



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 18, 2010

Mr. Thomas Joyce
President and Chief Nuclear Officer
PSEG Nuclear LLC
P.O. Box 236
Hancocks Bridge, NJ 08038

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE
HOPE CREEK GENERATING STATION LICENSE RENEWAL APPLICATION
FOR PRIMARY CONTAINMENT VALVE MATERIAL (TAC NO ME1832)

Dear Mr. Joyce:

By letter dated August 18, 2009, Public Service Enterprise Group Nuclear, LLC, submitted an application pursuant to Title 10 of the *Code of Federal Regulations* Part 54 for renewal of Operating License No. NPF-57 for the Hope Creek Generating Station. The staff of the U.S. Nuclear Regulatory Commission (NRC or the staff) is reviewing this application in accordance with the guidance in NUREG-1800, "Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants." During its review, the staff has identified areas where additional information is needed to complete the review. The staff's request for additional information is included in the Enclosure. Further requests for additional information may be issued in the future.

Items in the enclosure were provided to John Hufnagel and other members of your staff, and a mutually agreeable date for the response is within 30 days from the date of this letter. If you have any questions, please contact me by telephone at 301-415-2981 or by e-mail at bennett.brady@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read "Bennett M. Brady".

Bennett M. Brady, Project Manager
Projects Branch 1
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosure:
As stated

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE HOPE CREEK
GENERATING STATION LICENSE RENEWAL APPLICATION FOR PRIMARY CONTAINMENT
VALVE MATERIAL (TAC NO. ME1832)

3.2.1.20-01

Background

Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants (SRP LR) Table 3.2-1, item 20 addresses loss of fracture toughness due to thermal aging embrittlement in cast austenitic stainless steel (CASS) piping, piping components, and piping elements exposed to treated water (borated or unborated) with temperature greater than 250°C (482°F). License renewal application (LRA) item 3.2.1-20 states that this line item is not applicable because there are no CASS piping, piping components or piping elements, subject to treated water with temperature greater than 482°F in the engineered safety features systems. Final safety analysis report Table 6.1-2 states that core spray and residual heat removal system low pressure core injection (RHR LPCI) valves are constructed of CASS. LRA Section 2.3.2 states that both the core spray and RHR LPCI systems provide a primary containment and reactor coolant pressure boundary (RCPB) and are in scope as per 10 CFR 54.4(a)(1).

Issue

The staff does not have enough information to determine if (1) the core spray or RHR LPCI primary containment (reactor vessel side) or RCPB valves are constructed of CASS and (2) given the valves' positions whether they are subject to an environment exceeding 482°F.

Request

Are the core spray and RHR LPCI primary containment (reactor vessel side) or RCPB valves constructed of CASS and are they exposed to an environment exceeding 482°F?

ENCLOSURE

August 18, 2010

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/RA/

Bennett M. Brady, Project Manager
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Office of Nuclear Reactor Regulation

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NAME	B. Brady	IKing	B. Pham	B. Brady
DATE	08/09/10	08/04/10	08/16/10	08/18/10

OFFICIAL RECORD COPY

Letter to T. Joyce from B. Brady dated August 18, 2010

**SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE
HOPE CREEK GENERATING STATION LICENSE RENEWAL APPLICATION
FOR PRIMARY CONTAINMENT VALVE MATERIAL (TAC NO ME1832)**

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