

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 2, 2011

Mr. David A. Heacock President and Chief Nuclear Officer Dominion Energy Kewaunee, Inc. Innsbrook Technical Center 5000 Dominion Boulevard Glen Allen, VA 23060-6711

SUBJECT: KEWAUNEE POWER STATION (KPS) – ISSUANCE OF AMENDMENT FOR

THE CONVERSION TO THE IMPROVED TECHNICAL SPECIFICATIONS WITH BEYOND SCOPE ISSUES (TAC NOS. ME2139, ME2419, ME2420, ME2421,

ME3122, ME3460, AND ME3544)

Dear Mr. Heacock:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 207 to Facility Operating License No. DPR-43 for Kewaunee Power Station (KPS). The amendment consists of changes to the technical specifications (TS) and the license conditions for KPS in response to your application dated August 24, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML092440398), as supplemented by letters dated:

- October 22, 2009 (ADAMS Accession No. ML093070092),
- April 13, 2010 (ADAMS Accession Nos. ML101060517 and ML101040090).
- May 12, 2010 (ADAMS Accession No. ML101380399).
- July 1, 2010 (ADAMS Accession No. ML101890404),
- July 16, 2010 (ADAMS Accession No. ML102370370).
- August 18, 2010 (ADAMS Accession No. ML102371064).
- September 7, 2010 (ADAMS Accession No. ML1027303830,
- September 8, 2010 (ADAMS Accession No. ML102580700).
- October 15, 2010 (ADAMS Accession No. ML102920037), and
- December 2, 2010 (ADAMS Accession No. ML103400328).

- 2 -

The amendment converts the current TS (CTS) to the improved TS (ITS) and relocates certain requirements to other licensee-controlled documents. The ITS are based on:

- NUREG-1431, "Standard Technical Specifications (STS) Westinghouse Plants, " Revision 3.0:
- "NRC Final Policy Statement on Technical Specification Improvements for Nuclear Power Reactors," dated July 22, 1993 (58 FR 39132); and
- 10 CFR 50.36, "Technical Specifications."

Technical Specification Task Force changes were also incorporated. The purpose of the conversion is to provide clearer and more readily understandable requirements in the TS for KPS to ensure safe operation. In addition, the amendment includes a number of issues that were considered beyond the scope of NUREG -1431.

Included in the amendments are the following three conditions for the KPS operating license:

- the requirement to relocate certain CTS requirements into licensee-controlled documents during the implementation of the ITS;
- the schedule for the first performance of new and revised surveillance requirements for the ITS; and
- deferral of certain TS requirements.

These license conditions, which are discussed in the enclosed safety evaluation (SE), are part of the implementation of the ITS and constitute regulatory commitments that the NRC staff is relying upon in approving the amendment.

The ITS will become the governing TS for KPS upon the date of implementation. This means that until the implementation of the ITS is complete, the CTS shall remain in effect. Upon complete implementation of the ITS, please submit a letter stating as such within 14 days of the date of completion.

A copy of the related SE and its Attachments are enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

- 3 -

If you have any questions concerning this letter and the SE, contact me at 301-415-3049 or email Karl.Feintuch@nrc.gov.

Sincerely,

Karl Feintuch, Project Manager Plant Licensing Branch III-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Karl Feintuck

Docket No. 50-305

Enclosures:

- 1. Amendment No. 207 to DPR-43
- 2. Safety Evaluation

Safety Evaluation Attachments

- 1. Safety Evaluation Attachment 1 (from Balance of Plant Branch SBPB)
- 2. Safety Evaluation Attachment 2 (from Technical Specifications Branch ITSB)
- 3. Safety Evaluation Attachment 3 (from Instrumentation and Control Branch EICB)
- 4. Safety Evaluation Attachment 4 (from Electrical Engineering Branch EEEB)
- 5. Safety Evaluation Attachment 5 (from Accident Dose Branch AADB)
- 6. Safety Evaluation Attachment 6 (from Component Performance and Testing Branch CPTB)
- 3. List of Regulatory Commitments
- 4. Tables of Technical Specification Changes:
 - 1. Table A Administrative Changes
 - 2. Table L Less Restrictive Changes
 - 3. Table LA Removed Details
 - 4. Table M More Restrictive Changes
 - 5. Table R Relocated Specifications
- 5. List of Acronyms and Abbreviations
- 6. Improved Technical Specifications

cc w/encls: Distribution via listserv



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

DOMINION ENERGY KEWAUNEE, INC.

