## ES-401 PWR Examination Outline Form ES-401-2

Facility: Diab	lo Canyon	Rea	ctor	Opei	ator						Date	e of E	Exam:	Dece	mber 2	2009		
					F	RO K	/A C	ateg	ory I	Point	s				SR	O-On	ly Poin	ts
Tier	Group	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	,	<b>\</b> 2	C	3*	Total
1.	1	3	2	3				4	4			2	18					6
Emergency & Abnormal	2	1	2	2		N/A	N/A		1	N.	/A	2	9					4
Plant Evolutions	nt Tior Totals		4	5			5	5			4	27					10	
	1	3	3	3	4	2	3	2	2	2	2	2	28					5
2. Plant	2	1	1	0	1	1	1	1	1	1	1	1	10					3
Systems	Tier Totals					3	4	3	3	3	3	3	38					8
3. Generic k	3. Generic Knowledge and Abilities				5   3		-	2		3		4	10	1	2	3	4	7
	Categories				2		,	3	,	3	- 2	2						

Note:

- 1. Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two).
- 2. The point total for each group and tier in the proposed outline must match that specified in the table.

  The final point total for each group and tier may deviate by ±1 from that specified in the table based on NRC revisions.

  The final RO exam must total 75 points and the SRO-only exam must total 25 points.
- 3. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems/evolutions that are not included on the outline should be added. Refer to Section D.1.b of ES-401 for guidance regarding the elimination of inappropriate K/A statements.
- 4. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.
- 5. Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.
- 6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.
- 7.\* The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. Refer to Section D.1.b of ES-401 for the applicable K/As.
- 8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IRs) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above; if fuel handling equipment is sampled in other than Category A2 or G\* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2 (Note #1 does not apply). Use duplicate pages for RO and SRO-only exams.
- 9. For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43.

ES-401 Emergend	y ar	nd A	bno	PW rma	/R Ex I Plar	amin nt Evo	ation Outline Foolutions - Tier 1/Group 1 (RO / SRO)	orm ES	-401-2
E/APE # / Name / Safety Function	K 1		K 3	A 1	A 2	G	K/A Topic(s)	IR	#
000007 (BW/E02&E10 CE/E02) Reactor Trip - Stabilization - Recovery / 1									
000008 Pressurizer Vapor Space Accident / 3	х						AK 1.02 Knowledge of the operational implications of the following concepts as they apply to a Pressurizer Vapor Space Accident: Change in leak rate with change in pressure	3.1	39
000009 Small Break LOCA / 3		х					EK 2.03 Knowledge of the interrelations between the small break LOCA and the following: S/Gs	3.0	40
000011 Large Break LOCA / 3			х				EK 3.14 Knowledge of the reasons for the following responses as they apply to the Large Break LOCA: RCP tripping requirement	4.1	41
000015/17 RCP Malfunctions / 4				х			AA1.23 Ability to operate and/or monitor the following as they apply to the Reactor Coolant Pump Malfunctions (Loss of RC Flow): RCP vibrations	3.1	42
000022 Loss of Rx Coolant Makeup / 2					х		AA2.01 Ability to determine and interpret the following as they apply to the Loss of Reactor Coolant Makeup: Whether charging line leak exists	3.2	43
000025 Loss of RHR System / 4						х	2.4.8 Emergency Procedures/Plan: Knowledge of how abnormal operating procedures are used in conjunction with EOPs.	3.8	44
000026 Loss of Component Cooling Water / 8					x		AA202 Ability to determine and interpret the following as they apply to the Loss of Component Cooling Water: The cause of possible CCW loss	2.9	45
000027 Pressurizer Pressure Control System Malfunction / 3									
000029 ATWS / 1				x			EA1.08 Ability to operate and monitor the following as they apply to a ATWS: Reactor trip switch pushbutton	4.5	46
000038 Steam Gen. Tube Rupture / 3			Х				EK3.01 Knowledge of the reasons for the following responses as they apply to the SGTR: Equalizing pressure on primary and secondary sides of ruptured S/G	4.1	47
000040 (BW/E05; CE/E05; W/E12) Steam Line Rupture - Excessive Heat Transfer / 4		х					AK2.02 Knowledge of the interrelations between the Steam Line Rupture and the following: Sensors and detectors	2.6	48
000054 (CE/E06) Loss of Main Feedwater / 4									
000055 Station Blackout / 6	х						EK1.01 Knowledge of the operational implications of the following concepts as they apply to the Station Blackout: Effect of battery discharge rates on capacity	3.3	49
000056 Loss of Off-site Power / 6	х						AK1.04 Knowledge of the operational implications of the following concepts as they apply to Loss of Offsite Power: Definition of saturation conditions, implication for the systems	3.1	50

