Hand Calculation of Boric Acid and Primary Water Integrator Setting for Manual Makeup to the VCT

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Watts Bar Nuclear Plant 3-OT-JPMADA 1-7a

NUCLEAR TRAINING REVISION/USAGE LOG				
Rev. #	Description of Changes	Date	Pages Affected	Reviewed By
0 0	Initial issue, generated for NRC Initial License Exam		All	D. Hughes

Watts Bar Nuclear Plant 3-OT-JPMADA 1-7a

Task: Hand Calculation of Boric Acid and Primary Water Integrator Setting for Manual Makeup to the VCT

Alternate Path: None

Facility JPM #: 3-OT-JPMADA 1-7a Rev 0

K/A Rating(s):

2.1.20 Ability to interpret and execute procedure steps. (IMPORTANCE RO 4.6 SRO 4.6)
2.1.37 Knowledge of procedures, guidelines, or limitations associated with reactivity
| management. (IMPORTANCE RO 4.3 SRO 4.6)

2.1.43 Ability to use procedures to determine the effects on reactivity of plant changes, | such as reactor coolant system temperature, secondary plant, fuel depletion, etc. (IMPORTANCE RO 4.1 SRO 4.3)

TASK STANDARDS: Correctly perform Hand Calculation of Boric Acid and Primary Water Integrator Setting for Manual Makeup to the VCT

Preferred Evaluation Location:	Preferred Evaluation Method.
Simulator X In-Plant	Perform X Simulate
References: 1-SOI-62.02 Rev 0047 TI-59) Rev 0006
Task Number: RO-062-SOI-62-017,	APPLICABLE FOR: RO/SRO
<u>10CFR55.45</u> : 1, 5	
Validation Time: 15 min. Time Critical: No	
Candidate:NAME	Time Start: SSN/EIN Time Finish:
Performance Rating: SAT UNSAT	Performance Time
Examiner: NAME	SIGNATURE DATE
COM	IMENTS

SIMULATOR OPERATOR INSTRUCTIONS:

If JPM conducted in Simulator, reset to any EOL 100% IC

Tools/Equipment/Procedures Needed:

Copy of latest revision of 1-SOI-62.02 and TI-59 Calculator Access to controlled copies of plant procedures

READ TO OPERATOR

DIRECTION TO TRAINEE:

I will explain the initial conditions, and state the administrative task to be performed. I will provide initiating cues and reports on other actions when directed by you. Ensure you indicate to me when you understand your assigned task. To indicate that you have completed your assigned task return the handout sheet I provided you.

INITIAL CONDITIONS:

- The unit is in Mode 1 at 100% power.
- Current RCS C_B is 222 ppm.
- Current inservice BAT C_B is 6350 ppm
- Current B-10 Depletion value is 12 ppm.
- Currently the VCT is at 23%
- The Desired VCT Level is 40%
- You are an extra operator.

INITIATING CUES:

- Due to a suspected problem with REACTINW, the Unit SRO has directed you to perform a hand calculation of the Boric Acid and Primary Water integrator setting for a manual makeup to the VCT, in accordance with SOI-62.02
- Report the results of the surveillance to the Unit SRO.

<u>STEP 1</u> :	Obtain the necessary procedure.	
STANDARD:	Performer obtains copy of latest revision of 1-SOI-62.02. and finds section 6.5 Manual Makeup	SAT
COMMENTS:		
		UNSAT
		-
EXAMINER'S CUE:	NOTE: Provide copy of SOI 62.02 to candidate.	
STEP 2:	6.5 Manual Makeup	
	[1] PERFORM Appendix C, Calculation Of Boric Acid And Primary	SAT
STANDARD:	Performer turns to SOI 62.02 Appendix C	UNSAT
COMMENTS:	·	

		······
<u> 3TEP 3:</u>	Appendix C NOTE 1 Use page 1 of this appendix when using "VCT MAKEUP CALCULATION" program in REACTINW, otherwise use pages 2 and 3 when performing Hand Calculations.	SAT UNSAT
STANDARD:	Performer refers to iniating cues and realizes that "REACTINW" is suspect and goes to page 2 of Appendix C	
COMMENTS:		
		_
STEP 4:	2.0 HAND CALCULATION	
	[1] OBTAIN the following data for use in the HAND CALCULATION: [1.1] Current RCS Boric Acid Concentration. <u>222</u> PPM	SAT
	[1.2] Current BAT Boric Acid Concentration. <u>6350</u> PPM	LINSAT
	[1.3] B-10 Depletion Value from Reactivity Management Briefing Sheet12PPM	
	[1.4] Current VCT Level 23 %	
	[1.5] Desired VCT Level 40 %	
<u>STANDARD</u> <u>COMMENTS:</u>	Correctly obtains information from iniating cues.	

<u>STEP 5</u> :	2] CALCULATE BAT Boric Acid Concentration Ratio (BACR):	Critical Step
	6820 ppm ÷ Step 1.0[1.2] ppm = (ENTER 1.0 for BACR following a reactor shutdown or for a conservative calculation)	SAT
<u>STANDARD</u> :	Performer performs the following calculation: 6820 ppm ÷ 6350 ppm = <u>1.074</u>	UNSAT
COMMENTS:		
		-
STEP 6:	[3] CALCULATE B-10 corrected boron concentration:	Critical Step
	STEP 1.0[1.1] STEP 1.0[1.3] B-10 corrected boron	SAT
STANDARD:	Performer performs the following calculation:	
	222ppm - 12ppm = <u>210ppm</u> .	UNSAT
COMMENTS:		

<u>STEP 7</u> :	[4] CALCULATE total VCT addition volume needed for desired level.	<u>Critical Step</u>
	[%%] X 19.3 Gals/% = GALS Step 1.0[1.5] Step 1.0[1.4] Total Volume (Round to nearest whole number)	SAT
STANDARD:	Performer performs the following calculation:	UNSAT
	[% - %] X 19.3 Gals/% = <u>328.1</u> GALS	
COMMENTS:		-
STEP 8:	2.0 HAND CALCULATION (continued) NOTE Corrected Boric Acid Flow Rate in Step 2.0[5] should be obtained	<u>Critical Step</u>
	calculated in Step 2.0[3].	SAT
	GPM X GPM Corrected Boric BACR from Acid Flow Rate Step 2.0[2] [X] BA to Blender	UNSAT
STANDARD:	Performer performs the following calculation: <u>2.2</u> GPM X <u>1.074</u> <u>2.36</u> GPM Corrected Boric Acid Flow Rate Step 2.0[2] [X] BA to Blender	
COMMENTS:		





	STEP 13:	[10]	Critical Step
		X GALS GALS Step 2.0[9] [Z] Step 2.0[4] Primary Water Total Volume Batch Counter Setting	SAT
	STANDARD:	Performer performs the following calculation:	UNSAT
		0.967 Step 2.0[9] [Z]X 328.1 Step 2.0[4] Total VolumeGALS Frimary Water Batch Counter SettingPerformer can round PW counter down to 317 GALS	-
	COMMENTS:		
Sec	<u>STEP 14</u> :	[11] ENSURE independently verified by SRO.	
	<u>STANDARD</u> :	Performer reports that the task is completed and gives calculation to the SRO	SAT
	COMMENTS:		UNSAT
	Examiner cue:	Acknowledge report, notify performer that the task is complete. End task	

TIME STOP: _____

CANDIDATE CUE SHEET (TO BE RETURNED TO EXAMINER UPON COMPLETION OF TASK)

DIRECTION TO TRAINEE:

I will explain the initial conditions, and state the administrative task to be performed. I will provide initiating cues and reports on other actions when directed by you. Ensure you indicate to me when you understand your assigned task. To indicate that you have completed your assigned task return the handout sheet I provided you.

INITIAL CONDITIONS:

- The unit is in Mode 1 at 100% power.
- Current RCS C_B is 222 ppm.
- Current inservice BAT C_B is 6350 ppm
- Current B-10 Depletion value is 12 ppm.
- Currently the VCT is at 23%
- The Desired VCT Level is 40%
- You are an extra operator.

