

## ArevaEPRDCPEm Resource

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**From:** BRYAN Martin (EXTERNAL AREVA) [Martin.Bryan.ext@areva.com]  
**Sent:** Wednesday, August 04, 2010 6:18 PM  
**To:** Tesfaye, Getachew  
**Cc:** DELANO Karen (AREVA); ROMINE Judy (AREVA); BENNETT Kathy (AREVA); GUCWA Len (EXTERNAL AREVA)  
**Subject:** Response to U.S. EPR Design Certification Application RAI No. 266, FSAR Ch 6, Supplement 7  
**Attachments:** RAI 266 Supplement 7 Response US EPR DC.pdf

Getachew,

AREVA NP, Inc. provided a response to 1 of the 15 questions of RAI No. 266 on October 12, 2009. Supplement 1 response to RAI No. 266 was sent on November 4, 2009 to provide a response schedule for Questions 06.02.01.02-2, 06.02.01.02-3, 06.02.01.02-4. Supplement 2 response to RAI No. 266 was sent on December 10, 2009 to address 1 of the remaining questions. Supplement 3 response to RAI No. 266 was sent on December 18, 2009 to address 2 of the remaining questions. Supplement 4 response to RAI No. 266 was sent on February 25, 2010 to address 6 of the remaining questions and to provide a revised schedule for responding to one question. Supplement 5 response to RAI No. 266 was sent on May 5, 2010 to provide a revised response schedule for the remaining 5 questions. Supplement 6 response to RAI No. 266 was sent on June 30, 2010 to provide revised response dates for the remaining 5 questions.

On July 21, 2010, a telecon was held between the staffs of AREVA NP and the NRC and it was agreed to finalize the response to Question 06.02.01.04-4, but additional interaction is needed for Question 06.02.02-33.

The attached file, "RAI 266 Supplement 7 Response US EPR DC.pdf," provides a technically correct and complete response to Question 06.02.01.04-4.

The following table indicates the respective pages in the response document, "RAI 266 Supplement 7 Response US EPR DC.pdf," that contain AREVA NP's response to the subject questions.

<b>Question #</b>	<b>Start Page</b>	<b>End Page</b>
RAI 266 — 06.02.01.04-4	2	24

The response schedule for the 4 remaining RAI 266 questions is changed and is provided below:

<b>Question #</b>	<b>Final Response Date</b>
RAI 266 — 06.02.02-33	<b>August 25, 2010</b>
RAI 266 — 06.02.01.02-2	August 25, 2010
RAI 266 — 06.02.01.02-3	August 25, 2010
RAI 266 — 06.02.01.02-4	August 25, 2010

Sincerely,

Martin (Marty) C. Bryan  
U.S. EPR Design Certification Licensing Manager  
AREVA NP Inc.  
Tel: (434) 832-3016  
702 561-3528 cell  
[Martin.Bryan.ext@areva.com](mailto:Martin.Bryan.ext@areva.com)

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**From:** BRYAN Martin (EXT)  
**Sent:** Wednesday, June 30, 2010 12:03 PM  
**To:** 'Tefaye, Getachew'  
**Cc:** DELANO Karen V (AREVA NP INC); ROMINE Judy (AREVA NP INC); BENNETT Kathy A (OFR) (AREVA NP INC); GUCWA Len T (EXT)  
**Subject:** Response to U.S. EPR Design Certification Application RAI No. 266, FSAR Ch 6, Supplement 6

Getachew,

AREVA NP, Inc. provided a response to 1 of the 15 questions of RAI No. 266 on October 12, 2009. Supplement 1 response to RAI No. 266 was sent on November 4, 2009 to provide a response schedule for Questions 06.02.01.02-2, 06.02.01.02-3, 06.02.01.02-4. Supplement 2 response to RAI No. 266 was sent on December 10, 2009 to address 1 of the remaining questions. Supplement 3 response to RAI No. 266 was sent on December 18, 2009 to address 2 of the remaining questions. Supplement 4 response to RAI No. 266 was sent on February 25, 2010 to address 6 of the remaining questions and to provide a revised schedule for responding to one question. Supplement 5 response to RAI No. 266 was sent on May 5, 2010 to provide a revised response schedule for the remaining 5 questions.

As agreed with NRC, AREVA NP is providing a revised date for RAI 266 Supplement 6 at this time to accommodate interactions with the NRC on the responses to questions 06.02.01.04-4 and 06.02.02-33. The responses to the 3 remaining RAI 266 questions are dependent upon ongoing evaluations and subcompartment pressure analyses. The revised schedule for a technically correct and complete response to RAI 266 is provided below. The bases for the schedule change were discussed with NRC staff on June 30, 2010.

Question #	Response Date
RAI 266 — 06.02.01.04-4	August 4, 2010
RAI 266 — 06.02.02-33	August 4, 2010
RAI 266 — 06.02.01.02-2	August 25, 2010
RAI 266 — 06.02.01.02-3	August 25, 2010
RAI 266 — 06.02.01.02-4	August 25, 2010

Sincerely,

Martin (Marty) C. Bryan  
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**From:** BRYAN Martin (EXT)  
**Sent:** Wednesday, May 05, 2010 5:54 PM  
**To:** 'Tefaye, Getachew'  
**Cc:** DELANO Karen V (AREVA NP INC); ROMINE Judy (AREVA NP INC); BENNETT Kathy A (OFR) (AREVA NP INC); GUCWA Len T (EXT)  
**Subject:** Response to U.S. EPR Design Certification Application RAI No. 266, FSAR Ch 6, Supplement 5

Getachew,

AREVA NP, Inc. provided a response to 1 of the 15 questions of RAI No. 266 on October 12, 2009. Supplement 1 response to RAI No. 266 was sent on November 4, 2009 to provide a response schedule for Questions 06.02.01.02-2, 06.02.01.02-3, 06.02.01.02-4. Supplement 2 response to RAI No. 266 was sent on December 10, 2009 to address 1 of the remaining questions. Supplement 3 response to RAI No. 266 was sent on December 18, 2009 to address 2 of the remaining questions. Supplement 4 response to RAI No. 266 was sent on February 25, 2010 to address 6 of the remaining questions and to provide a revised schedule for responding to one question.

The responses to the 5 remaining RAI 266 questions are primarily dependent upon ongoing subcompartment pressure analyses and an analysis of the main steam line break using FSAR Chapter 15 assumptions. Because of these ongoing activities, AREVA NP is not providing a response at this time. The revised schedule for a technically correct and complete response to RAI 266 is provided below. The schedule for responding to Questions 06.02.01.02-2, 06.02.01.02-3, 06.02.01.02-4 and 06.02.01.04-4 has been revised while the schedule for providing a technically correct and complete response to Question 06.02.02-33 is unchanged. The bases for the schedule change were discussed with NRC staff during an audit conducted on April 30, 2010.

Question #	Response Date
RAI 266 — 06.02.01.02-2	June 30, 2010
RAI 266 — 06.02.01.02-3	June 30, 2010
RAI 266 — 06.02.01.02-4	June 30, 2010
RAI 266 — 06.02.01.04-4	June 30, 2010
RAI 266 — 06.02.02-33	June 30, 2010

Sincerely,

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 AREVA NP Inc.  
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 702 561-3528 cell  
[Martin.Bryan.ext@areva.com](mailto:Martin.Bryan.ext@areva.com)

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**From:** DUNCAN Leslie E (AREVA NP INC)  
**Sent:** Thursday, February 25, 2010 7:02 PM  
**To:** 'Tefsaye, Getachew'  
**Cc:** DELANO Karen V (AREVA NP INC); BENNETT Kathy A (OFR) (AREVA NP INC); ROMINE Judy (AREVA NP INC); GUCWA Len T (EXT); BRYAN Martin (EXT)  
**Subject:** Response to U.S. EPR Design Certification Application RAI No. 266, FSAR Ch 6, Supplement 4

Getachew,

AREVA NP, Inc. provided a response to 1 of the 15 questions of RAI No. 266 on October 12, 2009. Supplement 1 response to RAI No. 266 was sent on November 4, 2009 to provide a response schedule for Questions 06.02.01.02-2, 06.02.01.02-3, 06.02.01.02-4. Supplement 2 response to RAI No. 266 was sent on December 10, 2009 to address 1 of the remaining questions. Supplement 3 response to RAI No. 266 was sent on December 18, 2009 to address 2 of the remaining questions. The attached file, "RAI 266 Supplement 4 Response US EPR DC.pdf," provides a technically correct and complete response to 6 of the remaining 11 questions and a revised schedule for responding to Question 06.02.01.04-4.

The following table indicates the respective pages in the response document, "RAI 266 Supplement 4 Response US EPR DC.pdf," that contain AREVA NP's responses to the subject questions.

Question #	Start Page	End Page
RAI 266 — 06.02.01-48	2	3
RAI 266 — 06.02.01.03-2	4	19
RAI 266 — 06.02.01.03-3	20	20
RAI 266 — 06.02.01.04-2	21	29
RAI 266 — 06.02.01.04-3	30	30
RAI 266 — 06.02.01.04-4	31	31
RAI 266 — 06.02.01.04-5	32	37

The schedule for responding to the remaining RAI No. 266 questions is provided below. The schedule for responding to Question 06.02.01.04-4 was revised while the schedule for providing technically correct and complete responses to the remaining RAI No. 266 questions is unchanged.

Question #	Response Date
RAI 266 — 06.02.01.02-2	May 5, 2010
RAI 266 — 06.02.01.02-3	May 5, 2010
RAI 266 — 06.02.01.02-4	May 5, 2010
RAI 266 — 06.02.01.04-4	May 5, 2010
RAI 266 — 06.02.02-33	June 30, 2010

Sincerely,

Les Duncan  
Licensing Engineer  
**AREVA NP Inc.**  
An AREVA and Siemens Company  
Tel: (434) 832-2849  
[Leslie.Duncan@areva.com](mailto:Leslie.Duncan@areva.com)

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**From:** Pederson Ronda M (AREVA NP INC)  
**Sent:** Friday, December 18, 2009 3:59 PM  
**To:** 'Tesyfaye, Getachew'  
**Cc:** BENNETT Kathy A (OFR) (AREVA NP INC); DELANO Karen V (AREVA NP INC); BEELMAN Ronald J (AREVA NP INC)  
**Subject:** Response to U.S. EPR Design Certification Application RAI No. 266, FSAR Ch 6, Supplement 3

Getachew,

AREVA NP, Inc. provided a response to 1 of the 15 questions of RAI No. 266 on October 12, 2009. Supplement 1 response to RAI No. 266 was sent on November 4, 2009 to provide a response schedule for Questions 06.02.01.02-2, 06.02.01.02-3, 06.02.01.02-4. Supplement 2 response to RAI No. 266 was sent on December 10, 2009 to address 1 of the remaining questions. The attached file, "RAI 266 Supplement 3 Response US EPR DC.pdf," provides a technically correct and complete response to 2 of the remaining 13 questions.

The following table indicates the respective pages in the response document, "RAI 266 Supplement 3 Response US EPR DC.pdf," that contain AREVA NP's response to the subject question.

Question #	Start Page	End Page
RAI 266 — 06.02.01.04-6	2	2
RAI 266 — 06.02.01.04-7	3	3

The schedule for technically correct and complete responses to the remaining RAI No. 266 questions remains unchanged and is provided below.

Question #	Response Date
RAI 266 — 06.02.01-48	February 25, 2010
RAI 266 — 06.02.01.02-2	May 5, 2010
RAI 266 — 06.02.01.02-3	May 5, 2010
RAI 266 — 06.02.01.02-4	May 5, 2010
RAI 266 — 06.02.01.03-2	February 25, 2010
RAI 266 — 06.02.01.03-3	February 25, 2010
RAI 266 — 06.02.01.04-2	February 25, 2010
RAI 266 — 06.02.01.04-3	February 25, 2010
RAI 266 — 06.02.01.04-4	February 25, 2010
RAI 266 — 06.02.01.04-5	February 25, 2010
RAI 266 — 06.02.02-33	June 30, 2010

Sincerely,

*Ronda Pederson*

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Licensing Manager, U.S. EPR Design Certification

**AREVA NP Inc.**

An AREVA and Siemens company

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**From:** Pederson Ronda M (AREVA NP INC)

**Sent:** Thursday, December 10, 2009 6:06 PM

**To:** 'Tefsaye, Getachew'

**Cc:** BENNETT Kathy A (OFR) (AREVA NP INC); DELANO Karen V (AREVA NP INC); BEELMAN Ronald J (AREVA NP INC)

**Subject:** Response to U.S. EPR Design Certification Application RAI No. 266, FSAR Ch 6, Supplement 2

Getachew,

The response to RAI No. 266, Supplement 2, is submitted via AREVA NP Inc. letter, "Response Supplement 2 to U.S. EPR Design Certification Application RAI No. 266," NRC 09:123, dated December 10, 2009.

AREVA NP considers the information in the requested proprietary data files for the response to RAI No. 266 Question 06.02.01 - 47 submitted via that letter to be proprietary in their entirety and thus no non-proprietary version is provided. An affidavit to support withholding of information from public disclosure, per 10 CFR 2.390(b), is provided as an enclosure to that letter. The 2-page response document, alone, does not contain any proprietary information.

