	a) RCS temperature - LESS THAN 200°F	a) Continue cooling with RHR.
	b) RCS level - STABLE OR INCREASING	b) Control charging flow to maintain RCS level IAW Attachment 2 or 3.
29	CHECK RCS TEMPERATURE - LESS THAN 140°F	Continue cooling with RHR. RETURN TO Step 27g.

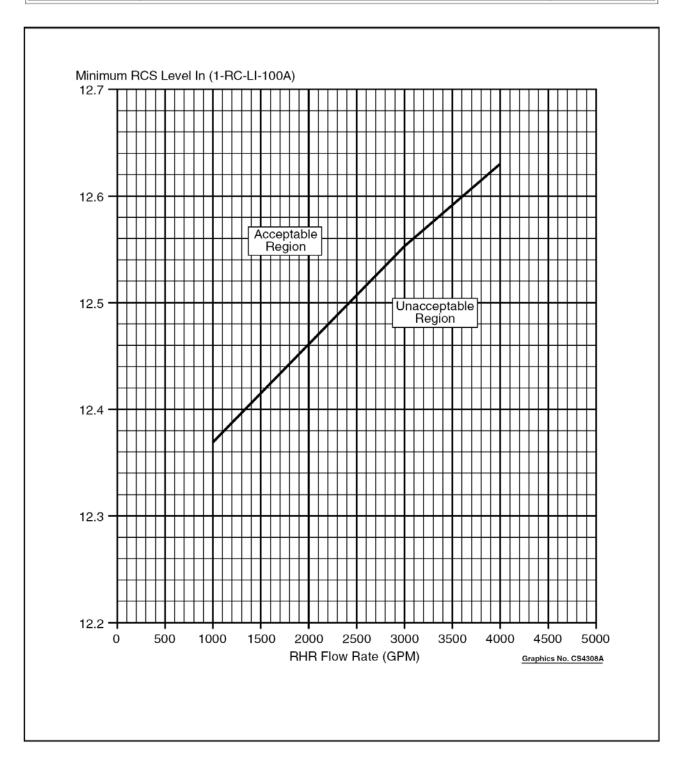
NUMBER	PROCEDURE TITLE	REVISION 20
1-AP-27.00	LOSS OF DECAY HEAT REMOVAL CAPABILITY	PAGE 18 of 18

STEP	ACTION/EXPECTED RESPONSE	7	RESPONSE NOT OBTAINED	
T SIEP T	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED	
30 F	RETURN TO PROCEDURE IN EFFECT			
		- END -		
I				

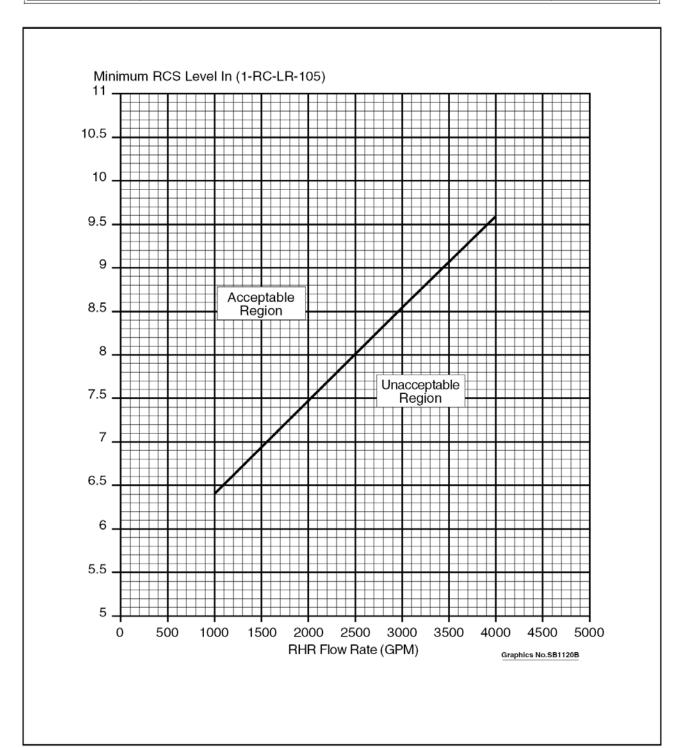
NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 1
REVISION 20	RHR FLOW REQUIREMENT VERSUS TIME AFTER SHUTDOWN	PAGE 1 of 1



NUMBER 1-AP-27.00		ATTACHMENT 2
REVISION 20	MINIMUM RCS LEVEL VERSUS RHR FLOW (1-RC-LI-100A)	PAGE 1 of 1



- 1			
	NUMBER	ATTACHMENT TITLE	ATTACHMENT
	1-AP-27.00	MINIMUM BOOLEVEL VERGUO RUR ELOW (4 DO LR 405)	3
	REVISION	MINIMUM RCS LEVEL VERSUS RHR FLOW (1-RC-LR-105)	PAGE
	20		1 of 1



NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 4
REVISION 20	NATURAL CIRCULATION COOLING	PAGE 1 of 1

NOTE: The RCS must be pressurized and SG tubes filled for Natural Circulation cooling to be effective.
<ol> <li>Attempt to start an RCP IAW 1-OP-RC-001, STARTING AND RUNNING ANY RCP. IF an RCP can be started, <u>THEN</u> RETURN TO procedure in effect. <u>IF</u> an RCP can <u>NOT</u> be started, <u>THEN</u> GO TO Step 2.</li> </ol>
2 Verify running or start three CRDM fans.
3 Control SG narrow range level between 11 and 65% in at least one SG:
□ • For AFW, control flow using 1-FW-MOV-151A through F.
□ • For CN, control flow using 1-FW-HCV-155A, B, or C.
4 Control Pressurizer level between 15 and 75% using charging and letdown.
5 Monitor RCS conditions for satisfactory Natural Circulation cooling:
□ • RCS Subcooling based on CETCs - GREATER THAN 30°F
CETCs - STABLE OR SLOWLY DECREASING
SG pressure - STABLE OR SLOWLY DECREASING
<ul> <li>WR hot leg temperature - STABLE OR SLOWLY DECREASING</li> </ul>
<ul> <li>WR cold leg temperature - AT SATURATION FOR SG PRESSURE</li> </ul>
6 <u>IF</u> any of the above parameters indicate a loss of Natural Circulation, <u>THEN</u> <u>slowly</u> increase steam flow rate using Steam Dumps or SG PORVs.
7 IF RCS cooldown is desired, <u>THEN</u> maintain cooldown rate less than or equal to 25°F/HR.
8 <u>WHEN</u> RHR system available, <u>THEN</u> GO TO procedure Step 23.
<ol> <li>IF Natural Circulation can NOT control RCS temperature AND CETC temperatures are approaching saturation, THEN GO TO Attachment 5.</li> </ol>

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 5
REVISION 20	REFLUX BOILING HEAT REMOVAL	PAGE 1 of 2

NOTE: • The number of SGs required for reflux cooling are as follows:
3 SGs, if shutdown less than 75 hours
<ul> <li>2 SGs, if shutdown greater than or equal to 75 hours and less than 375 hours</li> </ul>
<ul> <li>1 SG, if shutdown greater than or equal to 375 hours</li> </ul>
<ul> <li>Reflux cooling should occur when RCS temperature is maintained between 280 and 290°F.</li> </ul>
1 Send an Operator to CTMT to close 1-RC-184, Reactor Vessel Head vent isolation.
2 Verify closed or close both PRZR PORVs.
3 Allow RCS temperature to increase to between 280 and 290°F.
4 <u>WHEN</u> RCS temperature increases to between 280 and 290°F, <u>THEN</u> increase steam rate of SG(s) to maintain this temperature using SG PORV(s) or Steam Dumps to the Main Condenser.
5 Control SG narrow range level between 11 and 65% in the required number of SGs:
□ • For AFW, control flow using 1-FW-MOV-151A through F.
□ • For CN, control flow using 1-FW-HCV-155A, B, or C.
NOTE: RCS makeup will be needed only to account for losses due to leakage.
6 Control RCS level within the range of 1-RC-LR-105, COLD SHUTDOWN RCS LEVEL NARROW RANGE, using any of the following:
□ • Normal Charging
RCP Seal Injection
High Head or Low Head SI flow to the Cold or Hot legs
7 Monitor RCS conditions for satisfactory Reflux cooling:
RCS temperature - STABLE OR SLOWLY DECREASING
8 WHEN RHR System available, THEN GO TO procedure Step 23.

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 5
REVISION 20	REFLUX BOILING HEAT REMOVAL	PAGE 2 of 2

9 IF Reflux Boiling Heat Removal can NOT control RCS temperature, THEN GO TO Attachment 6.

NUMBER	ATTACHMENT TITLE	ATTACHMENT
1-AP-27.00	FORCED FEED COOLING	6
REVISION	TORROLD FILED GOGLING	PAGE
20		1 of 7

* * * * * * * * * * * * * * * * * * * *
<b>CAUTION:</b> • Personnel working in CTMT should be warned before the RCS is filled to avoid inadvertent contamination of personnel working near RCS openings.
<ul> <li>Only borated water should be added to the RCS to maintain adequate shutdown margin.</li> </ul>
<ul> <li>The intent of this Attachment is to maintain subcooled conditions in the RCS.</li> </ul>
* * * * * * * * * * * * * * * * * * * *
<b>NOTE:</b> The cold leg flow path is preferred for RCS feed and bleed. The hot leg should be used if the cold leg is <u>NOT</u> available and the PRZR can be filled.
<ol> <li>Determine which LHSI Pump is to be started. <u>IF</u> LHSI Pump <u>NOT</u> available, <u>THEN</u> GO TO Step 15.</li> </ol>
1-SI-P-1A 1-SI-P-1B
2 Verify open or open LHSI PUMP SUCTION FROM RWST MOV for LHSI Pump to be started:
1-SI-MOV-1862A or 1-SI-MOV-1862B
3 Verify open or open LHSI RECIRC PUMP MOVs for LHSI Pump to be started:
1-SI-MOV-1885A and 1-SI-MOV-1885D
<u>OR</u>
1-SI-MOV-1885B and 1-SI-MOV-1885C
4 Start LHSI Pump determined in Step 1.
5 Establish RCS bleed path IAW the following:
a. <u>IF</u> PRZR Safety Valve previously removed, <u>THEN</u> GO TO Step 6.
b. <u>IF</u> time since shutdown is greater than 14 hours and less than 113 hours, <u>THEN</u> open two PRZR PORVs and the associated Block MOVs.
c. <u>IF</u> time since shutdown is greater than or equal to 113 hours, <u>THEN</u> open <u>one</u> PRZR PORV and the associated Block MOV.

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 6
REVISION 20	FORCED FEED COOLING	PAGE 2 of 7

6 <u>IF</u> cold leg injection to be used, <u>THEN</u> perform Steps 7 through 11. <u>IF</u> hot leg injection to be used, <u>THEN</u> GO TO Step 12.
7 Verify energized or locally close breaker for 1-SI-MOV-1890C, LHSI TO COLD LEGS. (1H1-2N 9A)
8 Open 1-SI-MOV-1890C, LHSI TO COLD LEGS.
9 Throttle SI flow to RCS IAW Attachment 7 using LHSI PUMP TO COLD LEGS. <u>IF</u> RCS temperatures decrease, <u>THEN</u> throttle flow to maintain stable temperature.
1-SI-MOV-1864A <u>or</u> 1-SI-MOV-1864B
<ul> <li>10 <u>IF</u> RCS temperatures can <u>NOT</u> be maintained less than 200°F, <u>THEN</u> do <u>either</u> of the following:</li> <li>Increase SI flowrate.</li> </ul>
□ • Swap to hot leg injection.
11 GO TO Step 13.
12 Align for hot leg injection IAW the following:
a. Throttle SI flow to RCS IAW Attachment 7 using LHSI TO HOT LEGS for running LHSI Pump:
1-SI-MOV-1890A (Key 47) <u>or</u> 1-SI-MOV-1890B (Key 48)
b. <u>IF</u> hot leg injection <u>NOT</u> controlling RCS temperature as indicated by increasing PRZR liquid and vapor space temperatures or by decreasing subcooling, <u>THEN</u> do <u>either</u> of the following:
□ • Increase SI flowrate.
□ • Swap to cold leg injection.
13 IF RWST level decreases to 20% due to RCS feed and bleed, THEN GO TO Step 28.
14 WHEN RHR System available, THEN RETURN TO procedure Step 23.

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 6
REVISION 20	FORCED FEED COOLING	PAGE 3 of 7

5.	Align CHG Pump suction to	the RWST:	
	a. Open CHG PUMP S	UCT FROM RWST MOVs:	
	• 1-CH-MOV-1115B		
	• 1-CH-MOV-1115D		
	b. Close CHG PUMP S	UCTION FROM VCT MOVS	<b>S</b> :
	1 • 1-CH-MOV-1115C		
	• 1-CH-MOV-1115E		
16.	Determine which CHG Pum	p is to be started: -CH-P-1B 1-CH-P	-1C
		-CH-P-1B 1-CH-P	
	1-CH-P-1A 1  Verify open or open the follo  1-CH-P-1A	owing MOVs for the CHG Pu	ump selected in Step 16:
	1-CH-P-1A 1.  Verify open or open the follo	owing MOVs for the CHG Pu	ump selected in Step 16:
	1-CH-P-1A 1  Verify open or open the follo  1-CH-P-1A	owing MOVs for the CHG Pu	ump selected in Step 16:
	1-CH-P-1A 1 Verify open or open the follo  1-CH-P-1A  1-CH-MOV-1267A	1-CH-P-1B 1-CH-P-1D 1-CH-P-1B 1-CH-MOV-1269A	1-CH-P-1C 1-CH-MOV-1270A
	1-CH-P-1A 1 Verify open or open the folk  1-CH-P-1A 1-CH-MOV-1267A 1-CH-MOV-1267B	1-CH-P-1B 1-CH-P-20 1-CH-P-1B 1-CH-MOV-1269A 1-CH-MOV-1269B	1-CH-P-1C 1-CH-MOV-1270A 1-CH-MOV-1270B
	1-CH-P-1A1 Verify open or open the follow  1-CH-P-1A1-CH-MOV-1267A1-CH-MOV-1267B1-CH-MOV-1275A	1-CH-P-1B 1-CH-P-1D 1-CH-P-1B 1-CH-MOV-1269A 1-CH-MOV-1275B	1-CH-P-1C 1-CH-MOV-1270A 1-CH-MOV-1270B 1-CH-MOV-1275C

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 6
REVISION 20	FORCED FEED COOLING	PAGE 4 of 7

19.	Establish RCS bleed path IAW the following:
	a. Determine the required minimum SI flow for decay heat removal from Attachment 7.
	gpm
	b. <u>IF PRZR Safety Valve previously removed, THEN</u> GO TO Step 20.
	c. <u>IF</u> time since shutdown is greater than 112 hours and less than 120 hours, <u>THEN</u> open two PRZR PORVs and the associated Block MOVs.
	d. <u>IF</u> time since shutdown is greater than or equal to 120 hours, <u>THEN</u> open one PRZR PORV and the associated Block MOV.
NOT	E: The cold leg flow path is preferred for RCS feed and bleed.
20	<u>IF</u> cold leg injection to be used, <u>THEN</u> perform Steps 21 through 24. <u>IF</u> hot leg injection to be used, <u>THEN</u> GO TO Step 25.
21	Send an Operator to the Auxiliary Building basement.
22	Using one of the following MOVs, establish greater than the minimum SI flow for decay heat removal as determined in Step 19a by directing local throttling of SI flow by opening the MOV's breaker and throttling the selected MOV IAW Attachment 7. IF RCS temperatures decrease, THEN throttle flow to maintain stable temperature.
	1-SI-MOV-1867C, HHSI TO COLD LEGS (1H1-1 2C)
	1-SI-MOV-1867D, HHSI TO COLD LEGS (1J1-1 8C)
	1-SI-MOV-1842, ALT HHSI TO COLD LEGS (1H1-2N 4B)
23	<ul> <li>IF RCS temperatures can NOT be maintained less than 200°F, THEN do either of the following:</li> <li>Increase SI flowrate.</li> </ul>
	Swap to hot leg injection.
24	GO TO Step 26.