INITIATING CUES:

- Due to a suspected problem with REACTINW, the Unit SRO has directed you to perform a hand calculation of the Boric Acid and Primary Water integrator setting for a manual makeup to the VCT, in accordance with SOI-62.02
- Report the results of the surveillance to the Unit SRO.

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Watts Bar Nuclear Plant 3-OT-JPMADA 1-7b

SRO Verification of Hand Calculation of Boric Acid and Primary Water Integrator Setting for Manual Makeup to the VCT

NUCLEAR TRAINING REVISION/USAGE LOG				
Rev. #	Description of Changes	Date	Pages Affected	Reviewed By
0	Initial issue, generated for NRC Initial License Exam		All	D. Hughes

Proformed Evaluation Method

Watts Bar Nuclear Plant 3-OT-JPMADA 1-7b

Task: SRO Verification of Hand Calculation of Boric Acid and Primary Water Integrator Setting for Manual Makeup to the VCT

Alternate Path: None

3-OT-JPMADA 1-7b Rev 0 Facility JPM #:

K/A Rating(s):

2.1.20 Ability to interpret and execute procedure steps. (IMPORTANCE RO 4.6 SRO 4.6) 2.1.37 Knowledge of procedures, guidelines, or limitations associated with reactivity management. (IMPORTANCE RO 4.3 SRO 4.6) 2.1.43 Ability to use procedures to determine the effects on reactivity of plant changes, such as reactor coolant system temperature, secondary plant, fuel depletion, etc. (IMPORTANCE RO 4.1 SRO 4.3)

TASK STANDARDS: Correctly verify Hand Calculation of Boric Acid and Primary Water Integrator Setting for Manual Makeup to the VCT.. SRO finds error.

Preferred Evaluation Location:	Preferred Evaluation Method:	
Simulator In-Plant	Perform X Simulate	
<u>References</u> : 1-SOI-62.02 Rev 0047 TI-59	Rev 0006	
Task Number: SRO-113-SSP-12-013,	APPLICABLE FOR: SRO	
<u>10CFR55.45</u> : 1, 5		
Validation Time: 15 min. Time Critical: No		====
Candidate:NAME	Time Start: SSN/EIN Time Finish:	
Performance Rating: SAT UNSAT	Performance Tim	ne
Examiner:NAME	/	DATE
COM	MENTS	

SIMULATOR OPERATOR INSTRUCTIONS:

If JPM conducted in Simulator, reset to any EOL 100% IC

Tools/Equipment/Procedures Needed:

Copy of latest revision of 1-SOI-62.02 and TI-59 Calculator Access to controlled copies of plant procedures

READ TO OPERATOR

DIRECTION TO TRAINEE:

I will explain the initial conditions, and state the administrative task to be performed. I will provide initiating cues and reports on other actions when directed by you. Ensure you indicate to me when you understand your assigned task. To indicate that you have completed your assigned task return the handout sheet I provided you.

INITIAL CONDITIONS:

- The unit is in Mode 1 at 100% power.
- Current RCS C_B is 222 ppm.
- Current inservice BAT C_B is 6350 ppm
- Current B-10 Depletion value is 12 ppm.
- Currently the VCT is at 23%
- The Desired VCT Level is 40%

INITIATING CUES:

- Due to a suspected problem with REACTINW, an RO was directed to perform a hand calculation of the Boric Acid and Primary Water integrator setting for a manual makeup to the VCT, in accordance with SOI-62.02.
- The RO has reported the results of the surveillance to you, the Unit SRO.
- You are task with conducting a verification of the calculation.

START TIME: _____

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<u>STEP 1</u> :	Obtain the necessary procedure.	
STANDARD:	Performer obtains copy of latest revision of 1-SOI-62.02.	SAT
COMMENTS:		
		UNSAT
EXAMINER'S CUE:	NOTE: Provide copy of completed SOI 62.02 Appendix C to candidate.	-

<u>STEP 2:</u>		
	2.0 HAND CALCULATION	SAT
	[1] OBTAIN the following data for use in the HAND CALCULATION: [1.1] Current RCS Boric Acid Concentration. 222 PPM	
	[1.2] Current BAT Boric Acid Concentration. <u>6350</u> PPM	UNSAT
	[1.3] B-10 Depletion Value from Reactivity Management Briefing Sheet 12PPM	
	[1.4] Current VCT Level 23 %	
	[1.5] Desired VCT Level 40 %	
STANDARD	SRO Verifies the above information is correct:	
COMMENTS:		
	·	

<u>STEP 3</u> :	2] CALCULATE BAT Boric Acid Concentration Ratio (BACR):	Critical Step
	6820 ppm ÷ Step 1.0[1.2] ppm = (ENTER 1.0 for BACR following a reactor shutdown or for a conservative calculation)	SAT
<u>STANDARD</u> :	Performer performed the following calculation: 6820 ppm ÷ 6350 ppm = <u>1.074</u>	UNSAT
	SRO Verifies the above information is correct:	
COMMENTS:		-
STEP 4:	[3] CALCULATE B-10 corrected boron concentration:	<u>Critical Step</u>
	STEP 1.0[1.1] STEP 1.0[1.3] B-10 corrected boron : : :	SAT
STANDARD:	Performer performed the following calculation:	
	222ppm - 12ppm = <u>210ppm</u> . SRO Verifies the above information is correct:	UNSAT
COMMENTS:		

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<u>STEP 5</u> :	[4] CALCULATE total VCT addition volume needed for desired level.	<u>Critical Step</u>
	[%%] X 19.3 Gals/% = GALS Step 1.0[1.5] Step 1.0[1.4] Total Volume (Round to nearest whole number)	SAT
STANDARD:	Performer performed the following calculation:	UNSAT
	[<u>40</u> % – <u>23</u> %] X 19.3 Gals/% = <u>328.1</u> GALS	
	SRO Verifies the above information is correct:	-
COMMENTS:		

STEP 6:	2.0 HAND CALCULATION (continued) NOTE	<u>Critical Step</u>
	Corrected Boric Acid Flow Rate in Step 2.0[5] should be obtained from TI-59, Appendix C for the B-10 corrected boron value calculated in Step 2.0[3].	SVI
	[5]	0A1
	GPM X GPM Corrected Boric BACR from [X] BA to Blender Acid Flow Rate Step 2.0[2]	UNSAT
STANDARD:	Performer performed the following calculation:	
	2.2GPM X1.074=2.36GPMCorrected BoricBACR from Step 2.0[2][X] BA to BlenderGPM	-
	SRO Verifies the above information is correct:	
COMMENTS:		

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	STEP 9:	101	Critical Step
		Step 2.0[7] [Y] Step 2.0[4] Boric Acid Batch Total Volume Counter Setting	SAT
	STANDARD:	Performer performe the following calculation:	
		0.0326 Step 2.0[7] [Y]x 210 Step 2.0[4] Total VolumeGALS Boric Acid Batch Counter Setting	UNSAT
		SRO determines that the RO incorrectly obtained data from Step 2.0[3] (used 210). Should have used Step 2.0[4] which was 328.1.	-
		SRO calculates the correct value as follows:	
C	COMMENTS:	0.0326 Step 2.0[7] [Y]X 328.1 Step 2.0[4] Total VolumeGALS Boric Acid Batch Counter SettingSRO may round the BA up to 11 gals	



<u>STEP 12</u> :	[11] ENSURE independently verified by SRO.	
STANDARD:	Performer gives calculation back to the RO	SAT
COMMENTS:		UNSAT
Examiner cue:	Acknowledge report, notify performer that the task is complete. End task	-

TIME STOP: _____

CANDIDATE CUE SHEET (TO BE RETURNED TO EXAMINER UPON COMPLETION OF TASK)

DIRECTION TO TRAINEE:

I will explain the initial conditions, and state the administrative task to be performed. I will provide initiating cues and reports on other actions when directed by you. Ensure you indicate to me when you understand your assigned task. To indicate that you have completed your assigned task return the handout sheet I provided you.