K/A Category Totals:	3	2	3	4	4	2	Group Point Total:		18/6
000077 Generator Voltage and Electric Grid Disturbances / 6				X			AA1.02 Ability to operate and/or monitor the following as they apply to Generator Voltage and Electric Grid Disturbances: Turbine/generator controls	3.8	56
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4					х		EA2.1 Ability to determine and interpret the following as they apply to the (Loss of Secondary Heat Sink): Facility conditions and selection of appropriate procedures during abnormal and emergency operations.	3.4	55
W/E11 Loss of Emergency Coolant Recirc. / 4						х	2.1.30 Conduct of Operations: Ability to locate and operate components, including local controls	4.4	54
W/E04 LOCA Outside Containment / 3							reactor if instrument air pressure is decreasing		
000065 Loss of Instrument Air / 8					х		AA2.06 Ability to determine and interpret the following as they apply to the Loss of Instrument Air: When to trip reactor if instrument air pressure is decreasing	3.6	53
000062 Loss of Nuclear Svc Water / 4				х			AA1.02 Ability to operate and/or monitor the following as they apply to the Loss of Nuclear Service Water (SWS): Loads on the SWS in the control room	3.2	52
000058 Loss of DC Power / 6									
000057 Loss of Vital AC Inst. Bus / 6			х				AK3.01 Knowledge of the reasons for the following responses as they apply to the Loss of Vital AC Instrument Bus: Actions contained in EOP for loss of vital ac electrical instrument bus	4.1	51

ES-401 Emergency and Abn						_	utline Fo	orm ES	s-401-2
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
000001 Continuous Rod Withdrawal / 1									
000003 Dropped Control Rod / 1									
000005 Inoperable/Stuck Control Rod / 1									
000024 Emergency Boration / 1									
000028 Pressurizer Level Malfunction / 2						х	2.1.20 Conduct of Operations: Ability to interpret and execute procedure steps.	4.6	57
000032 Loss of Source Range NI / 7									
000033 Loss of Intermediate Range NI / 7									
000036 (BW/A08) Fuel Handling Accident / 8					х		AA2.01 Ability to determine and interpret the following as they apply to the Fuel Handling Incidents: ARM system indications	3.2	58
000037 Steam Generator Tube Leak / 3									
000051 Loss of Condenser Vacuum / 4				х			AA1.04 Ability to operate and/or monitor the following as they apply to the Loss of Condenser Vacuum: Rod position	2.5	59
000059 Accidental Liquid RadWaste Rel. / 9									
000060 Accidental Gaseous Radwaste Rel. / 9									
000061 ARM System Alarms / 7			х				AK3.02 Knowledge of the reasons for the following responses as they apply to the Area Radiation Monitoring (ARM) System Alarms: Guidance contained in alarm response for ARM system	3.4	60
000067 Plant Fire On-site / 8									
000068 (BW/A06) Control Room Evac. / 8		X					AK2.02 Knowledge of the interrelations between the Control Room Evacuation and the following: Reactor trip system	3.7	61
000069 (W/E14) Loss of CTMT Integrity / 5									
000074 (W/E06&E07) Inad. Core Cooling / 4							-		
000076 High Reactor Coolant Activity / 9							-		
W/EO1 & E02 Rediagnosis & SI Termination / 3	х						EK1.1 Knowledge of the operational implications of the following concepts as they apply to the (Reactor Trip or Safety Injection/Rediagnosis): components, capacity, and function of emergency systems.	3.1	62
N/E13 Steam Generator Over-pressure / 4		Х					EK2.1 Knowledge of the interrelations between the (Steam Generator Overpressure) and the following: Components, and functions of control and safety systems, including instrumentation, signals, interlocks, failure modes, and automatic and manual features.	3.0	63