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 6
REVISION 20	FORCED FEED COOLING	PAGE 5 of 7

25 Align for hot leg injection IAW the following:
a. Send an Operator to the Auxiliary Building basement.
b. Establish greater than the minimum SI flow for decay heat removal as determined in Step 19a by directing local throttling of SI flow by opening the breaker for the selected MOV and throttling the selected MOV:
1-SI-MOV-1869A, HHSI TO HOT LEGS (1H1-1 3C)
1-SI-MOV-1869B, HHSI TO HOT LEGS (1J1-1 9A)
c. <u>IF</u> hot leg injection <u>NOT</u> controlling RCS temperature as indicated by increasing PRZR liquid and vapor space temperatures <u>or</u> by decreasing subcooling, <u>THEN</u> do <u>either</u> of the following:
□ • Increase SI flowrate.
□ • Swap to cold leg injection.
26 <u>IF</u> RWST level decreases to 20% due to RCS feed and bleed, <u>THEN</u> GO TO Step 28.
27 <u>WHEN</u> RHR System available, <u>THEN</u> RETURN TO procedure Step 23.

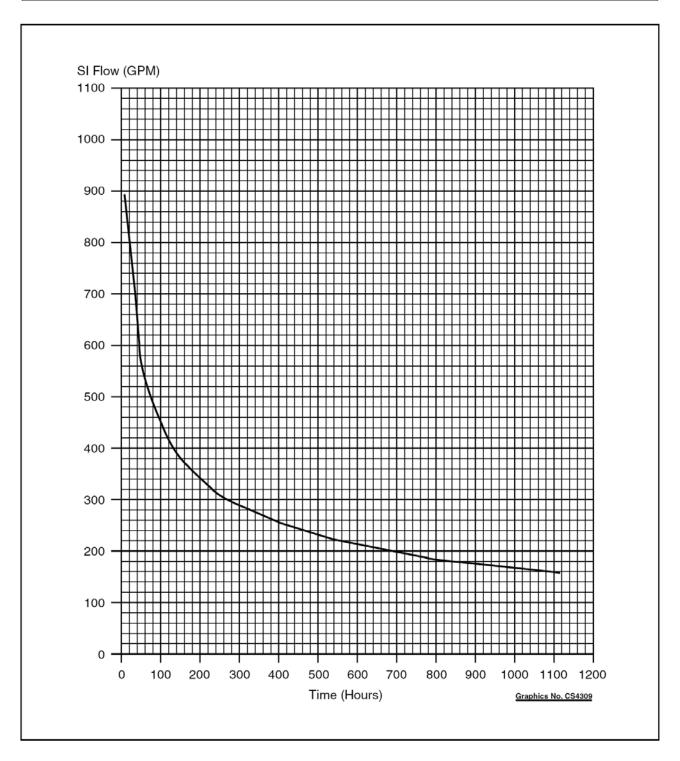
NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 6
REVISION 20	FORCED FEED COOLING	PAGE 6 of 7

************
<b>CAUTION:</b> • The SI system must not be aligned to the CTMT sump since adequate water level will not be available.
<ul> <li>If suction source is lost to any SI pump, the pump should be stopped.</li> </ul>
<ul> <li>Long-term loss of RHR may require manual alignment of one RS HX.</li> </ul>
<ul> <li>TSC should be consulted to determine preferred course of action.</li> </ul>
* * * * * * * * * * * * * * * * * * * *
28 Verify or place the RMT key switches in the REFUEL position.
29 <u>IF</u> RWST inventory can <u>NOT</u> support continued RCS feed and bleed, <u>THEN</u> align CHG pump suction crosstie from Unit 2 IAW the following:
NOTE: An LCO clock may be entered on Unit 2 if the RWST is crosstied.
1 Verify open or open CHG PUMP SUCT FROM RWST MOVs:
□ • 1-CH-MOV-1115B
□ • 1-CH-MOV-1115D
2 Close CHG PUMP SUCTION FROM VCT MOVs:
□ • 1-CH-MOV-1115C
□ • 1-CH-MOV-1115E
3 Open RWST CROSSTIE valves:
□ • 1-SI-TV-102A
□ • 1-SI-TV-102B
4 Direct Unit 2 Operator to open RWST CROSSTIE valves:
□ • 2-SI-TV-202A
□ • 2-SI-TV-202B

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 6
REVISION 20	FORCED FEED COOLING	PAGE 7 of 7

5	Start makeup to Unit 2 RWST.
6	WHEN RHR System available, THEN do the following:
	a. Close the RWST CROSSTIE valves on Unit 1 and Unit 2.
	b. Establish charging and letdown to maintain stable PRZR level and pressure.
	c. RETURN TO Procedure Step 23.

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 7
REVISION 20	MINIMUM SI FLOW REQUIRED FOR DECAY HEAT REMOVAL	PAGE 1 of 1



NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 8
REVISION 20	GRAVITY FEED COOLING	PAGE 1 of 5

***********
CAUTION: • A PRZR Safety Valve must be removed for gravity feed cooling to be effective.
<ul> <li>With an initially full RWST aligned to a LHSI Cold leg flowpath, gravity feed will suppress boiling for one hour if time after shutdown is greater than 110 hours and less than 325 hours.</li> </ul>
<ul> <li>With an initially full RWST aligned to a LHSI Cold leg flowpath, gravity feed will suppress boiling for three hours if time after shutdown is greater than 325 hours.</li> </ul>
* * * * * * * * * * * * * * * * * * * *
NOTE: • In order of priority, the flowpaths for aligning gravity feed are as follows:
•LHSI to Cold legs
•LHSI to Hot legs
•CHG Pump to Cold legs
•CHG Pump to Hot legs
<ul> <li>Attachment 9 may be used to determine the required RWST level to suppress boiling for one hour, based on time from shutdown.</li> </ul>
<ol> <li>IF CHG Pump crosstie from Unit 2 desired for RCS feed, <u>THEN</u> GO TO Step 13. <u>IF</u> gravity feed desired, <u>THEN</u> continue in this Attachment.</li> </ol>
2 <u>IF</u> LHSI Pump flowpath to Cold leg to be used, <u>THEN</u> do the following. <u>IF</u> LHSI Pump flowpath to Hot leg to be used, <u>THEN</u> GO TO Step 3. <u>IF</u> CHG Pump to be used, <u>THEN</u> GO TO Step 4.
a. Open LHSI PUMP SUCTION FROM RWST MOV:
□ • 1-SI-MOV-1862A or 1-SI-MOV-1862B
b. Open LHSI TO COLD LEGS MOV:
□ • 1-SI-MOV-1890C
c. Throttle SI flow to RCS IAW Attachment 7 using LHSI PUMP TO COLD LEGS MOV. <u>IF</u> RCS temperatures decrease, <u>THEN</u> throttle flow to maintain stable temperature.
□ • 1-SI-MOV-1864A or 1-SI-MOV-1864B

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 8
REVISION 20	GRAVITY FEED COOLING	PAGE 2 of 5

	d. <u>IF</u> RCS temperatures can <u>NOT</u> be maintained less than 200°F, <u>THEN</u> do either of the following:
	Swap to hot leg injection.
	Increase SI flowrate.
	e. WHEN RHR system available, THEN RETURN TO procedure Step 23.
3	<u>IF</u> LHSI Pump flowpath to Hot leg to be used, <u>THEN</u> do the following:
	a. Open LHSI PUMP SUCTION FROM RWST MOV:
	• 1-SI-MOV-1862A or 1-SI-MOV-1862B
	b. Throttle LHSI TO HOT LEGS MOV IAW Attachment 7:
	• 1-SI-MOV-1890A or 1-SI-MOV-1890B
	c. <u>IF</u> hot leg injection <u>NOT</u> controlling RCS temperature as indicated by increasing PRZR liquid and vapor space temperatures <u>OR</u> by decreasing subcooling, <u>THEN</u> do either of the following:
	Swap to cold leg injection.
	Increase SI flowrate.
	d. <u>WHEN</u> RHR system available, <u>THEN</u> RETURN TO procedure Step 23.
4.	Align CHG Pump suction to the RWST:
	a. Open CHG PUMP SUCT FROM RWST MOVs:
	• 1-CH-MOV-1115B
	• 1-CH-MOV-1115D
	b. Close CHG PUMP SUCTION FROM VCT MOVs:
	• 1-CH-MOV-1115C
	• 1-CH-MOV-1115E
5	Determine which CHG Pump to be used:
	1-CH-P-1A 1-CH-P-1B 1-CH-P-1C

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 8
REVISION 20	GRAVITY FEED COOLING	PAGE 3 of 5

6	Verify open or open the follow	ving MOVs for the CHG Pum	p selected in Step 5:	
	1-CH-P-1A	1-CH-P-1B	1-CH-P-1C	
	1-CH-MOV-1267A	1-CH-MOV-1269A	1-CH-MOV-1270A	
	1-CH-MOV-1267B	1-CH-MOV-1269B	1-CH-MOV-1270B	
	1-CH-MOV-1275A	1-CH-MOV-1275B	1-CH-MOV-1275C	
	1-CH-MOV-1286A	1-CH-MOV-1286B	1-CH-MOV-1286C	
	1-CH-MOV-1287A	1-CH-MOV-1287B	1-CH-MOV-1287C	
	1-CH-MOV-1373	1-CH-MOV-1373	1-CH-MOV-1373	
7	Hot leg flowpath to be used,  1-SI-MOV-1867C, HHSI  1-SI-MOV-1867D, HHSI	by opening the MOV's brea	ker and throttling the selected N	ЛОV. <u>IF</u>
8. <u> </u>	<ul> <li>IF RCS temperatures can NC</li> <li>Swap to hot leg injection</li> <li>Increase SI flowrate.</li> </ul>		00°F, <u>THEN</u> do either of the fol	lowing:
9	GO TO Step 12.			
10	Using Attachment 7, direct loand throttling the selected Mo		ening the breaker for the selecte	d MOV
	• 1-SI-MOV-1869A (1H1-	-1 3C) or 1-SI-MOV-18	69B (1J1-1 9A)	
	vapor space temperatures or	decreasing subcooling, THE	dicated by increasing PRZR liq <u>N</u> do either of the following:	uid and
	Swap to cold leg injecti	on.		
	Increase SI flowrate.			

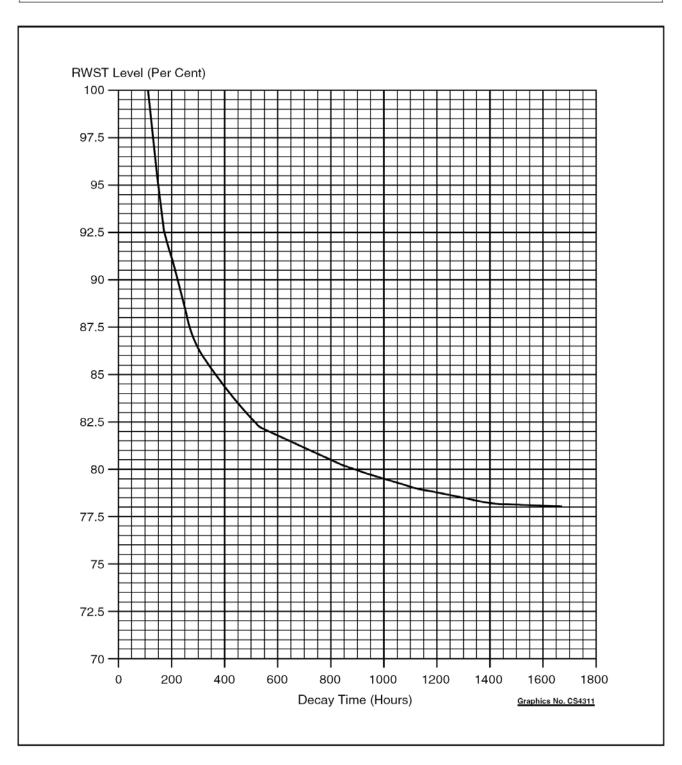
NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 8
REVISION 20	GRAVITY FEED COOLING	PAGE 4 of 5

12 WHEN RHR :	system available, <u>THEN</u> RETURN TO procedure Step 23.
NOTE: An LCO clo	ock may be entered on Unit 2 if Charging system is crosstied.
13 Isolate chargi	ng line IAW either of the following:
a. Locally	close 1-CH-304, Charging Line Isolation.
b. Close	1-CH-FCV-1122, CHG FLOW CNTRL.
14 Verify HHSI to	o cold leg MOVs closed:
□ • 1-SI-MC	OV-1867C
□ • 1-SI-MC	DV-1867D
15 Verify CHG lir	ne isolation MOVs open:
□ • 1-CH-M	OV-1289A
□ • 1-CH-M	OV-1289B
16 Verify Unit 1 0	CHG pump C discharge MOVs open:
□ • 1-CH-M	OV-1286C
□ • 1-CH-M	OV-1287C
17 Verify Unit 2.0	CHG Pump C discharge MOVs open:
□ • 2-CH-M	OV-2286C
□ • 2-CH-M	OV-2287C
18 Verify Unit 2 0	CHG Pump(s) suction aligned to the RWST.
19 Locally open	2-CH-447.
20 Locally vent of	crosstie piping by operating 1-CH-732.

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 8
REVISION 20	GRAVITY FEED COOLING	PAGE 5 of 5

21 Locally open 1-CH-728.
22 Commence makeup to Unit 2 RWST.
23 GO TO Step 19 of Attachment 6.

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 9
REVISION 20	REQUIRED RWST LEVEL TO SUPPRESS BOILING FOR ONE HOUR USING GRAVITY FEED	PAGE 1 of 1



NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 10
REVISION 20	COOLING THE RCS WITH SFP AND RWST COOLERS	PAGE 1 of 2

* * * * * * * * * * * * * * * * * * * *
<b>CAUTION:</b> • This mode of heat removal can <u>NOT</u> be used when large RCS openings exist, the RX head is on, or RX cavity level does <u>NOT</u> exist or can <u>NOT</u> be established.
<ul> <li>Overexposure of plant personnel due to degraded RCS conditions should be considered before this lineup is performed. This lineup should not be attempted with actual or suspected fuel damage.</li> </ul>
* * * * * * * * * * * * * * * * * * * *
1 Verify full or fill the RX cavity IAW 1-OP-SI-003, FILLING THE REACTOR CAVITY.
2 Open the Fuel Transfer Tube gate valve.
3 Verify in service or place in service one SFP Cooling Pump.
4 Stop RWST Recirc Pump, 1-CS-P-2A or 1-CS-P-2B.
5 Locally stop any running SFP Purification Pump.
• 1-FC-P-3A or 1-FC-P-3B
6 Locally perform the following valve line-up:
a. Open SFP Purification Pump suction for pump to be run:
1-FC-44, 1-FC-P-3A
1-FC-43, 1-FC-P-3B
b. Open SFP Purification Pump discharge for pump to be run:
1-FC-46, 1-FC-P-3A
1-FC-49, 1-FC-P-3B
c. Open 1-FC-72, Purification Pump header isolation.
d. Close 1-FC-29, Fuel Pit IX Bypass.
e. Close 1-FC-16, Fuel Pit IX inlet.
f. Close 2-FC-73, Unit 2 RWST discharge to SFP.
g. Open 1-FC-73, Unit 1 RWST discharge to SFP.
h. Open 1-FC-74, Unit 1 RWST discharge to SFP.

