



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**

REGION III  
2443 WARRENVILLE ROAD, SUITE 210  
LISLE, IL 60532-4352

August 9, 2010

Mr. Christopher J. Schwarz  
Vice President, Operations  
Entergy Nuclear Operations, Inc.  
Palisades Nuclear Plant  
27780 Blue Star Memorial Highway  
Covert, MI 49043-9530

**SUBJECT: PALISADES NUCLEAR PLANT –REQUEST FOR INFORMATION  
UNRESOLVED ITEM 05000255/2008009-03(DRS)**

Dear Mr. Schwarz:

During the 2008 the Component Design Bases Inspection at Palisades Nuclear Power Plant, the inspectors identified the station relied on non-safety-related equipment to mitigate the consequences of a steam generator tube rupture (SGTR) concurrent with a loss of offsite power event. Specifically, additional inspection and consultation with members of Nuclear Reactor Regulation (NRR) was needed to determine if the use of the non-safety-related Atmospheric Dump Valves (ADVs) was acceptable for a Final Safety Analysis Report (FSAR) Chapter 14 analyses. This issue was identified as Unresolved Item (URI 05000255/2008009-03).

In Task Interface Agreement Number 2009-03, dated June 29, 2010 (ML101260128), the NRR staff concluded the use of the non-safety-related ADVs was part of the licensing bases. The document further stated that the use of non-safety-related ADVs was allowed as long as the analyses using these non-safety-related systems, comply with regulations, and are approved by the NRC. In addition it requires a means to cool down the primary coolant system in the event the non-safety-related components are not available, using an established alternate method with procedures and evaluated to remain bounded by the worst case dose analysis using the ADVs.

The purpose of this letter is to request additional documents to facilitate the resolution of URI 05000255/2008009-03. The information requested should be available to the lead inspector at the Regional Office no later than August 18, 2010.

The lead inspector for this inspection is Jorge J. Corujo-Sandín. If there are questions about the material requested, or the inspection, please call Jorge J. Corujo-Sandín at (630) 829- 9741. Please send the information to the following e-mail address [jorge.corujo-sandin@nrc.gov](mailto:jorge.corujo-sandin@nrc.gov). A hard-copy with the required information is also an acceptable option.

It is important that these documents be as complete as possible, in order to minimize the number of documents requested during any additional follow up or during the on-site inspection, if one is determined to be required.

This letter does not contain new or amended information collection requirements subject to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 et seq.). Existing information collection requirements were approved by the Office of Management and Budget, control number 3150-0011. The NRC may not conduct or sponsor, and a person is not required to respond to a request for information or an information collection requirement unless the requesting document displays a currently valid Office of Management and Budget control number."

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Sincerely,

/RA/

Ann Marie Stone, Chief  
Engineering Branch 2  
Division of Reactor Safety

Docket No. 50-255  
License No. DPR-20

Enclosure: UNRESOLVED ITEM 05000255/2008009-03 INSPECTION DOCUMENT  
REQUEST

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## UNRESOLVED ITEM 05000255/2008009-03 INSPECTION DOCUMENT REQUEST

**Unresolved Item:**                   **05000255/2008009-03**

**Inspection Report:**               **05000255/2010**

**Lead Inspector:**                   **Jorge J. Corujo-Sandín**  
**(630) 829-9741**  
**jorge.corujo-sandin@nrc.gov**

### **A.     *Information Requested By August 18, 2010***

With respect to the Palisades design basis Steam Generator Tube Rupture (SGTR) with a concurrent Loss of Off-site Power (LOOP) event, please provide the following:

1. A brief description of the method used to cool down the plant using only safety-related equipment, in the event the non-safety-related Atmospheric Dump Valves (ADVs) are unavailable.
2. A brief explanation describing why the use of ADVs is the preferred method to mitigate the design bases SGTR coinciding with a LOOP.
3. A copy of the latest FSAR which describes the plant's response to a SGTR event. Include sections which describe the use of the ADVs and the safety-related method.
4. A copy of the assessment or analysis which demonstrates the use of only safety-related equipment for cooling the plant is bounded by the assessment performed for the use of ADVs. That is, the use of safety-related equipment is will not exceed worst case off-site dose assessment determined by the use of the preferred method to cope with a SGTR (the ADVs).
5. A copy of the analysis or calculation that demonstrates the use of only safety-related equipment for cooling the plant can remove the decay heat without impacting the primary system's mass inventory to the point that core damage results.
6. A copy of the analysis or calculation that demonstrates the use of only safety-related equipment for cooling the plant will not result in an overfill situation. That is, the margin to overfill the ruptured steam generator is bounded by the use of the ADVs.
7. A list of available safety-related options for supplying coolant, "feeding," into the Primary Coolant System. Include each option's coolant injection limitations (rate, shutoff head, etc).
8. If the use of only safety-related equipment for cooling the plant includes discharge into the pressurizer relief tank (PRT), please provide the following: the amount or rate of discharge into the PRT; and a description of the method used to vent, spray, and drain the PRT. Include a list of equipment needed as well as the safety classification of the equipment.

## UNRESOLVED ITEM 05000255/2008009-03 INSPECTION DOCUMENT REQUEST

9. A copy of the lesson plans, task training guide, or training material including Job Performance Measures and simulator scenarios used to train the operators to use the safety-related equipment when the nonsafety-related ADVs fail.
10. A copy of Corrective Action Documents generated as a result of this URI. Include associated evaluations, actions, status of actions, procedure changes, modifications, etc.

If the information requested above will not be available, please contact Jorge Corujo-Sandín as soon as possible at (630) 829-9741 or email [jorge.corujo-sandin@nrc.gov](mailto:jorge.corujo-sandin@nrc.gov).

C. Schwartz

-2-

request for information or an information collection requirement unless the requesting document displays a currently valid Office of Management and Budget control number.”

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Sincerely,

/RA/  
Ann Marie Stone, Chief  
Engineering Branch 2  
Division of Reactor Safety

Docket No. 50-255  
License No. DPR-20

Enclosure: UNRESOLVED ITEM 05000255/2008009-03 INSPECTION DOCUMENT REQUEST

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