The following table indicates the respective page in the response document, "RAI 266 Supplement 2 Response US EPR DC.pdf," that contains AREVA NP's response to the subject question.

Question #	Start Page	End Page
RAI 266 — 06.02.01 - 47	2	2

A response to Question 06.02.01.03 – 2 cannot be provided at this time. The schedule for technically correct and complete responses to the remaining RAI No. 266 questions has been changed and is provided below.

Question #	Response Date
RAI 266 — 06.02.01-48	February 25, 2010
RAI 266 — 06.02.01.02-2	May 5, 2010
RAI 266 — 06.02.01.02-3	May 5, 2010
RAI 266 — 06.02.01.02-4	May 5, 2010
RAI 266 — 06.02.01.03-2	February 25, 2010
RAI 266 — 06.02.01.03-3	February 25, 2010
RAI 266 — 06.02.01.04-2	February 25, 2010
RAI 266 — 06.02.01.04-3	February 25, 2010
RAI 266 — 06.02.01.04-4	February 25, 2010
RAI 266 — 06.02.01.04-5	February 25, 2010
RAI 266 — 06.02.01.04-6	December 18, 2009
RAI 266 — 06.02.01.04-7	December 18, 2009
RAI 266 — 06.02.02-33	June 30, 2010

Sincerely,

*Ronda Pederson*

[ronda.pederson@areva.com](mailto:ronda.pederson@areva.com)

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**From:** WELLS Russell D (AREVA NP INC)

**Sent:** Wednesday, November 04, 2009 3:20 PM

**To:** 'Getachew Tesfaye'

**Cc:** Pederson Ronda M (AREVA NP INC); BENNETT Kathy A (OFR) (AREVA NP INC); DELANO Karen V (AREVA NP INC)

**Subject:** Response to U.S. EPR Design Certification Application RAI No. 266, FSAR Ch 6, Supplement 1  
Getachew,

AREVA NP, Inc. provided a response to 1 of the 15 questions of RAI No. 266 on October 12, 2009. As indicated in our response, a schedule for the response to Questions 06.02.01.02-2, 06.02.01.02-3, 06.02.01.02-4, would be provided by November 5, 2009. Accordingly, the schedule for the response to these questions is provided in the attached file, RAI 266 Supplement 1 Response US EPR DC.pdf" and reflected in the below table.

The schedule for technically correct and complete responses to the remaining questions remains unchanged and is provided below.

Question #	Response Date
RAI 266 — 06.02.01-47	December 10, 2009
RAI 266 — 06.02.01-48	February 25, 2010
RAI 266 — 06.02.01.02-2	May 5, 2010
RAI 266 — 06.02.01.02-3	May 5, 2010
RAI 266 — 06.02.01.02-4	May 5, 2010
RAI 266 — 06.02.01.03-2	December 10, 2009
RAI 266 — 06.02.01.03-3	February 25, 2010

RAI 266 — 06.02.01.04-2	February 25, 2010
RAI 266 — 06.02.01.04-3	February 25, 2010
RAI 266 — 06.02.01.04-4	February 25, 2010
RAI 266 — 06.02.01.04-5	February 25, 2010
RAI 266 — 06.02.01.04-6	December 18, 2009
RAI 266 — 06.02.01.04-7	December 18, 2009
RAI 266 — 06.02.02-33	June 30, 2010

Sincerely,

(Russ Wells on behalf of)

*Ronda Pederson*

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Licensing Manager, U.S. EPR Design Certification

New Plants Deployment

**AREVA NP, Inc.**

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**From:** Pederson Ronda M (AREVA NP INC)

**Sent:** Monday, October 12, 2009 6:38 PM

**To:** Tesfaye, Getachew

**Cc:** BEELMAN Ronald J (AREVA NP INC); BENNETT Kathy A (OFR) (AREVA NP INC); DELANO Karen V (AREVA NP INC)

**Subject:** Response to U.S. EPR Design Certification Application RAI No. 266, FSAR Ch. 6

Getachew,

Attached please find AREVA NP Inc.'s response to the subject request for additional information (RAI). The attached file, "RAI 266 Response US EPR DC.pdf" provides technically correct and complete responses to 1 of the 15 questions.

The following table indicates the respective pages in the response document, "RAI 266 Response US EPR DC.pdf," that contain AREVA NP's response to the subject questions.

Question #	Start Page	End Page
RAI 266 — 06.02.01-47	2	2
RAI 266 — 06.02.01-48	3	3
RAI 266 — 06.02.01.02-2	4	4
RAI 266 — 06.02.01.02-3	5	5
RAI 266 — 06.02.01.02-4	6	6
RAI 266 — 06.02.01.03-2	7	7
RAI 266 — 06.02.01.03-3	8	8
RAI 266 — 06.02.01.04-2	9	9
RAI 266 — 06.02.01.04-3	10	10
RAI 266 — 06.02.01.04-4	11	11
RAI 266 — 06.02.01.04-5	12	12
RAI 266 — 06.02.01.04-6	13	13
RAI 266 — 06.02.01.04-7	14	14

RAI 266 — 06.02.02-33	15	15
RAI 266 — 06.02.02-34	16	16

A complete answer is not provided for 14 of the 15 questions. The schedule for a technically correct and complete response to these questions is provided below.

Question #	Response Date
RAI 266 — 06.02.01-47	December 10, 2009
RAI 266 — 06.02.01-48	February 25, 2010
RAI 266 — 06.02.01.02-2	Schedule to be provided by November 5, 2009
RAI 266 — 06.02.01.02-3	Schedule to be provided by November 5, 2009
RAI 266 — 06.02.01.02-4	Schedule to be provided by November 5, 2009
RAI 266 — 06.02.01.03-2	December 10, 2009
RAI 266 — 06.02.01.03-3	February 25, 2010
RAI 266 — 06.02.01.04-2	February 25, 2010
RAI 266 — 06.02.01.04-3	February 25, 2010
RAI 266 — 06.02.01.04-4	February 25, 2010
RAI 266 — 06.02.01.04-5	February 25, 2010
RAI 266 — 06.02.01.04-6	December 18, 2009
RAI 266 — 06.02.01.04-7	December 18, 2009
RAI 266 — 06.02.02-33	June 30, 2010

Sincerely,

*Ronda Pederson*

[ronda.pederson@areva.com](mailto:ronda.pederson@areva.com)

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**From:** Tesfaye, Getachew [mailto:Getachew.Tesfaye@nrc.gov]

**Sent:** Thursday, September 10, 2009 9:10 AM

**To:** ZZ-DL-A-USEPR-DL

**Cc:** Jensen, Walton; Jackson, Christopher; Snodderly, Michael; Carneal, Jason; Colaccino, Joseph; ArevaEPRDCPEm Resource

**Subject:** U.S. EPR Design Certification Application RAI No. 266(3408,3443,3444,3445,3446), FSAR Ch. 6

Attached please find the subject requests for additional information (RAI). A draft of the RAI was provided to you on August 3, 2009, and discussed with your staff on August 13, 2009. RAI Questions 06.02.01-47, 06.02.01-48, 06.02.01.04-2, 06.02.01.04-5, and 06.02.01.04-7 were revised as a result of that discussion. The schedule we have established for review of your application assumes technically correct and complete responses within 30 days of receipt of RAIs. For any RAIs that cannot be answered within 30 days, it is expected that a date for receipt of this information will be provided to the staff within the 30 day period so that the staff can assess how this information will impact the published schedule.

Thanks,



Getachew Tesfaye  
Sr. Project Manager  
NRO/DNRL/NARP  
(301) 415-3361

**Hearing Identifier:** AREVA\_EPR\_DC\_RAIs  
**Email Number:** 1783

**Mail Envelope Properties** (BC417D9255991046A37DD56CF597DB71071C6D26)

**Subject:** Response to U.S. EPR Design Certification Application RAI No. 266, FSAR Ch 6,  
Supplement 7  
**Sent Date:** 8/4/2010 6:17:59 PM  
**Received Date:** 8/4/2010 6:18:09 PM  
**From:** BRYAN Martin (EXTERNAL AREVA)

**Created By:** Martin.Bryan.ext@areva.com

**Recipients:**

"DELANO Karen (AREVA)" <Karen.Delano@areva.com>  
Tracking Status: None  
"ROMINE Judy (AREVA)" <Judy.Romine@areva.com>  
Tracking Status: None  
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Tracking Status: None  
"GUCWA Len (EXTERNAL AREVA)" <Len.Gucwa.ext@areva.com>  
Tracking Status: None  
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Tracking Status: None

**Post Office:** AUSLYNCMX02.adom.ad.corp

<b>Files</b>	<b>Size</b>	<b>Date &amp; Time</b>
MESSAGE	19265	8/4/2010 6:18:09 PM
RAI 266 Supplement 7 Response US EPR DC.pdf		1088427

**Options**

**Priority:** Standard  
**Return Notification:** No  
**Reply Requested:** No  
**Sensitivity:** Normal  
**Expiration Date:**  
**Recipients Received:**

**Response to**

**Request for Additional Information No. 266, Supplement 7**

**9/10/2009**

**U.S. EPR Standard Design Certification**

**AREVA NP Inc.**

**Docket No. 52-020**

**SRP Section: 06.02.01 - Containment Functional Design**

**SRP Section: 06.02.01.02 - Sub-compartment Analysis**

**SRP Section: 06.02.01.03 - Mass and Energy Release Analysis for Postulated  
Loss-of-Coolant Accidents (LOCAs)**

**SRP Section: 06.02.01.04 - Mass and Energy Release Analysis for Postulated  
Secondary System Pipe Ruptures**

**SRP Section: 06.02.02 - Containment Heat Removal Systems**

**Application Section: FSAR Chapter 6**

**QUESTIONS for Containment and Ventilation Branch 1 (AP1000/EPR Projects)  
(SPCV)**

**Question 06.02.01.04-4:**

This question relates to conservativeness of the secondary system break mass and energy release calculations in FSAR Section 6.2.1.4. In RAI #82 6.01.02.04-1d the staff questioned the assumptions used to calculate reactor power following a main steam line break and the effect of these assumptions on containment analysis. In the response AREVA provided a curve of reactor power vs. time for the MSLB case calculated to produce the highest calculated containment temperature and pressure. This was for a postulated double ended break with the reactor at an initial power level of 50%. Offsite power was assumed to remain available so that the reactor coolant pumps would continue to operate. The break flow was assumed to be limited by the area of the flow restrictors in the steam generator nozzles to 1.4 ft<sup>2</sup>. The RELAP5/Mod2-B&W computer code was used to model the reactor system. No return to power was calculated even though the most reactive control rod was assumed to be stuck and to not enter the core following reactor trip.

AREVA also evaluated the consequences of a main steam line break in FSAR Section 15.1.5 to determine the potential for reactor core damage. In these evaluations the S-RELAP5 computer code was used. For the limiting break a return to power was calculated which reached a maximum of 23.14% over a period of approximately 200 seconds. This energy generation is sufficient, if considered in the containment analysis to have a considerable effect on the calculated containment temperature and pressure. In the response to RAI #34 15.01.05-2, AREVA provided the results from the sensitivity study of postulated steam line breaks for which the potential for core damage was evaluated. Initial power levels of 100%, 60%, 25% and 0% were investigated using S-RELAP5. The core was calculated to return to power generation following reactor trip regardless of the initial power level. The staff understands that part of the reason that return to power was calculated for the Chapter 15 analyses but not for the Chapter 6 analysis, was that a much higher control rod shutdown margin was assumed for the Chapter 6 analyses as compared with the Chapter 15 analyses. Provide analyses of the containment response to postulated main steam line breaks for which the core physics assumptions are consistent or conservative as compared to those which the staff is reviewing to support the main steam line break analyses in FSAR chapter 15.

The above questions are follow-up questions to previously issued RAIs and the containment audit held in Lynchburg on July 14 and 15, 2009.

**Response to Question 06.02.01.04-4:**

The limiting S-RELAP5 main steam line break (MSLB) event scenario (in terms of core consequences from post-scrum return-to-criticality) has been previously identified in U.S. EPR FSAR Tier 2, Section 15.1.5 as a postulated 1.72 ft<sup>2</sup> break in a main steam line upstream of the affected steam generator (SG) main steam isolation valve (MSIV) at end-of-cycle (EOC) hot-zero-power (HZP) conditions with offsite power remaining available to operate the reactor coolant pumps. The MSLB effluent discharge, although assumed in U.S. EPR FSAR Tier 2, Chapter 15 to be outside containment but upstream of the MSIV (i.e., atmospheric back-pressure), constitutes a consistent basis upon which to formulate the mass and energy release rates (M&E) for an in-containment MSLB containment analysis in U.S. EPR FSAR Tier 2, Chapter 6. The single failure assumed in identifying the limiting case MSLB with respect to U.S. EPR FSAR Tier 2, Chapter 15 core consequences is a main steam relief control valve (MSRCV) that remains fully stuck open on an SG unaffected by the break location subsequent to MSIV

closure. The MSRCV effluent discharge to atmosphere is independent of MSLB mass and energy flow into containment and, therefore, does not contribute to containment M&Es.