INITIAL CONDITIONS:

- The unit is in Mode 1 at 100% power.
- Current RCS C_B is 222 ppm.
- Current inservice BAT C_B is 6350 ppm
- Current B-10 Depletion value is 12 ppm.
- Currently the VCT is at 23%
- The Desired VCT Level is 40%

INITIATING CUES:

- Due to a suspected problem with REACTINW, an RO was directed to perform a hand calculation of the Boric Acid and Primary Water integrator setting for a manual makeup to the VCT, in accordance with SOI-62.02.
- The RO has reported the results of the surveillance to you, the Unit SRO.
- You are task with conducting a verification of the calculation.

Evaluate Overtime Guidelines

NUCLEAR TRAINING REVISION/USAGE LOG				
Rev. #	Description of Changes	Date	Pages Affected	Reviewed By
0	Initial issue, generated for NRC Initial License Exam	03/21/08	All	D. Hughes

Task	Evaluate	Overtime	Guidelines
lasn.	Lvaluate	Overtime	Guidennes

Alternate Path: None

Facility JPM #: 3-OT-JPMADA 1-8 Rev 0

K/A Rating(s):

2.1.5 Ability to use procedures related to shift staffing, such as minimum crew | complement, overtime limitations, etc. (IMPORTANCE RO 2.9/ SRO 3.9)

<u>TASK STANDARDS</u>: Determine maximum hours that can be worked per request. Determine that a 12 hour shift cannot be worked on the day requested.

Preferred Evaluation Location:	Preferred Evaluation Method:
Simulator XIn-Plant	Perform X Simulate
References: SPP 1.5 Rev 0005	
Task Number: RO-119-OPDP-10.1-001,	APPLICABLE FOR: RO/SRO
<u>10CFR55.45</u> : 11	
Validation Time: 15 min. Time Critical: No	=======================================
Candidate:NAME	Time Start: SSN/EIN Time Finish:
Performance Rating: SAT UNSAT	Performance Time
Examiner: NAME	// SIGNATURE DATE
COM	MENTS

SPECIAL INSTRUCTIONS TO EVALUATOR:

a. Hand time (cue) sheet to candidate

READ TO OPERATOR

Directions to Trainee:

I will explain the initial conditions, and state the task to be performed. All control room steps shall be performed for this JPM. I will provide initiating cues and reports on other actions when directed by you. When you complete the task successfully, the objective for this job performance measure will be satisfied. Ensure you indicate to me when you understand your assigned task. To indicate that you have completed your assigned task return the handout sheet I provided you.

INITIAL CONDITIONS:

- 1. The ATC for the 0700-1900 shift has called in sick.
- 2. You are the only operator currently available , and You have been requested to work 12 hours tomorrow, 5/15/2008, from 0700-1900.

INITIATING CUES:

Determine the number of hours you could work, if any, starting at 0700 on 5/15/2008 without violating overtime guidelines. If required

		STEP/STANDARD	SAT/UNSAT
	<u>Evaluator</u> <u>Note:</u>	<i>If the performer locates a copy of SPP=1.5,</i> Overtime Restrictions (Regulatory), <i>give the performer a copy.</i>	Critical Step
	STEP 1.	Determine the number of hours that can be worked starting at 0700 on 5/15/08 without violating any overtime guidelines.	SAT UNSAT
	<u>STANDARD:</u>	May only work 6 hours of the 0700 – 1900 shift on 5/15/208. The Performer identifies that the requirements per SPP-1.5 of working more than 72 hours in 7 days will be violated if authorization per SPP- 1.5 is not approved prior to 1300 on 5/15/2008.	
-	<u>Evaluator</u> <u>Note:</u>	 This step has 2 critical components. Both of which must be satisfied to pass JPM: May only work 6 hours of the 0700 – 1900 shift without violating overtime restrictions. The Performer identifies that the requirements per SPP-1.5 of working more than 72 hours in 7 days will be violated if authorization per SPP- 1.5 is not approved prior to 1300 on 5/15/2008. If the performer only identifies the 6 hour restriction, then acknowledge report, and inform them to complete the appropriate paperwork as if they were to report 	
	<u>Comments:</u>	<i>to work at 0700. Inform them that they are the only person available.</i>	

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		STEP/STANDARD	SAT/UNSAT
nt (STEP 2.	Locates SPP-1.5, Overtime Restrictions, and fills out the following on Page 8 of the report:	SAT
		 Name SSN Organization Date # Hours Time limit will be exceeded 	UNSAT
	STANDARD:	# Hours and Time that the limit will be exceeded (6 hours – 1300 the limit will be exceeded)	
	<u>Evaluator</u> <u>Note:</u>	<i>If the performer ask for a copy of SPP=1.5,</i> Overtime Restrictions (Regulatory), <i>give the performer a copy.</i>	
	Comments:		
	STEP 3.	Specifies Reasons for cause of overtime.	SAT
	STANDARD:	Writes a reasonable reason on the form.	UNSAT
	Comments:		

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Job Performance Cr	necklist: STEP/STANDARD	SAT/UNSAT
 STEP 4.	Specifies requirements for exception and work to be performed.	Critical Step
		SAT
STANDARD:	Determines requirements per SPP-1.5 Page 8 item 2 and checks the 72 hours in 7 days block on Appendix A.	UNSAT
<u>Evaluator</u> <u>Note:</u>	This step is the second of the 2 critical components for this JPM (which may have been performed in step one of this JPM) :	
	• The Performer identifies that the requirements per SPP-1.5 of working more than 72 hours in 7 days will be violated if authorization per SPP- 1.5 is not approved prior to 1300 on 5/15/2008.	
<u>Evaluator Que:</u>	Shift Clerk will complete the remainder of SPP-1.5	
<u>Comments:</u>		
<u>, , , , , , , , , , , , , , , , , , , </u>	1	

End of JPM

CANDIDATE COPY TO BE RETURNED TO EXAMINER UPON COMPLETION OF ANSWER

Initial Conditions:

You are a Reactor Operator that has worked the following schedule:

Date	Hours	Status	Notes	
5/7/08	0700-1715	Normal Day Off	Call out to cover 10 hours of a 12 hour shift (15 minute turnover)	
5/8/08	1500-1915	Normal Day Off	Call out to cover 4 hour shift (15 minute turnover)	
5/9/08	0700-1700	Normal Day Off	Attended Simulator and Classroom JIT Training	
5/10/08	0700-1915	Normal Work Day	15 minute turnover	
5/11/08	0700-1915	Normal Work Day	15 minute turnover	
5/12/08	0700-1915	Normal Work Day	15 minute turnover	
5/13/08	0700-1915	Normal Work Day	15 minute turnover	
5/14/08	1100-1900	Normal Day Off	Attended Simulator and Classroom JIT Training	

The OAC for the 0700-1900 shift has called in sick.

You are the only operator currently available , and you have been requested to work 12 hours tomorrow, 5/15/2008 from 0700-1900.

INITIATING CUES:

Determine the number of hours you could work, if any, starting at 0700 on 5/15/2008 without violating overtime guidelines.

