ES-401 **Written Examination Quality Checklist** Draft **Form ES-401-6**

Facility: H. B. Robinson Date of Exam: 8/15/2008 Exam Level: RO [X] SRO [X]

Item Description					Initial b*	Initial c#
1.	Questions and answers are technically accurate and applicable to the facility.					
2.	a. NRC K/As are referenced for all questions. b. Facility learning objectives are referenced as available. ND					
3.	SRO questions are appropriate in accordance with Section D.2.d of ES-401 ND					
4.	The sampling process was random and systematic (If more than 4 RO or 2 SRO questions were repeated from the last 2 NRC licensing exams, consult the NRR OL program office).					
5.	Question duplication from the license screening/audit exam was controlled as indicated below (check the item that applies) and appears appropriate: __ the audit exam was systematically and randomly developed; or __ the audit exam was completed before the license exam was started; or __ the examinations were developed independently; or X the licensee certifies that there is no duplication; or __ other (explain)					
6.	Bank use meets limits (no more than 75 percent from the bank, at least 10 percent new, and the rest new or modified); enter the actual RO / SRO-only question distribution(s) at right.	Bank: 15 / 4	Modified: 7 / 1	New: 53 / 20		
7.	Between 50 and 60 percent of the questions on the RO exam are written at the comprehension/ analysis level; the SRO exam may exceed 60 percent if the randomly selected K/As support the higher cognitive levels; enter the actual RO / SRO question distribution(s) at right.	Memory: 35 / 10	C/A: 40 / 15			
8.	References/handouts provided do not give away answers or aid in the elimination of distractors.					
9.	Question content conforms with specific K/A statements in the previously approved examination outline and is appropriate for the tier to which they are assigned; deviations are justified.					
10.	Question psychometric quality and format meet the guidelines in ES Appendix B.					
11.	The exam contains the required number of one-point, multiple choice items; the total is correct and agrees with the value on the cover sheet.					

	Printed Name / Signature	Date
a. Author	R. O. Moore /	6/18/08
b. Facility Reviewer (*)	J. F. Jones /	6/18/08
c. NRC Chief Examiner (#)	** Edwin Lea, Jr. / Edwin Lea, Jr.	7/2/2008
d. NRC Regional Supervisor		

Note: * The facility reviewer's initials/signature are not applicable for NRC-developed examinations.
Independent NRC reviewer initial items in Column "c"; chief examiner concurrence required.

* See attached

The written exams, as submitted, were unacceptable. Per recommendations form the OL Branch Chief and the DRS Division Director, the exams were sent back to the licensee. We agreed to work with the licensee to improve the quality of the exam.

Edwin Lea, Jr.
7/2/2008

Facility: Date of Exam:

Tier	Group	RO K/A Category Points											SRO-Only Points		
		K1	K2	K3	K4	K5	K6	A1	A2	A3	A4	G*	Total	A2	G*	Total
1. Emergency & Abnormal Plant Evolutions	1	2	2	2	N/A			4	4	N/A		4	18			6
	2	2	2	1				1	1			2	9			4
	Tier Totals	4	4	3				5	5			6	27			10
2. Plant Systems	1	3	3	3	2	2	3	2	2	3	2	3	28			5
	2	3	1	1	1	0	1	0	1	0	2	0	10			3
	Tier Totals	6	4	4	3	2	4	2	3	3	4	3	38			8
3. Generic Knowledge and Abilities Categories		1			2			3			4		10	1	2	3	4	7
		2			2			3			3							

Note:

1. Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two).
2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ±1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.
3. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems/evolutions that are not included on the outline should be added. Refer to Section D.1.b of ES-401 for guidance regarding the elimination of inappropriate K/A statements.
4. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.
5. Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.
6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.
7.* The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. Refer to Section D.1.b of ES-401 for the applicable K/As.
8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IRs) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above; if fuel handling equipment is sampled in other than Category A2 or G* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2 (Note #1 does not apply). Use duplicate pages for RO and SRO-only exams.
9. For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43.