The limiting U.S. EPR FSAR Tier 2, Chapter 15 MSLB case, together with several other scenarios, was investigated using the U.S. EPR FSAR Tier 2, Chapter 15 S-RELAP5 methodology to confirm the limiting initial plant conditions and MSLB size and single failure relative to severity of containment response (see Table 06.02.01.04-4-1). These include:

- A 1.72 ft<sup>2</sup> MSLB from EOC HZP upstream of the MSIV with single failure of an unaffected SG MSRCV to close (offsite power available).
- A double-ended guillotine (DEG) MSLB from EOC HZP upstream of the MSIV with single failure of an unaffected SG MSRCV to close (offsite power available).
- A 1.72 ft<sup>2</sup> MSLB from EOC HZP upstream of the MSIV with single failure of the affected SG MSIV to close (offsite power available).
- A DEG MSLB from EOC 25 percent rated thermal power (RTP) upstream of the MSIV with single failure of the affected SG MSIV to close (offsite power available).

Figure 06.02.01.04-4-1 shows the results of the four U.S. EPR FSAR Tier 2, Chapter 15 S-RELAP5 methodology-based GOTHIC calculations previously mentioned, together with GOTHIC calculations (RELAP5-BW M&Es) of the limiting U.S. EPR FSAR Tier 2, Chapter 6 analysis of record (DEG MSLB from 20 percent RTP) (**green curve**) and a U.S. EPR FSAR Tier 2, Chapter 6-based double-ended MSLB from HZP (**blue curve**). The results indicate:

- The limiting U.S. EPR FSAR Tier 2, Chapter 15-based S-RELAP5 HZP 1.72ft<sup>2</sup> MSLB calculated peak containment pressure (with no credit taken for high  $P_{\text{cont}}$  pressure isolation) is not bounded by the limiting U.S. EPR FSAR Tier 2, Chapter 6-based RELAP5-BW 20 percent power DEG MSLB calculated peak containment pressure.
- The limiting single failure in terms of containment consequences is the failure of the affected SG MSIV to close.
- The limiting U.S. EPR FSAR Tier 2, Chapter 15-based MSLB scenario in terms of containment consequences is the 1.72 ft<sup>2</sup> split MSLB from HZP EOC conditions with failure of the affected SG MSIV to close and no loss of offsite power (LOOP).

The overall results of the S-RELAP5/GOTHIC containment analyses indicate that the limiting Chapter 15-based HZP MSLB scenario is not bounded by the corresponding U.S. EPR FSAR Tier 2, Chapter 6 limiting case MSLB scenario although no credit is taken for high containment pressure ( $P_{\text{cont}}$ ) isolation in any of the S-RELAP5 analyses.

In order to quantify the conservatism inherent in the U.S. EPR FSAR Tier 2, Chapter 6 methodology, the results of a postulated unisolable EOC HZP 1.72ft<sup>2</sup> MSLB, using a U.S. EPR S-RELAP5 best-estimate (BE) model, were compared to the U.S. EPR FSAR Tier 2, Chapter 6 limiting MSLB analysis of record and the limiting containment consequence U.S. EPR FSAR Tier 2, Chapter 15-based MSLB containment analysis results previously mentioned. Successive S-RELAP5 parameterization runs were undertaken implementing individual bounding assumptions one at a time while incrementally transforming the BE model into a best-estimate evaluation model (BEEM). The GOTHIC peak containment pressure results quantified the added conservatism afforded by each additional bounding assumption in assessing the overall conservatism in the calculated peak containment pressure figure of merit. All cases

were run using S-RELAP5 to generate M&Es for subsequent GOTHIC analyses. In order to afford a common basis of comparison for computed M&E release rates between methodologies, Moody single-velocity critical flow modeling at the break plane was used throughout. All cases assumed offsite power remained available. The individual cases included (in order of execution):

1. BE: base case - 1.72 ft<sup>2</sup> unisolable MSLB from EOC HZP (no single failure) using Moody critical flow modeling at the break.
2. BE+: base case PLUS implementation of a second RELAP5 SEPARATR component model at the SG#4 dryer elevation to afford nearly steam only exiting the affected SG.
3. BEEM: case #2 PLUS single failure of the affected SG#4 MSIV to close.
4. BEEM: case #3 PLUS bounding emergency feed water (EFW) flow to the affected SG initiated at break initiation ( $t_0$ ); PLUS post-scam shutdown margin (SDM) reduction to 6000 pcm.
5. BEEM: case #4 PLUS limiting main feed water (MFW) flow to the affected SG (complete replacement of affected SG inventory lost out the break until MFW isolation--analogous to the limiting U.S. EPR FSAR Tier 2, Chapter 15.1.5 MSLB case) at minimum temperature (250°F); PLUS degraded ECC injection flow/boron concentration and Technical Specification (TS) minimum emergency core coolant (ECC) temperature (59°F); PLUS minimum EFW temperature (50°F); PLUS TS minimum SDM (3000pcm).

No credit was taken for the high containment pressure isolation signal in any of these calculations, which would isolate main steam and main feed and, thereby, foreshorten the duration and extent of the overcooling event. This is an additional conservatism because crediting the isolation signal would curtail M&E releases to the containment. The quantification of conservatism results are shown in Table 06.02.01.04-4-2 and Figure 06.02.01.04-4-2.

Bounding MFW (Case 5) is the largest single contributor to added conservatism in terms of increased containment peak pressure, which is the figure of merit. This effect is magnified in the U.S. EPR FSAR Tier 2, Chapter 15 evaluation model (EM) due to specific modeling techniques designed to maximize core consequences.

The U.S. EPR FSAR Tier 2, Chapter 15.1.5 limiting MSLB scenario bounding peak containment pressure is attributed to modeling differences between the U.S. EPR FSAR Tier 2, Chapter 6 EM and U.S. EPR FSAR Tier 2, Chapter 15 EM. A comparison of the modeling features of the three models (BE, Chapter 6 EM, Chapter 15 EM) reveals three major modeling differences in the U.S. EPR FSAR Tier 2, Chapter 15 EM (shown in **bold** in Table 06.02.01.04-4-3) affecting the calculation of the peak containment pressure. These include:

- Single-node SG boiler section modeling
- Limiting MFW introduction (complete replacement of affected SG inventory lost out the break until MFW isolation).
- Skewed stuck-rod region reactivity weighting.

The conservatisms incorporated into the U.S. EPR FSAR Tier 2, Chapter 15 EM are intended to create circumstances maximizing core consequences. These circumstances are unnecessarily

conservative for the containment analysis. The conservatisms associated with these considerations are addressed in subsequent paragraphs.

- Compared to the U.S. EPR FSAR Tier 2, Chapter 6 EM commonly accepted SG nodalization scheme, the U.S. EPR FSAR Tier 2, Chapter 15 EM SG model consists of two nodes - a boiler section node and a steam dome node - joined by a special purpose junction that passes only steam. The net effect of this modeling approach is to enhance the primary-to-secondary heat transfer rate by placing all SG liquid inventory in contact with all SG secondary-side tube heat transfer surface area all the time. There is no progressive SG heat transfer surface area degradation during SG dryout as evidenced by the heat transfer precipice in Figure 06.02.01.04-4-3. The blowdown M&E release rates are artificially enhanced, producing relatively higher calculated peak containment pressures.
- Compared to the U.S. EPR FSAR Tier 2, Chapter 6 EM conservative MFW treatment during SG depressurization, the U.S. EPR FSAR Tier 2, Chapter 15 EM SG model completely replaces all affected SG inventory lost out the break until receipt of the MFW isolation signal. Approximately 46,000 lb<sub>m</sub>, in excess of a conservative 10,000 lb<sub>m</sub> bounding amount contained within the length of MFW piping under low-low MFW demand conditions, is introduced at 250°F, artificially delaying affected SG dryout and elevating M&Es (see Figure 06.02.01.04-4-4). The net effect of this modeling approach, coupled with the single-node boiler region noding, is to artificially prolong the viability of primary-to-secondary heat transfer enhancing the duration and extent of the overcooling event. The blowdown M&E release rates are artificially enhanced producing relatively higher calculated peak containment pressures.
- Compared to the U.S. EPR FSAR Tier 2, Chapter 6 EM conservative assumptions of 20 percent unaffected loop core flow inter-mixing with the affected loop (80 percent-20 percent flow split), together with 50 percent-50 percent affected-to-unaffected core sector reactivity weighting, the U.S. EPR FSAR Tier 2, Chapter 15 EM reactor vessel/core model assumes no loop inter-mixing, and, reactivity weighting heavily skewed toward the stuck rod region. The net effect of this modeling approach is to confine the overcooling to the affected loop only, resulting in a recriticality in which approximately 50 percent of the total return to power occurs in the stuck-rod region alone, with approximately 80 percent of the total return to power occurring in the affected quarter-core sector (see Figure 06.02.01.04-4-5). Coupled with the excessive MFW flow, this results in a quasi-steady return to approximately 22 percent power, artificially enhancing M&E releases, and producing relatively higher calculated peak containment pressures.

The intent of the guidelines in NUREG-0800 SRP Section 6.2.1.4 is to conservatively bound the MSLB M&E releases to containment in calculating the limiting peak containment pressure. Figure 06.02.01.04-4-6 illustrates a U.S. EPR FSAR Tier 2, Chapter 6 methodology-based 1.72 ft<sup>2</sup> counterpart MSLB scenario (3000 pcm SDM with credit for high P<sub>cont</sub> isolation) calculated peak containment pressure (**pink curve**) of approximately 63 psia, which includes conservatisms of approximately 9 percent based on containment design pressure (**red**) when compared to best-estimate calculations (**green curve**) (7 psi ΔP), and design margins of approximately 17 percent (13 psi ΔP) on the same basis. The U.S. EPR FSAR Tier 2, Chapter 15 methodology (**blue curve**), which is intended for core consequence analysis, includes conservatisms of approximately 18 percent based on containment design pressure when compared to best-estimate calculations (14 psi ΔP), when applied to containment analysis. The set of bounding circumstances for U.S. EPR FSAR Tier 2, Chapter 15 core consequence analysis is unnecessarily conservative for application to U.S. EPR FSAR Tier 2, Chapter 6

containment consequence analysis as demonstrated by the quantification of conservatism results, especially with regard to limiting MFW flow.

In order to validate the results, the limiting U.S. EPR FSAR Tier 2, Chapter 15.1.5 EM MSLB scenario, with alternate failure of the affected SG MSIV to close, was repeated using relaxed conservatism with respect to affected SG MFW flow. A limit of 10,000 lb<sub>m</sub> MFW flow into the affected SG was applied to the model consistent with bounding the mass of MFW in the main feed line downstream of the low flow control valve during very-low flow conditions.

The U.S. EPR plant response to a 1.72 ft<sup>2</sup> break in a main steam line upstream of the MSIV at EOC HZP conditions, assuming that (1) offsite power remains available, (2) the affected SG MSIV sticks in the fully open position, and, (3) a maximum of 10,000 lb<sub>m</sub> MFW flow is allowed into the affected SG during its depressurization, has been simulated with S-RELAP5 using the U.S. EPR FSAR Tier 2, Chapter 15 EM, as modified above, to generate M&Es for subsequent multi-node GOTHIC containment analysis. Key parameter results are shown in Figure 06.02.01.04-4-7 through Figure 06.02.01.04-4-15.

As shown in Figure 06.02.01.04-4-15, the relaxed MFW conservatism case (**blue curve**) produces a peak containment pressure approximately 7 psi lower than the limiting MFW case (**green curve**). As shown in Figure 06.02.01.04-4-16, the approximately 63 psia relaxed conservatism peak pressure (**orange curve**) is comparable to the result from the counterpart U.S. EPR FSAR Tier 2, Chapter 6 EM 1.72 ft<sup>2</sup> MSLB (**pink curve**) (3000 pcm SDM), which takes credit for high P<sub>cont</sub> isolation, whereas the relaxed MFW conservatism case does not. The net effect of taking credit for high P<sub>cont</sub> isolation is to lower calculated peak containment pressure. The counterpart U.S. EPR FSAR Tier 2, Chapter 6 EM 1.72 ft<sup>2</sup> MSLB calculated peak containment pressure, therefore, bounds the relaxed MFW conservatism calculated peak containment pressure and is itself bounded by the U.S. EPR FSAR Tier 2, Chapter 6 analysis of record peak containment pressure (**green curve**). The validation results clearly demonstrate that the Chapter 6 methodology conservatively bounds calculated peak containment pressure.



