

August 26, 2010

MEMORANDUM TO: Shana R. Helton, Chief
Rulemaking and International Projects Branch
Division of Policy and Rulemaking
Office of Nuclear Reactor Regulation

FROM: Tara Inverso, Project Manager **/RA/**
Rulemaking and International Projects Branch
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Office of Nuclear Reactor Regulation

SUBJECT: SUMMARY OF THE PUBLIC MEETING TO DISCUSS PLANT
SPECIFIC EMERGENCY CORE COOLING SYSTEM
PERFORMANCE ASSESSMENTS AND STATUS OF TITLE 10
OF THE *CODE OF FEDERAL REGULATIONS* SECTION
50.46(b) RULEMAKING

The U.S. Nuclear Regulatory Commission (NRC) held a Category 3 public meeting on August 12, 2010 to discuss the industry's project plan to provide plant specific Emergency Core Cooling System (ECCS) performance assessments to the NRC, and to provide an update on the status of Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.46(b) ECCS acceptance criteria rulemaking. The workshop was held in Rockville, MD at NRC headquarters. The meeting was attended by 40 individuals. Additional individuals participated in the meeting through audio teleconferencing and webinar. Since the NRC was not accepting public comments at the meeting, a transcript of the meeting was not obtained.

The major areas of discussion are summarized as follows:

1. 50.46(b) Rulemaking: Background, Status, and Related Information Request

The NRC staff gave an introductory presentation on the status of the 50.46(b) rulemaking project, and discussed its plans to provide the proposed rule package to the Commission in April, 2011. This schedule would allow draft regulatory guidance to be published for comment at the same time the proposed rule is published. The staff also discussed its plan to create three Regulatory Guides to:

- 1) Establish an acceptable experimental procedure for conducting post quench ductility (PQD) testing and developing analytical limits;
- 2) Establish an acceptable experimental procedure for conducting breakaway oxidation testing and developing analytical limits; and
- 3) Provide an acceptable set of PQD analytical limits.

2. Industry Plan to Assess Clad Performance Margins for Loss-Of-Coolant Accidents (LOCA)

During the previous April 28-29th 50.46(b) public workshop, the NRC staff presented the “Elements of a Prospective Information Request.” The purpose of the information request would be to confirm interim plant-specific safety in light of recent research data which discovered previously unknown embrittlement mechanisms. Since that workshop, the industry has been working on a plan to provide that information through an alternate means. Industry representatives presented the industry’s plan at this public meeting.

The plan to provide clad performance margin assessments was developed with the joint participation of the Nuclear Energy Institute, a Regulatory Technical Advisory Committee, the Pressurized Water Reactor Owners Group, the Boiling Water Reactor Owners Group, and fuel vendors (Areva, General Electric Hitachi/Global Nuclear Fuels, and Westinghouse). The plan overview, including a milestone schedule, can be found in the presentation “Industry Plan to Assess Clad Performance Margins for LOCA.” (ADAMS Accession No. ML102250415).

3. Summary of Follow-On Discussion

Following the industry’s presentation, the NRC staff provided feedback on the presented plan. The NRC staff indicated that it was encouraged by the information presented at the public meeting, but would need to see details of the provided information as the plan unfolds. The NRC staff and industry discussed conducting another public meeting in November or December to review a detailed example report.

Enclosure:

1. Attendee List

2. Industry Plan to Assess Clad Performance Margins for Loss-Of-Coolant Accidents (LOCA)

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1. Attendee List

DISTRIBUTION:

PUBLIC

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Charles Ader

Kathy Gibson

Timothy McGinty

William Ruland

ADAMS Accession No: ML102380149 (Pkg.); ML102360406 (Summary); ML102220473 (Presentation); ML102220487 (Handouts); ML102250415 (Industry Presentation)

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OFFICIAL RECORD COPY

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ENCLOSURE

