

July 13, 2010

Stephen E. Hedges Site Vice President

WO 10-0046

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555

Reference: Letter WO 10-0007, dated January 18, 2010, from M. W. Sunseri, WCNOC, to USNRC

Subject: Docket No. 50-482: Licensee Event Report 2009-010-01, Failure to Meet LCO 3.0.4b. During Transition from Mode 4 to Mode 3

Gentlemen:

The Reference submitted Licensee Event Report (LER) 2009-010-00 in accordance with 10 CFR 50.73, "Licensee event report system," paragraph (a)(2)(i)(B) as a condition prohibited by the plant's Technical Specifications (TS). The LER stated that a supplement would be sent providing the cause of the event and additional actions being taken. The enclosed LER provides that information. Changes are annotated in the margins.

The Attachment provides a list of regulatory commitments. If you have any questions concerning this matter, please contact me at (620) 364-4190, or Mr. Richard D. Flannigan, Manager Regulatory Affairs at (620) 364-4117.

Sincerely, E. Hedges Stepher

SEH/rlt

Attachment Enclosure

cc: E. E. Collins (NRC), w/a, w/e G. B. Miller (NRC), w/a, w/e B. K. Singal (NRC), w/a, w/e Senior Resident Inspector (NRC), w/a, w/e

IE22 HCR

P.O. Box 411 / Burlington, KS 66839 / Phone: (620) 364-8831 An Equal Opportunity Employer M/F/HC/VET Attachment to WO 10-0046 Page 1 of 1

# LIST OF REGULATORY COMMITMENTS

The following table identifies those actions committed to by WCNOC in this document. Any other statements in this submittal are provided for information purposes and are not considered to be regulatory commitments. Please direct questions regarding these commitments to Mr. Richard Flannigan at (620) 364-4117.

	DUE DATE/EVENT
A new procedure will be created to provide instructions and requirements for performing a Technical Specification LCO 3.0.4 Mode Change Review.	October 10, 2010

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transition fra (LCO) 3.0.4 progress to Mode 3 with driven AFW A risk asset	ber 17, 2 om Mod b. with f return t the exe f pump f ssment	2009, at e 4, Hot the turbin he plant ception o to a func was con	2353 I t Shutc ne driv to ser of com ctional	nours Cer lown, to M ven auxilia vice follow pleting so status. d as requi	ntral Star Node 3, I ary feedw wing a re ome wor	ndard Ti Hot Star water (A efueling k activiti	me (C ndby, i FW) p outage es and 4b. an	ST), Co n accord ump inc with th d post m d identif	dance wit operable. e plant re naintenan fied that p	h Limitir Heat-u eady to t ce testin	onnel authori ng Condition f p activities we ransition fron ng to restore d train signs hed from Mod	or Opera ere in n Mode 4 the turbin may be	4 to ne

posted as deemed necessary. On November 18, 2009, at 0024 hours, the plant transitioned from Mode 4 to Mode 3 under the provisions of LCO 3.0.4b. for the turbine driven AFW pump with no protected equipment signs posted for the motor driven AFW pump rooms.

On December 17, 2009, at a weekly meeting with the NRC Resident Inspector, the inspector identified a potential violation of LCO 3.0.4b. for not establishing risk management actions prior to transitioning from Mode 4 to Mode 3 as required by the technical specification. The risk management action not established was protected equipment signage for the motor driven AFW pump rooms.

NRC FORM 366A (9-2007)		•	U.S. NUCLEA	R REGU	LATORY	COMMISS	ION
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PLANT CONDITIONS PRIOR TO EVENT:					а 4.		
MODE – 4			. <b>*</b>				
Power - 0	· .				i.		
EVENT DESCRIPTION:							
activities and post maintenance testing to rest BA] to a functional status. Procedure AP 22C-003, "Operational Risk Ass Modes 1, 2, or 3. An Operational Risk Assess November 22, 2009, in preparation for transiti The day-shift Shift Manager had annotated Se activities" of the Operational Risk Assessment	sessment Progra sment was prepa oning from Mode ection 5.3, "Com	am," specif ared for the 4 to Mode pensatory (	ies that this proc period Novemb 3 and plant sta measures added	cedure is per 17, 2 irtup on d becaus	s applica 009 thro Novemi se of un	able in ough oer 17. schedule	
specified that when entering Mode 3 with the t should be placed on the motor driven AFW pu not in effect since the plant had not transitione A separate risk assessment for entering Mode completed as required by Limiting Condition for "Comments/Risk Management Actions/Comp	ump room doors ed to Mode 3. e 3 from Mode 4 or Operation (LC	However with the tu O) 3.0.4b.	, this Operationa	al Risk Á N pump	inopera	ient was able was	
It is expected that the PAL02 activity wil pump. Protected train signs may be po- practice ( <u>prior to the mode change</u> ) to v stability issues. This would be in addition Operator. No additional precautions or Attachment B are deemed necessary.	sted as deemed erify with the Tra on to the hourly p	necessary Insmission Iredictive g	<ul> <li>Without PAL0 Operator that the rid assessment</li> </ul>	2, it will nere are by the T	be a go no knov ransmis	od wn grid ssion	
The transition from Mode 4 to Mode 3 did not Shift Manager in preparation for transitioning f warrant the placement of the protected equipr Operational Risk Assessment was not update	from Mode 4 to 1 ment signs on the	Aode 3 det e motor dri	ermined that pla ven pump room	int cond	itions di	d not	
On November 17, 2009, at 2353 hours, Contro Shutdown, to Mode 3, Hot Standby, in accord and no specific risk management actions requ	ance with LCO 3						e
On November 18, 2009, at 0024 hours, the pl 3.0.4b. for the turbine driven AFW pump being AFW pump rooms.							
The turbine driven AFW pump was declared of	operable on Nov	ember 19.	2009 at 0433 hr	urs	,		