ES-401 PWR Examination Outline Form ES-401-2
Emergency and Abnormal Plant Evolutions - Tier 1/Group 1 (RO / SRO)

E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
000007 (BW/E02&E10; CE/E02) Reactor Trip - Stabilization - Recovery / 1 [1]									
000008 Pressurizer Vapor Space Accident / 3 [2]	X						AK1.01 **Knowledge of the operational implications of the following concepts as they apply to a Pressurizer Vapor Space Accident:** Thermodynamics and flow characteristics of open or leaking valves.	3.2 3.7	
000009 Small Break LOCA / 3 [3]									
000011 Large Break LOCA / 3 [4]				X			EA1.12: **Ability to operate and monitor the following as they apply to a Large Break LOCA:** Long-term containment of radioactivity.	4.1 4.4	
000015/17 RCP Malfunctions / 4 [5]		X					AK2.10: **Knowledge of the interrelations between the Reactor Coolant Pump Malfunctions (Loss of RC Flow) and the following:** RCP indicators and controls.	2.8 2.8	
000022 Loss of Rx Coolant Makeup / 2 [6]				X			AA1.01: **Ability to operate and / or monitor the following as they apply to the Loss of Reactor Coolant Makeup:** CVCS letdown and charging.	3.4 3.3	
000025 Loss of RHR System / 4 [7]					X		AA2.06: **Ability to determine and interpret the following as they apply to the Loss of Residual Heat Removal System:** Existence of proper RHR overpressure protection.	3.2 3.4	
000026 Loss of Component Cooling Water / 8 [8]				X			AA1.02: **Ability to operate and / or monitor the following as they apply to the Loss of Component Cooling Water:** Loads on the CCWS in the control room.	3.2 3.3	
000027 Pressurizer Pressure Control System Malfunction / 3 [9]									
000029 ATWS / 1 [10]						X	G2.1.28: **Knowledge of the purpose and function of major system components and controls.**	4.1 4.1	
000038 Steam Gen. Tube Rupture /3 [11]						X	G2.1.20: **Ability to interpret and execute procedure steps.**	4.6 4.6	
000040 (BW/E05; CE/E05; W/E12) Steam Line Rupture - Excessive Heat Transfer / 4 [12]				X			AA1.02: **Ability to operate and / or monitor the following as they apply to the Steam Line Rupture:** Feedwater isolation.	4.5 4.5	
000054 (CE/E06) Loss of Main Feedwater / 4 [13]				X			AA1.02: **Ability to operate and / or monitor the following as they apply to the Loss of Main Feedwater (MFW):** Manual startup of electric and steam-driven AFW pumps.	4.4 4.4	
000055 Station Blackout / 6 [14]									
000056 Loss of Off-site Power / 6 [15]					X		AA2.67: **Ability to determine and interpret the following as they apply to the Loss of Offsite Power:** Seal injection flow (for the RCPs).	2.9 3.1	
000057 Loss of Vital AC Inst. Bus / 6 [16]						X	G2.2.22: **Knowledge of limiting conditions for operations and safety limits.**	4.0 4.7	
000058 Loss of DC Power / 6 [17]						X	G2.2.36: **Ability to analyze the effect of maintenance activities, such as degraded power sources, on the status of limiting conditions for operations.**	3.1 4.2	

000062 Loss of Nuclear Svc Water/4 [18]			X				AK3.02: **Knowledge of the reasons for the following responses as they apply to the Loss of Nuclear Service Water:** The automatic actions (alignments) within the nuclear service water resulting from the actuation of the ESFAS.	3.6 3.9	
000065 Loss of Instrument Air / 8 [19]					X		AA2.05: **Ability to determine and interpret the following as they apply to the Loss of Instrument Air:** When to commence plant shutdown if instrument air pressure is decreasing.	3.4 4.1	
W/E04 LOCA Outside Containment/3 [20]			X				EK3.1: **Knowledge of the reasons for the following responses as they apply to the (LOCA Outside Containment):** Facility operating characteristics during transient conditions, including coolant chemistry and the effects of temperature, pressure, and reactivity changes and operating limitations and reasons for these operating characteristics.	3.2 3.5	
W/E11 Loss of Emergency Coolant Recirc. / 4 [21]	X						EK1.2: **Knowledge of the operational implications of the following concepts as they apply to the (Loss of Emergency Coolant Recirculation):** Normal, abnormal and emergency operating procedures associated with (Loss of Emergency Coolant Recirculation).	3.6 4.1	
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink/4 [22]									
000077 Generator Voltage and Electric Grid Disturbances / 6 [23]		X					AK2.06: **Knowledge of the interrelations between Generator Voltage and Electric Grid Disturbances and the following:** Reactor power.	3.9 4.0	
K/A Category Totals:	2	2	2	5	3	4	Group Point Total:		18