Hand Calculation of RCS Inventory Balance

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Rev 0

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Watts Bar Nuclear Plant 3-OT-JPMADA 2-3

NUCLEAR TRAINING REVISION/USAGE LOG					
 Rev. #	Description of Changes	Date	Pages Affected	Reviewed By	
Rev. #	Description of Changes Initial issue, generated for NRC Initial License Exam	Date	Pages Affected All	Reviewed By D. Hughes	

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Rev	0
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Task: I	Hand Calculation	of RCS	Inventory	Balance.
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Alternate Path: None

Facility JPM #: 3-OT-JPMADA 2-3 Rev 0

K/A Rating(s):

2.2.12 Knowledge of surveillance procedures. | (IMPORTANCE RO 3.7 SRO 4.1)

TASK STANDARDS: Correctly perform Hand Calculation of RCS Inventory Balance

TASK STANDARDS: Correctly perform Hand Ca	Iculation of RCS Inventory Dalance				
Preferred Evaluation Location:	Preferred Evaluation Method:				
Simulator X In-Plant	Perform XSimulate				
References: 1-SI-68-32 Rev 0012 TI-4 Part II	Rev 0012				
Task Number: RO-068-SI-68-32-001,	APPLICABLE FOR: RO/SRO				
<u>10CFR55.45</u> : 11					
Validation Time: 35 min. Time Critical: No					
Candidate:NAME	Time Start: SSN/EIN Time Finish:				
Performance Rating: SAT UNSAT	- Performance Time				
Examiner: NAME	/				
COMMENTS					
Watts Bar Nuclear Plant 3-OT-JPMADA 2-3

SIMULATOR OPERATOR INSTRUCTIONS:

none

Tools/Equipment/Procedures Needed:

Marked up copy of of latest revision of 1-SI-68-32 (see next page for data to put in 1-SI-68-32) Copy of TI-4 Part II. Steam Tables Calculator Access to controlled copies of plant procedures

READ TO OPERATOR

DIRECTION TO TRAINEE:

I will explain the initial conditions, and state the administrative task to be performed. I will provide initiating cues and reports on other actions when directed by you. Ensure you indicate to me when you understand your assigned task. To indicate that you have completed your assigned task return the handout sheet I provided you.

INITIAL CONDITIONS:

- The unit is in Mode 1 at 100% power.
- The Plant Computer program RCSWIB is unreliable.
- 1-SI-68-32 Appendix A has been completed through step 4 (2nd Data Set has been taken)
- You are an extra operator.

INITIATING CUES:

- Due to a suspected problem with RCSWIB, the Unit SRO has directed you to perform a manual calculation of RCS inventory balance.
- Report the results of the surveillance to the Unit SRO.

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Watts Bar Nuclear Plant 3-OT-JPMADA 2-3

Mark-Up instructions for 1-SI-68-32

- 1. Starting in section 4.1 initial/N/A steps as required. Fill in the following data at step 6:
 - [6] RECORD the most recent Steam Generator Leakage obtained from Chemistry in the appropriate column below , and CALCULATE Total SG Leakage (in gpm) as follows: ⇒ 0 gpd
- Continue to initial steps and/or fill in Handswitch/valve postions for all steps in sections 4.2, 4.3, and 6.0 (up to step 6.18). Ensure steps 6.5 thru 6.18 are marked N/A, since Appendix "A" will be performed.
- 3. Appendix "A" initial steps 1 thru 4.
- 4. Appendix "B" fill in the following data,(N/A all remaining data points):

	Dered	Initial Data Set	2nd Data Set
Parameter	Panel		0012
Time	N/A	0800	0912
VCT LEVEL	L0112A	31.5%	30.2%
PRESSURIZER	PO480A	2235.3 psig	2235.3 psig
PRESSURIZER	L0482A	60.24%	59.55%
RCS	T0460A	586.3 °F	586.3 °F
REACTOR	1-NI-41	100%	100%
PRT LEVEL	LO485A	72.6%	72.6%
RCDT LEVEL	1LI-77-1 0-PNL-276-L2	23%	23%
DATA SET RECORDED BY	N/A	Initials	Initials

START TIME: _____

	EXAMINER'S CUE:	Provide a marked up copy of 1-SI-68-32 to candidate.	
	STEP 1:	CALCULATE RCS leakage per Appendix C.4	SAT
(x_i, x_i)	<u>STANDARD:</u> COMMENTS:	Performer turns to 1-SI-68-32 Appendix C	UNSAT

 <u>STEP 3:</u>	[1] CALCULA ΔT = FINAL TI ΔT =	TE elapsed time (ΔT) a IME - INITIAL TIME	as follows:	<u>Critical Step</u> SAT
<u>STANDARD:</u> COMMENTS:	Performer det	termines that ∆T is 72 i	minutes	UNSAT
<u>STEP 4:</u>	[2] CONVERT measure to ps	INITIAL and FINAL PZR ia as follows: NITIAL =psig + 14.7 =psia	FINAL =psig + 14.7 =psia	SAT
<u>STANDARD</u> COMMENTS:	Performer com Pressure to be	rectly calculates Initial P2 e 2250 psia	ZR Pressure and final PZR	

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	3] CALCULATE VCT Leakage as follows:	Critical Step
<u>SIEP 5</u> :	VCT Leakage = (INITIAL VCT LEVEL - FINAL VCT LEVEL) x 19.27 gal/%	
	∆T (Step [1])	
	VCT Leakage <u>= (31.5% - 30.2 %) x 19.27 gal/%</u> 72 min	SAT
STANDARD:	Performer performs the following calculation:	UNSAT
	VCT Leakage = 0.3479 gpm	
COMMENTS:		

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					Critical Step
<u>STEP 6</u> :	 [4] CALCULATE Pressurizer (PZR) Leakage as follows: [a] RECORD INITIAL and FINAL PZR PRESSURE in tablebelow using values calculated in Step [2]. [b] RECORD INITIAL and FINAL PZR LEVEL in table below using values recorded in Appendix B. [c] DETERMINE and RECORD INITIAL and FINAL PZR Vf (volume of water) and Vg (volume of steam) at INITIAL and FINAL PZR PRESS (Step [2]) using ASME Steam Tables (Table 2, Properties of Saturated Steam and Saturated Water). 				SAT UNSAT
	STEP	PARAM	INITIAL	FINAL	
	[a]	PZR PRESS	psia	psia	
	[b]	PZR LEVEL	%	%	-
	[0]	PZR V _f	ft ³ /lbm	ft³/lbm	
	[C]	PZR Vg	ft ³ /lbm	ft ³ /lbm	
<u>STANDARD</u> :	Perform PZR V _f (calcula PZR V _g (calcula	ner determin is between ated value 0. g is between ated value 0.	es that: 0.026 ft ³ /lbm and 0 02698 ft ³ /lbm) 0.156 ft ³ /lbm and 0 .157025 ft ³ /lbm)	0.027 ft ³ /lbm 0.158 ft ³ /lbm	
COMMENTS:					
	<u>STEP 6</u> : <u>STANDARD</u> : <u>COMMENTS:</u>	STEP 6: [4] CALC [a] R ta [b] R [c] D (() a T STEP [a] [b] [c] STANDARD: Perform PZR V _f (calcula PZR V _g (calcula	STEP 6: [4] CALCULATE Press [a] RECORD INITI tablebelow usin [b] RECORD INITI using values registry [c] DETERMINE a (volume of wat and FINAL PZI Tables (Table staturated Wat STEP PARAM [a] PZR PRESS [b] PZR LEVEL [c] PZR V; [c] Staturated value 0. PZR V; Staturated value 0. PZR V; Staturated value 0. COMMENTS: Staturated value 0.	STEP 6: [4] CALCULATE Pressurizer (PZR) Leakage [a] RECORD INITIAL and FINAL PZR I tablebelow using values calculated i [b] RECORD INITIAL and FINAL PZR L using values recorded in Appendix B [c] DETERMINE and RECORD INITIAL (volume of water) and Vg (volume or and FINAL PZR PRESS (Step [2]) U Tables (Table 2, Properties of Satur Saturated Water). STEP [b] PZR PRESS [c] PZR V: [c] [c	STEP 6: [4] CALCULATE Pressurizer (PZR) Leakage as follows: [a] RECORD INITIAL and FINAL PZR PRESSURE in tablebelow using values calculated in Step [2]. [b] RECORD INITIAL and FINAL PZR RESSURE in table below using values recorded in Appendix B. [c] DETERMINE and RECORD INITIAL and FINAL PZR Vf (volume of water) and Vg (volume of steam) at INITIAL and FINAL PZR PRESS (Step [2]) using ASME Steam Tables (Table 2, Properties of Saturated Steam and Saturated Water). STEP PARAM INITIAL FINAL [a] PZR PRESS psia psia [b] PZR LEVEL % % [c] PZR V; ft*/Ibm ft*/Ibm [c] PZR V; ft*/Ibm ft*/Ibm [b] PZR LEVEL % % [c] PZR V; ft*/Ibm ft*/Ibm [c] PZR V; ft*/Ibm ft*/Ibm [c] PZR V; ft*/Ibm ft*/Ibm [c] PZR V; _ft*/Ibm _ft*/Ibm [c] PZR V; _ft*/Ibm _ft*/Ibm [c] PZR V; _ft*/Ibm _ft*/Ibm [c] PZR V; is between 0.026 ft*/Ibm and 0.027 ft*/Ibm _ft*/Ibm [c] PZR Vg is between 0.156 ft*/Ibm and 0.158 ft*/Ibm _ft _ft [c] COMMENTS:

