



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 30, 2010

Mr. M. J. Ajluni
Nuclear Licensing Director
Southern Nuclear Operating Company, Inc.
P.O. Box 1295
Bin - 038
Birmingham, Alabama 35201-1295

SUBJECT: VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2, REQUEST
FOR ADDITIONAL INFORMATION REGARDING RELIEF REQUEST
VEGP-ISI-ALT-03, VERSION 2 (TAC NOS. ME3753 AND ME3752)

Dear Mr. Ajluni:

By letter to the U.S. Nuclear Regulatory Commission (NRC) dated April 7, 2010, Southern Nuclear Operating Company, Inc. (SNC), submitted a proposed alternative, VEGP-ISI-ALT-03, Version 2, for the Vogtle Electric Generating Plants, Units 1 and 2, from certain examination qualification requirements of the American Society of Mechanical Engineers, *Boiler and Pressure Vessel Code* (ASME Code), Section XI. SNC proposes the use of alternate root mean square error (RMSE) values of 0.189 inch for Supplement 10 and 0.245 inch for Supplements 10 and 2 combined. We have reviewed the April 7, 2010, submittal and have determined that additional information is necessary as follows.

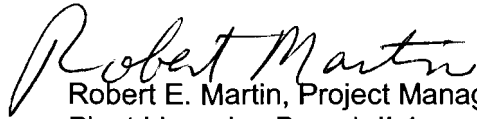
VEGP-ISI-ALT-03, Version 2 states that "In the event that an indication is detected that requires depth sizing, a process will be used where the difference between the required RMSE and vendor-demonstrated RMSE will be added to the measured through-wall depth for comparison with the Section IWB-3500 acceptance criteria." More specifically, the acceptance criteria specified for Class 1, Category R-A, Item R1.15, Reactor Pressure Vessel (RPV) nozzle-to-safe-end dissimilar metal (DM) butt welds and the adjacent Category R-A, Item B1.20, austenitic safe-end welds is IWB-3514. However, the 2007 Edition of the ASME Code with 2009 Addenda, no longer permits the use of IWB-3514 for pressurized-water reactors with nickel alloy welds that are susceptible to primary water stress corrosion cracking (PWSCC) for planar, surface-connected flaws that are in contact with the reactor coolant environment during normal operation. Additionally, MPR-139, Section 7, Evaluation Methodologies, specifies that "any flaw attributed to PWSCC, regardless of depth, will be evaluated even if it meets the IWB-3500 requirements." As such, please justify the use of IWB-3514 acceptance criteria when the flaw is determined to be surface-connected.

M. Ajluni

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Please provide a response within thirty (30) days of the date of this letter.

Sincerely,

A handwritten signature in cursive script that reads "Robert Martin".

Robert E. Martin, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-424 and 50-425

cc: Distribution via Listserv

M. Ajluni

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Please provide a response within thirty (30) days of the date of this letter.

Sincerely,

/RA/

Robert E. Martin, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-424 and 50-425

cc: Distribution via Listserv

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