DOCKET NO. 50-305

KEWAUNEE POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 207 License No. DPR-43

- 1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Dominion Energy Kewaunee, Inc. dated August 24, 2009 as supplemented by letters dated October 22, 2009, April 13, 2010, May 12, 2010, July 1, 2010, July 16, 2010, August 18, 2010, September 7, 2010, September 8, 2010, October 15, 2010, and December 2, 2010, complies with the standards and requirements of the Atomic Energy Act of 1954 as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-43 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 207, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

- 3. This amendment authorizes the relocation of certain Technical Specification requirements to other licensee-controlled documents. Implementation of this amendment shall include the relocation of these requirements to the other documents, as described in (1) Sections D and E of the NRC staff's Safety Evaluation, and (2) Table LA of Removed Details and Table R of Relocated Specifications attached to the NRC staff's Safety Evaluation, which is enclosed with this amendment.
- 4. License condition, 2.C.(8), is deleted.
- 5. License condition, 2.C.(9), is deleted.
- 6. New license conditions are added to Section C of this license to address performance of new and revised Surveillance Requirements (SRs):

For SRs that are new in this amendment, the first performance is due at the end of the first surveillance interval, which begins on the date of implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being reduced, the first reduced surveillance interval begins upon completion of the first surveillance performed after implementation of this amendment.

For SRs that existed prior to this amendment that have modified acceptance criteria, the first performance subject to the modified acceptance criteria is due at the end of the first Surveillance interval that began on the date the Surveillance was last performed prior to the implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being extended the first extended surveillance interval begins upon completion of the last surveillance performed prior to the implementation of this amendment.

7. New license condition is added to address a currently non-functional RCS Hot Leg A Temperature Indication, as follows:

Following implementation of License Amendment No. 207, the requirement for the RCS Hot Leg A Temperature Indication to be OPERABLE as required by TS 3.3.3 and TS 3.3.4 may be deferred until startup after the first outage of sufficient duration to repair the RCS Hot Leg A Temperature Indication. Specifically, TS Table 3.3.3-1, Function 3 will only require 1 channel to be OPERABLE, and TS Table B 3.3.4-1, Function 4.a will not be applicable. Following the startup after the first outage of sufficient duration to repair the RCS Hot Leg A Temperature Indication, TS Table 3.3.3-1 Function 3 and TS Table B 3.3.4-1, Function 4.a requirements will be applicable.

- 8. As part of the implementation of this amendment, the NRC expects that the actions identified in the "List of Regulatory Commitments" enclosed with this amendment will be carried out in accordance with their indicated due dates or events, including any interim actions necessary to make the commitments active as planned. The NRC accepts and relies upon the definition for the word "active" as applied in the previous sentence and as intended by Dominion Energy Kewaunee, Inc. Namely, when used in the context of USAR revisions, an "active" USAR change/revision is one that has been through all of the requisite reviews and approvals, and is, from the time designated as "active" forward, considered as part of the then current USAR in effect. Further, within the electronic document management system, "active" is used to indicate that the changed or revised document is the current document of record.
- 9. This license amendment is effective as of its date of issuance and shall be implemented on or before February 23, 2011.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert Pascarelli, Chief Plant Licensing Branch III-1

Marcarelly.

Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Attachment: Changes to the Facility Operating License and Technical Specifications

Date of Issuance: February 2, 2011

ATTACHMENT TO LICENSE AMENDMENT NO. 207

FACILITY OPERATING LICENSE NO. DPR-43

DOCKET NO. 50-305

Replace the following pages of the Facility Operating License No. DPR-43 with the revised pages attached. The changed area is identified by a marginal line.

