



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

October 25, 2010

Mr. Mark E. Reddemann  
Chief Executive Officer  
Energy Northwest  
P.O. Box 968 (Mail Drop 1023)  
Richland, WA 99352-0968

SUBJECT: COLUMBIA GENERATING STATION - ISSUANCE OF AMENDMENT RE:  
CONTROL ROD OPERABILITY AND CONTROL ROD SCRAM  
ACCUMULATORS (TAC NO. ME3326)

Dear Mr. Reddemann:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 216 to Facility Operating License No. NPF-21 for the Columbia Generating Station. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated February 8, 2010, as supplemented by letter dated August 17, 2010.

The amendment modifies TS requirements related to TS 3.1.3, "Control Rod OPERABILITY," and TS 3.1.5, "Control Rod Scram Accumulators," to be consistent with NUREG-1433, "Standard Technical Specifications General Electric Plants, BWR/4." The amendment also corrects certain typographical errors.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "C. Lyon".

Carl F. Lyon, Project Manager  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-397

Enclosures:

1. Amendment No. 216 to NPF-21
2. Safety Evaluation

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENERGY NORTHWEST

DOCKET NO. 50-397

COLUMBIA GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 216  
License No. NPF-21

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Energy Northwest (licensee), dated February 8, 2010, as supplemented by letter dated August 17, 2010, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-21 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 216 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

In addition, an editorial correction is made to Paragraph 2.C(33)(b).

3. The license amendment is effective as of its date of issuance and shall be implemented within 90 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Facility  
Operating License No. NPF-21  
and Technical Specifications

Date of Issuance: October 25, 2010

ATTACHMENT TO LICENSE AMENDMENT NO. 216

FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Facility Operating License No. NPF-21 and Appendix A, Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Facility Operating License

<u>REMOVE</u>	<u>INSERT</u>
-3-	-3-
-9b-	-9b-

Technical Specification

<u>REMOVE</u>	<u>INSERT</u>
3.1.3-2	3.1.3-2
3.1.3-3	3.1.3-3
3.1.3-4	3.1.3-4
3.1.3-5	--
3.1.5-1	3.1.5-1
3.1.5-2	3.1.5-2
3.1.5-3	3.1.5-3
3.7.3-3	3.7.3-3
5.5-13	5.5-13

- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source of special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- (6) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to store byproduct, source and special nuclear materials not intended for use at Columbia Generating Station. The materials shall be no more than 9 sealed neutron radiation sources designed for insertion into pressurized water reactors and no more than 40 sealed beta radiation sources designed for use in area radiation monitors. The total inventory shall not exceed 24 microcuries of strontium-90, 20 microcuries of uranium-235, 30 curies of plutonium-238, and 3 curies of americium-241.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of full power (3486 megawatts thermal). Items in Attachment 1 shall be completed as specified. Attachment 1 is hereby incorporated into this license.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 216 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- (a) For Surveillance Requirements (SRs) not previously performed by existing SRs or other plant tests, the requirement will be considered met on the implementation date and the next required test will be at the interval specified in the Technical Specifications as revised in Amendment No. 149.

- (b) The first performance of the periodic assessment of CRE habitability, Specification 5.5.14.c.(ii), shall be within 3 years, plus the 9-month allowance of SR 3.0.2, as measured from November 6, 2003, the date of the most recent successful tracer gas test, or within the next 9 months if the time period since the most recent successful tracer gas test is greater than 3 years.
- (c) The first performance of the periodic measurement of CRE pressure, Specification 5.5.14.d, shall be within 24 months, plus the 184 days allowed by SR 3.0.2, as measured from March 23, 2006, the date of the most recent successful pressure measurement test, or within 184 days if not performed previously.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. (continued)	<p>A.3 Perform SR 3.1.3.2 for each withdrawn OPERABLE control rod.</p> <p><u>AND</u></p> <p>A.4 Perform SR 3.1.1.1.</p>	<p>24 hours from discovery of Condition A concurrent with THERMAL POWER greater than the low power setpoint (LPSP) of the RWM</p> <p>72 hours</p>
B. Two or more withdrawn control rods stuck.	B.1 Be in MODE 3.	12 hours
C. One or more control rods inoperable for reasons other than Condition A or B.	<p>C.1 -----NOTE----- RWM may be bypassed as allowed by LCO 3.3.2.1, if required, to allow insertion of inoperable control rod and continued operation. -----</p> <p>Fully insert inoperable control rod.</p> <p><u>AND</u></p> <p>C.2 Disarm the associated CRD.</p>	<p>3 hours</p> <p>4 hours</p>