ES-401	PWR Examination Outline Emergency and Abnormal Plant Evolutions - Tier 1/Group 2 (RO / SRO)							Form ES-401-2	
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
000001 Continuous Rod Withdrawal / 1 [1]		X					AK2.08: **Knowledge of the interrelations between the Continuous Rod Withdrawal and the following:** Individual rod display lights and indications.	3.1 3.0	
000003 Dropped Control Rod / 1 [2]									
000005 Inoperable/Stuck Control Rod / 1 [3]			X				AK3.01: **Knowledge of the reasons for the following responses as they apply to the Inoperable / Stuck Control Rod:** Boration and emergency boration in the event of a stuck rod during trip or normal evolutions.	4.0 4.3	
000024 Emergency Boration / 1 [4]									
000028 Pressurizer Level Malfunction / 2 [5]						X	G2.4.4: **Ability to recognize abnormal indications for system operating parameters that are entry-level conditions for emergency and abnormal operating procedures.**	4.5 4.7	
000032 Loss of Source Range NI / 7 [6]	X						AK1.01: **Knowledge of the operational implications of the following concepts as they apply to Loss of Source Range Nuclear Instrumentation:** Effects of voltage changes on performance.	2.5 3.1	
000033 Loss of Intermediate Range NI / 7 [7]									
000036 (BW/A08) Fuel Handling Accident / 8 [8]									
000037 Steam Generator Tube Leak / 3 [9]				X			AA1.06: **Ability to operate and / or monitor the following as they apply to the Steam Generator Tube Leak:** Main steam line rad monitor meters.	3.8 3.9	
000051 Loss of Condenser Vacuum / 4 [10]									
000059 Accidental Liquid RadWaste Rel. / 9 [11]									
000060 Accidental Gaseous Radwaste Rel. / 9 [12]									
000061 ARM System Alarms / 7 [13]									
000067 Plant Fire On-site / 8 [14]									
000068 (BW/A06) Control Room Evac. / 8 [15]					X		AA2.05: **Ability to determine and interpret the following as they apply to the Control Room Evacuation:** Availability of heat sink.	4.2 4.3	
000069 (W/E14) Loss of CTMT Integrity / 5 [16]	X						AK1.01: **Knowledge of the operational implications of the following concepts as they apply to Loss of Containment Integrity:** Effect of pressure on leak rate.	2.6 3.1	
000074 (W/E06&E07) Inad. Core Cooling / 4 [17]									
000076 High Reactor Coolant Activity / 9 [18]									
W/EO1 & E02 Rediagnosis & SI Termination / 3 [19]									
W/E13 Steam Generator Over-pressure / 4 [20]		X					EK2.2: **Knowledge of the interrelations between the (Steam Generator Overpressure) and the following:** Facility's heat removal systems, including primary coolant, emergency coolant, the decay heat removal systems, and relations between the proper operation of these systems to the operation of the facility.	3.0 3.2	

W/E15 Containment Flooding / 5 [21]						X	G2.2.38: **Knowledge of conditions and limitations in the facility license.**	3.6 4.5	
W/E16 High Containment Radiation / 9 [22]									
BW/A01 Plant Runback / 1 [23]									
BW/A02&A03 Loss of NNI-X/Y / 7 [24]									
BW/A04 Turbine Trip / 4 [25]									
BW/A05 Emergency Diesel Actuation / 6 [26]									
BW/A07 Flooding / 8 [27]									
BW/E03 Inadequate Subcooling Margin / 4 [28]									
BW/E08; W/E03 LOCA Cooldown - Depress. / 4 [29]									
BW/E09; CE/A13; W/E09&E10 Natural Circ. / 4 [30]									
BW/E13&E14 EOP Rules and Enclosures [31]									
CE/A11; W/E08 RCS Overcooling - PTS / 4 [32]									
CE/A16 Excess RCS Leakage / 2 [33]									
CE/E09 Functional Recovery [34]									
K/A Category Point Totals:	2	2	1	1	1	2	Group Point Total:		9

ES-401 PWR Examination Outline Form ES-401-2
Plant Systems - Tier 2/Group 1 (RO / SRO)