**Table 06.02.01.04-4-1— MSLB Containment Peak Pressure Results**

<b>S-RELAP5/GOTHIC Chapter 15 Methodology-based Containment Peak Pressure Results</b>			
<b>Initial Power, (% Power)</b>	<b>Break Size</b>	<b>Single Failure</b>	<b>Peak Pressure, (psia)</b>
HZP	1.72 ft <sup>2</sup>	MSRCV	67.4
HZP	200%	MSRCV	62.9
HZP	1.72 ft <sup>2</sup>	MSIV	69.8
25	200%	MSIV	59.9

**Table 06.02.01.04-4-2—MSLB Quantification of Conservatism Results**

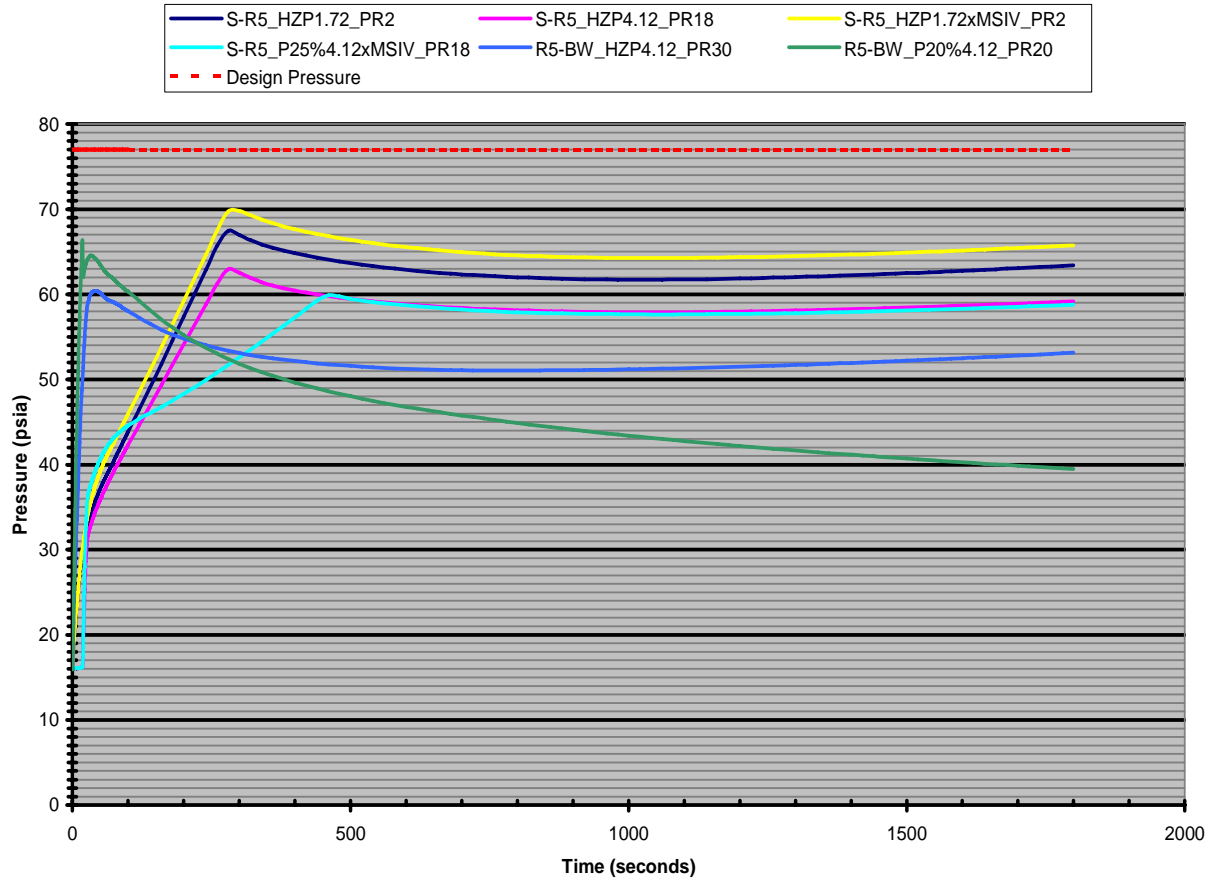
<b>Case</b>	<b>Scenario</b>	<b>Total Energy at Peak P<sub>cont</sub></b>	<b>Min inlet T<sub>core</sub> at SG Dryout</b>	<b>Peak P<sub>cont</sub> (psia)</b>	<b>Return to Power</b>
1	BE EOC HZP 1.72ft <sup>2</sup> MSLB (+Moody)	320.3x10 <sup>6</sup> BTUs BE base case	~453°F	~56* @ ~180s (base)	no
2	<u>BE+</u> EOC HZP 1.72ft <sup>2</sup> MSLB (+steam only)	~5.6x10 <sup>6</sup> BTUs additional	~448°F	~58* @ ~180s (+2psi)	no
3	<u>BEEM</u> EOC HZP 1.72ft <sup>2</sup> MSLB (+MSIV failure)	~34.1x10 <sup>6</sup> BTUs additional	~447°F	~61* @ ~200s (+3psi)	no
4	<u>BEEM</u> EOC HZP 1.72ft <sup>2</sup> MSLB (+maxEFW)	~3.7x10 <sup>6</sup> BTUs additional	~444°F	~62* @ ~200s (+1psi)	no
5	<u>BEEM</u> EOC HZP 1.72ft <sup>2</sup> MSLB (+limiting MFW)	~104.8x10 <sup>6</sup> BTUs additional	~408°F	~72* @ ~340s (+10psi)	yes @ 260s

\* No credit taken for high P<sub>cont</sub> isolation

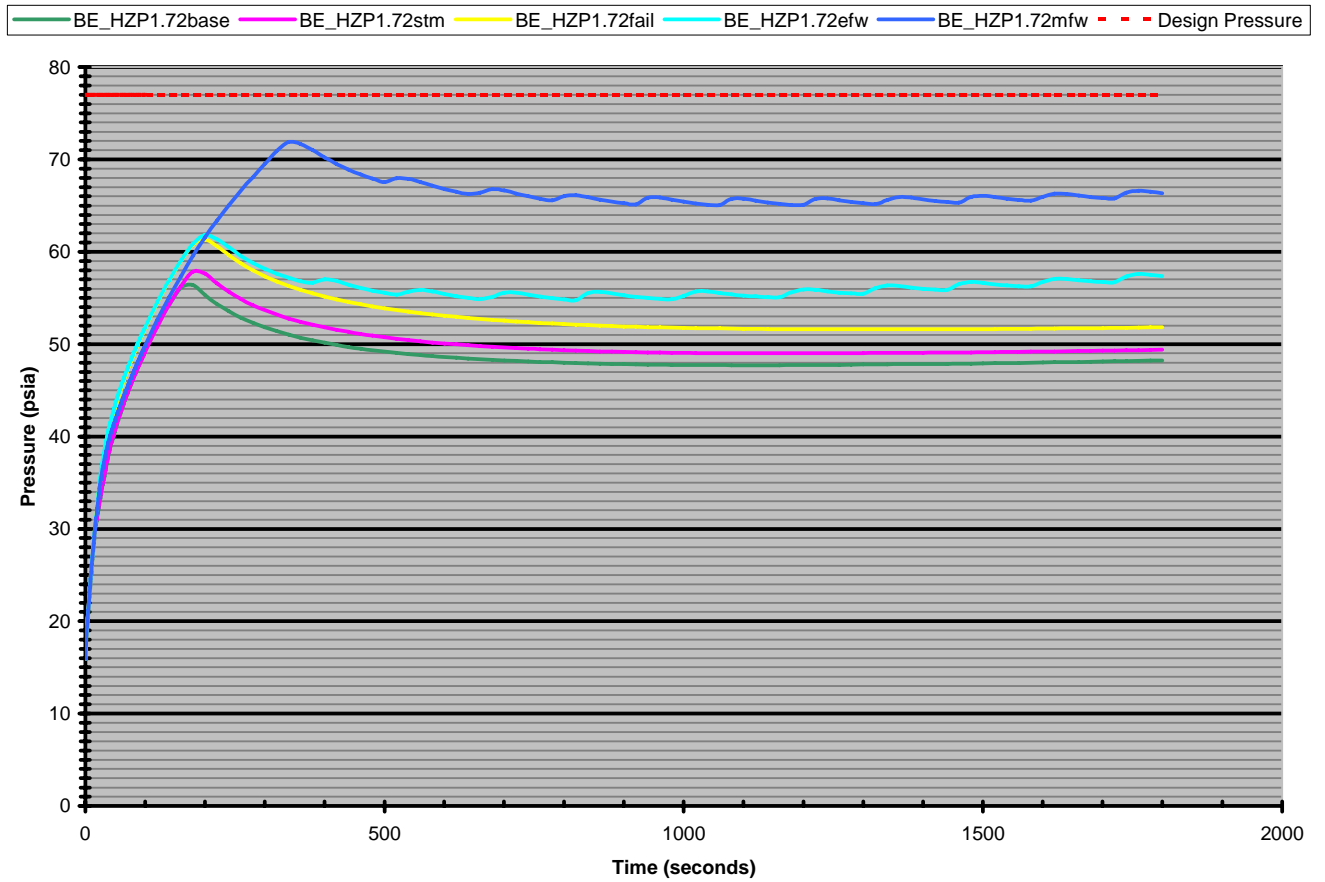
**Table 06.02.01.04-4-3—U.S. EPR HZP RELAP5 Input Model Comparisons**

<b>Feature</b>	<b>BE (base model)</b>	<b>Ch. 15 EM</b>	<b>Ch. 6 EM</b>
Reactivity	BE EOC HZP; Equal weighting	Min SDM/MTC; <b>Biased stuck rod region weighting</b>	Min SDM/MTC; 50-50 weighting
Boundary conditions	nominal EFW; nominal MFW; nominal ECC	Max EFW @ t <sub>0</sub> ; <b>limiting MFW flow</b> ; Degraded ECC	Max EFW @ t <sub>0</sub> ; Conservative MFW; Degraded ECC
Single failure	none	MSRCV	MSIV
Break fluid mixture	2-phase	1-phase	2-phase
Break critical flow model	Moody	Moody	Moody
Rx core model	1/4 - 3/4 core sectors; 60-40 loop mixing	1/4 - 3/4 core sectors +stuck rod region; no loop mixing	1/4 - 3/4 core sectors; 80-20 loop mixing
SG model	Representative nodalization	<b>1-node boiler region</b> w/ steam only junc.	Representative nodalization

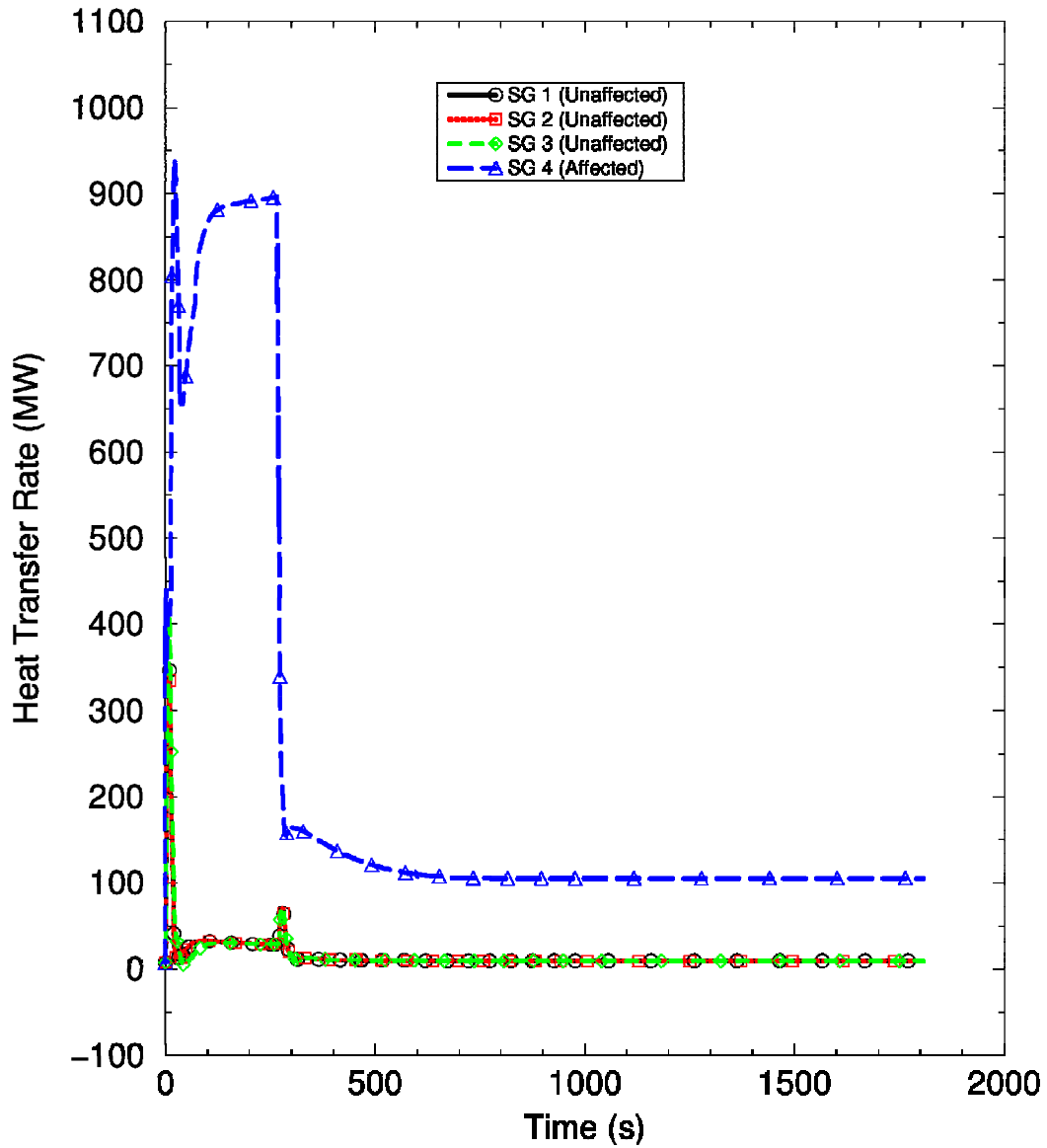
**Figure 06.02.01.04-4-1—MSLB Containment Peak Pressure Comparisons**