NRC FORM 366A (9-2007)

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NRC FORM 366A		U.S.	NUCLEAR REGULATORY COMMISSION
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LICENSEE EVENT RE	PORT (LER)	•	

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**BASIS FOR REPORTABILITY:** 

This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(i)(B), to report a TS LCO 3.0.4b. violation that occurred when the plant entered a specified mode without establishing the specified risk management action.

TS LCO 3.0.4 states, in part "When an LCO is not met, entry into a MODE or other specified condition in the Applicability shall only be made:

b. After performance of a risk assessment addressing inoperable systems and components, consideration of the results, determination of the acceptability of entering the MODE or other specified condition in the Applicability, and establishment of risk management actions, if appropriate; exceptions to this Specification are stated in the individual Specifications; or" .....

On December 17, 2009, at a weekly meeting with the NRC Resident Inspector, the inspector identified a potential violation of LCO 3.0.4b. for not establishing risk management actions prior to transitioning from Mode 4 to Mode 3 as required by the technical specification. The risk management action not established was protected equipment signage for the motor driven AFW pump rooms.

During the period of time when the turbine driven AFW pump was inoperable, at least one motor driven AFW pump was operable and capable of providing the feedwater flow required for removal of decay heat. As such, review of this event determined that it did not meet the criteria for reporting under 10 CFR 50.73(a)(2)(v) as an event or condition that could have prevented the fulfillment of a safety function.

### CAUSE:

The Shift Manager failed to ensure the protected train signs were placed as annotated in the Operational Risk Assessment. No formal process existed to ensure the risk assessment performed for LCO 3.0.4b. was compatible with the applicable Shutdown Risk Assessment or Operational Risk Assessment required for normal mode transition.

### **ACTIONS TAKEN:**

On November 18, 2009, the day-shift Shift Manager reviewed current plant conditions and after consultation with the Outage Control Center Operations Manager posted protected equipment signs on the motor driven AFW pump room doors based on procedure AI 22C-10, "Operations Work Controls."

Procedure AI 22C-010, "Operations Work Controls," has been revised to state that protected equipment signs will be placed to limit access to equipment that is protected in accordance with the risk assessment.

A new procedure will be created to provide instructions and requirements for performing a TS LCO 3.0.4 mode change review, by October 10, 2010.

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#### SAFETY SIGNIFICANCE:

The AFW System mitigates the consequences of any event with loss of normal feedwater. The design basis of the AFW System is to supply water to the steam generator to remove decay heat and other residual heat by delivering at least the minimum required flow rate to the steam generators at pressures corresponding to the lowest steam generator safety valve set pressure plus 3% accumulation. In addition, the AFW System must supply enough makeup water to replace steam generator secondary inventory lost as the unit cools to MODE 4 conditions. Sufficient AFW flow must also be available to account for flow losses such as pump recirculation and line breaks. The AFW System consists of two motor driven AFW pumps and one steam turbine driven pump configured into three trains. Each motor driven pump provides 100% of the feedwater flow required for removal of decay heat from the reactor. During the period of time when the turbine driven AFW pump was inoperable, at least one motor driven AFW pump was operable and capable of providing the feedwater flow required for removal of decay heat.

## **OPERATING EXPERIENCE/PREVIOUS EVENTS:**

Amendment No. 155 dated October 7, 2004, approved changes to the TS requirements for Mode change limitations in LCO 3.0.4 and Surveillance Requirement (SR) 3.0.4. The changes to the TSs were based on Technical Specification Task Force (TSTF) Standard TS (STS) Change Traveler TSTF-359, Revision 9, "Increase Flexibility in MODE Restraints." Since the implementation of Amendment No. 155, LCO 3.0.4b. has been invoked 3 times (including this event). A review was performed for similar events at WCGS where a Mode change was not performed in accordance with the requirements of LCO 3.0.4b. No events of this type were identified since the implementation of Amendment No. 155.

LER 2008-008-02 reported entering Mode 4 from Mode 3 at the beginning of Refueling Outage 16 and entering Mode 3 from Mode 4 at the end of Refueling Outage 16 without ensuring that the Residual Heat Removal System [EIIS Code: BP] was operable.

LER 2008-006-00 reported entering Mode 4 from Mode 5 with a containment spray train [EIIS Code: BE] inoperable. A post maintenance visual weld examination for leakage had not been completed as required by the In-service Testing Program prior to Mode 4 entry.