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
003 Reactor Coolant Pump [1]						X						K6.04: **Knowledge of the effect of a loss or malfunction on the following will have on the RCPS:** Containment isolation valves affecting RCP operation.	2.8 3.1	
004 Chemical and Volume Control [2]							X					A1.05: **Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the CVCS controls including:** S/G pressure and level.	2.9 3.2	
005 Residual Heat Removal [3]	X											K1.06: **Knowledge of the physical connections and/or cause-effect relationships between the RHRS and the following systems:** ECCS.	3.5 3.6	
006 Emergency Core Cooling [4]		X										K2.01: **Knowledge of bus power supplies to the following:** ECCS pumps.	3.6 3.9	
007 Pressurizer Relief/Quench Tank [5]				X								K4.01: **Knowledge of PRTS design feature(s) and/or interlock(s) which provide for the following:** Quench tank cooling.	2.6 2.9	
008 Component Cooling Water [6]										X		A4.05: **Ability to manually operate and/or monitor in the control room:** Normal CCW-header total flow rate and the flow rates to the components cooled by the CCWS.	2.7 2.5	
010 Pressurizer Pressure Control [7]					X							K5.01: **Knowledge of the operational implications of the following concepts as they apply to the PZR PCS:** Determination of condition of fluid in PZR, using steam tables.	3.5 4.0	
012 Reactor Protection[8]						X						K6.01: **Knowledge of the effect of a loss or malfunction of the following will have on the RPS:** Bistables and bistable test equipment.	2.8 3.3	
013 Engineered Safety Features Actuation [9]											X	G2.2.38: **Knowledge of conditions and limitations in the facility license.**	3.6 4.5	
022 Containment Cooling [10]									X			A3.01: **Ability to monitor automatic operation of the CCS, including:** Initiation of safeguards mode of operation.	4.1 4.3	
025 Ice Condenser [11]												REJECTED		
026 Containment Spray [12]							X					A1.01: **Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the CSS controls including:** Containment pressure.	3.9 4.2	
039 Main and Reheat Steam [13]											X	G2.4.20: **Knowledge of the operational implications of EOP warnings, cautions, and notes.**	3.8 4.3	

059 Main Feedwater [14]							X				A2.05: **Ability to (a) predict the impacts of the following malfunctions or operations on the MFW; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:** Rupture in the MFW suction or discharge line.	3.1 3.4	
061 Auxiliary/Emergency Feedwater [15]								X			A3.01: **Ability to monitor automatic operation of the AFW, including:** AFW startup and flows.	4.2 4.2	
062 AC Electrical Distribution [16]		X									K2.01: **Knowledge of bus power supplies to the following:** Major system loads.	3.3 3.4	
063 DC Electrical Distribution [17]			X								K3.02: **Knowledge of the effect that a loss or malfunction of the DC electrical system will have on the following:** Components using DC control power.	3.5 3.7	
064 Emergency Diesel Generator [18]										X	G2.1.7: **Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation.**	4.4 4.7	
073 Process Radiation Monitoring [19]									X		A4.01: **Ability to manually operate and/or monitor in the control room:** Effluent release.	3.9 3.9	
076 Service Water [20]				X			✓				A2.01 K4.03: **Knowledge of SWS design feature(s) and/or interlock(s) which provide for the following:** Automatic opening features associated with SWS isolation valves to CCW heat exchanges.	2.9 3.4	
078 Instrument Air [21]	X										K1.03: **Knowledge of the physical connections and/or cause-effect relationships between the IAS and the following systems:** Containment air.	3.3 3.4	
103 Containment [22]			X								K3.01: **Knowledge of the effect that a loss or malfunction of the containment system will have on the following:** Loss of containment integrity under shutdown conditions.	3.3 3.7	
004 Chemical and Volume Control [23]			X								K3.08: **Knowledge of the effect that a loss or malfunction of the CVCS will have on the following:** RCP seal injection.	3.6 3.8	
007 Pressurizer Relief/Quench Tank [24]					X	✓					A1.02 K5.02: **Knowledge of the operational implications of the following concepts as the apply to PRTS:** Method of forming a steam bubble in the PZR.	3.1 3.4	
039 Main and Reheat Steam [25]	X										K1.01: **Knowledge of the physical connections and/or cause-effect relationships between the MRSS and the following systems:** S/G.	3.1 3.2	
062 AC Electrical Distribution [26]								X			A3.05: **Ability to monitor automatic operation of the ac distribution system, including:** Safety-related indicators and controls.	3.5 3.6	
039 Main and Reheat Steam [27]			X								K3.06: **Knowledge of the effect that a loss or malfunction of the MRSS will have on the following:** SDS.	2.8 3.1	