(continued)



SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.1.3.1	Determine the position of each control rod.	24 hours
SR 3.1.3.2	<p>-----NOTE-----            Not required to be performed until 31 days after the control rod is withdrawn and THERMAL POWER is greater than the LPSP of the RWM.            -----</p> <p>Insert each withdrawn control rod at least one notch.</p>	31 days
SR 3.1.3.3	Verify each control rod scram time from fully withdrawn to notch position 5 is $\leq 7$ seconds.	In accordance with SR 3.1.4.1, SR 3.1.4.2, SR 3.1.4.3, and SR 3.1.4.4
SR 3.1.3.4	Verify each control rod does not go to the withdrawn overtravel position.	<p>Each time the control rod is withdrawn to "full out" position</p> <p><u>AND</u></p> <p>Prior to declaring control rod OPERABLE after work on control rod or CRD System that could affect coupling</p>

3.1 REACTIVITY CONTROL SYSTEMS

3.1.5 Control Rod Scram Accumulators

LCO 3.1.5 Each control rod scram accumulator shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTIONS

-----NOTE-----  
Separate Condition entry is allowed for each control rod scram accumulator.  
-----

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One control rod scram accumulator inoperable with reactor steam dome pressure $\geq 900$ psig.	A.1 -----NOTE----- Only applicable if the associated control rod scram time was within the limits of Table 3.1.4-1 during the last scram time Surveillance. ----- Declare the associated control rod scram time "slow."	8 hours
	OR A.2 Declare the associated control rod inoperable.	8 hours

(continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>B. Two or more control rod scram accumulators inoperable with reactor steam dome pressure <math>\geq</math> 900 psig.</p>	<p>B.1 Restore charging water header pressure to <math>\geq</math> 940 psig.</p>	<p>20 minutes from discovery of Condition B concurrent with charging water header pressure &lt; 940 psig</p>
	<p><u>AND</u></p> <p>B.2.1 -----NOTE----- Only applicable if the associated control rod scram time was within the limits of Table 3.1.4-1 during the last scram time Surveillance. -----</p>	
	<p>Declare the associated control rod scram time "slow."</p> <p><u>OR</u></p> <p>B.2.2 Declare the associated control rod inoperable.</p>	<p>1 hour</p> <p>1 hour</p>

(continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>C. One or more control rod scram accumulators inoperable with reactor steam dome pressure &lt; 900 psig.</p>	<p>C.1 Verify the associated control rod is fully inserted.</p> <p><u>AND</u></p> <p>C.2 Declare the associated control rod inoperable.</p>	<p>Immediately upon discovery of charging water header pressure &lt; 940 psig</p> <p>1 hour</p>
<p>D. Required Action B.1 or C.1 and associated Completion Time not met.</p>	<p>D.1 -----NOTE----- Not applicable if all inoperable control rod scram accumulators are associated with fully inserted control rods. -----</p> <p>Place the reactor mode switch in the shutdown position.</p>	<p>Immediately</p>

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>F. Two CREF subsystems inoperable during OPDRVs.</p> <p><u>OR</u></p> <p>One or more CREF subsystems inoperable due to inoperable CRE boundary during OPDRVs.</p>	<p>F.1 Initiate action to suspend OPDRVs.</p>	<p>Immediately</p>

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.7.3.1 Operate each CREF subsystem for <math>\geq 10</math> continuous hours with the heaters operating.</p>	<p>31 days</p>
<p>SR 3.7.3.2 Perform required CREF filter testing in accordance with the Ventilation Filter Testing Program (VFTP).</p>	<p>In accordance with the VFTP</p>

(continued)

5.5 Programs and Manuals

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5.5.13 Battery Monitoring and Maintenance Program (continued)

- b. Actions to equalize and test battery cells that had been discovered with electrolyte level below the top of the plates; and
  - c. Actions to verify that the remaining cells are  $\geq 2.07$  V when a cell or cells have been found to be  $< 2.13$  V.
- 