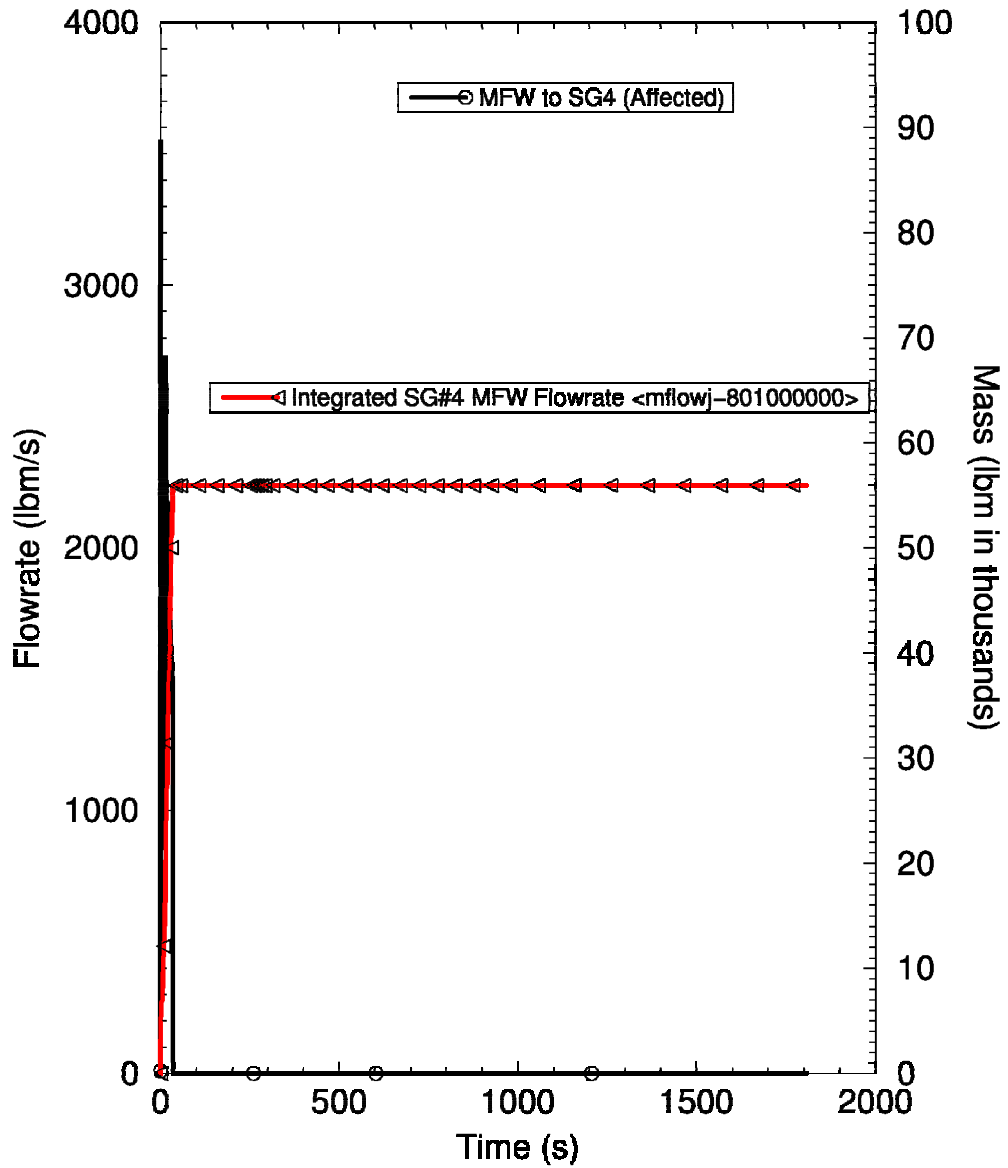
**Figure 06.02.01.04-4-2—Parametric Bounding Assumption Added  
Conservatism Containment Pressure Results**



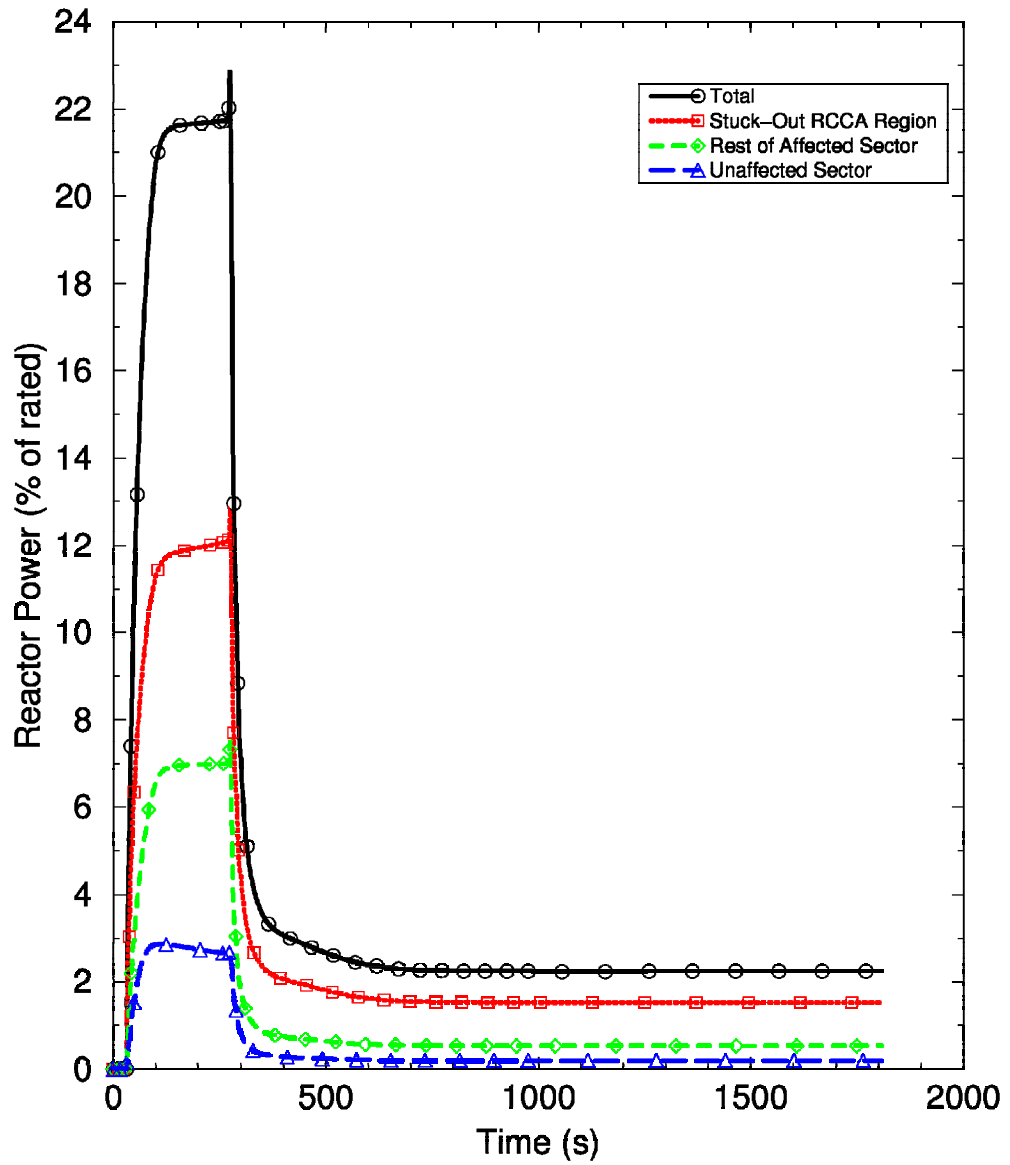
**Figure 06.02.01.04-4-3—Steam Generator Heat Transfer Rates for Limiting Case with MSIV Single Failure**



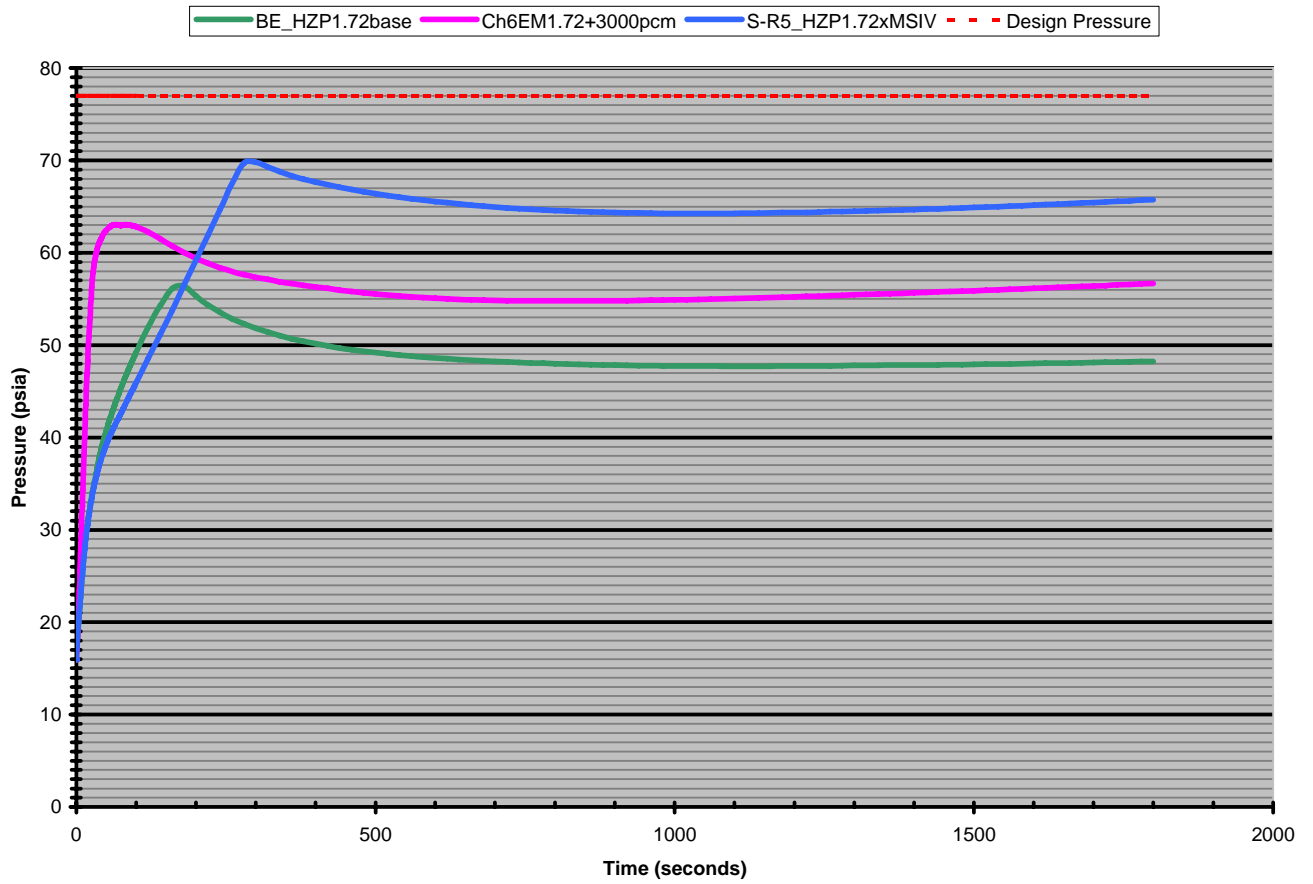
**Figure 06.02.01.04-4-4—Excess MFW in Limiting Chapter 15.1.5 MSLB Scenario**