W/E15 Containment Flooding / 5			x				EK3.4 Knowledge of the reasons for the following responses as they apply to the (Containment Flooding): RO or SRO function within the control room team as appropriate to the assigned position, in such a way that procedures are adhered to and the limitations in the facilities license and amendments are not violated.	2.9	64
W/E16 High Containment Radiation / 9						X	2.4.50 Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.	4.2	65
BW/A01 Plant Runback / 1									
BW/A02&A03 Loss of NNI-X/Y / 7									
BW/A04 Turbine Trip / 4									
BW/A05 Emergency Diesel Actuation / 6									
BW/A07 Flooding / 8									
BW/E03 Inadequate Subcooling Margin / 4									
BW/E08; W/E03 LOCA Cooldown - Depress. / 4									
BW/E09; CE/A13; W/E09&E10 Natural Circ. / 4									
BW/E13&E14 EOP Rules and Enclosures									
CE/A11; W/E08 RCS Overcooling - PTS / 4									
CE/A16 Excess RCS Leakage / 2									
CE/E09 Functional Recovery									
K/A Category Point Totals:	1	2	2	1	1	2	Group Point Total:		9/4

ES-401				Plar	nt S						Outlin p 1 (l	re RO / SRO)	orm ES	S-401-2
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
003 Reactor Coolant Pump	х											K1.04 Knowledge of the physical connections and/or cause-effect relationships between the RCPS (Reactor Coolant Pump System) and the following systems: CVCS		1
004 Chemical and Volume Control		х										K2.03 Knowledge of bus power supplies to the following: Charging pumps	3.3	2
005 Residual Heat Removal			Х									K3.01 Knowledge of the effect that a loss or malfunction of the RHRS will have on the following: RCS		3
006 Emergency Core Cooling				х								K4.13 Knowledge of ECCS design feature(s) and/or interlock(s) which provide for the following: Reset of containment isolation	3.8	4
007 Pressurizer Relief/Quench Tank				х								K4.01 Knowledge of PRTS design feature(s) and/or interlock(s) which provide for the following: Quench Tank Cooling	2.6	5
008 Component Cooling Water							х					A1.01 Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the CCWS controls including: CCW flow rate	2.8	6
010 Pressurizer Pressure Control					X	х						K6.03 Knowledge of the effect of a loss or malfunction of the following will have on the PZR PCS: PZR sprays and heaters	3.2	7
												K5.01 Knowledge of the operational implications of the following concepts as they apply to the PZR PCS: Determination of condition of fluid in PZR, using steam tables	3.5	8
012 Reactor Protection								x				A2.06 Ability to (a) predict the impacts of the following malfunctions or operations on the RPS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Failure of RPS signal to trip the reactor	4.4	9
013 Engineered Safety Features Actuation			х						х			K3.01 Knowledge of the effect that a loss or malfunction of the ESFAS will have on the following: Fuel	4.4	10
												A3.01 Ability to monitor automatic operation of the ESFAS including: Input channels and logic		11
022 Containment Cooling										х		A4.05 Ability to manually operate and/or monitor in the control room: Containment readings of temperature, pressure, and humidity system		12
225 Ice Condenser												humidity system		

