



Entergy Nuclear Operations, Inc.  
Vermont Yankee  
320 Governor Hunt Road  
Vernon, VT 05354  
Tel 802 257 7711

**Robert J. Wanczyk**  
Licensing Manager

BVY 10-049

August 23, 2010

ATTN: Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555-0001

SUBJECT: Vermont Yankee 2010 Summary Report –  
Inservice Inspection and Repair and Replacement  
Vermont Yankee Nuclear Power Station  
Docket No. 50-271  
License No. DPR-28

Dear Sir or Madam:

This letter, in accordance with IWA-4000, IWA-6000 and ASME Code Case N-532-4, provides a copy of the Inservice Inspection Summary Report for activities performed at the Vermont Yankee Nuclear Power Station for the period November 11, 2008 to May 29, 2010. Vermont Yankee hereby submits 2010 Form OAR-1, Owner Activity Report for Inservice Inspections. Vermont Yankee's fourth ten year Inservice Inspection interval began September 1, 2003.

There are no new regulatory commitments being made in this submittal.

If you have any questions concerning this submittal, please contact me at (802) 451-3166.

Sincerely,

A handwritten signature in black ink that reads "Robert J. Wanczyk".

[RJW/JTM]

Attachment: OAR-1 Owner Activity Report for Inservice Inspections (7 pages)

cc listing (next page)

A047  
NRC

cc: Region 1 Administrator  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406-1415

Mr. James S. Kim, Project Manager  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

USNRC Resident Inspector  
Vermont Yankee Nuclear Power Station  
320 Governor Hunt Road  
Vernon, VT 05354

Mr. David O'Brien, Commissioner  
VT Department of Public Service  
112 State Street, Drawer 20  
Montpelier, VT 05620-2601



## Entergy Nuclear Vermont Yankee




### OAR-1

#### Owner Activity Report Through RFO 28

November 11, 2008 through May 29, 2010

Reviewed By: Jeffrey A. Stevenson  Date: 8/19/10  
Inservice Inspection Program Owner

Reviewed By: William L. Rice  Date: 8/19/10  
Repair/Replacement Program Owner

Approved By: Mark LeFrancos  Date: 8/19/10  
Programs & Components Manager  
Acting for BKW

# Form OAR-1 Owner's Activity Report

Report Number: 10-01-01

Plant: Entergy Nuclear Vermont Yankee

Unit No.: 1 Commercial service date: 11/30/1972 Refueling Outage no.: RFO 28

Current inspection interval: Fourth (4<sup>th</sup>)  
(1<sup>st</sup>, 2<sup>nd</sup>, 3<sup>rd</sup>, 4<sup>th</sup>, other)

Current inspection period: Second (2<sup>nd</sup>)  
(1<sup>st</sup>, 2<sup>nd</sup>, 3<sup>rd</sup>)

Edition and Addenda of Section XI applicable to the inspection plans: 1998 Edition, through 2000 Addenda

Date and revision of inspection plans: PP7015 Vermont Yankee Inservice Inspection Program, Revision 10, 1/3/2008

Edition and Addenda of Section XI applicable to repair/replacement activities if different than the inspection plans: 2001 Edition, through 2003 Addenda for Repair & Replacement and Pressure Test Program

Code Cases used: ASME Code Case N-513-2  
(if applicable)

## Certificate of Conformance

I certify that (a) the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of RFO 28 conform to the requirements of Section XI.  
(refueling outage number)

Signed: Norman L. Rademacher / [Signature] Director of Engineering Date: 8/19/10  
Owner or Owner's Designee, Title

## Certificate of Inservice Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Vermont and employed by Hartford Steam Boiler of CT of Connecticut have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Curt Hanson / [Signature] Commissions NB 11916 ABNI VT 887  
Inspector's Signature National Board, State, Province, and Endorsements

Date: 8/19/2010

## OAR-1 Table 1

### Items with Flaws or Relevant Conditions that Required Evaluation for Continued Service

Examination Category and Item Number	Item Description	Evaluation Description
D-B	Service Water (SW) Piping Lines 8"SW-800, 8"SW-801, and 6"SW-801B	CR-VTY-2009-00238: During operator rounds, a leak was discovered in SW piping. An ASME Code Case N-513 structural evaluation was performed. The N-513 evaluation demonstrated that the indications detected in the four welds are structurally acceptable for all design basis loadings on a temporary basis and not exceeding the time to the next scheduled plant outage.
D-B	"A" Emergency Diesel Generator (DG-1-1-A) Aftercooler Stationary Head	CR-VTY-2009-03376: During operator rounds, a pinhole leak was discovered in the DG-1-1A aftercooler wall. The leak was approximately 10mL every 5 minutes. An Engineering evaluation concluded the DG was inoperable. Tech Spec LCO was entered for repairs.
D-B	SW/CW (Circulating Water) traveling screen backwash header	CR-VTY-2009-02093: During operator rounds, a leak was discovered on the traveling screen backwash header of approximately several drops per minute from the header section going to the "B" SW traveling screen. While assessing the affected piping for possible Ultrasonic (UT) examination the corroded area failed when touched with a hand tool. Tech Spec LCO was entered for repairs.
F-A	Core Spray (CS) Pipe Support CS-HD-57C	CR-VTY-2010-01557: During ISI VT-3 examination, a loose jam nut on CS-HD-57C pipe support threaded connection was discovered. Engineering evaluation concluded that the condition was acceptable since the spring can setting had not changed.