026 Containment Spray [28]								X				A2.04 **Ability to (a) predict the impacts of the following malfunctions or operations on the CSS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:** Failure of spray.	3.9 4.2	
064 Emergency Diesel Generator [29]						X						K6.07 **Knowledge of the effect of a loss or malfunction of the following will have on the ED/G system:** Air receivers.	2.7 2.9	
K/A Category Point Totals:	3	3	3	2	2	3	2	2	3	2	3	Group Point Total:		28

ES-401	PWR Examination Outline Plant Systems - Tier 2/Group 2 (RO / SRO)												Form ES-401-2	
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
001 Control Rod Drive [1]														
002 Reactor Coolant [2]						X						K6.07: **Knowledge of the effect or a loss or malfunction on the following RCS components:** Pumps.	2.5 2.8	
011 Pressurizer Level Control [3]														
014 Rod Position Indication [4]	X											K1.01: **Knowledge of the physical connections and/or cause-effect relationships between the RPIS and the following systems:** CRDS.	3.2 3.6	
015 Nuclear Instrumentation [5]				X								K4.01: **Knowledge of NIS design feature(s) and/or interlock(s) provide for the following:** Source-Range detector power shutoff at high powers.	3.1 3.3	
016 Non-nuclear Instrumentation [6]														
017 In-core Temperature Monitor [7]			X									K3.01: **Knowledge of the effect that a loss or malfunction of the ITM system will have on the following:** Natural circulation indications.	3.5 3.7	
027 Containment Iodine Removal [8]	X											K1.01: **Knowledge of the physical connections and/or cause-effect relationships between the CIRS and the following systems:** CSS.	3.4 3.7	
028 Hydrogen Recombiner and Purge Control [9]														
029 Containment Purge [10]														
033 Spent Fuel Pool Cooling [11]														
034 Fuel Handling Equipment [12]										X		A4.02: **Ability to manually operate and/or monitor in the control room:** Neutron levels.	3.5 3.9	
035 Steam Generator [13]								X				A2.06: **Ability to (a) predict the impacts of the following malfunctions or operations on the GS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:** Small break LOCA.	4.5 4.6	
041 Steam Dump/Turbine Bypass Control [14]										X		A4.06: **Ability to manually operate and/or monitor in the control room:** Atmospheric relief valve controllers.	2.9 3.1	
045 Main Turbine Generator [15]	X											K1.18: **Knowledge of the physical connections and/or cause-effect relationships between the MT/G system and the following systems**: RPS.	3.6 3.7	
055 Condenser Air Removal [16]														
056 Condensate [17]														
068 Liquid Radwaste [18]														

071 Waste Gas Disposal [19]													
072 Area Radiation Monitoring [20]													
075 Circulating Water [21]		X									K2.03: **Knowledge of bus power supplies to the following:** Emergency/essential SWS pumps.	2.6 2.7	
079 Station Air [22]													
086 Fire Protection [23]													
K/A Category Point Totals:	3	1	1	1	0	1	0	1	0	2		Group Point Total:	10

Facility:		Date of Exam:				
Category	K/A #	Topic	RO		SRO-Only	
			IR	#	IR	#
1. Conduct of Operations	2.1.26	**Knowledge of industrial safety procedures (such as rotating equipment, electrical, high temperature, high pressure, caustic, chlorine, oxygen and hydrogen).**	3.4	X		
	2.1.28	**Knowledge of the purpose and function of major system components and controls.**	4.1	X		
	2.1.					
	2.1.					
	2.1.					
	Subtotal					
2. Equipment Control	2.2.6	**Knowledge of the process for making changes to procedures.**	3.0	X		
	2.2.40	**Ability to apply Technical Specifications for a system.**	3.4	X		
	2.2.					
	2.2.					
	2.2.					
	Subtotal					
3. Radiation Control	2.3.4	**Knowledge of radiation exposure limits under normal or emergency conditions.**	3.2	X		
	2.3.13	**Knowledge of radiological safety procedures pertaining to licensed operator duties, such as response to radiation monitor alarms, containment entry requirements, fuel handling responsibilities, access to locked high-radiation areas, aligning filters, etc.**	3.4	X		
	2.3.7	**Ability to comply with radiation work permit requirements during normal or abnormal conditions.**	3.5	X		
	2.3.					
	2.3.					
	Subtotal					
4. Emergency Procedures / Plan	2.4.16	**Knowledge of EOP implementation hierarchy and coordination with other support procedures or guidelines such as, operating procedures, abnormal operating procedures, and severe accident management guidelines.**	3.5	X		
	2.4.22	**Knowledge of the bases for prioritizing safety functions during abnormal/emergency operations.**	3.6	X		
	2.4.23	**Knowledge of the bases for prioritizing emergency procedure implementation during emergency operations.**	3.4	X		
	2.4.					
	2.4.					
	Subtotal					
Tier 3 Point Total				10		

