

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

September 16, 2010

Mr. Mark A. Schimmel Site Vice President Prairie Island Nuclear Generating Plant Northern States Power Company - Minnesota 1717 Wakonade Drive East Welch, MN 55089-9642

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 – CORRECTION TO AMENDMENT NOS. 195 AND 184 REGARDING TECHNICAL SPECIFICATION CHANGES TO CONTROL ROOM HABITABILTY REQUIREMENTS (TAC NOS. ME1605 AND ME1606)

Dear Mr. Schimmel:

On May 20, 2010, the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 196 to Facility Operating License No. DPR-42 and Amendment No.185 to Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Units 1 and 2, respectively. These amendments consisted of changes to the Technical Specifications (TSs) in response to your application dated June 24, 2009, as supplemented by letter dated December 21, 2009. The amendments modify the technical specification requirements to control room envelope habitability in accordance with Technical Specification Task Force traveler (TSTF)-448, Revision 3, "Control Room Habitability."

Following issuance of the amendments, the NRC staff was informed by the PINGP licensee that the footer on TS page 5.0-29, which identifies the previous amendment numbers for that page, was incorrect. The error was entirely administrative in nature. This TS page has been updated to identify the correct previous amendment numbers.

M. Schimmel

The corrected TS page is enclosed. The correction does not change the approval conveyed by the amendments, or the conclusions reached by the associated safety evaluation. We regret any inconvenience this may have caused. If you have any questions regarding this matter, please call me at (301) 415-4037.

Sincerely,

Thanang Dee gert

Thomas J. Wengert, Senior Project Manager Plant Licensing Branch III-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosure: Corrected TS Page

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ENCLOSURE

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

DOCKET NOS. 50-282 AND 50-306

REPLACEMENT TECHNICAL SPECIFICATION PAGE 5.0-29 FOR AMENDMENT NOS. 195 AND 184

5.5 Programs and Manuals

5.5.14 <u>Containment Leakage Rate Testing Program</u> (continued)

- d. Leakage Rate acceptance criteria are:
 - 1. Primary containment leakage rate acceptance criterion is $\leq 1.0 L_a$. Prior to unit startup, following testing in accordance with the program, the combined leakage rate acceptance criteria are $\leq 0.60 L_a$ for all components subject to Type B and Type C tests and $\leq 0.75 L_a$ for Type A tests.
 - 2. Air lock testing acceptance criteria are:
 - a) Overall air lock leakage rate is $\leq 0.05 L_a$ when tested at ≥ 46 psig.
 - b) For each door intergasket test, leakage rate is $\leq 0.01 L_a$ when pressurized to ≥ 10 psig.
- e. The provisions of SR 3.0.3 are applicable to the Containment Leakage Rate Testing Program.
- f. Nothing in these Technical Specifications shall be construed to modify the testing Frequencies required by 10 CFR 50, Appendix J.

5.5.15 Battery Monitoring and Maintenance Program

This Program provides for restoration and maintenance of the 125V plant safeguards batteries and service building batteries, which may be used instead of the safeguards batteries during shutdown conditions in accordance with manufacturer's recommendations, as follows:

- a. Actions to restore battery cells with float voltage < 2.13 V will be in accordance with manufacturer's recommendations, and
- b. Actions to equalize and test battery cells that had been discovered with electrolyte level below the minimum established design limit

M. Schimmel

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Sincerely,

/RA/

Thomas J. Wengert, Senior Project Manager Plant Licensing Branch III-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

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