~			Critical Step
	<u>STEP 7</u> :	 [4] CALCULATE Pressurizer (PZR) Leakage as follows:4 (Continued) [d] CALCULATE INITIAL PZR VOLUME as follows: 	
		INITIAL = (PZR LVL + 100 - PZR LVL) x 16.73 ft3/% x 7.4805 gal/ft3 x 0.01605 ft3/lbm PZR Vf Vg	SAT
		INITIAL = $(60.24 + 100 - 60.24) \times 2$ gal fts/% lbm PZR Vf Vg VOLUME	UNSAT
	<u>STANDARD:</u>	Performer performs the following calculation and determines that: Initial Pzr Volume is 4971.946 gal	
	COMMENTS:		

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	T		Critical Step
$\langle \cdot \rangle$	<u>STEP 8:</u>	[4] CALCULATE Pressurizer (PZR) Leakage as follows:4	
		[e] CALCULATE INITIAL PZR VOLUME as follows:	
		FINAL = (PZR LVL + 100 - PZR LVL) x 16.73 ft3/% x 7.4805 gal/ft3 x 0.01605 ft3/lbm	SAT
		pzr Vf Vg	
		VOLOME	
		FINAL = $(59.55 + 100 - 59.55) \times 2$ gal ft ₃ /% lbm	UNSAT
		VOLUME	
		Durfamor performs the following calculation and	-
	<u>STANDARD:</u>	determines that:	
		Initial Pzr Volume is 4929.5856 gal	
C.	COMMENTS:		
C			

**Italicized Cues Are To Be Used Only If JPM Performance Is Being Simulated.

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 ز	STEP 9:	[4] CALCULATE Pressurizer (PZR) Leakage as follows:4	<u>Critical Step</u>
		[F] CALCULATE PZR Leakage as follows:	SAT
		(Step [d]) (Step [e]) PZR Leakage = INITIAL - FINAL <u>PZR VOLUME</u> PZR VOLUME ∆T(Step [1])	
		PZR Leakage = <u>gal - gal</u> min	UNSAT
		PZR Leakage = gpm	
	STANDARD:	4971.946198 gal - 4929.5856 gal / 72 min	-
		Performer determines that the PZR Leakage is 0.588 gpm:	
	COMMENTS:		
	<u>STEP 10</u> :	[5] IF RCS TEMP FINAL is NOT equal to RCS TEMP INITIAL ⁴ , THEN	SAT
	STANDARD:	Performer determines from data that RCS Temperature did not change therefore step 5 is N/A	
			UNSAT
	COMMENTS:		
August 12			

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**Italicized Cues Are To Be Used Only If JPM Performance Is Being Simulated.

_۲	STEP 11.		Critical Step
		[6] CALCULATE Total RCS Leakage as follows:	
		(Step [3]) (Step [4]) (Step [5] or 0) Total RCS Leakage = VCT Leakage + PZR Leakage + Temp Corr Total RCS Leakage = gpm + gpm + gpm	SAT
		Total RCS Leakage = gpm	UNSAT
	STANDARD:	Performer performs the following calculation:	
		.3479 gpm + .588 gpm + 0 gpm = 0.9359 gpm	-*
	COMMENTS:		
C			

**Italicized Cues Are To Be Used Only If JPM Performance Is Being Simulated.

 ` 	<u>STEP 12</u> :	[7] CALCUL as follow	ATE Leakage to Pressurizer Relief Tank (PRT) s:	
		[a] (CONVERT INITIAL and FINAL PRT LEVEL from percent (%) to gallons using TI-4,Part II, and	SAT
		I	RECORD the converted PRT LEVELS in Step [b].	
		[b]	CALCULATE Leakage to PRT using the following equation:	
		1	Leakage = <u>FINAL PRT LVL - INITIAL PRT LVL</u> To PRT∆T (Step [1])	0N3A1
		!	Leakage = <u>(galgal)</u> To PRT min	
			Leakage = gpm To PRT	
				-
	STANDARD:	Performer d change the	letermines that since the PRT level did not Leakage to the PRT is 0 gpm.	
hidaaa sa ta				
	COMMENTS:			

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	<u>STEP 13</u> :	 [8] CALCULATE Leakage to Reactor Coolant Drain Tank (RCDT) as follows: [a] CONVERT INITIAL and FINAL RCDT LEVEL from percent (%) to gallons using TI-4, Part II, and RECORD the converted RCDT LEVELS in Step [b]. [b] CALCULATE Leakage to RCDT using the following equation: Leakage = <u>FINAL RCDT LVL - INITIAL RCDT LVL</u> To RCDT <u>ΔT (Step [1])</u> Leakage = (<u>gal - gal)</u> To RCDT _ min Leakage = gpm To RCDT 	SAT UNSAT
	<u>STANDARD</u> :	Performer determines that since the RCDT level did not change the Leakage to the RCDT is 0 gpm.	
C	COMMENTS:		