**OAR-1 Table 1**

**Items with Flaws or Relevant Conditions that Required Evaluation for Continued Service**

<b>Examination Category and Item Number</b>	<b>Item Description</b>	<b>Evaluation Description</b>
F-A	Primary Containment Torus Support NB-H1951-TSS10	CR-VTY-2010-01638: During ISI VT-3 examination, Torus Support NB-H1951-TSS10 column anchor bolt was discovered cut-off. Engineering evaluation determined that this support bolt is abandoned equipment and that drawing had not been updated.
F-A	Residual Heat Removal Service Water (RHRSW) Spring Can Supports	CR-VTY-2010-01395: During a walk-down of the Service Water pipe trench, RHRSW pipe supports were identified as being in a degraded condition (exfoliation of the support surface due to general corrosion resulting from exposure to flooding in the pit). This condition was previously identified in 2005. In both instances an engineering evaluation concluded the degraded condition did not prevent the supports from performing their design function.
C-C	Feedwater (FDW) Anchor Piping Support Integral Attachment Weld FDW-HD39	CR-VTY-2010-01942: During an owner elected Penetrant Test (PT) exam, an approximate 1/2" long indication was discovered. The indication is located in the previously excavated area and is bounded by previous engineering evaluation supporting functionality and operability.

## OAR-1 Table 2

### **Abstract of Repair/Replacement Activities Required for Continued Service**

<b>Code Class</b>	<b>Repair, Replacement, Modification or Corrective Measure</b>	<b>Item Description</b>	<b>Description of Work</b>	<b>Reported Complete Date</b>	<b>Plan Number</b>
3	Replacement	Pipe Line 8" SW-801B (Standby Fuel Pool Cooling Service Water)	Thru wall leak in 8" SW-801B was repaired by installation of pre-fabricated Tee.	2/21/2009	180338-06
3	Replacement	Pipe Line 3" SW-5 (Service Water supply to traveling screens)	Thru wall leak in 3" SW-5 piping was repaired by replacement.	8/17/2009	197972-01
3	Replacement	Pipe Line SW-5 (Service Water supply to traveling screens)	All SW-5 piping outside and inside intake structure was repaired by replacement (prefabrication of piping).	8/17/2009	197972-23
3	Replacement	Standby Fuel Pool Cooling Service Water Piping per Engineering Change, EC 16328	Thru wall leak in Standby Fuel Pool Cooling Service Water Piping was repaired by replacement per EC 16328.	2/1/2010	202414-01
3	Replacement	Standby Fuel Pool Cooling Service Water Piping per Engineering Change, EC 16328	Standby Fuel Pool Cooling Service Water Pipe was replaced (Pre-fab).	2/1/2010	202414-19

## OAR-1 Table 2

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3	Replacement	Piping upstream of Standby Fuel Pool Cooling Service Water valve V70-261 (Engineering Change Notice, ECN-18983) on line 8"SW-800	8"SW-800 was replaced upstream of V70-261 Per ECN-18983.	1/21/2010	202414-41
3	Repair	12"X12"X8" Pipe Tee on Pipe Line 8"SW-800 (Standby Fuel Pool Cooling Service Water)	8"SW-800 was repaired by weld build-up on existing 12"X12"X8" Tee.	2/1/2010	202414-44
3	Replacement	"Alpha" Emergency Diesel Generator, DG-1-1A, Aftercooler	Pinhole leak on DG-1-1A Aftercooler was repaired by replacement.	2/8/2010	209099-01
3	Replacement	"Alpha" Service Water Pump P-7-1A Packing Gland	P-7-1A Packing Gland was replaced due to thru-wall leak on the stuffing box.	11/2/2009	212301-01
3	Replacement	"Alpha" Emergency Diesel Generator, DG-1-1A, Stationary Control Side Aftercooler Channel Head	Replace Stationary Control Side Aftercooler head on DG-1-1A("Alpha" Emergency Diesel Generator)	1/7/2010	213217-01



## OAR-1 Table 2

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<b>Code Class</b>	<b>Repair, Replacement, Modification or Corrective Measure</b>	<b>Item Description</b>	<b>Description of Work</b>	<b>Reported Complete Date</b>	<b>Plan Number</b>
3	Repair	"Alpha" Emergency Diesel Generator, DG-1-1A, Control Side Channel Head	Thru wall leak on DG-1-1A Control Side Channel Head wall damage was repaired by weld buildup from ID.	1/7/2010	213217-12
3	Replacement	"Alpha" Emergency Diesel Generator, DG-1-1A, Aftercooler Heat Exchanger Stationary Channel Flange	New DG-1-1A Aftercooler Heat Exchanger Stationary Channel Flange was modified to facilitate replacement.	1/7/2010	213217-13