



August 10, 2010

Docket No. 50-443

Docket No. 72-63

SBK-L-10118

ATTN: Document Control Desk  
Director, Spent Fuel Project Office,  
Office of Nuclear Material Safety and Safeguards  
Nuclear Regulatory Commission  
Washington, DC 20555-0001


Seabrook Station  
10 CFR 72.48 Report of Changes, Tests, and Experiments

10 CFR 72.48(d)(2) requires that a report containing a brief description of changes, tests, and experiments, including a summary of the evaluation of each be submitted at intervals not to exceed 24 months. The enclosure provides a summary report of changes, tests, and experiments performed during the time period of August 1, 2008 through July 31, 2010.

Should you have any questions regarding this submittal, please contact me at (603) 773-7745.

Sincerely,

NextEra Energy Seabrook, LLC

  
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Michael O'Keefe  
Licensing Manager

cc:

Regional Administrator, Region I  
Project Manager, Project Directorate I-2  
Seabrook Senior Resident Inspector

*NIMSSD*

**Enclosure to SBK-L-10118**

Summary Report of Facility Changes, Tests, and Experiments  
Completed in Accordance with the Requirements of 10 CFR 72.48  
for the Time Period August 1, 2008 through July 31, 2010

10CFR72.48 Evaluation 08-001

Revision to 10 CFR 72.212 Evaluation  
Report Dry Fuel Storage Manual  
(DFSM) Chapter 2

**Summary Description and Purpose:** The evaluated activity involved administrative corrections to the 10 CFR 72.212 Evaluation Report (Dry Fuel Storage Manual (DFSM) Chapter 2). The changes included the renumbering of tables and correcting the radiological classification of the Dry Fuel Storage (DFS) facility from a Radiologically Controlled Area to a Restricted Area in accordance with the Radiation Protection Manual.

**Evaluation Summary:** The 10 CFR 72.48 evaluation performed for this change concluded that the change was administrative in nature and did not affect the function of any structures, systems or components (SSC). Therefore, there was no increase to the probability or consequences of any accident or malfunction, nor was the possibility of a different accident or malfunction created. There was no impact to any design basis limit for a fission product barrier and no evaluation methodology was involved. The evaluation determined that the corrections could be implemented without prior NRC approval.