REMOVE	INSERT		
Page 3 Page 4 Page 5	Page 3 Page 4 Page 5		
Page 5a	Page 6		

Replace the following pages of Appendix A, Technical Specifications, with the revised pages attached. The revised pages are identified by amendment number. A marginal line to indicate the area of change is not used since all pages are new.

REMOVE	INSERT
All pages	All pages

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR, Chapter 1: (1) Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70, (2) is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and (3) is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady-state reactor core power levels not in excess of 1772 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 207, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Fire Protection

The licensee shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the licensee's Fire Plan, and as referenced in the Updated Safety Analysis Report, and as approved in the Safety Evaluation Reports, dated November 25, 1977, and December 12, 1978 (and supplement dated February 13, 1981) subject to the following provision:

The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission, only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

(4) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21, is entitled: "Nuclear Management Company Kewaunee Nuclear Power Plant Physical Security Plan (Revision 0)" submitted by letter dated October 18, as supplemented by letter dated October 21, 2004, July 26, 2005, and May 15, 2006.

(5) Deleted

(6) Steam Generator Upper Lateral Supports

The design of the steam generator upper lateral supports may be modified by reducing the number of snubbers from four (4) to one (1) per steam generator.

- (7) Deleted
- (8) Deleted
- (9) Deleted

(10) Mitigation Strategy License Condition

Develop and maintain strategies for addressing large fires and explosions and that include the following key areas:

- (a) Fire fighting response strategy with the following elements:
 - 1. Pre-defined coordinated fire response strategy and guidance
 - 2. Assessment of mutual aid fire fighting assets
 - 3. Designated staging areas for equipment and materials
 - 4. Command and control
 - 5. Training of response personnel
- (b) Operations to mitigate fuel damage considering the following:
 - 1. Protection and use of personnel assets
 - 2. Communications
 - 3. Minimizing fire spread
 - 4. Procedures for implementing integrated fire response strategy
 - 5. Identification of readily-available pre-staged equipment
 - 6. Training on integrated fire response strategy
 - 7. Spent fuel pool mitigation measures
- (c) Actions to minimize release to include consideration of:
 - 1. Water spray scrubbing
 - 2. Dose to onsite responders

(11) Seismic Analysis Methodology for Auxiliary Building Crane

The licensee shall use the seismic analysis methodology submitted by letter dated July 7, 2008, supplemented on September 19, 2008, and March 17, 2009, and approved by the NRC staff in Amendment No. 205, for analysis of the Auxiliary Building crane. The licensee shall update the USAR to reflect this approval and in accordance with the schedule specified by 10 CFR 50.71(e).

(12) Implementation of New and Revised Surveillance Requirements

- (a) For SRs that are new in Amendment No. 207, the first performance is due at the end of the first surveillance interval, which begins on the date of implementation of this amendment.
- (b) For SRs that existed prior to Amendment No. 207, whose intervals of performance are being reduced, the first reduced surveillance interval begins upon completion of the first surveillance performed after implementation of this amendment.
- (c) For SRs that existed prior to Amendment No. 207, that have modified acceptance criteria, the first performance subject to the modified acceptance criteria is due at the end of the surveillance interval that began on the date the surveillance was last performed prior to the implementation of this amendment.
- (d) For SRs that existed prior to Amendment No. 207, whose intervals of performance are being extended, the first extended surveillance interval begins upon completion of the last surveillance performed prior to the implementation of this amendment.

(13) Removed Details and Requirements Relocated to Other Controlled Documents

License Amendment No. 207 authorizes the relocation of certain technical specifications and operating license conditions, if applicable, to other licensee-controlled documents. Implementation of this amendment shall include relocation of these requirements to the specified documents.

(14) <u>Deferral of Certain Technical Specification Requirements</u>

Following implementation of License Amendment No. 207, the requirement for the RCS Hot Leg A Temperature Indication to be OPERABLE as required by TS 3.3.3 and TS 3.3.4 may be deferred until startup after the first outage of sufficient duration to repair the RCS Hot Leg A Temperature Indication.