STED 11.		<u>Critical Step</u>
<u>91L1 14</u> .	[9] CALCULATE Identified Leakage as follows:	
	(Section 4.1 (Step [7]) (Step [8]) Step [6]) Identified = Leakage + Leakage + Total SG Leakage To PRT to RCDT Leakage Identified =gpm +gpm +gpm Leakage	SAT UNSAT
	Identified =gpm (Acc Crit: ≤ 10 gpm)	
<u>STANDARD</u> :	Performer determines that Identified Leakage is 0 gpm.	-
COMMENTS:		
~~~^`		

[]		Critical Stop
<u>STEP 15</u> :	[10] CALCULATE Unidentified Leakage as follows:	<u>onical Step</u>
	(Step [6]) (Step [9]) Unidentified = Total RCS Leakage - Identified Leakage Leakage Unidentified = gpm gpm Leakage	SAT
	Unidentified = gpm (Acc Crit: ≤ 1 gpm)	UNSAT
STANDARD	Performer performs the following calculation:	
STANDARD.	r enormer performs the following baloaidation.	-
	0.9359 gpm + 0 gpm = 0.9359 gpm	

**Italicized Cues Are To Be Used Only If JPM Performance Is Being Simulated.

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<u>STEP 16</u> :	ENSURE independently verified.	SAT
<u>STANDARD</u> :	Performer reports that the task is completed and gives calculation to the SRO for independent verification.	
COMMENTS:		UNSAT
Examiner cue:	Acknowledge report, notify performer that the task is complete. End task	
		-

TIME STOP: _____

**Italicized Cues Are To Be Used Only If JPM Performance Is Being Simulated.

#### CANDIDATE CUE SHEET (TO BE RETURNED TO EXAMINER UPON COMPLETION OF TASK)

#### **DIRECTION TO TRAINEE:**

I will explain the initial conditions, and state the administrative task to be performed. I will provide initiating cues and reports on other actions when directed by you. Ensure you indicate to me when you understand your assigned task. To indicate that you have completed your assigned task return the handout sheet I provided you.

#### **INITIAL CONDITIONS:**

- The unit is in Mode 1 at 100% power.
- The Plant Computer program RCSWIB is unreliable.
- 1-SI-68-32 Appendix A has been completed through step 4 (2nd Data Set has been taken)
- You are an extra operator.

#### **INITIATING CUES:**

- Due to a suspected problem with RCSWIB, the Unit SRO has directed you to perform a manual calculation of RCS inventory balance.
- Report the results of the surveillance to the Unit SRO.

### WATTS BAR NUCLEAR PLANT 3-OT-JPMADA.3-1

TASK TITLE: DETERMINE IF PERSONNEL WILL EXCEED ADMINISTRATIVE DOSE LIMITS WHILE PERFORMING TASK

Rev. 1

# WATTS BAR NUCLEAR PLANT 3-OT-JPMADA.3-1

### NUCLEAR TRAINING REVISION/USAGE LOG

Rev #	Date	Description of changes	Pages Affected	Reviewed By
0		Initial Issue, NRC JPM exam question. Does not need to be word processed.	All	H.J. Voiles
1	03/04/08	Added the option for another route to the task area. Corrected K/A references to comply with revision 2 supplement 1 of NUREG 1122	All -	D.L.Hughes
	•			

#### REGION II INITIAL LICENSE EXAMINATION ADMINISTRATIVE JOB PERFORMANCE MEASURE

# Task: DETERMINE IF PERSONNEL WILL EXCEED ADMINISTRATIVE DOSE LIMITS WHILE PERFORMING TASK

Alternate Path: N/A

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- Facility JPM #: 3-OT-JPMADA3-1
- K/A Rating(s): 2.3.7 [3.5/3.6]

<u>Task Standard:</u> Ability to comply with radiation work permit requirements during normal or abnormal conditions.

Preferred Evaluation Location:	Preferred Evaluation Method:
Simulator In-Plant	Perform X Simulate
References: SPP 5.1 Rev 0006 Radiological C	ontrols
Task Number:: AUO-119-SSP-5.01-001	APLICABLE FOR: AUO/RO/SRO
10CFR55.45: 10	
Validation Time: 15 min. Time Critical: No	
Applicant:NAME	Time Start: SSN Time Finish:
Performance Rating: SAT UNSAT	Performance Time
Examiner: NAME	/SIGNATURE DATE
COM	MENTS

#### Tools/Equipment/Procedures Needed:

Calculator

# NOTE TO EVALUATOR: Hand out the Applicant Data Sheet along with the Applicant's Cue Sheet.

#### READ TO OPERATOR

#### **DIRECTION TO TRAINEE:**

I will explain the initial conditions, and state the administrative task to be performed. I will provide initiating cues and reports on other actions when directed by you. Ensure you indicate to me when you understand your assigned task. To indicate that you have completed your assigned task return the handout sheet I provided you.

#### INITIAL CONDITIONS:

The unit had experienced a valid safety injection signal.

The unit had also experienced a loss of both Unit 1 shutdown boards and RCP Seal Injection and thermal barrier cooling have been isolated per ECA-0.0 Appendix "A".

Shutdown Board 1A has been re-energized and control room dispatches you to locally start 1A TBBP and throttle open the discharge valve.

A survey map is available for elevation 737 of the auxiliary building, showing dose rates and travel times to reach the TBBP Discharge valve that you will throttle open.

Three NAUO's are available for assignment to perform this task. All 3 NAUOs have a current NRC FORM-4 documenting current year and previous years exposures.

RADCON personnel are currently unavailable to provide assistance.

#### INITIATING CUES:

You have been directed to determine the NAUO (s) that could perform task without exceeding NORMAL administrative limits (EPIP 15 Emergency Exposure Guidelines is not to be considered):

- NAUO #1 has TEDE of 631 mRem
- NAUO #2 has TEDE of 603 mRem
- NAUO #3 has TEDE of 613 mRem

START TIME: _____

NOTE TO EV	ALUATOR: The following steps may be performed out of order.	
<u>STEP 1</u> :	Calculate Exposure during pump start and throttle valve operation.	Critical Task
STANDARD:	(1.2 R/hr)(1000MR/R)(1HR/60Min)(10 Min) = 200 MR	SAT
EVALUATOR	QUE: If the operator locates the current revision of SPP 5.1 Radiological Controls, a copy may be given to him/her.	UNSAT
COMMENTS:		-
<u>STEP 2</u> :	. Calculate the exposure getting to task area and return.	
STANDARD:	Dose received for 2 way travel for survey area "B" :	SAT
	(2.7 R/hr)(1000MR/R)(1HR/60Min)(4 Min) = 180 MR	
	Dose received for 2 way travel for survey area "C" :	UNSAT
	(4.0 R/hr)(1000MR/R)(1HR/60Min)(3 Min) = 200 MR	
	Determines that Route "B" is the route to take.	
COMMENTS:		

**Italicized Cues Are To Be Used Only If JPM Performance Is Being Simulated.

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	Rev. 1	3-UT-JPMADA.3-1
STEP 3:	Calculate total dose for job.	Critical Task
STANDARD: D d 1 2	Dose received for 2 way travel for survey area "B" are added to lose received in Survey Area "A": ) Two way Survey Area B = 180 MR 2) Dose in Survey Area A = 200 MR	SAT
COMMENTS	() = 101a1  DOSE = 200  Mix + 100  Mix = 300  Mix	UNSAT
<u>COMMENTO</u> .		
<u>STEP 5</u> : D ir	Determine total dose that would be received for all NAUO.s nvolved.	-
STANDARD: F 1 2 3	Performer Determines dose that would be received by NAUO's. NAUO #1: 631 mrem + 380 mrem = 1011 mrem NAUO #2: 603 mrem + 380 mrem = 983 mrem NAUO #3: 613 mrem + 380 mrem = 993 mrem	SAT
COMMENTS:		
<u>STEP 5</u> : D A	Determines which NAUO can perform tasks without exceeding Administrative limits.	Critical Task
<u>STANDARD</u> : F	Performer determines NAUOs #2 and #3 are the only NAUOs that could perform task.	SAT
COMMENTS:	· · · · ·	UNSAT
	END OF TASK	

TIME STOP: _____

**Italicized Cues Are To Be Used Only If JPM Performance Is Being Simulated.

#### APPLICANT CUE SHEET (TO BE RETURNED TO EXAMINER UPON COMPLETION OF TASK)

#### **DIRECTION TO TRAINEE:**

I will explain the initial conditions, and state the administrative task to be performed. I will provide initiating cues and reports on other actions when directed by you. Ensure you indicate to me when you understand your assigned task. To indicate that you have completed your assigned task return the handout sheet I provided you.

#### **INITIAL CONDITIONS:**

The unit had experienced a valid safety injection signal.