**Figure 06.02.01.04-4-5—Reactor Power for Limiting Case with MSIV Single Failure**

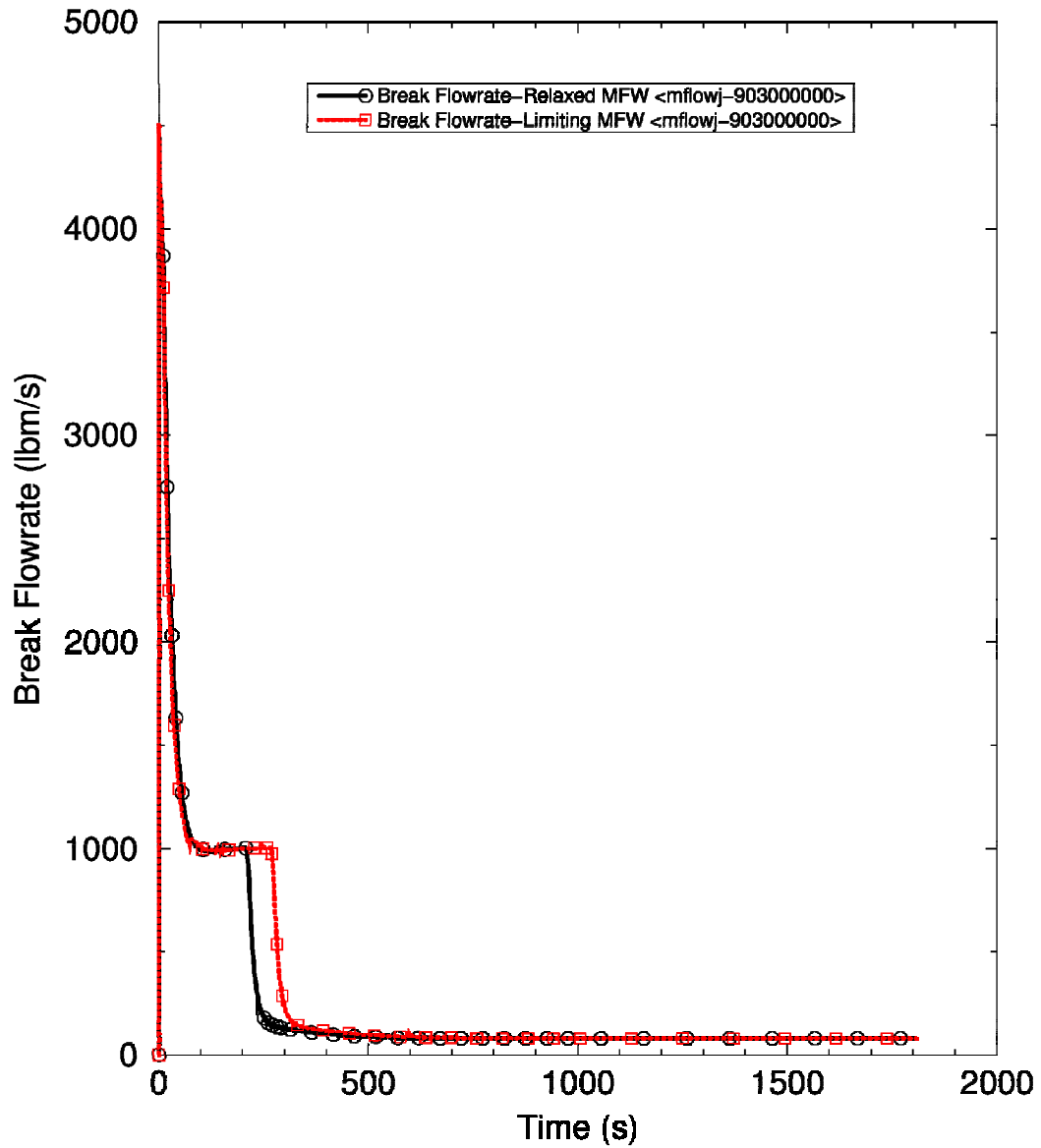


**Figure 06.02.01.04-4-6—GOTHIC Calculated Peak Containment Pressure Comparisons**

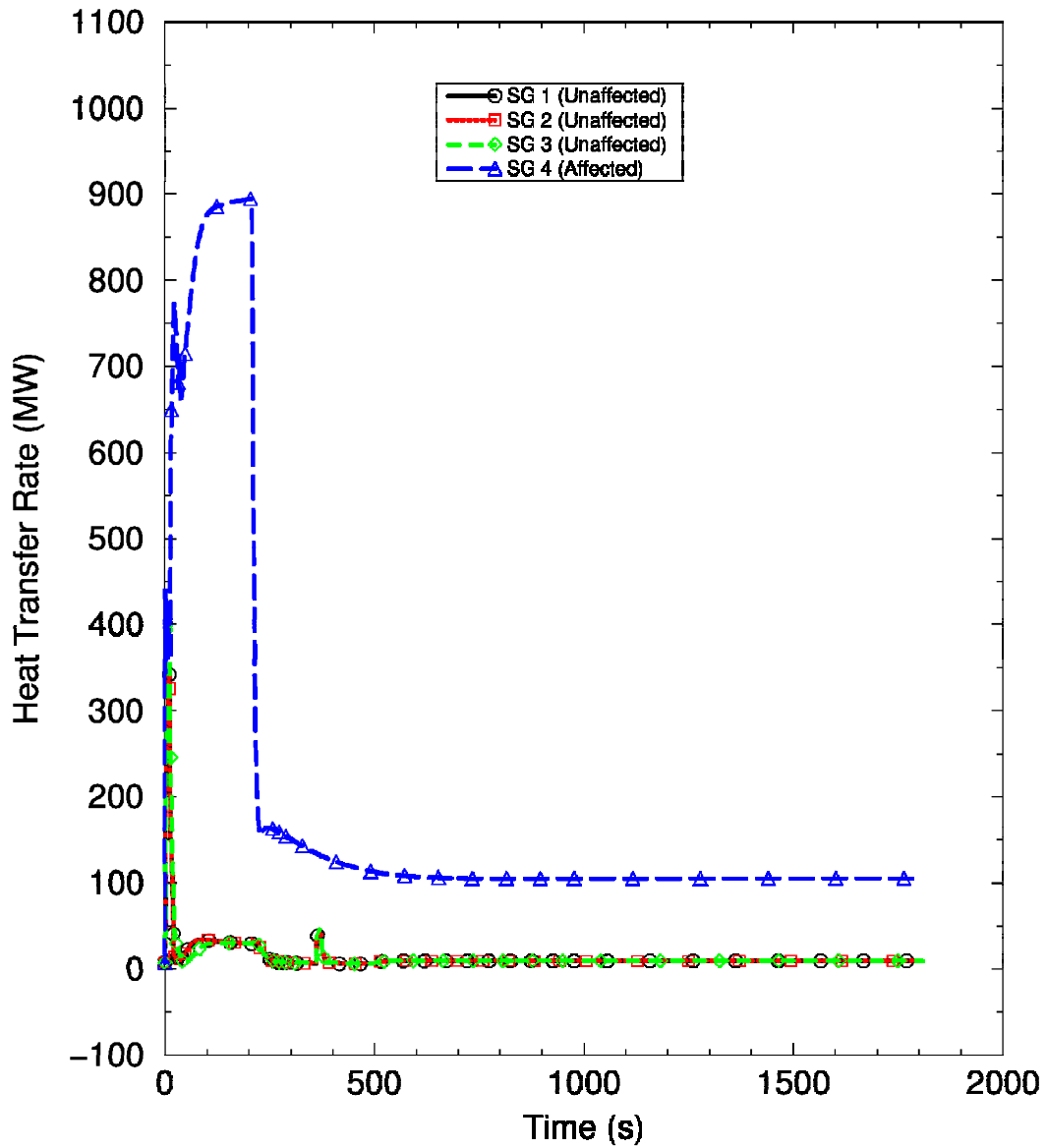




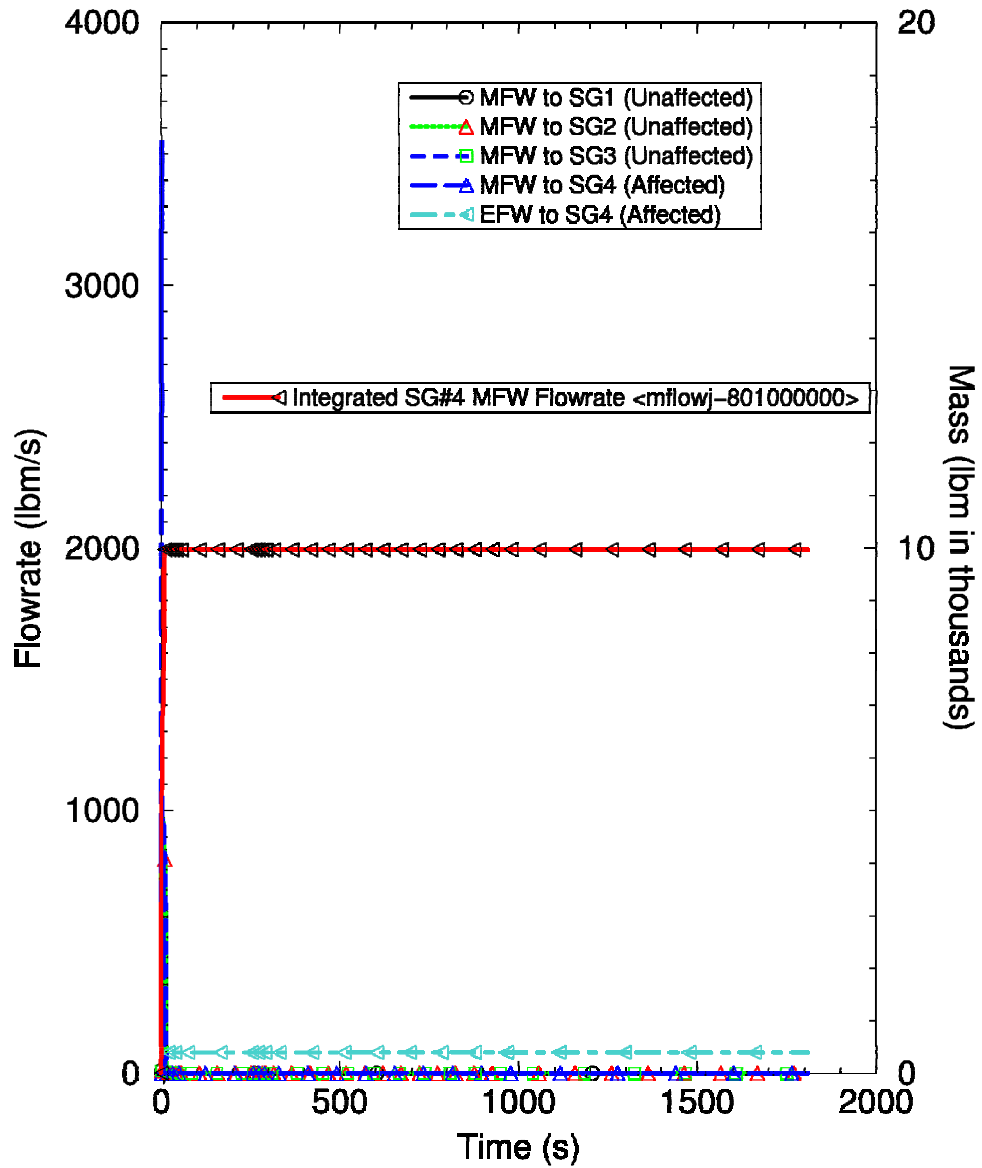
**Figure 06.02.01.04-4-7—Break Flow Rate Comparison for Limiting Case with MSIV Single Failure and Relaxed MFW Conservatism**