5.5.14 Control Room Envelope Habitability Program

A Control Room Envelope (CRE) Habitability Program shall be established and implemented to ensure that CRE habitability is maintained such that, with an OPERABLE Control Room Emergency Filtration (CREF) System, CRE occupants can control the reactor safely under normal conditions and maintain it in a safe condition following a radiological event, hazardous chemical release, or a smoke challenge. The program shall ensure that adequate radiation protection is provided to permit access and occupancy of the CRE under design basis accident (DBA) conditions without personnel receiving radiation exposures in excess of 5 rem total effective dose equivalent (TEDE) for the duration of the accident. The program shall include the following elements:

- a. The definition of the CRE and the CRE boundary.
- b. Requirements for maintaining the CRE boundary in its design condition including configuration control and preventive maintenance.
- c. Requirements for (i) determining the unfiltered air inleakage past the CRE boundary into the CRE in accordance with the testing methods and at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.

(continued)

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 216 TO

FACILITY OPERATING LICENSE NO. NPF-21

ENERGY NORTHWEST

COLUMBIA GENERATING STATION

DOCKET NO. 50-397

1.0 INTRODUCTION

By application dated February 8, 2010, as supplemented by letter dated August 17, 2010 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML100490749 and ML102360439, respectively), Energy Northwest (the licensee) requested changes to the Technical Specifications (TSs) (Appendix A to Facility Operating License No. NPF-21) for the Columbia Generating Station (CGS). The requested changes would modify TS requirements related to TS 3.1.3, "Control Rod OPERABILITY," and TS 3.1.5, "Control Rod Scram Accumulators," to be consistent with NUREG-1433, "Standard Technical Specifications [STS] General Electric Plants, BWR/4." The amendment would also correct certain typographical errors.

Specifically, the proposed changes revise TS 3.1.3, "Control Rod OPERABILITY," and TS 3.1.5, "Control Rod Scram Accumulators," to adopt the simplified approach of the STS. Adoption of the STS language will allow the licensee to remove restrictions imposed by analytical methods that are no longer utilized with the implementation of Amendment No. 211, dated May 5, 2009 (ADAMS Accession No. ML091100357).

The licensee also proposes to correct typographical errors introduced in Amendment No. 207, dated June 30, 2008 (ADAMS Accession No. ML081770003 package) in (1) operating license paragraph 2.C(33), (2) TS 3.7.3, "Control Room Emergency Filtration (CREF) System," Condition F, and (3) TS 5.5.14, "Control Room Envelope Habitability Program."

The supplemental letter dated August 17, 2010, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the U.S. Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on April 6, 2010 (75 FR 17442).

## 2.0 REGULATORY EVALUATION

The licensee stated in its application that the Control Rod Drive (CRD) System is designed to satisfy the requirements of the following Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Appendix A, General Design Criteria (GDC):

- GDC 26, "Reactivity control system redundancy and capability," requires, in part, that, "Two independent reactivity control systems of different design principles shall be provided. One of the systems shall use control rods, preferably including a positive means for inserting the rods, and shall be capable of reliably controlling reactivity changes to assure that under conditions of normal operation, including anticipated operational occurrences, and with appropriate margin for malfunctions such as stuck rods, specified acceptable fuel design limits are not exceeded."
- GDC 27, "Combined reactivity control systems capability," requires that, "The reactivity control systems shall be designed to have a combined capability, in conjunction with poison addition by the emergency core cooling system, of reliably controlling reactivity changes to assure that under postulated accident conditions and with appropriate margin for stuck rods the capability to cool the core is maintained."
- GDC 28, "Reactivity limits," requires, in part, that, "The reactivity control systems shall be designed with appropriate limits on the potential amount and rate of reactivity increase to assure that the effects of postulated reactivity accidents can neither (1) result in damage to the reactor coolant pressure boundary... nor (2) sufficiently disturb the core [and other vessel internals] to impair significantly the capability to cool the core."
- GDC 29, "Protection against anticipated operational occurrences," requires that, "The protection and reactivity control systems shall be designed to assure an extremely high probability of accomplishing their safety functions in the event of anticipated operational occurrences."

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include TSs as part of the license. These TSs are derived from the plant safety analyses. In Section 50.36, "Technical specifications," of 10 CFR, the NRC established its regulatory requirements related to the content of TSs. Pursuant to 10 CFR 50.36, TSs are required to include items in the following five specific categories related to station operation: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements (SRs); (4) design features; and (5) administrative controls. The rule does not specify the particular requirements to be included in a plant's TSs.

