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Kevin T. Walsh  
Vice President, Operations  
Arkansas Nuclear One

2CAN061004

June 23, 2010

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

SUBJECT: License Amendment Request  
Technical Specification Changes and Analyses Relating to Use of Alternative  
Source Term – Supplemental Information  
Arkansas Nuclear One, Unit 2  
Docket No. 50-368  
License No. NPF-6

REFERENCE 1. Entergy letter to the NRC, dated March 31, 2010, "License Amendment  
Request Technical Specification Changes and Analyses Relating to Use  
of Alternative Source Term" (2CAN031001)

2. Email from Kaly Kalyanam (NRC) to Robert W. Clark (Entergy), dated  
May 5, 2010, "Transmission of the Acceptance Review Result to Licensee  
and Request for Supplemental Information (TAC No. ME3678)

Dear Sir or Madam:

With Reference 1, Entergy Operations, Inc. (Entergy) submitted a request to change the Technical Specifications (TSs) for Arkansas Nuclear One, Unit 2 (ANO-2). In particular the proposed amendment would modify the TS requirements related to the use of an alternate source term (AST) associated with accident offsite and control room dose consequences.

In Reference 2, the NRC provided the results of their acceptance review of Reference 1, which concluded that the Reference 1 submittal provided technical information in sufficient detail to enable the Staff to proceed with its detailed technical review; however, the NRC requested additional supplementary information prior to the beginning of the detailed review. The supplementary information is the design analyses used to determine the dose consequences for each of the design basis accidents that were analyzed.

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Attachment 3 of Reference 1 included a summary of the results of dose analyses for the following events that are expected to produce the most limiting dose consequences.

- Steam Generator Tube Rupture (CN-OA-08-15)
- Fuel Handling Accident (CN-OA-08-16)
- Main Steam Line Break (CN-OA-08-17)
- Loss-of-Coolant Accident (CN-OA-08-18)
- Control Rod Ejection Accident (CN-OA-08-20)
- Locked Reactor Coolant Pump Rotor Accident (CN-OA-08-21)

Enclosed is one copy of each of the design analyses for the above events. In addition, the calculation of the AST using the ORIGEN- S computer code and the determination of the control room attenuation factors is included (CN-OA-08-57).

Also, enclosed with each individual analysis is the Westinghouse Application for Withholding Proprietary Information from Public Disclosure, accompanying Affidavit, Proprietary Information Notice, and Copyright Notice.

As each individual analysis contains information proprietary to Westinghouse Electric Company LLC, each analysis is supported by its own individual affidavit signed by Westinghouse, the owner of the information. The affidavit sets forth the basis on which the information may be withheld from public disclosure by the Commission and addresses with specificity the consideration listed in paragraph (b)(4) of Section 2.390 of the Commission's regulations.

Accordingly, it is respectfully requested that the information which is proprietary to Westinghouse be withheld from public disclosure in accordance with 10 CFR Section 2.390 of the Commission's regulations.

Correspondence with respect to the copyright or proprietary aspects of the enclosed analyses or the supporting Westinghouse affidavits should reference the individual Westinghouse Application for Withholding Proprietary Information from Public Disclosure and should be addressed to J. A. Gresham, Manager, Regulatory Compliance and Plant Licensing, Westinghouse Electric Company LLC, P. O. Box 355, Pittsburgh, Pennsylvania 15230-0355.

This letter contains no new commitments.

If you have any questions or require additional information, please contact David Bice at 479-858-5338.

I declare under penalty of perjury that the foregoing is true and correct. Executed on June 23, 2010.

Sincerely,



KTW/rwc

- Attachments:
1. Calculation Note CN-OA-08-15, "ANO-2 Steam Generator Tube Rupture Dose Analysis Using Alternative Source Terms"
  2. Calculation Note CN-OA-08-16, "ANO-2 Fuel Handling Accident Dose Analysis Using Alternative Source Terms"
  3. Calculation Note CN-OA-08-17, "ANO-2 Main Steam Line Break Dose Analysis Using Alternative Source Terms"
  4. Calculation Note CN-OA-08-18, "ANO-2 LOCA Dose Analysis Using Alternative Source Terms"
  5. Calculation Note CN-OA-08-20, "ANO-2 Rod Ejection Accident Dose Analysis Using Alternative Source Terms"
  6. Calculation Note CN-OA-08-21, "ANO-2 Locked Rotor Accident Dose Analysis Using Alternative Source Terms"
  7. Calculation Note CN-OA-08-57, "ANO-2 Calculation of Alternative Source Terms Using ORIGEN-S and Determination of Control Room Attenuation Factors"

cc: Mr. Elmo E. Collins  
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