



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 17, 2010

Mr. David J. Bannister
Vice President and CNO
Omaha Public Power District
Fort Calhoun Station
444 South 16th St. Mall
Omaha, NE 68102-2247

SUBJECT: FORT CALHOUN STATION – SUPPLEMENTAL INFORMATION NEEDED FOR
ACCEPTANCE OF REQUESTED LICENSING ACTION RE: AMENDMENT
REQUEST (TAC NO. ME4542)

Dear Mr. Bannister:

By letter dated August 16, 2010 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML102290067), Omaha Public Power District (OPPD, or the licensee) submitted a license amendment request for Fort Calhoun Station, Unit 1 (FCS). The proposed amendment would remove the Technical Specification (TS) limiting condition for operation (LCO) 2.15, *Instrumentation and Control Systems*, Table 2-5, *Instrumentation Operating Requirements for Other Safety Feature Functions*, Items 3, 4, and 5, and the associated Notes a, b, c, and d, for power-operated relief valve (PORV) and pressurizer safety valve (PSV) acoustic position indication and tail pipe temperature from the FCS TSs. The proposed change would also revise the surveillance requirement (SR), TS 3.1, *Instrumentation and Control*, Table 3-3, *Minimum Frequencies for Checks, Calibrations and Testing of Miscellaneous Instrumentation and Controls*, Items 21, 23, and 24 for *PORV Operation and Acoustic Position Indication*, *Safety Valve Acoustic Position Indication*, and *PORV/Safety Valve Tail Pipe Temperature*, respectively.

The purpose of this letter is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of this amendment request. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical review. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Consistent with Section 50.90 of Title 10 of the *Code of Federal Regulations* (10 CFR), an amendment to the license (including the technical specifications) must fully describe the changes requested, and following as far as applicable, the form prescribed for original applications. Section 50.34 of 10 CFR addresses the content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

The NRC staff has reviewed your application and concluded that the information delineated in the enclosure to this letter is necessary to enable the NRC staff to make an independent assessment regarding the acceptability of the proposed amendment request in terms of

regulatory requirements and the protection of public health and safety and the environment. In order to make the application complete, the NRC staff requests that OPPD supplement the application to address the information requested in the enclosure by September 27, 2010. This will enable the NRC staff to complete its detailed technical review. If the information responsive to the NRC staff's request is not received by the above date, the application will not be accepted for review pursuant to 10 CFR 2.101, and the NRC staff will cease its review activities associated with the application. If the application is subsequently accepted for review, you will be advised of any further information needed to support the NRC staffs detailed technical review by separate correspondence.

The information requested and associated time frame in this letter were discussed with Bill Hansher of your staff on September 13, 2010. If circumstances result in the need to revise your submittal date, or if you have any questions, please contact me.

Sincerely,



Lynnea Wilkins, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure:
As stated

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SUPPLEMENTAL INFORMATION NEEDED

AMENDMENT REQUEST

OMAHA PUBLIC POWER DISTRICT

FORT CALHOUN STATION, UNIT NO. 1

DOCKET NO. 50-285

1. Please provide a technical analysis for deleting the tail pipe temperature. Only a technical analysis discussion for deletion of the power-operated relief valve (PORV) and pressurizer safety valve (PSV) acoustic monitors is provided. Include an analysis that discusses the tail pipe temperature and the limiting condition for operation (LCO) four criteria, much like what is currently stated for the PORV/PSV valve position indication.
2. Please provide the P&ID and schematic diagrams and the depiction of the control room indicators for those PORV/PSV valve acoustic position indications.
3. Please explain what indication and/or recording each tail pipe temperature provides. Provide the schematic diagrams and the depiction of the control room indicators.
4. Please explain if all related Technical Specifications will relocate to the Updated Safety Analysis Report. If not, specify each document where the operability and surveillance requirements will be controlled.

Enclosure

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Sincerely,

/RA by Alan Wang for/

Lynnea Wilkins, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure:
As stated

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DATE	9/15/10	9/15/10	9/17/10	9/15/10	9/17/10	9/17/10

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