- The TS Bases for TS 3.1.3, "Control Rod OPERABILITY," states, in part, that "In conjunction with the Reactor Protection System, the CRD System provides the means for the reliable control of reactivity changes to ensure that under

conditions of normal operation, including anticipated operational occurrences, specified acceptable fuel design limits are not exceeded.”

- The TS Bases for TS 3.1.5, “Control Rod Scram Accumulators,” states, in part, that, “The control rod accumulators...are provided to ensure that the control rods scram under varying reactor conditions....The scram accumulators are necessary to scram the control rods within the required insertion times of LCO 3.1.4, “Control Rod Scram Times.”

CGS Final Safety Analysis Report (FSAR) Section 4.3, “Nuclear Design,” incorporates by reference the CGS specific fuel cycle design and reload analyses. These analyses detail the assumptions used in modeling control rod scram times.

The NRC staff reviewed the proposed changes as provided by the licensee in its February 8, 2010, submittal and concludes that the changes do not affect the licensee’s continued compliance with regulatory requirements. The proposed changes adopt STS language, align the TSs with previously approved Amendment No. 211, and correct typographical errors. The licensee continues to meet the above applicable GDCs and TSs.

### 3.0 TECHNICAL EVALUATION

#### 3.1 TS 3.1.3

Required Action C.1 of LCO 3.1.3 states “Verify the total number of “slow” and inoperable control rods is  $\leq$ eight.” The licensee proposes to eliminate this statement and renumber subsequent Required Actions accordingly.

The statement reflects an analytical methodology that is no longer applicable to CGS since the licensee changed to a new fuel vendor, as reflected in the changes to TS 3.1.4, “Control Rod Scram Times,” and TS 5.6.3, “CORE OPERATING LIMITS REPORT (COLR),” approved by Amendment No. 211. Amendment No. 211 allowed incorporation of the analytical methodologies associated with the operation of Global Nuclear Fuel-Americas (GNF) fuel into the licensing basis to support transition to GNF GE14 fuel.

The proposed changes involve the removal of a TS statement pertaining to the number of allowed inoperable or “slow” control rods. The TS controls for the allowed number of inoperable control rods continue to be specified with Condition F of LCO 3.1.3 which requires that if there are nine or more inoperable control rods that the plant be placed in MODE 3 with a Completion Time of 12 hours. The controls for distribution of inoperable control rods continue to be delineated in Required Action A.1 of LCO 3.1.3 which ensures that rod separation criteria are met. The proposed deletion of Required Action C.1 from LCO 3.1.3 does not remove any pertinent controls placed on the number or distribution of inoperable control rods.

The proposed deletion also involves TS controls that limit the number of “slow” control rods to eight. This statement is inconsistent with the current analytical bases and LCO 3.1.4.a, which limits the number and spacing of allowed “slow” control rods. LCO 3.1.4 was revised with Amendment No. 211 and provides the necessary TS controls to ensure that the number and distribution of “slow” control rods continue to meet analytical requirements.

The NRC staff concludes that the proposed change is acceptable, since the assumptions of the accident analyses and transient analyses continue to be protected by TS 3.1.3 and TS 3.1.4.

3.2 TS 3.1.5

Required Action A.1 of LCO 3.1.5 currently states:

-----NOTE-----  
Only applicable if the average scram times of the two-by-two arrays associated with the control rod with the inoperable accumulator are within the limits of Table 3.1.4-1 during the last scram time Surveillance.

-----  
Declare the average scram time in all two-by-two arrays associated with the control rod with the inoperable accumulator not within the limits of Table 3.1.4-1 and declare the associated control rod "slow."

The licensee proposes to revise Required Action A.1 of LCO 3.1.5 in accordance with the STS by simplifying the characterization of scram timing results from a "grouped average" to an individual control rod/accumulator basis as follows:

-----NOTE-----  
Only applicable if the associated control rod scram time was within the limits of Table 3.1.4-1 during the last scram time Surveillance.

-----  
Declare the associated control rod scram time "slow."

Required Action B.2.1 of LCO 3.1.5 currently states:

-----NOTE-----  
Only applicable if the average scram times of the two-by-two arrays associated with the control rod with the inoperable accumulator are within the limits of Table 3.1.4-1 during the last scram time Surveillance.