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT 10
REVISION 20	COOLING THE RCS WITH SFP AND RWST COOLERS	PAGE 2 of 2

7	Locally perform valve line-up for RWST refrigeration units.
	a. 1-CS-MR-1A
	1. Open 1-CS-37, 1-CS-MR-1A inlet.
	2. Open 1-CS-40, 1-CS-MR-1A outlet.
	b. 1-CS-MR-1B
	1. Open 1-CS-41, 1-CS-MR-1B inlet.
	2. Open 1-CS-44, 1-CS-MR-1B outlet.
8	Close 1-CS-46, 1-CD-E-2A outlet.
9	Close 1-CS-47, 1-CD-E-2B outlet.
10.	$\overline{\text{IF}}$ RWST coolers (Chilled Water required) to be placed in service, $\overline{\text{THEN}}$ locally perform the following valve line-up.
	a. 1-CD-E-2A
	1. Open 1-CS-35, 1-CD-E-2A inlet.
	2. Open 1-CS-46, 1-CD-E-2A outlet.
	b. 1-CD-E-2B
	1. Open 1-CS-36, 1-CD-E-2B inlet.
	2. Open 1-CS-47, 1-CD-E-2B outlet.
11	Open 1-CS-48, RWST cooler discharge to RWST.
12	Locally start the SFP Purification Pump aligned in Step 6.
13	Monitor RWST level for increase. As RWST level increases, align any available LHSI pump flowpath to provide cavity makeup.
14	Maintain RCS heat removal. WHEN RHR restored, THEN RETURN TO procedure Step 23.

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT
REVISION 20	PROBABLE CAUSES AND REFERENCES	PAGE 1 of 2

#### 1. PROBABLE CAUSES:

- a. Loss of RCS inventory
- b. Valve mis-positioned
- c. Loss of both RHR pumps
- d. Loss of CC while on RHR

#### 2. REFERENCES:

- a. Generic Letter 88-17, Loss of Decay Heat Removal
- b. UFSAR Section 9.3
- c. 11448-FM-81A, 84A, 86A, 87A
- d. Tech Spec 3.5
- e. Background Information for WOG ARG-1, LOSS OF RHR WHILE OPERATING AT MID-LOOP CONDITIONS
- f. CTS 189, Entire procedure
- g. CTS 832, Step 1 eighth bullet
- h. CTS 982, Step 29 and Attachment 6
- i. CTS 1984, Step 2e
- j. NE Technical Report 865, Rev. 3, Background and Guidance for Ensuring Adequate Backup Decay Heat Removal Following Loss of RHR, Attachments 4, 5, 6 and 9
- k. NSA-93005, SPS, RHR OPERATION AT MID-LOOP IMPACT OF UPDATED INSTRUMENT ERRORS, Attachment 1
- I. 1-OSP-CT-214, CONTAINMENT CLOSURE FOR REDUCED OR POTENTIALLY REDUCED INVENTORY CONDITIONS
- m. 0-AP-40.00, NON-RECOVERABLE LOSS OF INSTRUMENT AIR
- n. 1-OP-SI-003, FILLING THE REACTOR CAVITY
- o. 1-AP-16.01, SHUTDOWN LOCA
- p. CTS 2746 (Steps 16 and 24)
- q. CTS 2743 (DCP 94-030-3, Core Uprate)
- r. TSCR 313, Amendment 207
- s. DCP 04-033, Setpoint change from 16% to 20%

NUMBER 1-AP-27.00	ATTACHMENT TITLE	ATTACHMENT
REVISION	PROBABLE CAUSES AND REFERENCES	PAGE
20		2 of 2

20.ET NAF 06-0088, Justification For Not Having a LHSI Flowpath From The Containment Sump At CSD.
21.SAA000911, Shutdown Cooling Defense In Depth Assessment
22.ET-NAF-08-0062, Hot Leg Versus Cold Leg Injection for Loss of Residual Heat Removal

## U.S. Nuclear Regulatory Commission Surry Power Station

## SR10301 Simulator Job Performance Measure EPEE05.EA1.1 (4.1/4.0) Alternate

Applicant	Start Time
Examiner	
Date	Stop Time

#### **Title**

ESTABLISH FEED FLOW FROM THE CONDENSATE SYSTEM IN RESPONSE TO A LOSS OF SECONDARY HEAT SINK.

K/A: EPEE05.EA1.1 Ability to operate and/or monitor the following as they apply to the (Loss of Secondary Heat Sink): Components, and functions of control and safety systems, including instrumentation, signals, interlocks, failure modes, and automatic and manual features.

Applicability Estimated Time

RO/SRO(I) 20 minutes

#### **Conditions**

- Task is to be PERFORMED in the simulator.
- Plant is shutdown following a trip.
- 1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK, is in effect at step 8.

#### **Standards**

 Establishes feed flow from the main condensate system IAW 1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK, step 8.

#### **Initiating Cues**

- 1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK, step 8.
- Shift Manager direction.

#### **Terminating Cues**

• 1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK, step 8 completed.

#### **Procedures**

1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK.

#### **Tools and Equipment**

#### **Safety Considerations**

None

None

#### **Simulator Setup**

- Recall saved IC-328 OR
- Call up 100% IC and initialize.
- Enter the following malfunctions: FW0701 (1-FW-P-3A Overcurrent Trip), FW0702 (1-FW-P-3B Overcurrent Trip), and MS16 (1-FW-P-2 Trip Throttle Valve Failed Close) with a 0 second time delay, FW1701 (1-FW-P-1A Impeller Degradation) and FW1702 (1-FW-P-1B Impeller Degradation) with a 2 second time delay 100% degradation.
- Place the simulator in run and implement the malfunctions, immediately trip to conserve SG level.
- Depressurize the RCS to 1950# while cooling the plant to <540°F, block Low Pressure and High Steam Flow SI signals, Trip the RCPs, ensure RCS temp and pressure are <u>stable</u> below the SI block setpoints. Go to Bypass Interlock on the steam dumps.
- SG levels should be about 30-50% WR when the setup is completed. If needed, delete a MFW pump degradation malfunction and feed the SGs. After feeding stop the MFP and re-insert the deleted MFW pump degradation.
- Set up the following triggers:
  - Trigger #5 Places 'A' MFP motor breakers in test (use remote functions FWP1A1\_TEST and FWP1A2 TEST and set the final value to IN TEST)
  - Trigger #7 Places 'B' MFP motor breakers in test (use remote functions FWP1B1\_TEST and FWP1B2\_TEST and set the final value to IN TEST)
- Place the simulator in freeze. Save conditions if necessary.
- Insert the following malfunctions to fail the steam dumps: MSPC464B\_RAISE to OFF, MSPC464B\_LOWER to OFF, MSPC464B MANUAL to ON.
- Make sure RCS pressure is low enough that SI will not unblock before the cooldown (depressurization of 1 SG) begine.

#### **Initiating Cues**

- 1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK, step 8.
- Shift Manager direction.

#### **Directions to the Applicant**

- You are the Unit RO and I am the Shift Manager. There is a loss of secondary heat sink in progress on Unit 1.
- Unit conditions are as follows, Unit 1 tripped due to mechanical damage to both main feed pumps, none of the aux feed pumps are working on Unit 1 and crosstie attempts from unit 2 have proven unsuccessful.
- 1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK, has been completed to step 8. RCPs are secured, low pressure and hi steam flow SI signals have been blocked.
- Here is a copy of FR-H.1 Step 8, I need you to ESTABLISH FEED FLOW FROM THE CONDENSATE SYSTEM IN RESPONSE TO A LOSS OF SECONDARY HEAT SINK in accordance with step 8.
- When you finish the actions necessary to accomplish this, please inform me.

#### **Notes**

## PERFORMANCE CHECKLIST

## Notes to the Evaluator

- Task critical elements are bolded and noted at the end of the step as CRITIAL STEP.
- An additional instructor may be needed to silence and acknowledge alarms for the examinee.
- START TIME:

STANDARD:	Flow on each Main Steamline should be kept less than 1.0 x 106 PPH to prevent Main Steamline isolation during RCS cooldown with the Steam Dumps.	SATUNSAT
COMMENTS:		
STEP 2: STEP 8a -	TRY TO ESTABLISH FEED FLOW FROM CONDENSATE SYSTEM BY DEPRESSURIZING ONE INTACT SG:  a) Select one intact SG to depressurize	SAT UNSAT
<ul> <li>Selects one SG to depressurize (A, B, or C)</li> <li>EVALUATOR'S NOTE: If asked, there is no preferred SG.</li> <li>COMMENTS:</li> </ul>		

	TRY TO ESTABLISH FEED FLOW FROM CONDENSATE SYSTEM BY DEPRESSURIZING ONE INTACT SG:  b) Close MSTVs on the non-selected SGs  s MSTVs on the 2 non-selected SGs by taking control switch to close and ng green light on and red light off.  S NOTE: N/A	SATUNSAT
STEP 4:		
Step 8c -	TRY TO ESTABLISH FEED FLOW FROM CONDENSATE SYSTEM BY DEPRESSURIZING ONE INTACT SG:  c) Align CHG pump suction to the RWST:  1) Verify open or open CHG pump suction from RWST MOVs • 1-CH-MOV-1115B • 1-CH-MOV-1115D  2) Verify closed or close CHG pump suction from VCT MOVs • 1-CH-MOV-1115C • 1-CH-MOV-1115E	SATUNSAT
ar • C	pens 1-CH-MOV-1115B and 1-CH-MOV-1115D by taking control switch to open nd verifying red light on and green light off. loses 1-CH-MOV-1115C and 1-CH-MOV-1115E by taking control switch to close nd verifying green light on and red light off.	

STEP 5:	SAT	
Step 8d -	TRY TO ESTABLISH FEED FLOW FROM CONDENSATE SYSTEM BY DEPRESSURIZING ONE INTACT SG:	UNSAT
	d) Verify closed or close letdown isolation valves  • 1-CH-LCV-1460A  • 1-CH-LCV-1460B	
STANDARD:		
verifying	1-CH-LCV-1460A and 1-CH-LCV-1460B by taking control swich to close and g green light on and red light off. letdown flow goes to zero.	
EVALUATOR'S	NOTE: N/A	
COMMENTS:		
STEP 6:		0.17
Step 8e -	TRY TO ESTABLISH FEED FLOW FROM CONDENSATE SYSTEM BY DEPRESSURIZING ONE INTACT SG:	SAT
	e) Control charging flow to maintain PRZR level	
STANDARD:		
	1-CH-FCV-1122 is controlling in automatic or places in manual to control izer level.	
EVALUATOR'S	NOTE: N/A	
COMMENTS:		
STEP 7:		CAT
Step 8f –	TRY TO ESTABLISH FEED FLOW FROM CONDENSATE SYSTEM BY DEPRESSURIZING ONE INTACT SG:	SAT
	f) Dump steam to condenser at maximum rate to depressurize one intact SG to less than 550 psig.	
STANDARD:		
	ts to use steam dumps to depressurize selected intact SG.	

EVALUATOR'S	S NOTE: N/A	
COMMENTS:		
STEP 8:		
		SAT
Step 8t RNO-	TRY TO ESTABLISH FEED FLOW FROM CONDENSATE SYSTEM BY DEPRESSURIZING ONE INTACT SG:	UNSAT
	f) Dump steam from intact SG:  • Manually use SG PORV.  OR	
	Locally use SG PORV IAW Attachment 6.	
	IF at least one SG can NOT be depressurized, THEN GO TO Step 10.	
STANDARD:		
either p	intact SG PORV and depressurizes selected intact SG to less than 550 psig by lacing SG PORV in manual and opening it OR lowering setpoint for SG PORV. ot go to step 10, goes to step 8g.	
EVALUATOR'S	S NOTE: N/A	
COMMENTS:		
STEP 9:		
Step 8g –	TRY TO ESTABLISH FEED FLOW FROM CONDENSATE SYSTEM BY DEPRESSURIZING ONE INTACT SG:	SAT
	g) Establish condensate flow:	
	1) Locally put one set of feed pump breakers in test	
	2) Close feed pump breakers in test	
	3) Open appropriate feed pump discharge MOV	
	Control flow with SG FW bypass flow valve(s) or feed REG valve(s) as necessary	
	<ol> <li>Depressurize SG as necessary to establish adequate Delta-P for feeding</li> </ol>	

STANDARD:	
<ul> <li>Directs operator to close either 'A' or 'B' MFP breakers in test.</li> <li>Once MFP breakers are reported to be closed in test, takes the control switches to close for the selected MFPs.</li> <li>Opens appropriate discharge MOV (1-FW-MOV-150A or 1-FW-MOV-150B).</li> <li>Controls FW flow to selected SG using the FW bypass flow valve for that selected SG.</li> <li>Operator may continue to depressurize SG using SG PORV.</li> <li>Verifies positive indication of SG level increase for selected SG.</li> <li>All the above are Critical Tasks.</li> </ul>	
EVALUATOR'S NOTE: N/A	
COMMENTS:	
STEP 11:	SAT
REPORT TO SHIFT SUPERVISOR (EVALUATOR).	
STANDARD:	UNSAT
Verbal status report that feed flow has been established to at least one intact SG.	
EVALUATOR'S NOTE: N/A	
COMMENTS:	
STOP TIME:	

# Operator Directions Handout (TO BE READ TO APPLICANT BY EXAMINER)

#### **Task**

- Task is to be performed in the simulator.
- Establish feed flow from the main condensate system IAW 1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK, step 8.

#### **Directions**

The evaluator will explain the initial conditions of the task to be performed and will provide the initiating cue. Ensure you indicate to the evaluator when you understand your assigned task

#### **Initial Conditions**

- Unit 1 tripped due to mechanical damage to both main feed pumps, none of the aux feed pumps are working on Unit 1 and crosstie attempts from unit 2 have proven unsuccessful.
- 1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK, has been completed to step 8. RCPs are secured, low pressure and hi steam flow SI signals have been blocked.

#### **Initiating Cues**

- You are the Unit RO and I am the Shift Manager. There is a loss of secondary heat sink in progress on Unit 1.
- Unit conditions are as follows, Unit 1 tripped due to mechanical damage to both main feed pumps, none of the aux feed pumps are working on Unit 1 and crosstie attempts from unit 2 have proven unsuccessful.
- 1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK, has been completed to step 8. RCPs are secured, low pressure and hi steam flow SI signals have been blocked.
- Here is a copy of FR-H.1 Step 8, I need you to ESTABLISH FEED FLOW FROM THE CONDENSATE SYSTEM IN RESPONSE TO A LOSS OF SECONDARY HEAT SINK in accordance with step 8.
- When you finish the actions necessary to accomplish this, please inform me.

## Operator Directions Handout (TO BE GIVEN TO APPLICANT)

#### **Initial Conditions**

- Unit 1 tripped due to mechanical damage to both main feed pumps, none of the aux feed pumps are working on Unit 1 and crosstie attempts from unit 2 have proven unsuccessful.
- 1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK, has been completed to step 8. RCPs are secured, low pressure and hi steam flow SI signals have been blocked.

#### **Initiating Cues**

- You are the Unit RO and I am the Shift Manager. There is a loss of secondary heat sink in progress on Unit 1.
- Unit conditions are as follows, Unit 1 tripped due to mechanical damage to both main feed pumps, none of the aux feed pumps are working on Unit 1 and crosstie attempts from unit 2 have proven unsuccessful.
- 1-FR-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK, has been completed to step 8. RCPs are secured, low pressure and hi steam flow SI signals have been blocked.
- Here is a copy of FR-H.1 Step 8, I need you to ESTABLISH FEED FLOW FROM THE CONDENSATE SYSTEM IN RESPONSE TO A LOSS OF SECONDARY HEAT SINK in accordance with step 8.
- When you finish the actions necessary to accomplish this, please inform me.

