

August 24, 2010

NRC 2010-0122
10 CFR 50.90

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Units 1 and 2
Dockets 50-266 and 50-301
Renewed License Nos. DPR-24 and DPR-27

License Amendment Request 261
Extended Power Uprate
Response to Request for Additional Information

- References:
- (1) FPL Energy Point Beach, LLC letter to NRC, dated April 7, 2009, License Amendment Request 261, Extended Power Uprate (ML091250564)
 - (2) Public Meeting between NextEra Energy Point Beach, LLC and NRC Regarding AST, EPU, AFW and Non-Conservative Setpoint Amendments, July 22, 2010 (ML102180030)

NextEra Energy Point Beach, LLC (NextEra) submitted License Amendment Request (LAR) 261 (Reference 1) to the NRC pursuant to 10 CFR 50.90. The proposed amendment would increase each unit's licensed thermal power level from 1540 megawatts thermal (MWt) to 1800 MWt, and revise the Technical Specifications to support operation at the increased thermal power level.

Via Reference (2), the NRC staff determined that additional information is required to enable the staff's continued review of the request. Enclosure 1 provides the NextEra response to the NRC staff's request for additional information.

This letter contains no new Regulatory Commitments and no revisions to existing Regulatory Commitments.

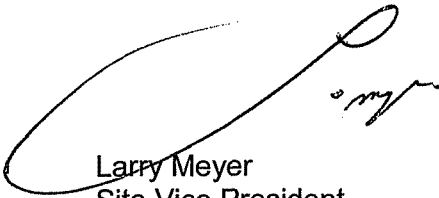
The information contained in this letter does not alter the no significant hazards consideration contained in Reference (1) and continues to satisfy the criteria of 10 CFR 51.22 for categorical exclusion from the requirements of an environmental assessment.

In accordance with 10 CFR 50.91, a copy of this letter is being provided to the designated Wisconsin Official.

I declare under penalty of perjury that the foregoing is true and correct.
Executed on August 24, 2010.

Very truly yours,

NextEra Energy Point Beach, LLC

A handwritten signature in black ink, appearing to read "Larry Meyer", is written over a large, stylized, handwritten "L" that spans across the signature text.

Larry Meyer
Site Vice President

Enclosure

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
PSCW

ENCLOSURE 1

NEXTERA ENERGY POINT BEACH, LLC POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

LICENSE AMENDMENT REQUEST 261 EXTENDED POWER UPRATE RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION

The NRC staff determined that additional information was required (Reference 1) to enable the Balance of Plant Branch to complete its review of License Amendment Request (LAR) 261, Extended Power Uprate (EPU) (Reference 2). The following information is provided by NextEra Energy Point Beach, LLC (NextEra) in response to the NRC staff's request.

Question 1

With reference to SBPB RAI 2.5-6, please describe procedural changes, if any, for cooldown to maximum RHR temperature conditions. How during a normal cooldown is RHR temperature controlled by Operations?

NextEra Response

With regard to the NextEra response to SBPB RAI 2.5-6 (Reference 3), the cooldown to maximum RHR temperature conditions is controlled in Operations procedure OP 3C, Hot Standby to Cold Shutdown. No changes to the method for this cooldown are required to support EPU conditions. During a normal cooldown, once the reactor coolant system (RCS) temperature is less than 350°F and pressure is between 275 and 325 psig, then the residual heat removal (RHR) system is placed in operation. The RCS cooldown rate is controlled by modulating the RHR heat exchanger outlet and bypass valves, while controlling the component cooling water (CCW) temperatures. The CCW supply header temperature is normally maintained between 90 - 105°F, but maximum supply header temperature is allowed to reach 125°F for two hours during RHR cooldown. Evaluations of the RHR and CCW demonstrate that the systems continue to support normal and design basis accident/event cooling requirements at EPU conditions.

Question 2

With reference to SBPB RAI 2.5-7, in the dynamic evaluation for FWIV closure, why is the feedwater pump trip assumed at the same time?

NextEra Response

As stated in the NextEra response to SBPB RAI 2.5-7 (Reference 3), two scenarios were analyzed for the feedwater pipe stress analysis at EPU conditions:

- Case 1: Existing feedwater regulating valves close during steady-state operation, while the main feedwater pumps continue to operate.
- Case 2: Newly added main feedwater isolation valves close during steady-state operation, while the main feedwater pumps trip at the same time.

The above two cases were selected to bound the possible valve fast closure scenarios. A loss of air or power to the feedwater regulating valves (FRVs) will cause them to fail closed and these valves stroke faster than the feedwater isolation valves (FIVs). The FIVs fail as-is on loss of power or air. Therefore, the first case represents a worst-case flow cessation transient due to spurious valve actuation.

By design, a feedwater isolation signal (initiated by a safety injection signal) causes the FIVs and FRVs to close, and the main feedwater pumps to trip. Postulating a single failure of a main feedwater pump to trip during a feedwater isolation, the dynamic effects are bounded by the analysis of the faster acting FRVs closing with the main feedwater pumps still operating.

References

- (1) Public Meeting between NextEra Energy Point Beach, LLC and NRC Regarding AST, EPU, AFW and Non-Conservative Setpoint Amendments, July 22, 2010, (ML102180030)
- (2) FPL Energy Point Beach, LLC letter to NRC, dated April 7, 2009, License Amendment Request 261, Extended Power Uprate (ML091250564)
- (3) NextEra Energy Point Beach, LLC letter to NRC, dated June 24, 2010, License Amendment Request 261, Extended Power Uprate, Response to Request for Additional Information (ML101760119)