-----  
Declare the average scram time in all two-by-two arrays associated with the control rod with the inoperable accumulator not within the limits of Table 3.1.4-1 and declare the associated control rod "slow."

The licensee also proposes to revise Required Action B.2.1 of LCO 3.1.5 in accordance with the STS as follows:

-----NOTE-----  
Only applicable if the associated control rod scram time was within the limits of Table 3.1.4-1 during the last scram time Surveillance.

-----  
Declare the associated control rod scram time "slow."

The design basis and transient analyses assume that all of the control rods scram at a specified insertion rate. OPERABILITY of each individual control rod scram accumulator, along with LCOs 3.1.3 and 3.1.4, ensures the scram reactivity assumed in the design-basis accident and transient analyses can be met. The existence of an inoperable accumulator may invalidate prior scram time measurements for the associated control rod. The scram function of the control rod drive system and, therefore, the OPERABILITY of the accumulators, protects the Minimum Critical Power Ratio Safety Limit as well as other fuel design limits to ensure that no fuel damage will occur if those limits are not exceeded.

The proposed changes to Required Actions A.1 and B.2.1 reflect the change in the licensee's analytical approach for modeling scram times from the averaged two-by-two array method to an individual control rod modeling method. The analytical methodologies were approved by the NRC in Amendment No. 211. This change entails a more conservative approach in that the former methodology, the averaged two-by-two array, would allow a control rod that did not meet the times specified in LCO 3.1.4 to be considered fully OPERABLE and not be declared "slow" if separation criteria for "slow" and inoperable rods were met, as well as if the average value of the scram times in the adjacent control rods continued to meet the negative reactivity insertion rate assumed in the scram time analysis. The proposed change requires declaration of a control rod not in compliance with LCO 3.1.4 times to be declared "slow," with no credit for adjacent rod speeds being allowed to offset the rod in question. The proposed change aligns the TSs with the STS and is consistent with the analysis methodology implemented with Amendment No. 211 for scram time modeling. Therefore, the proposed changes are acceptable.

### 3.3 Typographical Errors

Amendment No. 207 introduced the following typographical/administrative errors, resulting from the licensee's application:

- In the operating license, paragraph 2.C(33), "Control Room Envelope Habitability Program (CRE)," paragraph (b), the last sentence states "...or within the next 9 months if the- time period since the most recent..." The licensee proposes to remove the superfluous hyphen symbol (changing "the-" to "the").
- In Condition F of TS 3.7.3, "Control Room Emergency Filtration (CREF) System," the OR statement is left justified against the Condition field left margin. The licensee proposes to align the OR statement with the two text statements in the Condition column. Repositioning the OR statement will match the formatting recommendations of Nuclear Energy Institute 01-03, "Writer's Guide for the Improved [STS]," November 2001, for Condition statements.
- In TS 5.5.14, "Control Room Envelope Habitability Program," the first sentence reads "A Control Room Envelope (CRE) Habitability Program shall be established and implemented to ensure [the] CRE habitability is maintained such that, with an ..." The word "the" in front of CRE has been bracketed to reflect the word transposition that was introduced with Amendment No. 207. The licensee proposes to replace the bracketed word "the" with the word "that." Correcting this word transposition will conform the TSs to the NRC-approved language specified

in TS Task Force (TSTF) Traveler TSTF-448, Revision 3, "Control Room Habitability."

The proposed changes do not impact the intent or substance of the license or TSs, and are therefore acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Washington State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20, and makes editorial, corrective, or other minor revisions. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding published in the *Federal Register* on April 6, 2010 (75 FR 17442). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Lyon

Date: October 25, 2010

October 25, 2010

Mr. Mark E. Reddemann  
Chief Executive Officer  
Energy Northwest  
P.O. Box 968 (Mail Drop 1023)  
Richland, WA 99352-0968

SUBJECT: COLUMBIA GENERATING STATION - ISSUANCE OF AMENDMENT RE:  
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ACCUMULATORS (TAC NO. ME3326)

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A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

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/RA/

Carl F. Lyon, Project Manager  
Plant Licensing Branch IV  
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Docket No. 50-397

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1. Amendment No. 216 to NPF-21
2. Safety Evaluation

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