

Crystal River Nuclear Power Plant, Florida

Update: September 10, 2009
 Next Update: September 2010

Crystal River 3 Nuclear Power Plant Net Generation and Capacity, 2008

Unit	Net Capacity MW(e)	Generation (Million Kilowatt Hours)	Capacity Factor (Percent)	Type	On Line Date	License Expiration Date
3	838	7,000	95	PWR	3/13/1977	12/3/2016

PWR =pressurized light water reactors.

Sources

Description: The Crystal River Energy Complex is located in Citrus County, Florida. The site consists of approximately 4,700 acres. The single nuclear unit shares the site with 4 fossil-fueled generators.

Crystal River, Unit 3

Nuclear Steam System Supplier (NSSS Vendor) = Babcock & Wilcox
 Architect Engineer = Gilbert Associates
 Owner = [Progress Energy Corporation](#) is majority owner. The distribution of ownership is shown in the table that follows
 Operator (Licensee) = Progress Energy Corporation.

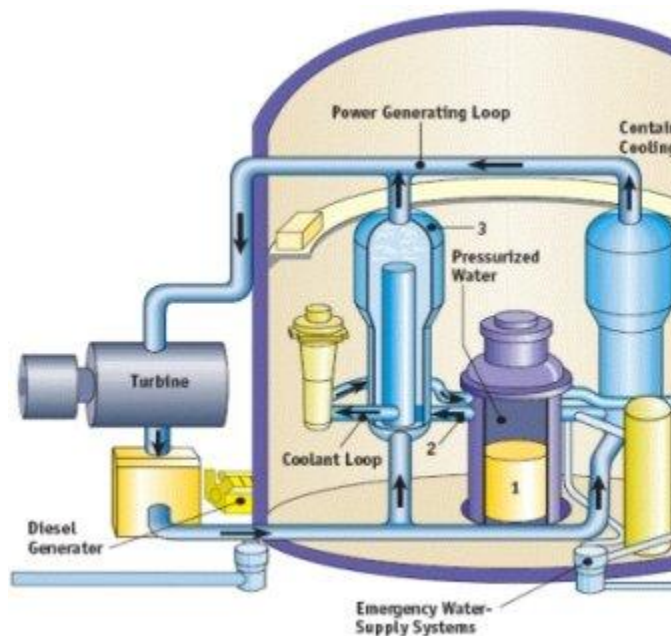
Ownership of Crystal River 3

Company	Percent of Total
Progress Energy Florida	91.8
Seminole Electric Cooperative	1.7
Orlando Utilities Commission	1.6
Gainesville Regional Utilities	1.4
City of Ocala, Florida	1.3
Leesburg Electric Department	0.8
Kissimmee Utility Authority	0.7
Utilities Commission of New Smyrna Beach	0.6
City of Alachua, Florida	0.1
City of Bushnell, Florida	*
* Less than 0.1 percent	

U.S. Nuclear Power Plants by State	Plants
Alabama	Browns Ferry
	Farley (Joseph M. Farley)
Arizona	Palo Verde
Arkansas	Arkansas Nuclear One
California	Diablo Canyon
	San Onofre
Connecticut	Millstone
Florida	Crystal River 3
	St Lucie
	Turkey Point
Georgia	Hatch (Edwin I. Hatch)
	Vogtle
Illinois	Braidwood
	Byron
	Clinton
	Dresden
	LaSalle County
	Quad Cities
Iowa	Duane Arnold
Kansas	Wolf Creek
Louisiana	River Bend
	Waterford
Maryland	Calvert Cliff
Massachusetts	Pilgrim
Michigan	Donald C. Cook
	Enrico Fermi (Fermi)
	Palisades
Minnesota	Monticello
	Prairie Island
Mississippi	Grand Gulf
Missouri	Callaway
Nebraska	Cooper
	Fort Calhoun
New Hampshire	Seabrook
New Jersey	Hope Creek
	Oyster Creek
	Salem Creek
New York	Fitzpatrick (James A. Fitzpatrick)

Pressurized-Water Reactor (PWR)

In a typical commercial pressurized light-water reactor (1) the reactor core generates heat, (2) pressurized-water in the primary coolant loop carries the heat to the steam generator, (3) inside the steam generator heat from the primary coolant loop vaporizes the water in a secondary loop producing steam, (4) the steam line directs the steam to the main turbine causing it to turn the turbine generator, which produces electricity. The unused steam is exhausted to the condenser where it is condensed into water. The resulting water is pumped out of the condenser with a series of pumps, reheated, and pumped back to the steam generator. The reactor core contains fuel assemblies which are cooled by water, which is force-circulated by electrically powered pumps. Emergency cooling water is supplied by other pumps, which can be powered by onsite diesel generators. Other safety systems, such as the containment cooling system, also need power.



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Containment: According to the U.S. Nuclear Regulatory Commission, unit 3 is has dry, ambient pressure.¹

	Indian Point
	Nile Mile Point
	R.E. Ginna (Ginna, or Robert E. Ginna)
North Carolina	Brunswick
	McGuire
	Shearon-Harris(Harris)
Ohio	Davis-Besse
	Perry
Pennsylvania	Beaver Valley
	Limerick
	Peach Bottom
	Susquehanna
	Three Mile Island
South Carolina	Catawba
	H.B. Robinson
	Oconee
	Virgil C. Summer (Summer)
Tennessee	Sequoyah
	Watts Bar
Texas	Comanche Peak
	South Texas
Vermont	Vermont Yankee
Virginia	North Anna
	Surry
Washington	Columbia Generating Station
Wisconsin	Kewaunee
	Point Beach

¹Dry, Ambient Pressure: a reactor containment design whose safety has been evaluated on the basis of having a dry air atmosphere at ambient pressure (0 psig) prior to the onset of a loss of coolant accident or steam pipe break. The containment design (concrete and steel tendons) must be able to take the full thermal and pressure stresses associated with the rapid energy release (steam) from a major pipe break.

Sources for Data in Table: **Capacity**, for purposes of this report, is the net summer capability as reported in Energy Information Administration (EIA) Form EIA-860, "Annual Electric Generator Report." **Capacity Factor** is a percentage calculation in which the maximum possible generation (based on net summer capability) is divided into the actual generation then multiplied by 100. **Generation** is the net electricity output reported by plant owners on Form EIA-906, "Power Plant Report." **Type of Unit:** All U.S. commercial reactors currently in operation are one of two types: **BWR** (boiling water reactor) or **PWR** (pressurized light water reactor). The type, on-line date, and the license expiration date are published annually in *Information Digest* by the U.S. Nuclear Regulatory Commission.

see also:

[annual nuclear statistics back to 1953](#)

[projected electricity capacity to 2030](#)

[international electricity statistics](#)