	T			$\overline{}$	$\overline{}$				$\overline{}$				$\overline{}$	
026 Containment Spray				X							х	K4.06 Knowledge of CSS design features and/or interlock(s) which provide for the following: Iodine scavenging via the CSS	2.8	13
												2.2.37 Ability to determine operability and/or availability of safety related equipment	3.6	14
039 Main and Reheat Steam	Х										х	2.1.28 Knowledge of the purpose and function of major system components and controls.	4.1	15
												K1.01 Knowledge of the physical connections and/or cause-effect relationships between the MRSS and the following systems: S/G	3.1	16
059 Main Feedwater										х		A4.11 ability to manually operate and monitor in the control room: Recovery from automatic feedwater isolation	3.1	17
061 Auxiliary/Emergency Feedwater						X			X			K6.02 Knowledge of the effect of a loss or malfunction of the following will have on the AFW components: Pumps	2.6	18
												A3.01 Ability to monitor automatic operation of the AFW including: AFW startup and flow.	4.1	19
062 AC Electrical Distribution								x				A2.09 Ability to (a) predict the impacts of the following malfunctions or operations on the ac distribution system; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:  Consequences of exceeding current limitations	2.7	20
063 DC Electrical Distribution							х					A1.01 Ability to predict and/or monitor changes in parameters associated with operating the DC electrical system controls including: Battery capacity as it is affected by discharge rate.	2.5	21
064 Emergency Diesel Generator						X						K6.07 Knowledge of the effect of a loss or malfunction of the following will have on the ED/G system: Air receivers	2.7	22
073 Process Radiation Monitoring					x							K5.01 Knowledge of the operational implications as they apply to concepts as they apply to the PRM system: Radiation theory, including sources, types, units, and effects	2.5	23
076 Service Water		х		X								K4.06 Knowledge of SWS design feature(s) and/or interlock(s) which provide for the following: Service water train separation	2.8	24
												K2.01 Knowledge of bus power supplies to the following: Service water	2.7	25
078 Instrument Air		х	х									K2.02 Knowledge of bus power supplies to the following: Emergency air compressor	3.3	26
												K3.01 Knowledge of the effect that a loss or malfunction of the IAS will have on the following: Containment air system	3.1	27

103 Containment	Х											K1.08 Knowledge of the physical connections and/or cause-effect relationships between the containment system and the following systems: SIS, including action of safety injection reset	28
K/A Category Point Totals:	3	3	3	4	2	3	2	2	2	2	2	Group Point Total:	28/5

ES-401				Plar	nt S							tline Fc 2 (RO / SRO)	orm ES	5-401-2
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
001 Control Rod Drive		х										K2.02 Knowledge of bus power supplies to the following: One-line diagram of power supply to trip breakers		29
002 Reactor Coolant											x	2.4.31 Knowledge of annunciator alarms, indications, or response procedures	4.2	30
011 Pressurizer Level Control														
014 Rod Position Indication														
015 Nuclear Instrumentation										х		A4.02 Ability to manually operate and/or monitor in the control room: NIS indicators	3.9	31
016 Non-nuclear Instrumentation									х			A3.01 Ability to monitor automatic operation of the NNIS, including: Automatic selection of NNIS inputs to control systems	2.9	32
017 In-core Temperature Monitor								х				A2.02 Ability to (a) predict the impacts of the following malfunctions or operations on the ITM system; and (b) based on those predictions, use procedures to correct, control or mitigate the consequences of those malfunctions or operations: Core damage	3.6	33
027 Containment Iodine Removal														
028 Hydrogen Recombiner and Purge Control							Х					A1.01 Ability to predict and/or monitor changes in parameter (to prevent exceeding design limits) associated with operating the HRPS controls including: Hydrogen concentration	3.4	34
29 Containment Purge														
033 Spent Fuel Pool Cooling														
034 Fuel Handling Equipment														
035 Steam Generator														
041 Steam Dump/Turbine Bypass Control						х						K6.03 Knowledge of the effect of a loss or malfunction on the following will have on the SDS: Controller and positioners, including ICS, S/G, CRDS	2.7	35
045 Main Turbine Generator					х							K5.18 Knowledge of the operational implications of the following concepts as they apply to the MT/B System: Purpose of lowpower reactor trips (limited to 25% power)		36
055 Condenser Air Removal														
056 Condensate														
068 Liquid Radwaste														
071 Waste Gas Disposal														
072 Area Radiation Monitoring				х								K4.02 Knowledge of ARM system design feature(s) and/or interlock(s) which provide for the following: Fuel Building isolation	3.2	37

075 Circulating Water	х											K1.08 Knowledge of the physical connections and/or cause-effect relationships between the circulating water system and the following systems: Emergency/essential SWS	38
079 Station Air													
086 Fire Protection													
K/A Category Point Totals:	1	1	0	1	1	1	1	1	1	1	1	Group Point Total:	10/3