NUMBER	PROCEDURE TITLE	REVISION 28
1-FR-H.1	RESPONSE TO LOSS OF SECONDARY HEAT SINK	PAGE 8 of 22

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
NOTE	E: Flow on each Main Steamline should be ke isolation during RCS cooldown with the Ste	kept less than 1.0 x 10 <sup>6</sup> PPH to prevent Main Steamline team Dumps.
8	TRY TO ESTABLISH FEED FLOW FROM CONDENSATE SYSTEM BY DEPRESSURIZING ONE INTACT SG:	
	a) Select one intact SG to depressurize	
	b) Close MSTVs on the non-selected SGs	
	c) Align CHG pump suction to the RWST:	
	Verify open or open CHG pump suction from RWST MOVs	
	<ul><li>1-CH-MOV-1115B</li><li>1-CH-MOV-1115D</li></ul>	
	Verify closed or close CHG pump suction from VCT MOVs	
	<ul><li>1-CH-MOV-1115C</li><li>1-CH-MOV-1115E</li></ul>	
	d) Verify closed or close letdown isolation valves	
	<ul><li>1-CH-LCV-1460A</li><li>1-CH-LCV-1460B</li></ul>	
	e) Control charging flow to maintain PRZR level	
(STEP 8 C	CONTINUED ON NEXT PAGE)	

NUMBER	DECRONOS TO LOCA OF OF OCNIDARY LIFAT CINIC	REVISION 28
1-FR-H.1	RESPONSE TO LOSS OF SECONDART HEAT SINK	PAGE 9 of 22

COTED	ACTION/EXPECTED DECDONOR		DESPONSE NOT OBTAINED
STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
8.	TRY TO ESTABLISH FEED FLOW FROM CONDENSATE SYSTEM BY DEPRESSURIZING ONE INTACT SG: (Continued)		
	f) Dump steam to condenser at maximum		f) Dump steam from intact SG:
	rate to depressurize one intact SG to lea	ss 🗆	Manually use SG PORV. OR
			Locally use SG PORV IAW     Attachment 6.
			<u>IF</u> at least one SG can <u>NOT</u> be depressurized, <u>THEN</u> GO TO Step 10.
	g) Establish condensate flow:		g) GO TO Step 10.
	<ol> <li>Locally put one set of feed pump breakers in test</li> </ol>		
	2) Close feed pump breakers in test		
	<ol> <li>Open appropriate feed pump discharge MOV</li> </ol>		
	<ol> <li>Control flow with SG FW bypass flow valve(s) or feed REG valve(s) as necessary</li> </ol>		
	5) Depressurize SG as necessary to establish adequate $\Delta P$ for feeding		
9	CHECK SG LEVELS:		
	a) Check narrow range level in at least one SG - GREATER THAN 12% [18%]	e 🗆	a) IF feed flow to at least one SG verified,  THEN maintain flow to restore narrow range level to greater than 12% [18%]. IF flow NOT verified, THEN GO TO Step 10.
	b) RETURN TO procedure and step in effe	ect	

# U.S. Nuclear Regulatory Commission Surry Power Station

## SR10301 Simulator Job Performance Measure 026.A4.01 (4.5/4.3) Alternate Path

Applicant	Start Tir	me	
Examiner			
Date	Stop Tir	me	
<u>Title</u>			
RESPOND TO A SPURIOUS CS ACTUATION.			
K/A: 026.A4. 01 Ability to manually operate and/or monitor in the control room: CSS Controls.			
<u>Applicability</u>	Estimated Time	Actual Time	
RO/SRO(I)	12 minutes		

#### **Conditions**

- Task is to be PERFORMED in the simulator.
- · Spurious CS actuation has occurred at HSD.

#### **Standards**

- Applicant recognizes spurious Hi Hi CLS signal.
- Applicant secures 1-CS-P-1A, closes discharge MOVs 1-CS-MOV-101A, B from MCR.
- Applicant directs actions to secure 1-CS-P-1B, close discharge MOVs 1-CS-MOV-101C, D from field / MCR.

#### **Initiating Cues**

- 1B-A6/B6, CTMT PART PRESS -0.1 PSI CH 1 / 2 remain locked in.
- Shift Manager direction.

## **Terminating Cues**

• 1B-A6 Attachment 1, steps 1-6 complete.

#### **Procedures**

• 1B-A6, CTMT PART PRESS -0.1 PSI CH 1.

#### **Tools and Equipment**

#### **Safety Considerations**

None

None

#### **Simulator Setup**

- Recall IC-326 OR
- Call up HSD IC and initialize.
- Start 1-CS-P-1A/1B, freeze simulator, green flag both pumps and then place simulator in run.
- Open 1-CS-MOV-101A-D.
- Implement the following overrides to prevent 1-CS-P-1B from being secured from MCR and 1-CS-MOV-101C/D from closing with control switch: MOVCS101C\_OPEN & MOVCS101D\_OPEN to ON, CSP1B\_STOP & CSP1B\_LOCK to OFF.
- Place the following Remotes on triggers: CSMOV101C & CSMOV101D to rack-out, CSMOV101C & CSMOV101D manual valves to 0% (closed).

#### **Initiating Cues**

- 1B-A6/B6, CTMT PART PRESS -0.1 PSI CH 1 / 2 remain locked in.
- Shift Manager direction.

#### **Directions to the Applicant**

- I am the Shift Manager and you are the Unit RO.
- Unit 1 is at Hot Shutdown.
- There are NO personnel in Unit 1 containment.
- An inadvertent CS actuation occurred and we have received annunciators 1B-A6 and 1B-B6.
- Annunciator response 1B-A6 has been initiated.
- Here's a copy of annunciator response 1B-A6. I need you to perform attachment 1 of ARP 1B-A6, steps 1-6 only.
- When you finish the actions necessary to complete attachment 1, please inform me.

#### <u>Notes</u>

## PERFORMANCE CHECKLIST

## Notes to the Evaluator

- Task critical elements are bolded and noted at the end of the step as CRITICAL STEP.
- An additional instructor may be needed to silence and acknowledge alarms for the examinee.
- START TIME:

STANDARD:	es annunciators 1B-C4 and 1B-C5 are not lit.	SATUNSAT
STEP 2:		SAT
STEP 2 -	Place control switch(es) for running CS pump(s) in PTL.	UNSAT
•	1-CS-P-1A	ONSAT
•	1-CS-P-1B	
STANDARD:		
Critical T  Places co	ontrol switch 1-CS-P-1A in PTL and verifies zero amps. This is a lask. Introl switch for 1-CS-P-1B in PTL. Introl switch for 1-CS-P-1B in PTL. Introl switch for 1-CS-P-1B in PTL. In the second switch for 1-CS-P-1B in PTL. In the second switch for 1-CS-P-1B in PTL. In the second switch 1-CS-P-1B in PTL and verifies zero amps. This is a last second switch 1-CS-P-1B in PTL. In the second switch 1-CS-P-1B in PTL and verifies zero amps. This is a last switch 1-CS-P-1B in PTL. In the second switch 1-CS-P-1B in PTL and verifies zero amps. This is a last switch 1-CS-P-1B in PTL. In the second switch 1-CS	
EVALUATOR'S N		
Booth Operator:	When called to locally open 1-EP-BKR-14J-5, implement trigger 9 and inform the candidate that a time compression has occurred and 1-EP-BKR-14J-5 is open.	
COMMENTS:		

STEP 3:		SAT
Step 3 –	IF CS pump(s) will NOT stop from the MCR, THEN locally open supply breaker(s):	UNSAT
	• 1-CS-P-1A, 14H-5	
	• 1-CS-P-1B, 14J-5	
STANDARD:		
	o Operator to locally open supply breaker 1-CS-P-1B supply breaker 14J-5. a Critical Task.	
	local operator reports 14J-5 is open, recognizes breaker for 1-CS-P-1B is red light off). This is a Critical Task.	
EVALUATOR'S	S NOTE: N/A	
BOOTH OPER	ATOR'S NOTE: When directed to locally open 14J-5, inform the candidate that a time compression has occurred and 14J-5 is open.	
COMMENTS:		
STEP 4:		SAT
Note prior to S	itep 4	UNSAT
NOTE:	The CS Pump Discharge MOVs will not close if the HI HI CLS signal is present.	UNSAT
STANDARD:		
• Acl	knowledges note.	
EVALUATOR'S	S NOTE: N/A	
COMMENTS:		

STEP 5:		SAT
Step 4 -	Verify closed or close CS PUMP DISCH MOV(s):	UNSAT
	• 1-CS-MOV-101A	ONOAT
	• 1-CS-MOV-101B	
	• 1-CS-MOV-101C	
	• 1-CS-MOV-101D	
STANDAR	D:	
•	Closes 1-CS-MOV-101A (red light off, green light on). This is a Critical Task.	
•	Closes 1-CS-MOV-101B (red light off, green light on). This is a Critical Task.	
•	Closes 1-CS-MOV-101C (red light off, green light on) Recognizes 1-CS-MOV-101C did NOT close	
•	Closes 1-CS-MOV-101D (red light off, green light on) Recognizes 1-CS-MOV-101D did NOT close	
EVALUAT	OR'S NOTE: N/A	
COMMEN	TS:	
STEP 6:		CAT
Step 5 -	IF CS PUMP DISCH MOV(s) will NOT close, THEN do the following:	SAT
	<ul> <li>a. Send an Operator to Unit 1 Cable Vault and an Operator to Unit 1 Safeguards.</li> </ul>	UNSAT
	<ul> <li>b. Direct Operator in Unit 1 Cable Vault to open MOV supply breaker(s) for MOV(s) which will not close.</li> </ul>	
	<ul> <li>1-CS-MOV-101A, 1H1-2S-1A</li> <li>1-CS-MOV-101B, 1J1-2W-5A</li> <li>1-CS-MOV-101C, 1H1-2S-2B</li> <li>1-CS-MOV-101D, 1J1-2W-4A</li> </ul>	
	c. WHEN breaker(s) open, THEN have Operator in Safeguards locally close	

STANDARD:	
<ul> <li>Directs Operator to locally open breaker for 1-CS-MOV-101C, 1H1-2S-2B.</li> <li>This is a Critical Task.</li> </ul>	
<ul> <li>Directs Operator to locally open breaker for 1-CS-MOV-101D, 1J1-2W-4A.</li> <li>This is a Critical Task.</li> </ul>	
<ul> <li>Once breakers reported open, directs local closure of 1-CS-MOV-101C and 1-CS-MOV-101D.</li> </ul>	
EVALUATOR'S NOTE: N/A	
<b>Booth Operator:</b> When called to locally open 1-EP-BKR-1H1-2S-2B & 1-EP-BKR-1J1-2W-4A, implement triggers 3 & 5 and inform the candidate that a time compression has occurred and 1-EP-BKR-1H1-2S-2B & 1-EP-BKR-1J1-2W-4A are open.	
<b>Booth Operator:</b> When called to locally close 1-CS-MOV-101C & 1-CS-MOV-101D, implement trigger 7 and inform the candidate that a time compression has occurred and 1-CS-MOV-101C & 1-CS-MOV-101D are closed.	
COMMENTS:	
STEP 7:	SAT
Step 6 – Verify closed or close CHEM ADD TK OUTLET MOVs:	UNSAT
• 1-CS-MOV-102A	
• 1-CS-MOV-102B	
STANDARD:	
<ul> <li>Verifies 1-CS-MOV-102A and 1-CS-MOV-102B closed by observing red light off and green light on.</li> </ul>	
EVALUATOR'S NOTE: N/A	
COMMENTS:	

STEP 8:	SAT
REPORT TO SHIFT SUPERVISOR (EVALUATOR).	
STANDARD:	UNSAT
Verbal status report that Attachment 1 of Annunciator response 1B-A6, steps 1-6 are complete.	
EVALUATOR'S NOTE: N/A	
COMMENTS:	
STOP TIME:	

# Operator Directions Handout (TO BE READ TO APPLICANT BY EXAMINER)

#### **Task**

- Task is to be performed in the simulator.
- Perform Attachment 1, steps 1-6, of ARP 1B-A6.

#### **Directions**

The evaluator will explain the initial conditions of the task to be performed and will provide the initiating cue. Ensure you indicate to the evaluator when you understand your assigned task

#### **Initial Conditions**

Spurious CS actuation has occurred at HSD.

#### **Initiating Cues**

- I am the Shift Manager and you are the Unit RO.
- Unit 1 is at Hot Shutdown.
- There are NO personnel in Unit 1 containment.
- An inadvertent CS actuation occurred and we have received annunciators 1B-A6 and 1B-B6.
- Here's a copy of annunciator response 1B-A6. I need you to perform attachment 1 of ARP 1B-A6, steps 1-6 only.
- When you finish the actions necessary to complete attachment 1, please inform me.

## Operator Directions Handout (TO BE GIVEN TO APPLICANT)

### **Initial Conditions**

Spurious CS actuation has occurred at HSD.

- I am the Shift Manager and you are the Unit RO.
- Unit 1 is at Hot Shutdown.
- There are NO personnel in Unit 1 containment.
- An inadvertent CS actuation occurred and we have received annunciators 1B-A6 and 1B-B6.
- Here's a copy of annunciator response 1B-A6. I need you to perform attachment 1 of ARP 1B-A6, steps 1-6 only.
- When you finish the actions necessary to complete attachment 1, please inform me.

NUMBER 1B-A6	ATTACHMENT TITLE	ATTACHMENT 1
REVISION 8	SPURIOUS CS ACTUATION - CONTINGENCY ACTIONS	PAGE 1 of 2

1 Verify reset or reset HI HI CLS signal.
2 Place control switch(es) for running CS pump(s) in PTL.
□ • 1-CS-P-1A
□ • 1-CS-P-1B
3 IF_CS pump(s) will NOT stop from the MCR, THEN locally open supply breaker(s):
□ • 1-CS-P-1A, 14H-5
□ • 1-CS-P-1B, 14J-5
NOTE: The CS Pump Discharge MOVs will not close if the HI HI CLS signal is present.
4 Verify closed or close CS PUMP DISCH MOV(s):
□ • 1-CS-MOV-101A
□ • 1-CS-MOV-101B
□ • 1-CS-MOV-101C
□ • 1-CS-MOV-101D
5 IF CS PUMP DISCH MOV(s) will NOT close, THEN do the following:
a. Send an Operator to Unit 1 Cable Vault and an Operator to Unit 1 Safeguards.
b. Direct Operator in Unit 1 Cable Vault to open MOV supply breaker(s) for MOV(s) which will not close.
□ • 1-CS-MOV-101A, 1H1-2S-1A
□ • 1-CS-MOV-101B, 1J1-2W-5A
□ • 1-CS-MOV-101C, 1H1-2S-2B
□ • 1-CS-MOV-101D, 1J1-2W-4A
c. <u>WHEN</u> breaker(s) open, <u>THEN</u> have Operator in Safeguards locally close affected MOV(s).

NUMBER 1B-A6	ATTACHMENT TITLE	ATTACHMENT 1
REVISION 8	SPURIOUS CS ACTUATION - CONTINGENCY ACTIONS	PAGE 2 of 2

6 Verify closed or close CHEM ADD TK OUTLET MOVs:  □ • 1-CS-MOV-102A  □ • 1-CS-MOV-102B
7 Consult with Station Management on desired course of action.
8 WHEN both of the following conditions are met, THEN return CS pump control switches to AUTO.
<ul> <li>The cause of the spurious actuation has been corrected</li> </ul>
□ • Station Management permission has been granted
9 Review Tech Spec 3.8, Containment.
10 Increase surveillance of RCP parameters.
11 RETURN TO procedure Step 10.