The unit had also experienced a loss of both Unit 1 shutdown boards and RCP Seal Injection and thermal barrier cooling have been isolated per ECA-0.0 Appendix "A".

Shutdown Board 1A has been re-energized and control room dispatches you to locally start 1A TBBP and throttle open the discharge valve.

A survey map is available for elevation 737 of the auxiliary building, showing dose rates and travel times to reach the TBBP Discharge valve that you will throttle open.

Three NAUO's are available for assignment to perform this task. All 3 NAUOs have a current NRC FORM-4 documenting current year and previous years exposures.

RADCON personnel are currently unavailable to provide assistance.

#### INITIATING CUES:

You have been directed to determine the NAUO (s) that could perform task without exceeding NORMAL administrative limits (EPIP 15 Emergency Exposure Guideline is not to be considered):

- NAUO #1 has TEDE of 631 mRem
- NAUO #2 has TEDE of 603 mRem
- NAUO #3 has TEDE of 613 mRem

#### APPLICANT DATA SHEET (TO BE RETURNED TO EXAMINER UPON COMPLETION OF TASK)

### SURVEY DATA:

- TBBP control switches and pump discharge FCV are located in Survey area "A".
- Estimated time at control switch and discharge valve is 10 minutes.
- ONE-way travel time to Valve & control switches via route "B" is 2 minutes.
- ONE-way travel time to Valve & control switches via route "C" is 1 minute 30 seconds.
- General Area Dose rate for Survey Area "A" is 1.2 R/hr.
- General Area Dose rate for Survey Area "B" is 2.7 R/hr.
- General Area Dose rate for Survey Area "C" is 4.0 R/hr.
  - NAUO #1 has TEDE of 631 mRem
  - NAUO #2 has TEDE of 603 mRem
  - NAUO #3 has TEDE of 613 mRem

### **RESULTS**:

Indicate below by checking yes or no if the respective NAUO can be allowed to perform task without exceeding administrative dose limits.

	YES	NO
NAUO #1		
NAUO #2		
NAUO #3		



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REV. 0

### WATTS BAR NUCLEAR PLANT 3-OT-JPMS081b

TASK TITLE: CLASSIFY THE EVENT PER THE REP (LOCA WITH SIGNIFICANT FUEL FAILURE AND POTENTIAL LOSS OF CONTAINMENT) REV. 0

PAGE 2 0F 17

# WATTS BAR NUCLEAR PLANT 3-OT-JPMS081b

#### NUCLEAR TRAINING REVISION/USAGE LOG

REV	# Date	Description of Changes	Pages Affected	Reviewed By
0	03/05/08	Initial issue, generated for NRC Initial License Exam	ALL	D.L.Hughes
			-	
				<u> </u>

#### EVALUATION SHEET

Task: Classify The Event Per The REP (LOCA WITH SIGNIFICANT FUEL FAILURE AND POTENTIAL LOSS OF CONTAINMENT)

Alternate Path: N/A

Facility JPM #: 3-OT-JPMS081b

K/A Rating(s): 2.4.29 Knowledge of the emergency plan. IMPORTANCE RO 3.1 SRO 4.4

Task Standard: The event is classified as an GENERAL EMERGENCY based on "Loss Of Any Two Barriers" And A Potential Loss Of Third Barrier" Notifications are made per EPIP-5.

Preferred Evaluation Location:	Preferred Evaluation Method:
Simulator X In-Plant	Perform X Simulate This JPM will be simulated.
<b><u>References</u></b> : EPIP-1 "Emergency Plan Classification EMERGENCY", Rev. 34	Flowpath", Rev. 27; EPIP-5 "GENERAL
Task Number: SRO-113-EPIP-001	APPLICABLE FOR: SRO
<u>10CFR55.45</u> : 10, 11, 12	
Validation Time: 20 min. Time Critical: Yes	
Applicant:NAME	Time Start: SSN/EIN Time Finish:
Performance Rating: SAT UNSAT	Performance Time
Examiner:	/
COMM	IENTS

EVALUATOR INFORMATION SHEET

TASK TITLE: CLASSIFY THE EVENT PER THE REP (LOCA WITH SIGNIFICANT FUEL FAILURE AND POTENTIAL LOSS OF CONTAINMENT)

#### SIMULATOR SETUP INSTRUCTIONS:

NONE: THIS JPM IS SIMULATED and the simulator will not be used to reflect the conditions.

#### REQUIRED MATERIALS:

EPIP-1 and EPIP-5

SAFETY CONSIDERATIONS:

None

### Tools/Equipment/Procedures Needed:

Ensure clean copy of EPIP-1 and EPIP-5 in all copies of Emergency Instructions on the Simulator Floor and in the file drawer of Unit Supervisor's desk.

Print out Met Tower Cue Sheets on following 3 pages. The Met Tower cue sheets are to be given to the performer when they are obtaining information from the Integrated Computer System (ICS).

### SIMULATOR OPERATOR INSTRUCTIONS:

- 1. Obtain copy of EPIP-5 to facilitate role plays for contacts made by the performer.
- 2. Performer must use NRC ring down phone.





05-MAK-2008 ID:14						LAST HOUR: 0.00 IN	SOLAR	RADIATION*: 0.85 LANGLEY		DIFFUSER POND ELEVATION: 700.99 FEET		RIVER FLOM: 4877 CFS	STARTI ITY CI ASS: D		MET-TOMER LINK IS UP		* - Hourly Averages	
Metdata >	DLOGICAL DATA 15 MIN. AVG.	AIR TEMPERATURE: 42.59 DEGF	VEC WIND DIR: 70 DEG	VEC WIND SPEED: 4.9 MI/HR	AVG WIND SPEED: 5.0 MI/HR	HORIZ SIG-THETA: 16 DEG	AIR TEMPERATURE: 43.45 DEGF	VEC WIND DIR: 73 DEG	VEC WIND SPEED: 4.6 MI/HR	AVG WIND SPEED: 4.8 MI/HR	HORIZ SIG-THETA: 17 DEG	ATR TEMPERATURE: 44.56 DEGF	VEC WIND DIR: 80 DEG	VEC WIND SPEED: 4.1 MI/HR	AVG MIND SPEED: 4.4 MI/HR	HORIZ SIG-THETA; 24 DEG	10 METER DEW POINT*: 30.57 DEGF	
SELECT FUNC. KEY OR TURN-ON CODE	METEOR								46 METER									

Ensure clean copy of EPIP-1 and EPIP-5 in all copies of Emergency Instructions on the Simulator Floor and in the file drawer of Unit Supervisor's desk.

#### READ TO OPERATOR

#### **DIRECTION TO TRAINEE:**

I will explain the initial conditions, and state the task to be performed. All control room steps shall be performed for this JPM, including any required communications. I will provide initiating cues and reports on other actions when directed by you. Ensure you indicate to me when you understand your assigned task. To indicate that you have completed your assigned task return the handout sheet I provided you.

### THE CURRENT SIMULATOR CONDITIONS ARE NOT REPRESENTATIVE OF THIS JPM.

#### **INITIAL CONDITIONS:**

- Unit 1 was at 100% RTP when a LOCA occurred.
- An SI occurred due to high containment pressure.
- RCS pressure stabilized at 350 psig saturated conditions.
- The CLAs are injecting.
- The ONLY ECCS pump in service is the 1B-B RHR pump, all other ECCS pumps tripped shortly after the SI.
- Containment pressure is 1.8 psig and stable.
- 1-RM-90-273 & 1-RM-90-274 have risen to 35 rem/hr.
- 1-RM-90-271 & 1-RM-90-272 have risen to 60 rem/hr.
- Containment Hydrogen concentration is 6% as indicated on the H₂ analyzers.
- → The STA reports a RED path condition exists on core cooling.
  - You are the SM/SRO

#### **INITIATING CUES:**

The Unit Supervisor has informed you of the above conditions.

The operators are taking actions per the emergency instructions (currently transitioning from E-1 to FR-C.1).

You are to make the initial classification of the event per the REP EPIPs and make required notifications.

Portions of this JPM are time critical.

REV. 0

# WATTS BAR NUCLEAR PLANT 3-OT-JPMS081b

<u>STEP 1</u> :	Refers to EPIP-1 to determine level of event.	CRITICAL STEP
<u>STANDARD</u> :	Performer refers to EPIP-1 and declares a GENERAL EMERGENCY based on "Loss Of Any Two Barriers And A Potential Loss Of Third Barrier" (1.1.1 Loss, 1.2.2 Loss, 1.3.2 Potential Loss) This must be completed within <b>15</b> <b>minutes</b> of task assignment.	SAT
	This step is critical to ensure proper activation of TVA resources for event in progress.	UNSA
COMMENTS:	RECORD time that declaration was made:	-
<u>STEP 2</u> :	Implements EPIP-5, GENERAL EMERGENCY	SAT
STANDARD:	EPIP-5, GENERAL EMERGENCY, is implemented.	
<u>COMMENTS</u> :		UNS

<u>STEP 3</u> : <b>[1]</b>	<ul> <li>IF the onsite emergency centers are not staffed, THEN DIRECT Shift Personnel to activate the Emergency Paging System (EPS) to staff the Technical Support Center (TSC) and Operations Support Center (OSC). Shift Personnel should confirm activation and provide the 20 minute printed report to the SM for review.</li> <li>A IF the EPS system fails, call the ODS, ringdown or (5- 751-1700) and DIRECT him to activate the EPS.</li> <li>B IF the above methods of activating the EPS fail, THEN DIRECT Shift Personnel to use the Watts Bar Nuclear Plant Emergency Response Call-List to staff the TSC and OSC. (This list is located in the EPS Manual near the terminal.)</li> </ul>	CRITICAL STEP
