



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 27, 2010

LICENSEE: Pacific Gas and Electric Company

FACILITY: Diablo Canyon Nuclear Power Plant, Units 1 and 2

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON AUGUST 17, 2010, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND PACIFIC GAS AND ELECTRIC COMPANY CONCERNING DRAFT REQUEST FOR ADDITIONAL INFORMATION RELATED TO THE DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2, LICENSE RENEWAL APPLICATION - METAL FATIGUE

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Pacific Gas and Electric Company (PG&E) held a telephone conference call on August 17, 2010, to obtain clarification on the staff's draft request for additional information (D-RAI) regarding the Diablo Canyon Nuclear Power Plant license renewal application (LRA).

By e-mail dated August 4, 2010, the staff sent D-RAIs to PG&E regarding metal fatigue time limited aging analysis review. PG&E reviewed the information contained therein, and requested a telephone conference call. The telephone conference call was useful in clarifying the intent of the staff's D-RAIs. Enclosure 1 provides a listing of the participants. Enclosure 2 provides discussions on D-RAIs for which the applicant requested clarification. No changes to other D-RAIs were necessary as a result of this telephone conference call. Formal RAIs will be issued by a separate letter.

The applicant had an opportunity to comment on this summary.

A handwritten signature in black ink, appearing to read "N. Ferrer", is positioned above the typed name.

Nathaniel Ferrer, Safety Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket Nos. 50-275 and 50-323

Enclosures:
As stated

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**TELEPHONE CONFERENCE CALL
DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION**

**LIST OF PARTICIPANTS
AUGUST 17, 2010**

PARTICIPANTS

Nate Ferrer
James Medoff
Todd Mintz
Philippe Soenen
Michelle Albright
Kevin Braico
Dave Kunsemiller
Brett Lynch

AFFILIATIONS

U.S. Nuclear Regulatory Commission (NRC)
NRC
Center for Nuclear Waste Regulatory Analyses
Pacific Gas and Electric Company (PG&E)
PG&E
PG&E
Strategic Teaming And Resource Sharing (STARS)
STARS

Diablo Canyon Nuclear Power Plant, Units 1 and 2
License Renewal Application
Draft Request for Additional Information
Metal Fatigue

D-RAI 4.3-8

Background:

In the license renewal application (LRA) Table 4.3-1, the applicant credits the "Global" monitoring (i.e., cycle count monitoring) of aging management program (AMP) B3.1 as the 10 CFR 54.21(c)(1)(iii) aging management monitoring basis for dispositioning the cumulative usage factor (CUF) analyses for the reactor pressure vessel (RPV) Core Support Pads, Pressurizer Spray Nozzle, and Pressurizer Heater Penetration.

Issue:

The LRA Table 4.3-1 or LRA Table 4.3-6 indicated that the RPV Core Support Pads, Pressurizer Spray Nozzle, and Pressurizer Heater Penetration in Unit 1 have a maximum limiting design basis CUFs of ~0.89, ~0.95, and ~0.94 respectively and limiting 60-year projected CUFs of ~1.07, ~1.14, and ~2.97.

Request:

Justify your basis using the "Global" monitoring method of AMP B3.1 to monitor these components during the period of extended operation in accordance with 10 CFR 54.21(c)(1)(iii), and why it would not be more appropriate to monitor for these components using the cycle based fatigue (CBF) monitoring method.

Discussion: Based on discussion with the applicant, the staff determined that the Unit 1 pressurizer heater penetration 60-year projected CUF value was 0.9391, not 2.97, as indicated in LRA Table 4.3-6. The staff will revise the question as follows. The revised question will be sent as a formal RAI.

RAI 4.3-8

Background:

In the LRA Table 4.3-1, the applicant credits the "Global" monitoring (i.e. cycle count monitoring) of AMP B3.1 as the 10 CFR 54.21(c)(1)(iii) aging management monitoring basis for dispositioning the CUF analyses for the RPV Core Support Pads, Pressurizer Spray Nozzle, and Pressurizer Heater Penetration.

Issue:

The LRA Table 4.3-1 or LRA Table 4.3-6 indicated that the RPV Core Support Pads, Pressurizer Spray Nozzle, and Pressurizer Heater Penetration in Unit 1 have a maximum limiting design basis CUFs of ~0.89, ~0.95, and ~0.94 respectively and limiting 60-year projected CUFs of ~1.07, ~1.14, and ~0.9391.

Request:

Justify your basis for using the "Global" monitoring method of AMP B3.1 to monitor these components during the period of extended operation in accordance with 10 CFR 54.21(c)(1)(iii), and why it would not be more appropriate to monitor for these components using the CBF monitoring method.

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The applicant had an opportunity to comment on this summary.

/RA/

Nathaniel Ferrer, Safety Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket Nos. 50-275 and 50-323

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DATE	08/25/10	08/25/10	08/26/10	08/27/10

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Letter to John Conway from Nathaniel B. Ferrer dated August 27, 2010

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