# NRC INSPECTION MANUAL

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93813	Reserved for Part 52 Operational Readiness Assessment Team
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93814	Reserved for Independent Design Verification Program
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94300	Status of Plant Readiness for an Operating Licensee 02/14/86
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94301	Reserved for Part 52 Status of 10 CFR 52 Plant Readiness for Fuel
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95001	Inspection for One or Two White Inputs in a Strategic
05000	Performance Area 11/09/09 (09-026)
95002	Inspection for One Degraded Cornerstone or Any three White Inputs in a Strategic Performance Area 11/09/09 (09-026)
95003	Inspection for Repetitive Degraded Cornerstones, Multiple
*•	Degraded Cornerstones, Multiple Yellow Inputs, or One
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	·····

95003.01 Emergency Preparedness 01/15/09 (09-002)

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- 95003.02 Guidance for Conducting an Independent NRC Safety Culture Assessment) 01/15/09 (09-002)
- 9600 Materials Safety and Security
  - 96001 Contingency Response Annual Force-On-Force Testing Category 1 Fuel Cycle Facilities 02/14/06 (06-003)
- 9700 <u>Reserved</u>
- 9800 <u>Reserved</u>

<u>GUIDANCE</u>

9900 Guidance<sup>2</sup> 99000 Technical Guidance 05/12/86 (86-26) **Technical Guidance** ASME Section III American Society of Mechanical Engineers ASME Boiler and Pressure Vessel Code, Section III 06/16/89 (89-011) ASME Section III American Society of Mechanical Engineers ASME Boiler and & XI Pressure Vessel Code, Section III & XI 11/12/96 (96-022) ASME Section XI American Society of Mechanical Engineers ASME Boiler and Pressure Vessel Code, Section XI 12/08/86 (86-54) ASME Section XI American Society of Mechanical Engineers ASME, Section XI 05/12/86 (86-26) Oper Det Process **Operability Determinations & Functionality Assessments For** Resolution of Degraded or Nonconforming Conditions Adverse to Quality or Safety 09/26/05 (05-026) STS Sec 1.0 Operability 05/12/86 (86-26) STS Sec 1.0 Definitions 12/08/86 (86-54) STS Sec 3.0 Acceptable Measurement Tolerances for Technical Specification Limits 10/01/78 (78-35) STS Sec 3/4.6.4 Containment and Drywell Isolation Valves 06/09/88 (88-06) STS Sec 3.7.5 Ultimate Heat Sink 11/25/88 (88-17) STS Sec 3/4.8.1 TDI Diesel Generator Air Roll Tests 06/09/88 (88-06) STS Sec 4.2.8.X.X Surveillance Requirements - 125-Volt Battery Bank 10/01/79 (79-22) STS Sec 4.5.1 Standard Technical Specifications (BWR) 05/12/86 (86-26) STS Sec 4.8.1.1.2 Surveillance Testing of Diesel Generator Lockout Feature 05/12/86 (86-26) STS Surveillance Sections - Locked or Otherwise Secured Components 10/01/77 (77-25)

<sup>&</sup>lt;sup>2</sup>[XXXX.XX] attached to each document in Part 9900, is the file name for that document.

PART GUIDANCE	
STS Interp	Licensee Technical Specification Interpretations 02/03/97 (97-001)
Maintenance	
	Voluntary Entry into Limiting Conditions For Operation Action Statements to Perform Preventive Maintenance 01/17/02 (02-001)
	Assessing On-Line Leak Sealing of ASME Code Class 1 & 2 07/15/97 (97-011)
	Filled Organic Coatings Used in Maintenance of Safety Related Equipment 10/11/94 (94-018)
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	Steam Generator Tube-to-Secondary Leakage 09/09/03 (03-033) [TG_Mech SGTUBELKG.wpd]
	Ice Condenser System Inspections 06/06/05 (05-015)
<u>Operations</u>	
	Notice of Enforcement Discretion 02/07/05 (05-005) [NOED.TG]
	Notice of Enforcement Discretion for Gaseous Diffusion Plants 05/03/99 (99-008) [NOEDGAS.TG]
	Operability Determinations & Functionality Assessments for Resolution of Degraded or Nonconforming Conditions Adverse to Quality or Safety 04/16/08 (08-016) [TG STSODP]
	Operation - Safety and Compliance [TG 9900 SAFETY] 09/11/07 (07-028)

# PART GUIDANCE

# PART 10 CFR GUIDANCE

10	Title 10 of the Code of Federal Regulations 12/08/86 (86-54)
10CFR71 and 49CFR171-178	Transportation of Radioactive Material 12/01/04 (04-027)
50.55(e)	Construction Deficiency Reporting 01/31/89 (89-002)
50.55a	Codes and Standards 02/22/73 (73-9)
50.59	10 CFR 50.59, Changes, Tests, and Experiments 03/13/01 (01-008) [50_59.CFR]
Part 50, App A	Definition of Leak-Before-Break Analysis 09/26/96 (96-020)
Part 50	Appendix J, Containment Integrated Leak Rate Testing 12/08/86 (86-54)
70.51	Material Balance, Inventory, and Records Requirements 12/19/74 (74-11)
70.52	Report of Accidental Criticality or Loss of Special Nuclear Material 11/15/74 (74-010)

Interpretations

END