# U.S. Nuclear Regulatory Commission Surry Power Station

### SR10301 Simulator Job Performance Measure 064.A2.05 (3.1/3.2) Alternate

Applicant	Start Time
Examiner	
Date	Stop Time

#### **Title**

#### ALIGN THE EMERGENCY BUS IN ACCORDANCE WITH AP-10.07 ATTACHMENT 4

K/A: 064.A2.05 Ability to (a) predict the impacts of the following malfunctions or operations on the ED/G system; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Loading the ED/G.

Applicability Estimated Time Actual Time

RO/SRO(I) 15 minutes

#### **Conditions**

- Task is to be PERFORMED in the simulator.
- 1-CH-P-1B is in service. 1-CH-P-1A and 1C are in AUTO. 1-VS-F-58A is in service on the fuel building. Breaker
   15H8 spuriously opened and #1 EDG is supplying Unit 1 H Bus Loads.

#### **Standards**

1-AP-10.07, LOSS OF UNIT 1 POWER, Attachment 4 completed.

#### **Initiating Cues**

- 1-AP-10.07, LOSS OF UNIT 1 POWER, Step 41.
- Shift Manager Direction.

#### **Terminating Cues**

Attachment 4 of 1-AP-10.07, LOSS OF UNIT 1 POWER, completed.

#### **Procedures**

Attachment 4 of 1-AP-10.07, LOSS OF UNIT 1 POWER.

#### **Tools and Equipment**

#### **Safety Considerations**

None

None

#### **Simulator Setup**

- Recall IC-325 OR
- Call up 100% power IC & initialize. Verify that 1-CH-P-1B is running and 1-CH-P-1A and 1-CH-P-1C are in auto after stop.
- Place the 58A fan in service on the fuel building.
- Implement the following malfunction to cause breaker 15H8 to open: EL4901
- Implement the following override to automatically actuate when 1-CH-P-1A or 1-CH-P-1C is started: EDG1\_MW = 0.32 (this will cause the #1 EDG MWs to read approximately 1600kW).
- Reset the stub bus and start 1-CC-P-1A.
- Perform 1-AP-10.07 up to step 41 and place the simulator in freeze to await JPM performance.
- Place an orange magnet on 15H8.
- RED FLAG 1-VS-F-58A.

#### **Initiating Cues**

- 1-AP-10.07, Step 41.
- Shift Manager Direction.

#### **Directions to the Applicant**

- I am the Shift Manager and you are the Unit RO. 1-VS-F-58A is in service on the fuel building at the request of the fuel handlers in preparations for upcoming activities in the fuel building.
- 1-EP-BKR-15H8 spuriously opened and we are performing 1-AP-10.07, LOSS OF UNIT 1 POWER. #1 EDG is supplying the 1H Bus solely.
- Here is a copy of Attachment 4 of 1-AP-10.07 (EMERGENCY BUS LOAD ALIGNMENT). I need you to perform this attachment in its entirety.
- Inform the Shift Manager prior to performing any component manipulations.
- When you finish the actions necessary to accomplish this, please inform me.

#### **Notes**

## PERFORMANCE CHECKLIST

## Notes to the Evaluator

- Task critical elements are bolded and noted at the end of the step as CRITICAL STEP.
- An additional instructor may be needed to silence and acknowledge alarms for the examinee.
- START TIME:

STEP 1:

CALITIONS AND NOT	TO DOUGD TO STED 4	SAT
CAUTIONS AND NOTE	ES PRIOR TO STEP 1	UNSAT
CAUTION:	An EDG can NOT initially be loaded to greater than 1300 KW prior to an SI or HI HI CLS, with the exception of loads that will trip when a HI HI CLS occurs, or loads which are running yet assumed to start on a HI HI CLS.	
CAUTION:	Total EDG load shall not exceed 2750 KW.	
NOTE:	The purpose of this Attachment is to limit EDG loading so that a subsequent HI HI CLS actuation will not overload the EDG.	
	EDG load must be monitored closely during pump operations.	
	One CHG pump must be running on each Emergency Bus solely supplied from an EDG. The other CHG pump supplied from that Emergency Bus must remain in PTL.	
	A 6/30 hour TS 3.0.1 LCO is in effect until one CHG pump is running on an Emergency Bus supplied by an EDG, AND the non-running CHG pump is in PTL. This Tech LCO is entered due to an inoperable Emergency Bus due to EDG loading.	
STANDARD:		
•	ges cautions and notes. ate may inform the SRO of the tech spec clock.	
COMMENTS:		

STEP 2:		SAT
STEP 1a -	IF EDG 1 is the sole source of power to Emergency Bus 1H, THEN do the following to limit the amount of instantaneous loading that could occur in the event of a HI HI CLS.	UNSAT
	a. Verify running or start 1-CH-P-1A or 1-CH-P-1C on Emergency Bus 1H IAW Shift Supervision direction.	
STANDARD:		
	1-CH-P-1A or 1-CH-P-1C by placing control switch to start and observing ndicated. This is a Critical Task.	
EVALUATOR'S	NOTE: If asked, you have no preference as to which charging pump is started.	
COMMENTS:		
STEP 3:		SAT
Step 1b -	<ul> <li>b. IF a CHG pump can NOT be started on Emergency Bus 1H, THEN do the following. Otherwise, enter N/A.</li> <li> 1. Verify running or start a CHG pump on Emergency Bus 1J.</li> <li> 2. Place CHG pumps supplied from Emergency Bus 1H in PTL. (A supplied by Emergency Bus 1H are in PTL)</li> <li> 3. GO TO Step 2.</li> </ul>	UNSAT
STANDARD:		
<ul> <li>N/A ste</li> </ul>	p 1b.	
EVALUATOR'S	S NOTE: N/A	
COMMENTS:		
STEP 4:		SAT
Step 1c -		UNSAT
	<ul> <li>c. Verify or place the remaining CHG pump supplied from Emergency Bus 1H in PTL. (1-CH-P-1C must remain running if pump is supplied by Emergency Bus 1J)</li> </ul>	

STANDARD:				
<ul> <li>Places non-running Critical Task.</li> </ul>	charging pum	p (1-CH-P-1A or 1-CH-	P-1C) in PTL. This is a	
EVALUATOR'S NOTE: If in	formed, acknowl	edge that the 6/30 hr TS	S clock can be stopped.	
COMMENTS:				
STEP 5:				
				SAT
Step 1d -				UNSAT
		upplied by offsite powe -P-1B and place in Auto		
STANDARD:				
<ul><li>Secures 1-CH-P-1B I</li><li>Observes zero amps</li></ul>		switch to stop and retur	ning to auto.	
EVALUATOR'S NOTE: N/A				
COMMENTS:				
Oommervio.				
STEP 6:				SAT
Step 1e -				
e Determine	Emergency Bu	s 1H load limit IAW the	following equation	UNSAT
where (x)			RUNNING equipment	
•		00 KW =	129	
	(x)	EDG Load	LIMIT	
	KW	LOAD		
	100	1-VS-F-1A		
	125	1-VS-F-58A (if		
		supplied by 14H1-1)		
	310	1-FW-P-3A		

STANDARD:	
<ul> <li>Identifies that 1-VS-F-1A &amp; 1-VS-F-58A are in service and 1-FW-P-3A is not running.</li> <li>Places 225 kW on the line in step 1e above the letter (x).</li> <li>Determines that the load limit is 1525 kW.</li> <li>Records load limit in step 1e.</li> </ul>	
EVALUATOR'S NOTE: N/A	
COMMENTS:	
STEP 7:	
Step 1f –	SAT
f. IF existing EDG load is greater than the limit calculated in substep e. above, THEN reduce EDG load to less than or equal to the calculated load.	UNSAT
STANDARD:	
<ul> <li>Identifies that the current EDG loading is approximately 1600 kW. This exceeds the current EDG load limit.</li> <li>Informs the SM that the EDG is exceeding its load limit and the procedure states we should reduce load. This is a Critical Task.</li> </ul>	
<b>EVALUATOR'S NOTE:</b> If asked about EDG loading: Ask what the candidate recommends to reduce. Acknowledge and inform the candidate that you understand the EDG is exceeding its load limit and will have another operator investigate what loads can be secured to reduce EDG loading; please continue this attachment.	
COMMENTS:	
STEP 8:	SAT
Step 2 – IF EDG 3 is the sole source of power to Emergency Bus 1J, <u>THEN</u> do the following to limit the amount of instantaneous loading that could occur in the event of a HI HI CLS.	UNSAT
STANDARD:	
N/A's step 2.	
EVALUATOR'S NOTE: N/A	

COMMENTS:		
STEP 9: Step 3 –	Notify Auxiliary Building Operator to increase monitoring of CHG pump lube oil temperature.	SAT
	ets the Auxiliary Building Operator and directs him to increase monitoring of arging Pump Bearing Temperatures.  S NOTE: N/A	
COMMENTS:		
STEP 10: Step 4 –	Increasing monitoring of CHG pump bearing temperatures using the Plant Computer system.	SAT
CHAR( require		
EVALUATOR'S COMMENTS:	S NOTE: N/A	
STEP 11: REPORT TO S	HIFT SUPERVISOR (EVALUATOR).	SAT
STANDARD:		
Verbal status re	eport that Attachment 4 of 1-AP-10.07 is complete.	
EVALUATOR'S	S NOTE: N/A	
COMMENTS:		

STOP TIME:			

## Operator Directions Handout (TO BE READ TO APPLICANT BY EXAMINER)

#### **Task**

- Task is to be performed in the simulator.
- Perform Attachment 4 of 1-AP-10.07, LOSS OF UNIT 1 POWER.

#### **Directions**

The evaluator will explain the initial conditions of the task to be performed and will provide the initiating cue. Ensure you indicate to the evaluator when you understand your assigned task

#### **Initial Conditions**

 1-CH-P-1B is in service. 1-CH-P-1A and 1C are in AUTO. 1-VS-F-58A is in service on the fuel building. Breaker 15H8 spuriously opened and #1 EDG is supplying Unit 1 H Bus Loads.

- I am the Shift Manager and you are the Unit RO. 1-VS-F-58A is in service on the fuel building at the request of the fuel handlers in preparations for upcoming activities in the fuel building.
- 1-EP-BKR-15H8 spuriously opened and we are performing 1-AP-10.07, LOSS OF UNIT 1 POWER. #1 EDG is supplying the 1H Bus solely.
- Here is a copy of Attachment 4 of 1-AP-10.07 (EMERGENCY BUS LOAD ALIGNMENT). I need you to perform
  this attachment in its entirety.
- Inform the Shift Manager prior to performing any component manipulations.
- When you finish the actions necessary to accomplish this, please inform me.

## Operator Directions Handout (TO BE GIVEN TO APPLICANT)

#### **Initial Conditions**

• 1-CH-P-1B is in service. 1-CH-P-1A and 1C are in AUTO. 1-VS-F-58A is in service on the fuel building. Breaker 15H8 spuriously opened and #1 EDG is supplying Unit 1 H Bus Loads.

- I am the Shift Manager and you are the Unit RO. 1-VS-F-58A is in service on the fuel building at the request of the fuel handlers in preparations for upcoming activities in the fuel building.
- 1-EP-BKR-15H8 spuriously opened and we are performing 1-AP-10.07, LOSS OF UNIT 1 POWER. #1 EDG is supplying the 1H Bus solely.
- Here is a copy of Attachment 4 of 1-AP-10.07 (EMERGENCY BUS LOAD ALIGNMENT). I need you to perform this attachment in its entirety.
- Inform the Shift Manager prior to performing any component manipulations.
- When you finish the actions necessary to accomplish this, please inform me.

NUMBER 1-AP-10.07	ATTACHMENT TITLE	ATTACHMENT 4
REVISION 55	EMERGENCY BUS LOAD ALIGNMENT	PAGE 1 of 3

* * * * * * * * * * * * * * * * * * * *
<b>CAUTION:</b> An EDG can NOT initially be loaded to greater than 1300 KW prior to an SI or HI HI CLS, with the exception of loads that will trip when a HI HI CLS occurs, or loads which are running yet assumed to start on a HI HI CLS.
* * * * * * * * * * * * * * * * * * * *
* * * * * * * * * * * * * * * * * * * *
CAUTION: Total EDG load shall not exceed 2750 KW.
* * * * * * * * * * * * * * * * * * * *
NOTE: • The purpose of this Attachment is to limit EDG loading so that a subsequent HI HI CLS actuation will not overload the EDG.
<ul> <li>EDG load must be monitored closely during pump operations.</li> </ul>
<ul> <li>One CHG pump must be running on each Emergency Bus solely supplied from an EDG.</li> <li>The other CHG pump supplied from that Emergency Bus must remain in PTL.</li> </ul>
<ul> <li>A 6/30 hour TS 3.0.1 LCO is in effect until <u>one</u> CHG pump is running on an Emergency Bus supplied by an EDG, <u>AND</u> the non-running CHG pump is in PTL. This Tech LCO is entered due to an inoperable Emergency Bus due to EDG loading.</li> </ul>
<ol> <li>IF EDG 1 is the sole source of power to Emergency Bus 1H, <u>THEN</u> do the following to limit the amount of instantaneous loading that could occur in the event of a HI HI CLS.</li> </ol>
a. Verify running or start 1-CH-P-1A or 1-CH-P-1C on Emergency Bus 1H IAW Shift Supervision direction.
b. <u>IF</u> a CHG pump can NOT be started on Emergency Bus 1H, <u>THEN</u> do the following. Otherwise, enter N/A.
1. Verify running or start a CHG pump on Emergency Bus 1J.
2. Place CHG pumps supplied from Emergency Bus 1H in PTL. (A 6/30 hour Tech Spec 3.0.1 LCO is in effect until CHG pumps supplied by Emergency Bus 1H are in PTL)
3. GO TO Step 2.
c. Verify or place the remaining CHG pump supplied from Emergency Bus 1H in PTL. (1-CH-P-1C must remain running if pump is supplied by Emergency Bus 1J)
d. <u>IF</u> Emergency Bus 1J is supplied by offsite power, and 1-CH-P-1B is running, <u>THEN</u> stop 1-CH-P-1B and place in Auto.

NUMBER 1-AP-10.07	ATTACHMENT TITLE	ATTACHMENT 4
REVISION 55	EMERGENCY BUS LOAD ALIGNMENT	PAGE 2 of 3

		-		ollowing equation, where (x) equals the trom the table below.		
	•+ 1300	) KW =				
	(x)	E	EDG Load Limit			
		ĸw	LOAD			
		100	1-VS-F-1A			
		125	1-VS-F-58A (if supplied by 14H1-1)			
		310	1-FW-P-3A			
	EDG load to less tha	n or equal t	to the calculated lo			
NO		the non-ru	inning CHG pump	ump is running on an Emergency Bus is in PTL. This Tech LCO is entered ling.		
2.	IF EDG 3 is the sole source amount of instantaneous loa		T .	1J, <u>THEN</u> do the following to limit the event of a HI HI CLS.		
	a. Verify running or start 1-CH-P-1B or 1-CH-P-1C on Emergency Bus 1J IAW Shift Supervision direction.					
	<ul> <li>b. <u>IF</u> a CHG pump can NOT be started on Emergency Bus 1J, <u>THEN</u> do the following.</li> <li>Otherwise, enter N/A.</li> </ul>					
	1. Verify running or start a CHG pump on Emergency Bus 1H.					
2. Place CHG pumps supplied from Emergency Bus 1J in PTL. (A 6/30 hour Tech Spec 3.0.1 LCO is in effect until CHG pumps supplied by Emergency Bus 1J are in PTL)						
	3. GO TO Step 3.					
	c. Verify or place the re	maining CH	IG pump supplied	from Emergency Bus 1J in PTL.		
d. <u>IF</u> Emergency Bus 1H is supplied by offsite power, and 1-CH-P-1A or 1-CH-P-1C is running, <u>THEN</u> stop CHG pump(s) supplied from Emergency Bus 1H and place in Auto.						

