

Attachment 3

License Application Change Pages

ABB INC.

**COMBUSTION ENGINEERING SITE
Windsor, CT.**

**Application for Amendment
US NRC License Number 06-00217-06
Docket Number 030-03754**

August 6, 2010

NRC License No. 06-00217-06

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10.0 Technical Requirements

10.1 Exposure Control and Monitoring

Personnel dose control and monitoring for licensed activities is described in Sections 10.1.1 and 10.1.2. Monitoring shall be performed for individuals likely to receive in excess of 10% of the applicable limit in 10 CFR 20. Work restrictions shall be implemented if an individual reaches 50% of the applicable limit.

The licensee will survey applicable portions of the facility and maintain contamination levels in accordance with the survey frequencies contained in written procedures. Release of equipment and materials from restricted areas shall be in accordance with the NRC's "Guidelines for Decontamination of Facilities and Equipment Prior to Release for Unrestricted Use or Termination of Licenses for Byproduct, Source, or Special Nuclear Material," dated April, 1993. The building DCGLs for the CE Windsor Site are uranium 20,148 dpm/100cm² (total alpha plus beta) and Co-60 6,980 dpm/100 cm² (total beta) as accepted under Decommissioning Plan Revision 1. The DCGLs for the CE Windsor Site for soils and sediments are:

Uranium – 557 pCi/g total uranium	Thorium – 4.0 pCi/g Th-232
Reactor Byproduct Material – 5.0 pCi/g Co-60	Radium – 4.5 pCi/g Ra-226

Radiological surveys are conducted on a routine basis, both in and adjacent to restricted areas. The type, location and frequency of surveys shall be specified in written procedures. Radiation and contamination surveys are performed only by qualified personnel, with qualification as determined by the RSO.

10.1.1 External Dose Monitoring

Personnel monitoring devices (e.g., TLDs; extremity badges) are used as determined by the RSO. If used, they shall be processed at least quarterly by a vendor accredited by NVLAP.

10.1.2 Internal Dose Monitoring

In vivo and/or in-vitro monitoring may be performed, upon direction of the RSO, to assess and monitor workers' internal dose. Such monitoring may include body/lung counts, bioassay monitoring and/or breathing zone air sampling.

10.2 Environmental Monitoring

10.2.1 Environmental Monitoring, Sampling and Counting

The Site Radiation Protection Program includes environmental monitoring to demonstrate compliance with 10 CFR 20.1301. Samples obtained from surface and well water, sediment, and soil are analyzed for the presence of radioactive materials. Sample location and frequency is specified in written procedures, or at the direction of the RSO.