Facility:		Date of Exam:				
Category	K/A #	Topic	F	RO	SRC	-Only
			IR	#	IR	#
1.	2.1.	2.1.31 Ability to locate control room switches, controls, and indications, and to determine that they correctly reflect the desired plant lineup.	4.6	66		
Conduct of Operations	2.1.	2.1.1 Knowledge of conduct of operations requirements.	3.8	67		
or operanone	2.1.					
	2.1.					
	2.1.					
	2.1.					
	Subtotal			2		
	2.2.	2.2.6 Knowledge of the process for making changes to procedures.	3.0	68		
2.	2.2.	2.2.13 Knowledge of tagging and clearance procedures.	4.1	69		
Equipment Control	2.2.	2.2.20 Knowledge of the process for managing troubleshooting activities.	2.6	70		
	2.2.					
	2.2.					
	2.2.					
	Subtotal			3		
	2.3.	2.3.4 Knowledge of radiation exposure limits under normal or emergency conditions.	3.2	71		
3.	2.3.	2.3.11 Ability to control radiation releases.	3.8	72		
Radiation Control	2.3.	2.3.7 Ability to comply with radiation work permit requirements during normal or abnormal conditions.	3.5	73		
	2.3.					
	2.3.					
	2.3.					
	Subtotal			3		
	2.4.	2.4.3 Ability to identify post accident instrumentation.	3.7	74		
4.	2.4.	2.4.19 Knowledge of EOP layout, symbols, and icons.	3.4	75		
Emergency	2.4.					
Procedures / Plan	2.4.					
	2.4.					
	2.4.					
	Subtotal			2		
Tier 3 Point Tota	al			10		

## **RO** exam

INO CX		
Tier / Group	Randomly Selected K/A	Reason for Rejection
Tier 1/Group 2	2.1.39	Suppressed per Diablo Canyon as not applicable
T2/G2	028 A1.02	Not a good tie in for Diablo Canyon. Would make better JPM. Randomly selected A1.01 as a replacement.
T2/G2	072 K4.03	Similar question already sampled on this exam. Replaced with K4.02
T2/G2	075 K3.07	No connection at DC between ESFAS and Circulation Water System. Replaced with K1.08
T2/G1	006 K5.01	Could not write discriminating question due to lack of system complexity.
T2/G1	061 A3.04	No Automatic isolations.

## SRO exam

Facility: Diab	lo Canyon	Senio	or Re	eacto	or Op	erate	or						Date of E	Exam:	De	cemb	er 2009	9
					F	RO K	/A C	ateg	ory F	oint	S				SR	O-On	ly Poin	ts
Tier	Group	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	A	\2	(	3*	Total
1.	1												18		3		3	6
Emergency & Abnormal	2					N/A				N/	/A		9		2		2	4
Plant Evolutions	Tier Totals												27		5		5	10
	1												28		2		3	5
2. Plant	2												10		2		1	3
Systems	Tier Totals												38		4		4	8
	(nowledge and	Abil	ities		,	1	:	2	(	3	4	1	10	1	2	3	4	7
	Categories													2	2	1	2	

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ES-401 Emergence	cy ar	nd A	.bno	PW rma	/R Ex I Plar	camin	nation Outline Foolutions - Tier 1/Group 1 (RO / SRO)	orm ES	-401-2
E/APE # / Name / Safety Function	K 1	K 2		A 1	A 2	G	K/A Topic(s)	IR	#
000007 (BW/E02&E10 CE/E02) Reactor Trip - Stabilization - Recovery / 1									
000008 Pressurizer Vapor Space Accident / 3									
000009 Small Break LOCA / 3						х	2.1.20 Ability to Interpret and execute procedure steps.	4.6	76
000011 Large Break LOCA / 3									
000015/17 RCP Malfunctions / 4									
000022 Loss of Rx Coolant Makeup / 2									
000025 Loss of RHR System / 4					х		AA2.02 Ability to determine and interpret the following as they apply to the Loss of Residual Heat Removal System: Leakage of reactor coolant from RHR into closed cooling water system or into reactor building atmosphere.	3.8	77
000026 Loss of Component Cooling Water / 8						х	2.1.28 Knowledge of the purpose and function of major system components and controls.	4.1	78
000027 Pressurizer Pressure Control System Malfunction / 3									
000029 ATWS / 1									
000038 Steam Gen. Tube Rupture / 3									
000040 (BW/E05; CE/E05; W/E12) Steam Line Rupture - Excessive Heat Transfer / 4									
000054 (CE/E06) Loss of Main Feedwater / 4									
000055 Station Blackout / 6					x		EA2.03 Ability to determine or interpret the following as they apply to a Station Blackout: Actions necessary to restore power	4.7	79
000056 Loss of Off-site Power / 6						х	2.4.11 Knowledge of abnormal condition procedures.	4.2	80
000057 Loss of Vital AC Inst. Bus / 6					x		AA2.14 Ability to determine and interpret the following as they apply to the Loss of vital AC Instrument Bus: That substitute power sources have come on line on a loss of initial AC.	3.6	81
000058 Loss of DC Power / 6									
000062 Loss of Nuclear Svc Water / 4									
000065 Loss of Instrument Air / 8									