NUMBER 1-AP-10.07	ATTACHMENT TITLE	ATTACHMENT 4
REVISION 55	EMERGENCY BUS LOAD ALIGNMENT	PAGE 3 of 3

55					3 of 3
e.	_	-		llowing equation, wher from the table below.	e (x) equals the
	•+ 130	0 KW =	DG Load Limit		
		KW	LOAD		

KW	LOAD
100	1-VS-F-1B
125	1-VS-F-58B (if supplied by 14J1-3)
310	1-FW-P-3B

- \_\_\_\_ f. <u>IF</u> existing EDG load is greater than the limit calculated in substep e. above, <u>THEN</u> reduce EDG load to less than or equal to the calculated load.
- 3. Notify Auxiliary Building Operator to increase monitoring of CHG pump lube oil temperature.
- 4. Increasing monitoring of CHG pump bearing temperatures using the Plant Computer system.

# U.S. Nuclear Regulatory Commission Surry Power Station

### SR10301 Simulator Job Performance Measure 071.A3.03 (3.6/3.8) Alternate

Applicant	Start Time
Examiner	
Date	Stop Time

### <u>Title</u>

#### WASTE GAS DECAY TANK RELEASE ISOLATION.

K/A: 071.A3.03 Ability to monitor automatic operation of the Waste Gas Disposal System including: Radiation monitoring system alarm and actuating signals.

 Applicability
 Estimated Time
 Actual Time

 RO/SRO(I)
 13 minutes

#### **Conditions**

- Task is to be PERFORMED in the simulator.
- Release of 'B' Waste Gas Decay Tank is in progress. The team is making preparations to run #1 EDG for its monthly performance test.

#### **Standards**

Release isolated. 1-GW-FCV-101 closed, 1-GW-FCV-160 closed, and 2-GW-FCV-260 closed IAW RMA-C6 and/or RMA-C7.

#### **Initiating Cues**

- Annunciators RMA-C6, PROCESS VENT PART ALERT/HI, and/or 0-RMA-C7, PROCESS VENT NORMAL RANGE GAS ALERT/HI.
- Shift Manager direction.

#### **Terminating Cues**

• Steps 1-5 of RMA-C6 and/or RMA-C7 completed.

#### **Procedures**

 0-RMA-C6, PROCESS VENT PART ALERT/HI, and/or 0-RMA-C7, PROCESS VENT NORMAL RANGE GAS ALERT/HI.

#### **Tools and Equipment**

#### **Safety Considerations**

None

None

#### **Simulator Setup**

- Recall IC-329 OR
- Call up 100% power IC and initialize.
- Enter malfunctions for Disable AUTO CLOSURE for 1-GW-FCV-101, 1-GW-FCV-160, 2-GW-FCV-260. Enter
  overrides for Process Vent Particulate and Gas RMs: RM0601 to 40% and RM0602 to 60% and place on trigger
  1.
- Place simulator in run and implement trigger 1 when ready to perform JPM.
- HAVE A SIGNED OFF COPY OF OP-23.2.4, RELEASE OF WASTE GAS DECAY TANK 1B, AVAILABLE IN CASE THE RO ASKS TO REVIEW/LOOK AT PROCEDURE.

#### **Initiating Cues**

- Annunciators RMA-C6, PROCESS VENT PART ALERT/HI, and/or 0-RMA-C7, PROCESS VENT NORMAL RANGE GAS ALERT/HI.
- Shift Manager direction.

#### **Directions to the Applicant**

- I am the Shift Manager. You are the 3<sup>rd</sup> RO. The unit is at 100% power and stable. 'B" Waste Gas Decay Tank release is in progress and we are currently making preparations to run #1 EDG for its monthly surveillance test.
- An event is about to occur and I need you to respond IAW applicable procedures.

#### **Notes**

## PERFORMANCE CHECKLIST

## Notes to the Evaluator

- Task critical elements are bolded and noted at the end of the step as CRITICAL STEP.
- An additional instructor may be needed to silence and acknowledge alarms for the examinee.
- START TIME:

NORMAL RAN	RMA-C6, PROCESS VENT PART ALERT/HI, and RMA-C7, PROCESS VENT NGE GAS ALERT/HI will alarm. Performing either ARP will satisfy the JPM. Steps 1-6 of either ARP are identical.	
STEP 1:		
NOTES PRIOR	TO STEP 1	SAT UNSAT
NOTE:	The MGP Process Vent channel sensing high radiation will buzz with an audible alarm. Pressing the blue SEL key on the display unit will silence the alarm.	UNSAT
	This annunciator does NOT have reflash capability. This MGP remote display unit indicating high radiation will buzz with an audible alarm.	
STANDARD:		
• Acl	knowledges notes.	
EVALUATOR'S	S NOTE: N/A	
COMMENTS:		
STEP 2:		CAT
STEP 1 -	VERIFY ALARM - READING ON MONITOR OR CHART RECORDER GREATER THAN OR EQUAL TO ALERT SETPOINT	SAT
	• 1-GW-RI-130A • 1-RM-RR-132, Ch 1 • 1-RM-RR-132, Ch 4	
STANDARD:		
	readings on radiation monitors greater than alert setpoint or checks that chart or indication is greater than alert setpoint.	

EVALUATOR'S NOT	E: N/A	
COMMENTS:		
STEP 3:		
	IFY PROCESS VENT FLOW RATE DEVICE - OPERABLE	SAT
STANDARD:		ONOA!
light off for ei	one process vent blower is in service by checking red light on and green ther 1-GW-F-1A or 1-GW-F-1B. ess vent flow indicated.	
EVALUATOR'S NOT	E: N/A	
COMMENTS:		
STEP 4:		
	vior to Stop 3	SAT
Caution and Note p	ior to step s	UNSAT
CAUTION:	When CTMT Vacuum Pump Discharge Isolation valve 1-GW-FCV-160 or 2-GW-FCV-260 is closed, the associated Vacuum Pumps must be placed in OFF.	
NOTE:	If a high alarm has actuated, the automatic functions associated with that monitor shall be verified or performed.	
STANDARD:		
Acknowle	edges caution and note.	
EVALUATOR'S NOT	E: N/A	
COMMENTS:		

STEP 5:  Step 3 - CONSULT SHIFT SUPERVISION TO DETERMINE IF GW SHOULD BE ISOLATED  STANDARD:  • Asks SM if GW should be isolated.  EVALUATOR'S NOTE: If asked, respond by saying "I need you to perform the actions for a high radiation alarm".  COMMENTS:  STEP 6:	SATUNSAT
Step 4 - VERIFY OR PERFORM AUTO ACTIONS  • Decay Tank Bleed Isolation valve 1-GW-FCV-101 - CLOSED  • CTMT Vacuum Pump Discharge Isolation valve 1-GW-FCV-160 - CLOSED  • CTMT Vacuum Pump Discharge Isolation valve 2-GW-FCV-260 - CLOSED  STANDARD:	SATUNSAT
<ul> <li>Closes 1-GW-FCV-101 by taking control switch to close and observing green light on and red light off OR by pressing the auto/manual station close arrow to close the valve. This is a Critical Task.</li> <li>Closes 1-GW-FCV-160 by taking control switch to close and observing green light on and red light off. This is a Critical Task.</li> <li>Closes 2-GW-FCV-260 by taking control switch to close and observing green light on and red light off. This is a Critical Task.</li> <li>EVALUATOR'S NOTE: N/A</li> <li>COMMENTS:</li> </ul>	1

STEP 7: Step 5 –	PLACE CTMT VACUUM PUMPS IN OFF  • 1-CV-P-1A • 1-CV-P-1B • 2-CV-P-1A • 2-CV-P-1B	SAT UNSAT
STANDARD:  • Places	control switches to the "off" position for 1-CV-P-1A and 1-CV-P-1B.	
	sts Unit 2 to place control switches for 2-CV-P-1A and 2-CV-P-1B to the "off"	
EVALUATOR'S	S NOTE: N/A	
COMMENTS:		
STEP 8:		SAT
Step 6 -	NOTIFY HP TO DO THE FOLLOWING:	
	<ul> <li>a) Verify area evacuated as necessary</li> <li>b) Control access as necessary</li> <li>c) Sample</li> <li>d) Contact HP to verify activity within limits</li> <li>e) Investigate cause</li> </ul>	UNSAT
STANDARD:		
• NONE		
EVALUATOR'S	<b>S NOTE:</b> When candidate picks up phone or gaitronics to make notifications, inform the candidate that another RO will perform that action and complete the ARP. This JPM is complete.	
COMMENTS:		

STOP TIME:			

## Operator Directions Handout (TO BE READ TO APPLICANT BY EXAMINER)

#### **Task**

- Task is to be performed in the simulator.
- Isolate release by performing Steps 1-5 of RMA-C6 and/or RMA-C7 (1-GW-FCV-101 closed, 1-GW-FCV-160 closed, 2-GW-FCV-260 closed).

#### **Directions**

The evaluator will explain the initial conditions of the task to be performed and will provide the initiating cue. Ensure you indicate to the evaluator when you understand your assigned task

#### **Initial Conditions**

 Release of 'B' Waste Gas Decay Tank is in progress. The team is making preparations to run #1 EDG for its monthly performance test.

- I am the Shift Manager. You are the 3<sup>rd</sup> RO. The unit is at 100% power and stable. 'B" Waste Gas Decay Tank release is in progress and we are currently making preparations to run #1 EDG for its monthly surveillance test.
- An event is about to occur and I need you to respond IAW applicable procedures.

## Operator Directions Handout (TO BE GIVEN TO APPLICANT)

### **Initial Conditions**

• Release of 'B' Waste Gas Decay Tank is in progress. The team is making preparations to run #1 EDG for its monthly performance test.

- I am the Shift Manager. You are the 3<sup>rd</sup> RO. The unit is at 100% power and stable. 'B" Waste Gas Decay Tank release is in progress and we are currently making preparations to run #1 EDG for its monthly surveillance test.
- An event is about to occur and I need you to respond IAW applicable procedures.



#### SURRY POWER STATION

#### ANNUNCIATOR RESPONSE PROCEDURE

NUMBER	PROCEDURE TITLE	REVISION 1
0-RMA-C6	PROCESS VENT PART ALERT / HI (WITH 1 ATTACHMENT)	PAGE 1 of 7

REFERENCES RMA-27

- 1) UFSAR 11.3
- 2) VPAP-2103, Offsite Dose Calculation Manual, Section 6.3.2
- 3) DCP 01-022, Ventilation Radiation Monitoring (Kaman) System Replacement

#### PROBABLE CAUSE

1) Alarm actuates when 1-GW-RM-130A, Rad Monitor Process Vent Particulate Detector, detects Process Vent activity greater than or equal to setpoint.

High activity may be caused by one or more of the following:

- · Discharge of Waste Gas Decay Tank.
- · High RCS radioactivity.
- · Overpressure or relief valve failure on a system discharging to Process Vent.
- · Faulty monitor filter operation.
- Radioactive material transport in vicinity of monitor.
- 2) Instrumentation failure has occurred.

## CONTINUOUS USE

NUMBER	PROCEDURE TITLE	REVISION
0-RMA-C6	PROCESS VENT PART ALERT / HI	PAGE 2 of 7

STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
NOTE: • The MGP Process Vent channel sensin the blue SEL key on the display unit wi	g high radiation will buzz with an audible alarm. Pressing II silence the alarm.
<ul> <li>This annunciator does NOT have reflace high radiation will buzz with an audible</li> </ul>	sh capability. This MGP remote display unit indicating alarm.
1 VERIFY ALARM - READING ON MONITO OR CHART RECORDER GREATER THAN	
OR EQUAL TO ALERT SETPOINT	a) Increase surveillance on the following
□ • 1-GW-RI-130A	monitors:
<ul><li>□ • 1-RM-RR-132, Ch 1</li><li>□ • 1-RM-RR-132, Ch 4</li></ul>	<ul><li>□ • 1-GW-RI-130B</li><li>□ • 1-GW-RI-130C</li></ul>
	□ b) Evaluate entry into 0-AP-10.13, LOSS OF MAIN CONTROL ROOM ANNUNCIATORS.
	☐ c) Initiate a Work Request.
	□ d) GO TO Step 15.
2 VERIFY PROCESS VENT FLOW RATE DEVICE - OPERABLE	Do the following:
524162 61 214 (522	<ul> <li>a) Verify one Process Vent Blower in service:</li> </ul>
	<ul><li>□ • 1-GW-F-1A</li><li>□ • 1-GW-F-1B</li></ul>
	☐ b) Estimate flow rate every 4 hrs.
	<ul> <li>c) Record flow rate in Unit 1 Narrative Log in MCR.</li> </ul>

NUMBER	PROCEDURE TITLE	REVISION 1
0-RMA-C6	PROCESS VENT PART ALERT / HI	PAGE 3 of 7

STEP ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED	
***************	****
CAUTION: When CTMT Vacuum Pump Discharge Isolation valve 1-GW-FCV-160 or 2-GW-FCV-26	1
closed, the associated Vacuum Pumps must be placed in OFF.	) 15
* * * * * * * * * * * * * * * * * * * *	****
<b>NOTE:</b> If a high alarm has actuated, the automatic functions associated with that monitor shall be or performed.	e verified
3 CONSULT SHIFT SUPERVISION TO GO TO Step 6.  DETERMINE IF GW SHOULD BE ISOLATED	
4 VERIFY OR PERFORM AUTO ACTIONS:	
<ul> <li>Decay Tank Bleed Isolation valve 1-GW-FCV-101 - CLOSED</li> </ul>	
<ul> <li>CTMT Vacuum Pump Discharge Isolation valve 1-GW-FCV-160 - CLOSED</li> </ul>	
<ul> <li>CTMT Vacuum Pump Discharge Isolation valve 2-GW-FCV-260 - CLOSED</li> </ul>	
5 PLACE CTMT VACUUM PUMPS IN OFF	
□ • 1-CV-P-1A	
□ • 1-CV-P-1B	
- 2-CV-P-1A	
□ • 2-CV-P-1B	

NUMBER	PROCEDURE TITLE	REVISION
0-RMA-C6	PROCESS VENT PART ALERT / HI	PAGE 4 of 7

STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED	
6	NOTIFY HP TO DO THE FOLLOWING:			
	a) Verify area evacuated as necessary			
	b) Control access as necessary			
	c) Sample			
	d) Contact HP to verify activity within limit	s 🗆	d) Initiate a Work Request.	
	e) Investigate cause			
7	ISOLATE KNOWN RELEASES TO PROCE VENTS:	ESS		
	a) Secure any known venting evolution:			
	• SI ACCUM(s) • PRT			
	b) Attempt to isolate leakage:			
	• WGDT			
	CTMT VAC PUMPS     PRT			
	• SI ACCUM(s)			
	c) Coordinate with HP to determine need setpoint change	for		

NUMBER	PROCEDURE TITLE PROCESS VENT PART ALERT / HI	REVISION 1
0-RMA-C6	PROCESS VENT PART ALERT / HI	PAGE 5 of 7

ACTION/EVENTED DECROVES	DESDONSE NOT OBTAINED
STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
*8 CHECK ALARM - READING ON	Do the following:
MONITOR OR CHART RECORDER GREATER THAN OR EQUAL TO HI	☐ a) Increase surveillance of radiation trend.
SETPOINT	<ul> <li>b) Continue efforts to isolate leakage, as necessary.</li> </ul>
	<ul> <li>c) <u>IF CTMT Vacuum Pumps stopped AND</u> WGDT isolated in Step 4, <u>THEN</u> realign when directed by Shift Supervision.</li> </ul>
	<ul> <li>d) <u>IF</u> HI setpoint exceeded, <u>THEN</u> perform Steps 9 through 14.</li> </ul>
	□ e) GO TO Step 15.
9 CHECK ACTIVITY LEVEL - DECREASING	☐ GO TO Step 11.
10 GO TO STEP 12	