Specifically, TS Table 3.3.3-1, Function 3 will only require 1 channel to be OPERABLE, and TS Table B 3.3.4-1, Function 4.a will not be applicable. Following the startup after the first outage of sufficient duration to repair the RCS Hot Leg A Temperature Indication, TS Table 3.3.3-1 Function 3 and TS Table B 3.3.4-1, Function 4.a requirements will be applicable.

- D. The licensee shall comply with applicable effluent limitations and other limitations and monitoring requirements, if any, specified pursuant to Section 401(d) of the Federal Water Pollution Control Act Amendments of 1972.
- E. This license is effective as of the date of issuance, and shall expire at midnight on December 21, 2013.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by

A. Giambusso, Deputy Director for Reactor Projects Directorate of Licensing

Attachment:

Appendices A and B - Technical Specifications

Date of Issuance: December 21, 1973

D. Heacock

- 3 -

If you have any questions concerning this letter and the SE, contact me at 301-415-3049 or email Karl.Feintuch@nrc.gov.

Sincerely,

/RA/

Karl Feintuch, Project Manager Plant Licensing Branch III-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket No. 50-305 Enclosures:

- 1. Amendment No. 207 to DPR-43
- 2. Safety Evaluation

Safety Evaluation Attachments

- 1. Safety Evaluation Attachment 1 (from Balance of Plant Branch SBPB)
- 2. Safety Evaluation Attachment 2 (from Technical Specifications Branch ITSB)
- 3. Safety Evaluation Attachment 3 (from Instrumentation and Control Branch EICB)
- 4. Safety Evaluation Attachment 4 (from Electrical Engineering Branch EEEB)
- 5. Safety Evaluation Attachment 5 (from Accident Dose Branch AADB)
- 6. Safety Evaluation Attachment 6 (from Component Performance and Testing Branch CPTB)
- 3. List of Regulatory Commitments
- 4. Tables of Technical Specification Changes:
 - 1. Table A Administrative Changes
 - 2. Table L Less Restrictive Changes
 - 3. Table LA Removed Details
 - 4. Table M More Restrictive Changes
 - Table R Relocated Specifications
- 5. List of Acronyms and Abbreviations
- 6. Improved Technical Specifications

cc w/encls: Distribution via listsery

DISTRIBUTION:

KScales, NRR

RidsNrrDorlLpl3-1 Resource

PUBLIC LPL3-1 R/F VCusumano,NRR
RidsNrrDirsItsb Resource KBucholtz, NRR
RidsOgcRp Resource GWaig, NRR
RidsNrrLABTully Resource RidsNrrPMKewaunee Resource RidsNrrDorlDpr Resource
RidsArrSAcnw MailCTR Resource

MMcConnell, NRR BMarcus, NRR DRahn, NRR WBlumberg, NRR LBenton, NRR RWolfgang, NRR SGardocki GMatheru, NRR

Package Accession Number: ML103350500 Amendment Accession Number: ML102020662

Safety Evaluation and Safety Evaluation Attachments Accession Number: ML103360504

List of Regulatory Commitments Accession Number: ML110260316 Tables A, L, LA, M, and R Accession Number: ML103480093 List of Acronyms and Abbreviations Accession Number: ML103370691 Technical Specification pages Accession Number: ML102020664

Note: Safety evaluation input on beyond-scope issues has been provided by the respective technical branches. The memos

transmitting the SEs may be found in ADAMS under the TAC Nos. listed in the title above.

OFFICE	LPL3-1/PM	LPL3-1/LA	Tech Branch	DE/EEEB	DIRS/ITSB	OGC NLO	LPL3-1/BC
NAME	KFeintuch	BTully	See Note	RMathew	RElliott	LSubin	RPascarelli
DATE	1/10/11	1/10/11	Various	1/31/11	1/11/11	1/25/11	2/02/11