W/E04 LOCA Outside Containment / 3						
W/E11 Loss of Emergency Coolant Recirc. / 4						
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4						
000077 Generator Voltage and Electric Grid Disturbances / 6						
K/A Category Totals:			3	3	Group Point Total:	18/6

ES-401 Emergency and A						_	utline - Tier 1/Group 2 (RO / SRO)	orm ES	3-401-2
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
000001 Continuous Rod Withdrawal / 1									
000003 Dropped Control Rod / 1					Х		AA2.01 Ability to determine and interpret the following as they apply to the dropped rod: Rod position indication to actual rod position	3.9	84
000005 Inoperable/Stuck Control Rod / 1									
000024 Emergency Boration / 1							<u> </u>		
000028 Pressurizer Level Malfunction / 2					Х		AA2.11 Ability to determine and interpret the following as they apply to the Pressurizer Level Control Malfunctions: Leak in PZR	3.6	82
000032 Loss of Source Range NI / 7									
000033 Loss of Intermediate Range NI / 7									
000036 (BW/A08) Fuel Handling Accident / 8									
000037 Steam Generator Tube Leak / 3									
000051 Loss of Condenser Vacuum / 4						x	2.1.7 Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation.	4.7	83
000059 Accidental Liquid RadWaste Rel. / 9									
000060 Accidental Gaseous Radwaste Rel. / 9									
000061 ARM System Alarms / 7									
000067 Plant Fire On-site / 8									
000068 (BW/A06) Control Room Evac. / 8									
000069 (W/E14) Loss of CTMT Integrity / 5									
000074 (W/E06&E07) Inad. Core Cooling / 4									
000076 High Reactor Coolant Activity / 9									
W/EO1 & E02 Rediagnosis & SI Termination / 3									
W/E13 Steam Generator Over-pressure / 4									
W/E15 Containment Flooding / 5									
W/E16 High Containment Radiation / 9									
BW/A01 Plant Runback / 1									
BW/A02&A03 Loss of NNI-X/Y / 7									
BW/A04 Turbine Trip / 4									
BW/A05 Emergency Diesel Actuation / 6									
BW/A07 Flooding / 8									
BW/E03 Inadequate Subcooling Margin / 4									
BW/E08; W/E03 LOCA Cooldown - Depress. / 4									
BW/E09; CE/A13; W/E09&E10 Natural Circ. / 4									
BW/E13&E14 EOP Rules and Enclosures									
CE/A11; W/E08 RCS Overcooling - PTS / 4						х	2.2.22 Knowledge of limiting conditions for operations and safety limits.	4.7	85

CE/A16 Excess RCS Leakage / 2						
CE/E09 Functional Recovery						
K/A Category Point Totals:			2	2	Group Point Total:	9/4