NUMBER	PROCEDURE TITLE	REVISION 1
0-RMA-C6	PROCESS VENT PART ALERT / HI	PAGE 6 of 7

OTED	ACTION/EXPECTED DECIDING	ı	DESPONSE NOT OBTAINED
STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
11	DENTIFY SOURCE OF RELEASE:		
□ a	a) Close trip valve 1-GW-HCV-106		
□ы	o) Close isolation valves IAW Attachment	1	
0	c) Open trip valve 1-GW-HCV-106		
□ d	Open isolation valves closed in     Attachment 1 individually to identify sou     of release	ırce	
□ e	e) Verify source of release isolated		e) Do the following:
			<ol> <li>Monitor systems with relief valves discharging to process vent.</li> </ol>
			2) Isolate release.
□ f)	) Return unaffected systems to service		
*12 \	/ERIFY HIGH ALARM - CLEAR		Do the following:
			a) <u>WHEN</u> high alarm clears, <u>THEN</u> perform Steps 13 and 14.
			b) GO TO Step 15.
	OPEN CTMT VACUUM PUMP DISCHARO SOLATION VALVES 1-GW-FCV-160 AND 2-GW-FCV-260 AS NECESSARY	ΞE	
14 S	START CTMT VAC PUMPS AS NECESSA	ARY	

NUMBER	PROCEDURE TITLE	REVISION 1
0-RMA-C6	PROCESS VENT PART ALERT / HI	PAGE 7 of 7

				_
STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED	
15 F	PROVIDE NOTIFICATIONS AS NECESSA	ARY:		
	• OMOC			
_ ·	• STA			
.	Shift Supervision			
.	· HP			
	• I&C			
	_	END -		
		LIVD -		

NUMBER	ATTACHMENT TITLE	ATTACHMENT
0-RMA-C6	RELEASE ISOLATION	1
REVISION	RELEASE ISOLATION	PAGE
1		1 of 1

#### 1. Close the following isolation valves:

- CC Surge Tank, HCV-CC-100 (1-CC-TK-1)
- Laboratory Drain Tank, 1-LW-16 (1-PL-TK-4) (2 ft North of South Wall, 2 ft East of West Wall, at Gate 5)
- Spent Resin Catch Tank, 1-LW-676 (1-LW-TK-14) (Decon Bldg, Gate 27)
- Spent Resin Blend Tank, 1-LW-646 (1-LW-TK-15) (Decon Bldg, Gate 22, 1 ft North of South Wall)
- SFP IX, 1-FC-17 (1-FC-I-1) (IX Alley, Reach Rod, 15 ft Northwest of Gate 14)
- BR Cleanup IX 2A, 1-BR-255 (1-BR-I-2A) (1 ft Southeast of Boron Cleanup IX A 13 ft off floor)
- BR Cleanup IX 2B, 1-BR-256 (1-BR-I-2B) (1 ft Southwest of Boron Cleanup IX A 13 ft off floor)
- BR Cs Removal IX 1A, 1-BR-282 (1-BR-I-1A) (IX Alley, 7 ft off floor)
- BR Cs Removal IX 1B, 1-BR-280 (1-BR-I-1B) (IX Alley, 7 ft off floor)
- Unit 1 Primary Vent, 1-VA-1 (1-DA-TK-1) (Valve normally locked closed)
- Unit 2 Primary vent, 2-VA-1 (2-DA-TK-1) (Valve normally locked closed)
- Unit 1 CVCS Deborating Demineralizer 3A, 1-CH-41 (1-CH-I-3A) (IX Alley, 8 ft off floor)
- Unit 1 CVCS Deborating Demineralizer 3B, 1-CH-48 (1-CH-I-3B) (IX Alley, 8 ft off floor)
- Unit 1 CVCS Cation Demineralizer, 1-CH-54 (1-CH-I-2) (IX Alley, 8 ft off floor)
- Unit 1 CVCS Mixed Bed Demineralizer 1A, 1-CH-60 (1-CH-I-1A) (IX Alley, 8 ft off floor)
- Unit 1 CVCS Mixed Bed Demineralizer 1B, 1-CH-66 (1-CH-I-1B) (IX Alley, 8 ft off floor)
- Unit 2 CVCS Deborating Demineralizer 3A, 2-CH-41 (2-CH-I-3A) (IX Alley, 7 ft off floor)
- Unit 2 CVCS Deborating Demineralizer 3B, 2-CH-48 (2-CH-I-3B) (IX Alley, 7 ft off floor)
- Unit 2 CVCS Cation Demineralizer, 2-CH-54 (2-CH-I-2) (IX Alley, 7 ft off floor)
- Unit 2 CVCS Mixed Bed Demineralizer 1A, 2-CH-60 (2-CH-I-1A) (IX Alley, 5.5 ft off floor)
- Unit 2 CVCS Mixed Bed Demineralizer 1B, 2-CH-66 (2-CH-I-1B) (IX Alley, 7 ft off floor)



#### SURRY POWER STATION

#### ANNUNCIATOR RESPONSE PROCEDURE

NUMBER 0-RMA-C7	PROCEDURE TITLE	REVISION 1
0-KIVIA-C7	PROCESS VENT NORMAL RANGE GAS ALERT / HI (WITH 1 ATTACHMENT)	PAGE 1 of 8

REFERENCES RMA-29

- 1) UFSAR 11.3
- 2) VPAP-2103, Offsite Dose Calculation Manual, Section 6.3.2
- 3) DCP 01-022, Ventilation Radiation Monitoring (Kaman) System Replacement

#### PROBABLE CAUSE

1) Alarm actuates when 1-GW-RM-130B, Rad Monitor Process Vent Noble Gas Detector, detects Process Vent activity greater than or equal to setpoint.

High activity may be caused by one or more of the following:

- · Discharge of Waste Gas Decay Tank.
- · High RCS radioactivity.
- Overpressure or relief valve failure on a system discharging to Process Vent.
- · Faulty monitor filter operation.
- · Radioactive material transport in vicinity of monitor.
- 2) Instrumentation failure has occurred.

# CONTINUOUS USE

NUMBER	PROCEDURE TITLE	REVISION 1
0-RMA-C7	PROCESS VENT NORMAL RANGE GAS ALERT / HI	PAGE 2 of 8

ACTION/EXPECTED PEODONICE	DESPONSE NOT OPTAINED		
STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED		
NOTE: • The MGP Process Vent channel sensing high radiation will buzz with an audible alarm. Pressing the blue SEL key on the display unit will silence the alarm.			
<ul> <li>This annunciator does NOT have reflash high radiation will buzz with an audible a</li> </ul>	h capability. This MGP remote display unit indicating alarm.		
1 VERIFY ALARM - READING ON MONITOR OR CHART RECORDER GREATER THAN	_		
OR EQUAL TO ALERT SETPOINT	a) Increase surveillance on the following     monitors:		
□ • 1-GW-RI-130B	_		
<ul><li>□ • 1-RM-RR-132, Ch 2</li><li>□ • 1-RM-RR-132, Ch 5</li></ul>	□ • 1-GW-RI-130A □ • 1-GW-RI-130C		
	<ul> <li>b) Evaluate entry into 0-AP-10.13, LOSS</li> <li>OF MAIN CONTROL ROOM</li> <li>ANNUNCIATORS.</li> </ul>		
	□ c) Initiate a Work Request.		
	□ d) GO TO Step 18.		
2 VERIFY PROCESS VENT FLOW RATE DEVICE - OPERABLE	Do the following:		
	<ul> <li>a) Verify one Process Vent Blower in service:</li> </ul>		
	<ul><li>□ • 1-GW-F-1A</li><li>□ • 1-GW-F-1B</li></ul>		
	□ b) Estimate flow rate every 4 hrs.		
	<ul> <li>c) Record flow rate in Unit 1 Narrative Log in MCR.</li> </ul>		

NUMBER	PROCEDURE TITLE	REVISION
0-RMA-C7	PROCESS VENT NORMAL RANGE GAS ALERT / HI	PAGE 3 of 8

STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
ACTION EXPEDITED NEST CHOE	RESPONSE NOT OBTAINED
* * * * * * * * * * * * * * * * * * * *	* * * * * * * * * * * * * * * * * * * *
CAUTION: When CTMT Vacuum Pump Discharge Is closed, the associated Vacuum Pumps m	
* * * * * * * * * * * * * * * * * * * *	·
<b>NOTE:</b> If a high alarm has actuated, the automati or performed.	ic functions associated with that monitor shall be verified
3 CONSULT SHIFT SUPERVISION TO DETERMINE IF GW SHOULD BE ISOLAT	☐ GO TO Step 6. ED
4 VERIFY OR PERFORM AUTO ACTIONS:	
<ul> <li>Decay Tank Bleed Isolation valve 1-GW-FCV-101 - CLOSED</li> </ul>	
<ul> <li>CTMT Vacuum Pump Discharge Isolation valve 1-GW-FCV-160 - CLOSED</li> </ul>	
<ul> <li>CTMT Vacuum Pump Discharge Isolation valve 1-GW-FCV-260 - CLOSED</li> </ul>	
5 PLACE CTMT VACUUM PUMPS IN OFF	
<ul> <li>□ • 1-CV-P-1A</li> <li>□ • 1-CV-P-1B</li> <li>□ • 2-CV-P-1A</li> <li>□ • 2-CV-P-1B</li> </ul>	

NUMBER	PROCEDURE TITLE	REVISION 1
0-RMA-C7	PROCESS VENT NORMAL RANGE GAS ALERT / HI	PAGE 4 of 8

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
6	NOTIFY HP TO DO THE FOLLOWING:	
	a) Verify area evacuated as necessary	
	b) Control access as necessary	
	c) Sample	
	d) Contact HP to verify activity within limits	□ d) Initiate a Work Request.
	e) Investigate cause	
7	ISOLATE KNOWN RELEASES TO PROCESS VENTS:	
	a) Secure any known venting evolution:	
	• SI ACCUM(s) • PRT	
	b) Attempt to isolate leakage:	
	<ul><li>WGDT</li><li>CTMT VAC PUMPS</li><li>PRT</li><li>SI ACCUM(s)</li></ul>	
	c) Coordinate with HP to determine need for setpoint change	

NUMB	ER	PROCEDURE TITLE	REVISION 1
0-RMA	-C7	PROCESS VENT NORMAL RANGE GAS ALERT / HI	PAGE 5 of 8

Г	STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	_
*8 CHECK ALARM - READING ON		CHECK ALARM - READING ON MONITOR OR CHART RECORDER	Do the following:	
		GREATER THAN OR EQUAL TO HI SETPOINT	<ul> <li>a) Increase surveillance of radiation trend.</li> </ul>	
			□ b) Continue efforts to isolate leakage, as necessary.	
			<ul> <li>c) <u>IF CTMT Vacuum Pumps stopped AND WGDT isolated in Step 4, THEN realign when directed by Shift Supervision.</u></li> </ul>	
			<ul> <li>d) <u>IF</u> HI setpoint exceeded, <u>THEN</u> perform</li> <li>Steps 9 through 16.</li> </ul>	
			☐ e) GO TO Step 18.	
	9	CHECK ACTIVITY LEVEL - DECREASING	☐ GO TO Step 11.	
	10	GO TO STEP 14		

NUMBER	PROCEDURE TITLE	REVISION 1
0-RMA-C7	PROCESS VENT NORMAL RANGE GAS ALERT / HI	PAGE 6 of 8

11 IDENTIFY SOURCE OF RELEASE:	
☐ a) Close trip valve 1-GW-HCV-106	
□ b) Close isolation valves IAW Attachment 1	
□ c) Open trip valve 1-GW-HCV-106	
☐ d) Open isolation valves closed in  Attachment 1 individually to identify source  of release	
☐ e) Verify source of release isolated e) Do the following:	
☐ 1) Monitor systems with relief valve discharging to process vent.	s
☐ 2) Isolate release.	
☐ f) Return unaffected systems to service	
NOTE: A HI HI (H/H) alarm on 1-GW-RI-130B, Remote Indication Process Vent Gas, will swap monit to 1-GW-RI-130C, Remote Indication Process Vent AR Gas. The normal range monitor is retuto service by I&C.	
*12 CHECK MGPI PROCESS VENT GAS	

NUMBER	PROCEDURE TITLE PROCESS VENT NORMAL RANGE GAS ALERT / HI	REVISION 1
0-RMA-C7	PROCESS VENT NORMAL RANGE GAS ALERT / HI	PAGE 7 of 8

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
	ACTION/EXI ECTED RESI ONSE	RESI GNOE NOT GETAINED
*13	RETURN MGPI PROCESS VENT GAS RADIATION MONITORING TO NORMAL RANGE (1-GW-RI-130B):	
	a) Check the following:	a) Do the following:
_	<ul> <li>Reading on 1-GW-RI-130C - NORMAL</li> </ul>	<ol> <li>WHEN conditions met, <u>THEN</u> perform Steps 13b through 17.</li> </ol>
	AND	2) GO TO Step 18.
	HP concurrence to return monitoring to normal - OBTAINED	
	b) Direct I&C to restore MGP Process Vent Rad Monitor to normal range IAW IMP-C-RM-36, CHECKING, REPAIRING, OR REPLACING A COMPONENT IN THE RADIATION MONITORING SYSTEM	
14	VERIFY HIGH ALARM - CLEAR	Do the following:
*14	VERIFY HIGH ALARM - CLEAR	Do the following:  a) WHEN high alarm clears, THEN perform Steps 15 through 17.
^14	VERIFY HIGH ALARM - CLEAR	a) WHEN high alarm clears, THEN perform
	OPEN CTMT VACUUM PUMP DISCHARGE ISOLATION VALVES 1-GW-FCV-160 AND 2-GW-FCV-260 AS NECESSARY	a) <u>WHEN</u> high alarm clears, <u>THEN</u> perform Steps 15 through 17.
15	OPEN CTMT VACUUM PUMP DISCHARGE ISOLATION VALVES 1-GW-FCV-160	a) <u>WHEN</u> high alarm clears, <u>THEN</u> perform Steps 15 through 17.
15 16	OPEN CTMT VACUUM PUMP DISCHARGE ISOLATION VALVES 1-GW-FCV-160 AND 2-GW-FCV-260 AS NECESSARY	a) <u>WHEN</u> high alarm clears, <u>THEN</u> perform Steps 15 through 17.
15 16	OPEN CTMT VACUUM PUMP DISCHARGE ISOLATION VALVES 1-GW-FCV-160 AND 2-GW-FCV-260 AS NECESSARY  START CTMT VAC PUMPS AS NECESSARY  REOPEN 1-GW-HCV-106 AS DIRECTED BY	a) <u>WHEN</u> high alarm clears, <u>THEN</u> perform Steps 15 through 17.
15 16	OPEN CTMT VACUUM PUMP DISCHARGE ISOLATION VALVES 1-GW-FCV-160 AND 2-GW-FCV-260 AS NECESSARY  START CTMT VAC PUMPS AS NECESSARY  REOPEN 1-GW-HCV-106 AS DIRECTED BY	a) <u>WHEN</u> high alarm clears, <u>THEN</u> perform Steps 15 through 17.