STANDARD:	The SRO activates the emergency paging system (EPS) to staff the TSC and Operations Support Center (OSC) or EPS is activated in the control room. This step is critical to ensure proper activation of TVA resources for event in progress.	
**CUE	After the shift personnel are notified, acknowledge the request to activate the Emergency Paging System.	
<u>COMMENTS</u>		

<ul> <li>STEP 4: [2] IF the TSC has <u>not</u> been activated, THEN</li> <li>a. INITIATE Appendix A and B, Initial Notification FORM for GENERAL EMERGENCY and Protective Action Recommendations.</li> </ul>	CRITICAL STEP
STANDARD: Appendix A is accurately completed with 1. This is a Drill 2. Their name, Shift Manager (SED) at WBN Plant.	SAT
<ol> <li>General Emergency declared on UNIT 1</li> <li>EAL Designators: [LOSS 1.1.1 and LOSS 1.2.2, and Potential LOSS 1.3.2]</li> <li>Brief description of incident: [RCS LOCA with a Loss of Subcooled Margin, Loss of Core Cooling and High Hydrogen concentration in Containment] (or something similar)</li> <li>Radiological Conditions [Either Release information not known or Minor releases within federally approval limits]</li> <li>Event Declared: [Time and Date]</li> <li>Wind direction at 46 meters, coming from [73] degrees</li> </ol>	UNSAT
Evaluator Cue (1): The Met Tower cue sheets are to be given to the performer when they are obtaining information from the Integrated Computer System (ICS).	
<u>Evaluator Cue (2)</u> : When release data addressed, state "Release data not available for Appendix B."	
<ul> <li>NOTE TO EVALUATOR: Items 2, 3, 6, and 7 above are the critical parts.</li> <li>Items 2 &amp; 3 are critical due to correct EALs must be used for General Emergency declaration.</li> <li>Item 6 is critical and it must be within 15 minutes of the performer stating that they understand their task and the JPM is started.</li> <li>Item 7 is criticalto ensure proper protective actions are recommended.</li> </ul>	
<u>COMMENTS</u> :	
### REV. 0

# WATTS BAR NUCLEAR PLANT 3-OT-JPMS081b

6			
	STEP 5:	Appendix B:	<u>Critical Step</u>
	<u>STANDARD:</u>	Operator determines from page 15, logic chart in EPIP-5, that appropriate protective action recommendation is <b>RECOMMENDATION 2</b> . This should be identified on the notification form in the next JPM step.	SAT
	<u>COMMENTS</u>		UNSAT
	STEP 6:	Appendix A completion:	Critical Step
			SAT
	STANDARD:	Appendix A is accurately completed with:	
		Recommendation 2 is checked	UNSAT
		69-110 A-1, B-1, C-1, D-1, A-3, D-2, -4, -5 is checked	
		(Both blocks should be checked)	
	COMMENTS		

## WATTS BAR NUCLEAR PLANT 3-OT-JPMS081b

<u>STEP 7</u> :	<ul> <li>[2.b] NOTIFY the ODS direct by ODS Ring-down or by USING 5-751-1700 or 5-751-2495 and</li> <li>1) PROVIDE the information from Appendix A.</li> <li>2) FAX Appendix A to the ODS. (# pre-programmed or 5-751-8620).</li> </ul>	CRITICAL STEP
STANDARD:	The ODS is notified and provided the information on Appendix A. This notification must be made within <b>10</b> <b>minutes</b> of event declaration. Appendix A is placed in the fax machine and transmission simulated.	UNSAT
EVALUATOR		
**CUE:	(Booth Operator) Role play as the ODS and repeat back the report.	
**CUE:	Do NOT allow fax to be actually transmitted.	
COMMENTS:	RECORD time that ODS was notified:	

<u>STEP 8</u> : <b>[2.c]</b>	<ul> <li>IF the ODS CANNOT be contacted within <u>10 minutes</u>, THEN</li> <li>1) NOTIFY Rhea County, Meigs County, McMinn County, and the Tennessee Emergency Management Agency (TEMA) of the Classification USING the contact information in Appendix E.</li> <li>2) FAX Appendix A to TEMA at 9-1-615-242-9635.</li> </ul>	SAT UNSAT
STANDARD:	Performer N/As step since ODS was previously notified.	
COMMENTS:		
<u>STEP 9</u> : <b>[2.d</b>	<b>ANNOUNCE</b> to the crew: "A GENERAL EMERGENCY is being declared based on Loss of Two Barriers and Potential Loss Of A Third Barrier. I will be the Site Emergency Director."	CRITICAL STEP SAT
<u>STANDARD</u> :	The above announcement is made to the crew. Wording describing the event may vary. Step is critical to alert crew to the declaration of the GENERAL EMERGENCY and provide for tracking personnel.	UNSAT
**CUE:	After the crew is notified, acknowledge the report.	
COMMENTS:		

# WATTS BAR NUCLEAR PLANT 3-OT-JPMS081b

<u>STEP 10</u> : <b>[2.e]</b>	<ul> <li>TRACK dispatched personnel by name, and PERFORM one of the following:</li> <li>IF OSC is <u>not</u> staffed, THEN INFORM Maintenance Shift Supervisor of names for team tracking.</li> <li>WHEN OSC is staffed, THEN INFORM OSC manager of names for team tracking.</li> </ul>	SAT UNSAT
<u>STANDARD</u> :	Maintenance Shift Supervisor is notified of names for team tracking.	
**CUE: **CUE:	(As Crew Member) If asked, State that Control Bldg AUO John Doe was dispatched to evaluate 1B 6.9 KV Shutdown Board. (Booth Operator) Role play as the Maintenance Shift Supervisor or OSC manager and repeat back the report.	
COMMENTS:		
<u>STEP 11</u> : <b>[5]</b>	ANNOUNCE to the plant: "ATTENTION ALL SITE PERSONNEL. ATTENTION ALL SITE PERSONNEL. A GENERAL EMERGENCY has been declared based on Loss of two barriers and potential loss of a third barrier. Staff the TSC and OSC (Repeat)	CRITICAL STEP
<u>STANDARD</u> :	A public address announcement for Plant Emergency Response Personnel to staff the TSC and OSC is made over the plant paging system. Wording describing the event may vary. Step is critical to alert plant to the declaration of the GENERAL EMERGENCY.	UNSAT
COMMENTS:		

### REV. 0

## WATTS BAR NUCLEAR PLANT 3-OT-JPMS081b

STEP 12: [6] INITIATE WBN EPIP-8, Personnel Accountability and Evacuation, Appendix D.	SAT
EVALUATOR CUE: When the performer starts to refer to EPIP-8, inform him/her that another SRO will continue on from here. Notify performer that the task is complete.	UNSAT
End of task	
<u>COMMENTS</u> :	

TIME STOP: _____

## PERFORMER HANDOUT SHEET (TO BE RETURNED TO EXAMINER UPON COMPLETION OF TASK)

### **DIRECTION TO TRAINEE:**

I will explain the initial conditions, and state the task to be performed. All control room steps shall be performed for this JPM, including any required communications. I will provide initiating cues and reports on other actions when directed by you. Ensure you indicate to me when you understand your assigned task. To indicate that you have completed your assigned task return the handout sheet I provided you.

### THE CURRENT SIMULATOR CONDITIONS ARE NOT REPRESENTATIVE OF THIS JPM.

## **INITIAL CONDITIONS:**

- Unit 1 was at 100% RTP when a LOCA occurred.
- An SI occurred due to high containment pressure.
- RCS pressure stabilized at 350 psig saturated conditions.
- The CLAs are injecting.
- The ONLY ECCS pump in service is the 1B-B RHR pump, all other ECCS pumps tripped shortly after the SI.
- Containment pressure is 1.8 psig and stable.
- 1-RM-90-273 & 1-RM-90-274 have risen to 35 rem/hr.
- 1-RM-90-271 & 1-RM-90-272 have risen to 60 rem/hr.
- Containment Hydrogen concentration is 6% as indicated on the H2 analyzers.
- The STA reports a RED path condition exists on core cooling.
- You are the SM/SRO

## **INITIATING CUES:**

The Unit Supervisor has informed you of the above conditions.

The operators are taking actions per the emergency instructions (currently transitioning from E-1 to FR-C.1).

You are to make the initial classification of the event per the REP EPIPs and make required notifications.

Portions of this JPM are time critical.