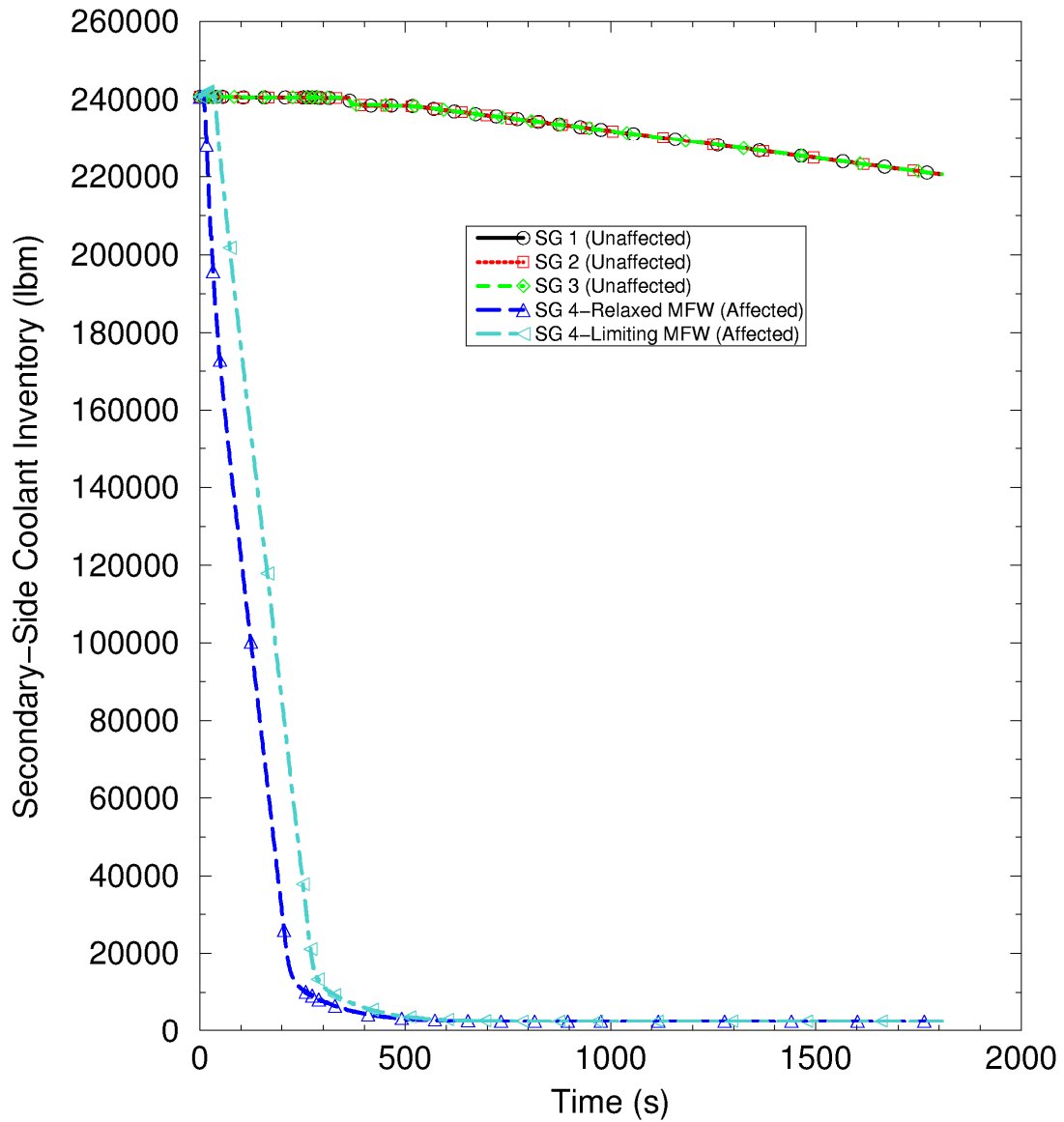
**Figure 06.02.01.04-4-8—Steam Generator Heat Transfer Rates for Limiting Case with MSIV Single Failure and Relaxed MFW Conservatism**



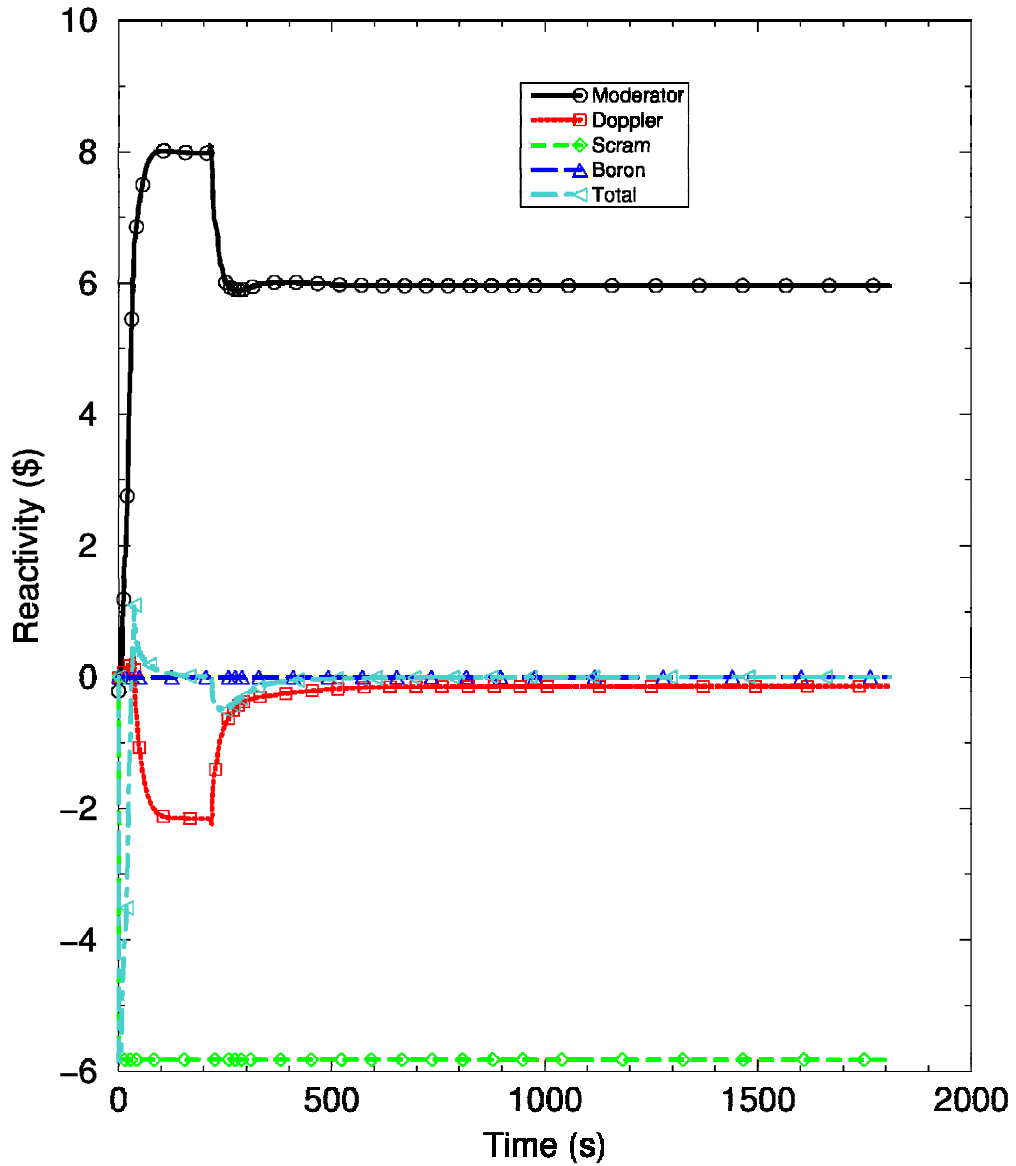
**Figure 06.02.01.04-4-9—Feedwater Flow Rates for Limiting Case with MSIV Single Failure and Relaxed MFW Conservatism**



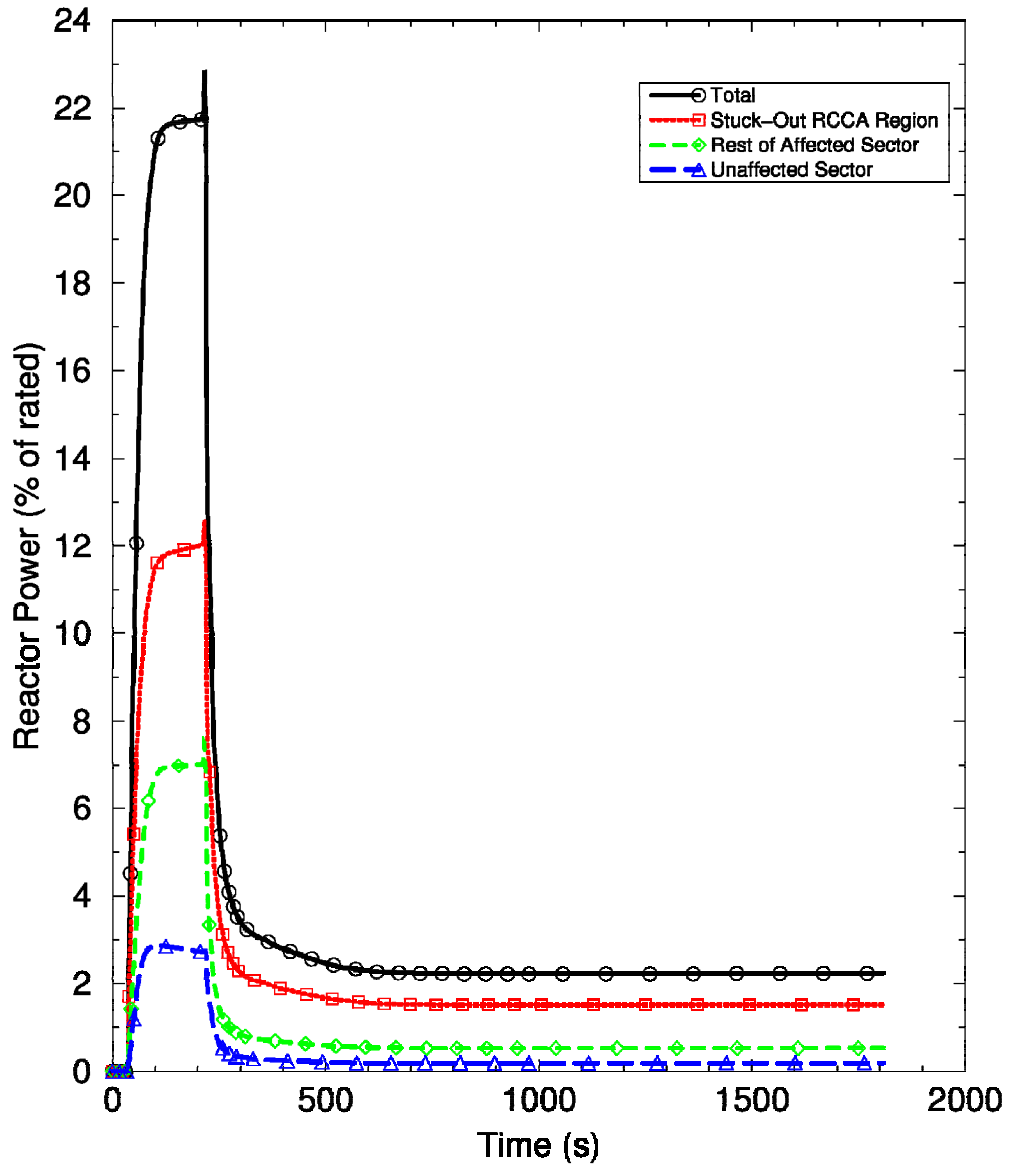
**Figure 06.02.01.04-4-10—Steam Generator Secondary-Side Coolant Inventories for Limiting Case with MSIV Single Failure and Relaxed MFW Conservatism**



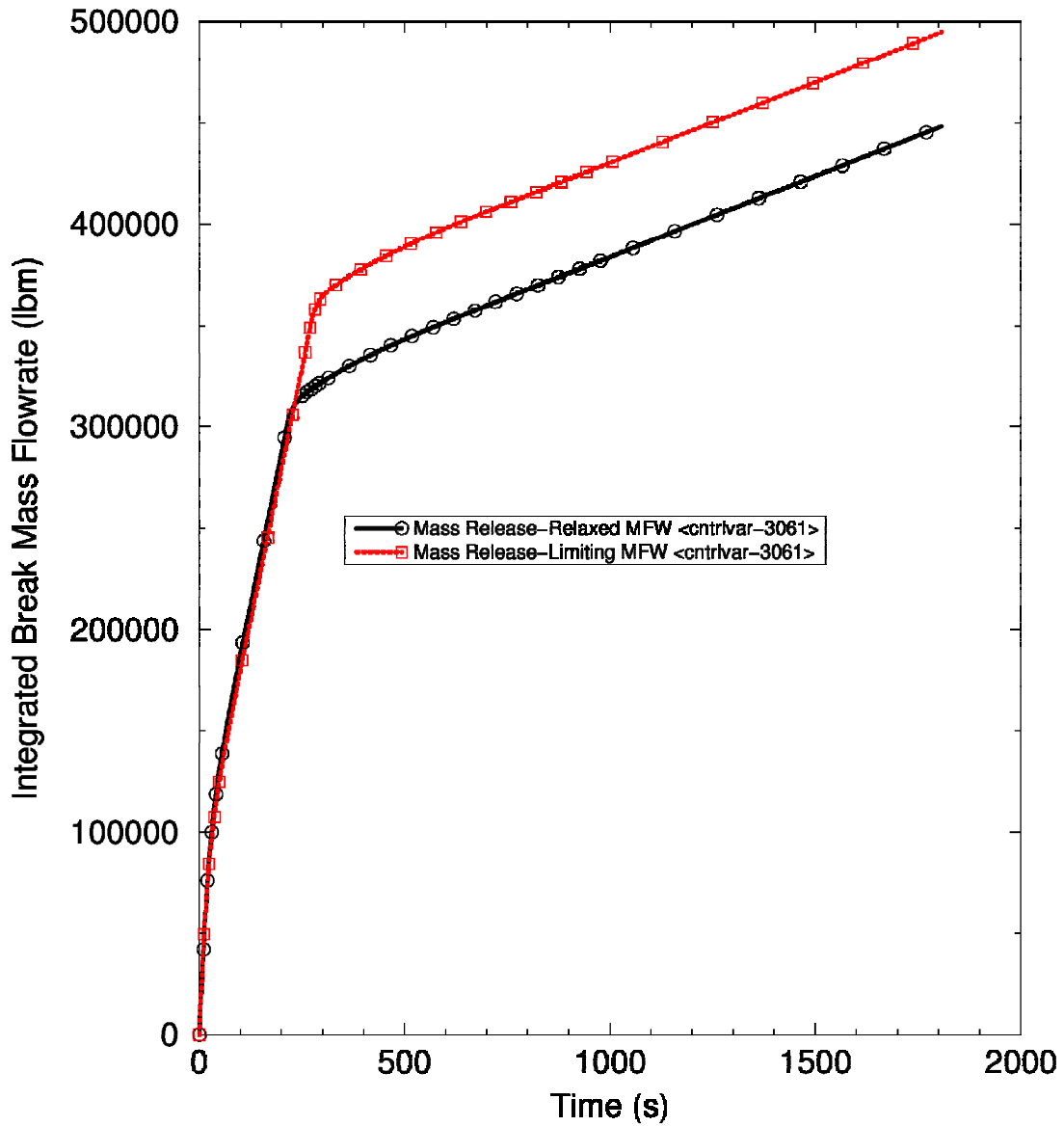
**Figure 06.02.01.04-4-11—Reactivity for Limiting Case with MSIV Single Failure and Relaxed MFW Conservatism**