ES-401				Plar	nt S						Outlin p 1 (F	ne RO / SRO)	orm ES	-401-2
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
003 Reactor Coolant Pump											Х	2.2.44 Ability to interpret control room indications to verify the status and operations of a system, and understand how operator actions and directives affect plant and system conditions.	4.4	90
004 Chemical and Volume Control														
005 Residual Heat Removal											х	2.4.45 Ability to prioritize and interpret the significance of each annunciator or alarm.	4.3	88
006 Emergency Core Cooling														
007 Pressurizer Relief/Quench Tank											х	2.2.44 Ability to interpret control room indications to verify the status and operation of a system, and understand how operator actions and directives affect plant and system conditions.	4.4	86
008 Component Cooling Water														
010 Pressurizer Pressure Control														
012 Reactor Protection														
013 Engineered Safety Features Actuation														
022 Containment Cooling														
025 Ice Condenser														
026 Containment Spray								х				A2.04 Ability to (a) predict the impacts of the following malfunctions or operations on the CSS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Failure of spray pump	4.2	87
039 Main and Reheat Steam														
059 Main Feedwater														
061 Auxiliary/Emergency Feedwater														
062 AC Electrical Distribution														

063 DC Electrical Distribution				х			A2.02 Ability to (a) predict the impacts of the following malfunctions or operations on the DC electrical systems; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Loss of ventilation during battery charging	89
064 Emergency Diesel Generator								
073 Process Radiation Monitoring								
076 Service Water								
078 Instrument Air								
103 Containment								
K/A Category Point Totals:				2		3	Group Point Total:	28/5

ES-401			Plaı	nt S							tline 2 (RO / SRO)	orm ES	3-401-2
System # / Name	K 1	K 2		K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
001 Control Rod Drive													
002 Reactor Coolant													
011 Pressurizer Level Control													
014 Rod Position Indication													
015 Nuclear Instrumentation							х				A2.01 Ability to (a) predict the impacts of the following malfunctions or operations on the NIS: and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Power supply loss or erratic operation	3.9	91
016 Non-nuclear Instrumentation													
017 In-core Temperature Monitor													
027 Containment Iodine Removal													
028 Hydrogen Recombiner and Purge Control							X				A2.02 Malfunctions or operations on the HRPS; and 9 (b) based on those predictions, use procedures to correct, control or mitigate the consequences of those malfunctions or operations.	3.9	92
029 Containment Purge													
033 Spent Fuel Pool Cooling													
034 Fuel Handling Equipment													
035 Steam Generator													
041 Steam Dump/Turbine Bypass Control													
045 Main Turbine Generator													
055 Condenser Air Removal													
056 Condensate													
068 Liquid Radwaste										x	2.2.36 Ability to analyze the effect of maintenance activities, such as degraded power sources, on the status of limiting conditions for operations.	4.2	93
071 Waste Gas Disposal													
072 Area Radiation Monitoring													
075 Circulating Water													
079 Station Air													
086 Fire Protection													

K/A Category Point Totals:				2		1	Group Point Total:	10/3

Facility:		Date of Exam:				
Category	K/A #	Topic	F	RO	SRO	O-Only
			IR	#	IR	#
	2.1.	2.1.5 Ability to use procedures related to shift staffing, such as minimum crew complement, overtime limitations, etc.			3.9	94
1. Conduct of Operations	2.1.	2.1.17 Ability to make accurate, clear, and concise <u>verbal</u> <u>reports.</u>			4.0	95
or operations	2.1.					
	2.1.					
	2.1.					
	2.1.					
	Subtotal					
	2.2.	2.2.14 Knowledge of the process for controlling equipment configuration or status.			4.3	96
2. Equipment	2.2.	2.2.25 Knowledge of the bases in Technical Specifications for limiting conditions for operations and safety limits.			4.2	97
Control	2.2.					
	2.2.					
	2.2.					
	2.2.					
	Subtotal					
	2.3.	2.3.6 Ability to approve release permits.			3.8	98
	2.3.					
3.	2.3.					
Radiation Control	2.3.					
	2.3.					
	2.3.					
	Subtotal					
	2.4.	2.4.17 Knowledge of EOP terms and definitions.			4.3	99
4. Emergency	2.4.	2.4.22 Knowledge of the bases for prioritizing safety functions during abnormal/emergency operations.			4.4	100
Procedures /	2.4.					
Plan	2.4.					
	2.4.					
	2.4.					
	Subtotal					
Tier 3 Point Tota	al —			10		7

**ES-401**