NUMBER	PROCEDURE TITLE	REVISION 1
0-RMA-C7	PROCESS VENT NORMAL RANGE GAS ALERT / HI	PAGE 8 of 8

STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
7.0.1.0.1.0.1.0.1.0.1.0.1.0.1.0.1.0.1.0.	1,231 31132 1131 331 11123	
18 PROVIDE NOTIFICATIONS AS NECESSARY:		
□ • OMOC		
□ · STA		
□ • Shift Supervision		
□•нР		
□ • I&C		
- END -		
- LND -		

NUMBER 0-RMA-C7	ATTACHMENT TITLE	ATTACHMENT 1
REVISION 1	RELEASE ISOLATION	PAGE 1 of 1

#### 1. Close the following isolation valves:

- CC Surge Tank, HCV-CC-100 (1-CC-TK-1)
- Laboratory Drain Tank, 1-LW-16 (1-PL-TK-4) (2 ft North of South Wall, 2 ft East of West Wall, at Gate 5)
- Spent Resin Catch Tank, 1-LW-676 (1-LW-TK-14) (Decon Bldg, Gate 27)
- Spent Resin Blend Tank, 1-LW-646 (1-LW-TK-15) (Decon Bldg, Gate 22, 1 ft North of South Wall)
- SFP IX, 1-FC-17 (1-FC-I-1) (IX Alley, Reach Rod, 15 ft Northwest of Gate 14)
- BR Cleanup IX 2A, 1-BR-255 (1-BR-I-2A) (1 ft Southeast of Boron Cleanup IX A 13 ft off floor)
- BR Cleanup IX 2B, 1-BR-256 (1-BR-I-2B) (1 ft Southwest of Boron Cleanup IX A 13 ft off floor)
- BR Cs Removal IX 1A, 1-BR-282 (1-BR-I-1A) (IX Alley, 7 ft off floor)
- BR Cs Removal IX 1B, 1-BR-280 (1-BR-I-1B) (IX Alley, 7 ft off floor)
- Unit 1 Primary Vent, 1-VA-1 (1-DA-TK-1) (Valve normally locked closed)
- Unit 2 Primary vent, 2-VA-1 (2-DA-TK-1) (Valve normally locked closed)
- Unit 1 CVCS Deborating Demineralizer 3A, 1-CH-41 (1-CH-I-3A) (IX Alley, 8 ft off floor)
- Unit 1 CVCS Deborating Demineralizer 3B, 1-CH-48 (1-CH-I-3B) (IX Alley, 8 ft off floor)
- Unit 1 CVCS Cation Demineralizer, 1-CH-54 (1-CH-I-2) (IX Alley, 8 ft off floor)
- Unit 1 CVCS Mixed Bed Demineralizer 1A, 1-CH-60 (1-CH-I-1A) (IX Alley, 8 ft off floor)
- Unit 1 CVCS Mixed Bed Demineralizer 1B, 1-CH-66 (1-CH-I-1B) (IX Alley, 8 ft off floor)
- Unit 2 CVCS Deborating Demineralizer 3A, 2-CH-41 (2-CH-I-3A) (IX Alley, 7 ft off floor)
- Unit 2 CVCS Deborating Demineralizer 3B, 2-CH-48 (2-CH-I-3B) (IX Alley, 7 ft off floor)
- Unit 2 CVCS Cation Demineralizer, 2-CH-54 (2-CH-I-2) (IX Alley, 7 ft off floor)
- Unit 2 CVCS Mixed Bed Demineralizer 1A, 2-CH-60 (2-CH-I-1A) (IX Alley, 5.5 ft off floor)
- Unit 2 CVCS Mixed Bed Demineralizer 1B, 2-CH-66 (2-CH-I-1B) (IX Alley, 7 ft off floor)

## U.S. Nuclear Regulatory Commission Surry Power Station

# SR10301 Simulator Job Performance Measure 010.A4.01(3.7/3.5)

Applicant	_ Start Tir	ne
Examiner		
Date	Stop Tir	ne
<u>Title</u>		
DEPRESSURIZE THE RCS WITH AUX SPRAY	DUE TO LOSS OF RCPS IN	I AP-24.01.
K/A: 010.A4.01 Ability to manually operate and	l/or monitor in the control ro	om: PZR Spray Valve
<u>Applicability</u>	Estimated Time	Actual Time
RO	15 minutes	

## **Conditions**

- Task is to be PERFORMED in the simulator.
- A steam generator tube leak of approximately 65 gpm has developed on "C" SG. Reactor has been tripped and 1-AP-24.01, LARGE STEAM GENERATOR TUBE LEAK, is in progress. The team has cooled down the RCS to target temperature and is currently at step 29 "Depressurize RCS to stop breakflow".

## **Standards**

RCS depressurized using Aux Spray until termination criteria satisfied (Step 29 complete).

## **Initiating Cues**

- 1-AP-24.01, LARGE STEAM GENERATOR TUBE LEAK, step 29.
- Shift Manager direction.

## **Terminating Cues**

RCS depressurized using Aux Spray until termination criteria satisfied (Step 29 complete).

## **Procedures**

• 1-AP-24.01, LARGE STEAM GENERATOR TUBE LEAK

## **Tools and Equipment**

## **Safety Considerations**

None

None

## **Simulator Setup**

- Recall IC-324 OR
- Call up 100% power IC and initialize.
- Enter malfunctions for SG tube leakage (RC2403) at 7% deg.
- Enter malfunctions to have RCPs trip spuriously trip on swapover (RC5601, RC5602, RC5603).
- Defeat SG level alarms to avoid nuisance alarms (HG5, HG6, HG7).
- Implement all malfunctions and perform AP-16.00, E-0, and AP-24.01 through step 28 to cooldown the RCS to target temperature and stabilize.
- Place the simulator until ready for JPM performance.
- Place orange magnets on 1-RC-P-1A/1B/1C control switches.

## **Initiating Cues**

- 1-AP-24.01, LARGE STEAM GENERATOR TUBE LEAK, step 29.
- Shift Manager direction.

## **Directions to the Applicant**

- I am the Shift Manager. A steam generator tube leak of approximately 65 gpm has developed on "C" SG.
- 1-AP-16.00, RCS LEAKAGE, has been performed. The Reactor has been tripped IAW 1-E-0, REACTOR TRIP
  OR SAFETY INJECTION, and 1-AP-24.01, LARGE STEAM GENERATOR TUBE LEAK, is in progress.
- All 3 RCPs tripped on swapover.
- The team has cooled down the RCS to target temperature and is currently at step 29 "Depressurize RCS to stop breakflow".
- I need you to perform step 29 of 1-AP-24.01 in its entirety to depressurize the RCS to stop breakflow.
- When you finish the actions necessary to accomplish this, please inform me.

## <u>Notes</u>

## PERFORMANCE CHECKLIST

## Notes to the Evaluator

- Task critical elements are bolded and noted at the end of the step as CRITICAL STEP.
- An additional instructor may be needed to silence and acknowledge alarms for the examinee.
- START TIME:

STANDARD:  • Not	DEPRESSURIZE RCS TO STOP BREAKFLOW:  a) Normal PRZR spray – AVAILABLE  es that no RCPs are running and normal spray is not available. es to the RNO.  NOTE: N/A	SATUNSAT
STEP 2: STEP 29aRNO	DEPRESSURIZE RCS TO STOP BREAKFLOW:  a) Do the following:  1) Verify or place normal letdown in service.  2) Close normal PRZR spray valves.  3) Open 1-CH-HCV-1311, CHG AUX spray valve.  4) Close 1-CH-HCV-1310A, CHG Line ISOL Valve.  5) Monitor PRZR level	SATUNSAT

STANDARD:		
<ul> <li>Verifies</li> <li>Places This is</li> <li>Opens green</li> <li>Closes red lig</li> <li>Monito</li> </ul>		
EVALUATOR'S	S NOTE: N/A	
COMMENTS:		
69%, 0	DEPRESSURIZE RCS TO STOP BREAKFLOW:  b) Spray PRZR with maximum available spray until ANY of the following satisfied:  • PRZR level - GREATER THAN 69% OR • RCS subcooling based on CETCs - LESS THAN 30°F OR • RCS Pressure - LESS THAN AFFECTED SG PRESSURE  s with Aux Spray using 1-CH-HCV 1311 until pressurizer level reaches DR RCS subcooling is less the 30°F, OR RCS pressure is less than "C" essure. This is a Critical Task.  S NOTE: N/A	SATUNSAT
STEP 4:		SAT
Step 29c -	DEPRESSURIZE RCS TO STOP BREAKFLOW:	UNSAT
	c) Close spray valves	
	<ul> <li>Normal spray valves</li> <li>OR</li> <li>Auxiliary spray valve, 1-CH-HCV-1311</li> </ul>	

STANDARD:  • Closes 1-CH-HCV-1311 when termination criteria satisfied by observing green light on and red light off.  EVALUATOR'S NOTE: N/A  COMMENTS:	
STEP 5:  Step 29d - DEPRESSURIZE RCS TO STOP BREAKFLOW:  d) Verify open or open 1-CH-HCV-1310A, CHG Line ISOL Valve.  STANDARD:  • Opens 1-CH-HCV-1310A by observing red light on and green light off.  EVALUATOR'S NOTE: N/A  COMMENTS:	SATUNSAT
STEP 6:  Step 29e - DEPRESSURIZE RCS TO STOP BREAKFLOW:  e) Check CHG pumps - ONLY ONE RUNNING  STANDARD:  • Checks 1-CH-P-1C in service by observing red light indication and amps and the 1-CH-P-1A & 1B secure in AUTO.  EVALUATOR'S NOTE: N/A  COMMENTS:	SATUNSAT

STEP 7:  REPORT TO SHIFT SUPERVISOR (EVALUATOR).	SAT
STANDARD:	
Verbal status report that the RCS is depressurized IAW step 29 of 1-AP-24.01, LARGE STEAM GENERATOR TUBE LEAKAGE.	
EVALUATOR'S NOTE: N/A	
COMMENTS:	
STOP TIME:	

## Operator Directions Handout (TO BE READ TO APPLICANT BY EXAMINER)

## **Task**

- Task is to be performed in the simulator.
- Perform Step 29 of 1-AP-24.01, LARGE STEAM GENERATOR TUBE LEAKAGE.

## **Directions**

The evaluator will explain the initial conditions of the task to be performed and will provide the initiating cue. Ensure you indicate to the evaluator when you understand your assigned task

## **Initial Conditions**

 A steam generator tube leak of approximately 65 gpm has developed on "C" SG. Reactor has been tripped and 1-AP-24.01, LARGE STEAM GENERATOR TUBE LEAK, is in progress. The team has cooled down the RCS to target temperature and is currently at step 29 "Depressurize RCS to stop breakflow".

## **Initiating Cues**

- I am the Shift Manager. A steam generator tube leak of approximately 65 gpm has developed on "C" SG.
- 1-AP-16.00, RCS LEAKAGE, has been performed. The Reactor has been tripped IAW 1-E-0, REACTOR TRIP OR SAFETY INJECTION, and 1-AP-24.01, LARGE STEAM GENERATOR TUBE LEAK, is in progress.
- All 3 RCPs tripped on swapover.
- The team has cooled down the RCS to target temperature and is currently at step 29 "Depressurize RCS to stop breakflow".
- I need you to perform step 29 of 1-AP-24.01 in its entirety to depressurize the RCS to stop breakflow.
- When you finish the actions necessary to accomplish this, please inform me.

## Operator Directions Handout (TO BE GIVEN TO APPLICANT)

## **Initial Conditions**

 A steam generator tube leak of approximately 65 gpm has developed on "C" SG. Reactor has been tripped and 1-AP-24.01, LARGE STEAM GENERATOR TUBE LEAK, is in progress. The team has cooled down the RCS to target temperature and is currently at step 29 "Depressurize RCS to stop breakflow".

## **Initiating Cues**

- I am the Shift Manager. A steam generator tube leak of approximately 65 gpm has developed on "C" SG.
- 1-AP-16.00, RCS LEAKAGE, has been performed. The Reactor has been tripped IAW 1-E-0, REACTOR TRIP
  OR SAFETY INJECTION, and 1-AP-24.01, LARGE STEAM GENERATOR TUBE LEAK, is in progress.
- All 3 RCPs tripped on swapover.
- The team has cooled down the RCS to target temperature and is currently at step 29 "Depressurize RCS to stop breakflow".
- I need you to perform step 29 of 1-AP-24.01 in its entirety to depressurize the RCS to stop breakflow.
- When you finish the actions necessary to accomplish this, please inform me.

NUMBER	PROCEDURE TITLE  LARGE STEAM GENERATOR TUBE LEAK	REVISION 29
1-AP-24.01	LARGE STEAM GENERATOR TOBE LEAR	PAGE 15 of 36

STEP	ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
OTE	ACTION/EXI ESTED REST SINCE		NEOF ONCE NOT OBTAINED
29	DEPRESSURIZE RCS TO STOP BREAKFLOW:		
	a) Normal PRZR spray - AVAILABLE		a) Do the following:
			<ol> <li>Verify or place normal letdown in service.</li> </ol>
			2) Close normal PRZR spray valves.
			<ol><li>Open 1-CH-HCV-1311, CHG AUX spray valve.</li></ol>
			<ol> <li>Close 1-CH-HCV-1310A, CHG Line ISOL Valve.</li> </ol>
			5) Monitor PRZR level.
	b) Spray PRZR with maximum available spray until ANY of the following satisfied:		
	<ul> <li>PRZR level - GREATER THAN 69%</li> </ul>		
	<u>OR</u>		
	<ul> <li>RCS subcooling based on CETCs - LESS THAN 30°F</li> </ul>		
	<u>OR</u>		
	<ul> <li>RCS Pressure - LESS THAN AFFECTED SG PRESSURE</li> </ul>		
	c) Close spray valves		
	Normal spray valves		Stop RCPs supplying failed spray valves:
			• RCP A, 1-RC-PCV-1455A
			• RCP C, 1-RC-PCV-1455B
	OR		
	<ul> <li>Auxiliary spray valve, 1-CH-HCV-1311</li> </ul>		
(STEP 29 CONTINUED ON NEXT PAGE)			
	•		

NUMBER	PROCEDURE TITLE	REVISION 29
1-AP-24.01	LARGE STEAM GENERATOR TUBE LEAK	PAGE 16 of 36

 STEP	$\Box$	ACTION/EXPECTED RESPONSE	<u> </u>		Г	RESPONSE NOT OBTAINED
, , <u>, , , , , , , , , , , , , , , , , </u>	L	ACTION EXTENTION OF THE PROPERTY OF THE PROPER	J		Ь	AZGI GIOZING I GDIA IIIZD
29.	DE	EPRESSURIZE RCS TO STOP BREAKFL	.OW: (Co	ntin	uec	3)
	d)	Verify open or open 1-CH-HCV-1310A CHG Line ISOL Valve.	,			
	e)	Check CHG pumps - ONLY ONE RUNNING			e)	Stop all but one CHG pump.
30	CI	HECK CC SYSTEM STATUS:				
	a)	Check SW to CC HXs - IN SERVICE			a)	Restore SW to CC HXs IAW 0-AP-12.01, LOSS OF INTAKE CANAL LEVEL.
	b)	Check CC pumps - AT LEAST ONE			b)	Do the following:
	RUNNING				1) Put CC pump control switches in PTL.	
						Locally throttle CC pump discharge valve to approximately 25% open:
						<ul><li>1-CC-558 for 1-CC-P-1A</li><li>1-CC-564 for 1-CC-P-1B</li></ul>
						3) Locally close stub bus tie breaker.
						4) Start one CC pump.
						5) Locally open discharge valve.
						<u>IF</u> a CC pump can <u>NOT</u> be started, <u>THEN</u> do the following:
						1) Attempt to crosstie CC systems.
						<ol> <li>WHEN CC restored, <u>THEN</u> perform Step 31.</li> </ol>
						3) GO TO Step 32.