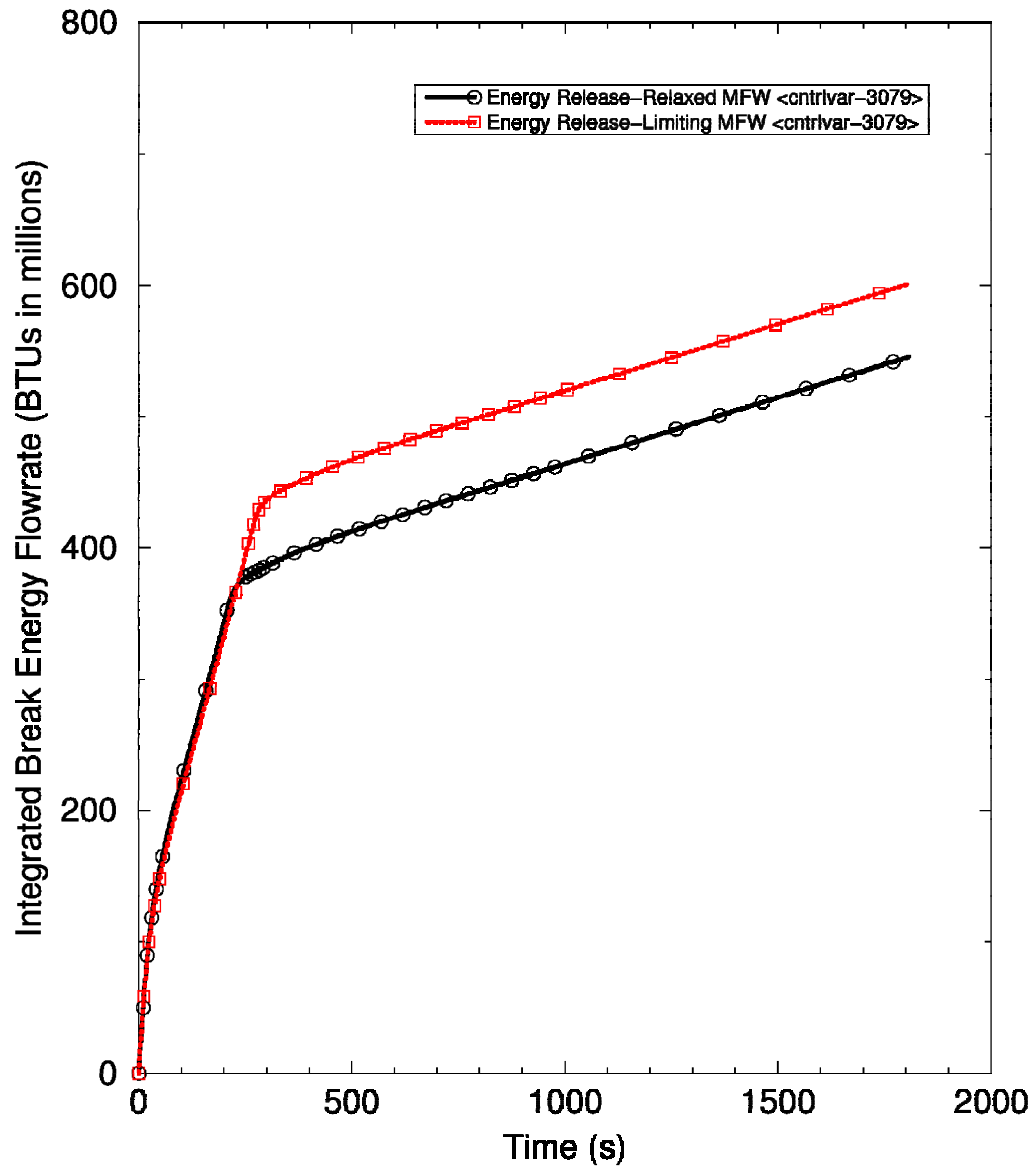
**Figure 06.02.01.04-4-12—Reactor Power for Limiting Case with MSIV Single Failure and Relaxed MFW Conservatism**



**Figure 06.02.01.04-4-13—Integrated Break Mass Flowrate Comparison for Limiting Case with MSIV Single Failure and Relaxed MFW Conservatism**

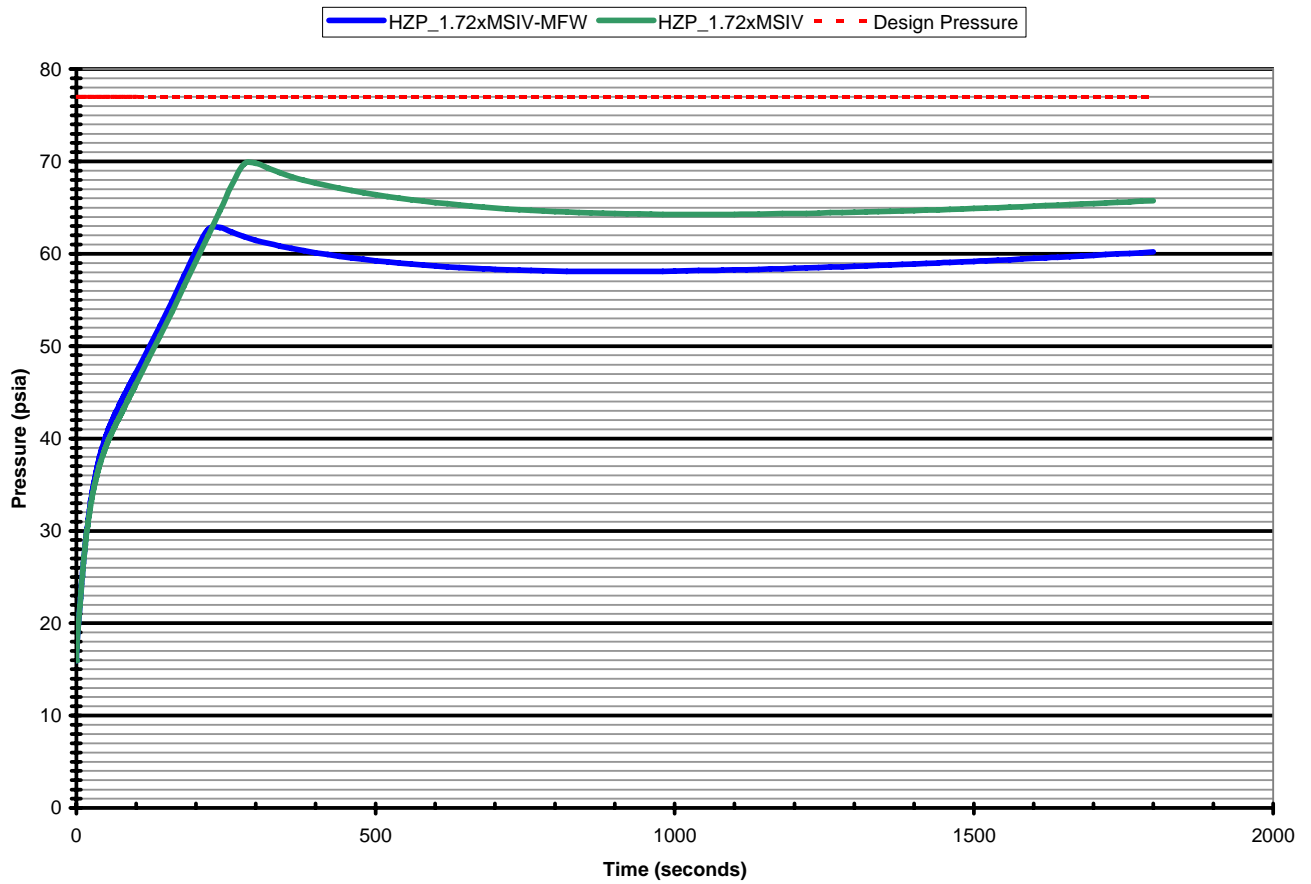


**Figure 06.02.01.04-4-14—Integrated Break Energy Flowrate Comparison for Limiting Case with MSIV Single Failure and Relaxed MFW Conservatism**

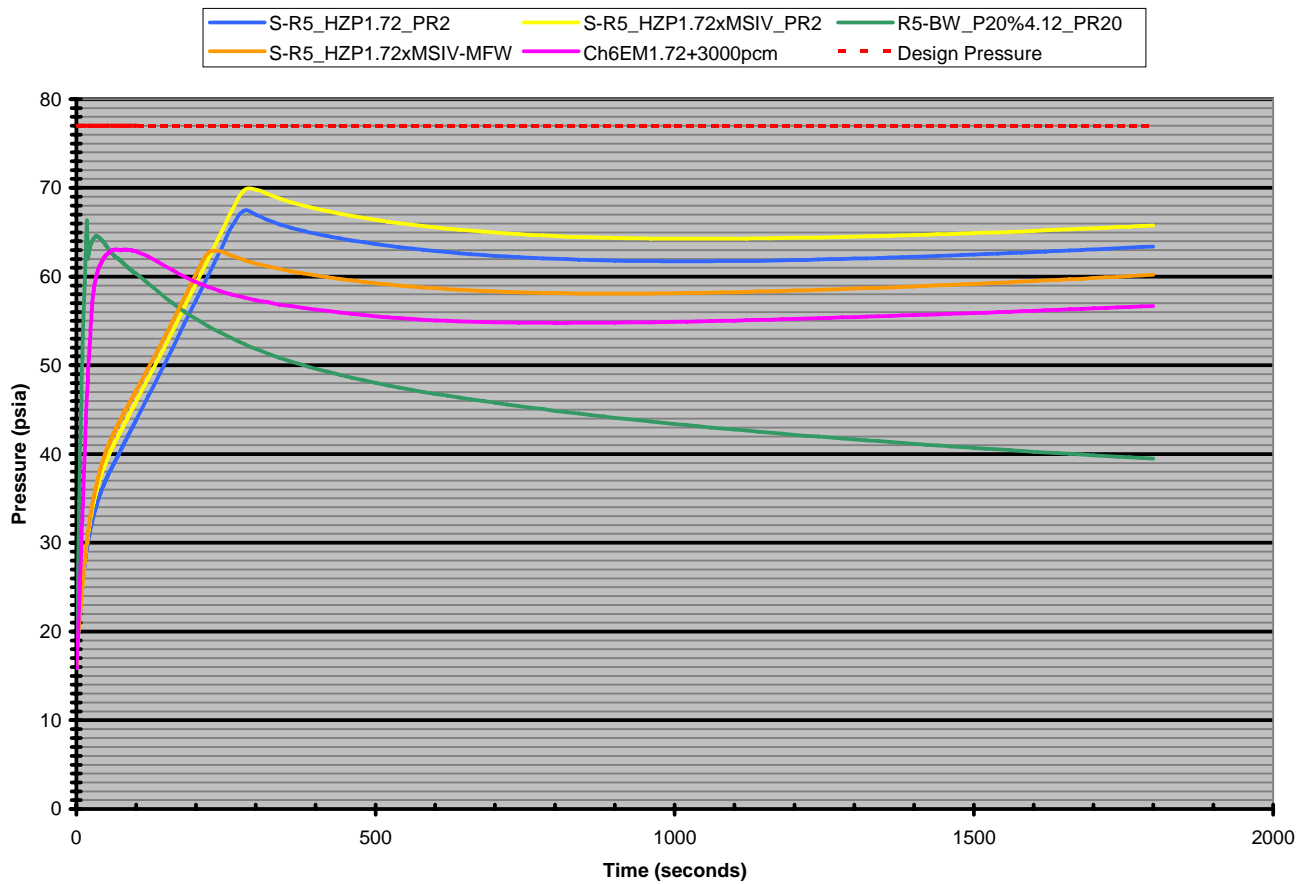




**Figure 06.02.01.04-4-15—Containment Peak Pressure Comparison for Limiting Case with MSIV Single Failure and Relaxed MFW Conservatism**



**Figure 06.02.01.04-4-16—Comparison of Results**



**FSAR Impact:**

The U.S. EPR FSAR will not be changed as a result of this question.