Tier / Group	Randomly Selected K/A	Reason for Rejection
1/1	000029.G2.3.13	Cannot write a quality question concerning radiological actions for an ATWS event. (Replaced with G2.1.28)
1/1	000057.G2.1.41	Does not meet NUREG 1021, ES-401, D.1.b guidelines. (Replaced with G2.2.22)
1/2	000028.G2.4.39	Does not meet NUREG 1021, ES-401, D.1.b guidelines. (Replaced with 2.4.4)
1/2	W/E 15. G2.2.22	RNP has no LCO actions for Containment Sump high level addressed in Technical Specifications. (Replaced with G2.2.38)
2/1	026.A1.04	RNP does not have the capability to monitor Containment Humidity remotely. (Replaced with A1.01)
2/1	059.A2.07	RNP does not have Turbine Driven Feed Pumps. (Replaced with A2.05)
2/1	061.A3.04	RNP does not have automatic isolation features for AFW to the S/Gs. (Replaced with A3.01)
2/2	014.K2.02	RNP does not correct the NIS for power. RPI is an independent system which is temperature corrected only. (Replaced with K1.01)
2/1	076.K4.03	RNP does not have any automatic isolations for the CCW heat exchangers. (Replaced with)

Facility: Date of Exam:

Tier	Group	RO K/A Category Points											SRO-Only Points		
		K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	A2	G*	Total
1. Emergency & Abnormal Plant Evolutions	1				N/A					N/A			18	3	3	6
	2												9	2	2	4
	Tier Totals												27	5	5	10
2. Plant Systems	1												28	2	3	5
	2												10	0	3	3
	Tier Totals												38	2	6	8
3. Generic Knowledge and Abilities Categories		1		2		3		4					10	1	2	3	4	7
														1	3	0	3	

Note:

1. Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two).
2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ±1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.
3. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems/evolutions that are not included on the outline should be added. Refer to Section D.1.b of ES-401 for guidance regarding the elimination of inappropriate K/A statements.
4. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.
5. Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.
6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.
7.* The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. Refer to Section D.1.b of ES-401 for the applicable K/As.
8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IRs) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above; if fuel handling equipment is sampled in other than Category A2 or G* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2 (Note #1 does not apply). Use duplicate pages for RO and SRO-only exams.
9. For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43.

ES-401 PWR Examination Outline Form ES-401-2
Emergency and Abnormal Plant Evolutions - Tier 1/Group 1 (RO / SRO)

E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
000007 (BW/E02&E10; CE/E02) Reactor Trip - Stabilization - Recovery / 1 [1]									
000008 Pressurizer Vapor Space Accident / 3 [2]						X	G2.4.11: **Knowledge of abnormal condition procedures.** (CFR: 41.10 / 43.5 / 45.13)	4.0 4.2	
000009 Small Break LOCA / 3 [3]									
000011 Large Break LOCA / 3 [4]									
000015/17 RCP Malfunctions / 4 [5]									
000022 Loss of Rx Coolant Makeup / 2 [6]					X		AA2.03: **Ability to determine and interpret the following as they apply to the Loss of Reactor Coolant Makeup:** Failures of flow control valve or controller (CFR: 43.5/ 45.13)	3.1 3.6	
000025 Loss of RHR System / 4 [7]					X		AA2.07: **Ability to determine and interpret the following as they apply to the Loss of Residual Heat Removal System:** Pump Cavitation. (CFR 43.5/45.13)	3.4 3.7	
000026 Loss of Component Cooling Water / 8 [8]						X	G2.2.38: **Knowledge of conditions and limitations in the facility license.** (CFR: 41.7/41.10/43.1/45.13)	3.6 4.5	
000027 Pressurizer Pressure Control System Malfunction / 3 [9]									
000029 ATWS / 1 [10]									
000038 Steam Gen. Tube Rupture /3 [11]									
000040 (BW/E05; CE/E05; W/E12) Steam Line Rupture - Excessive Heat Transfer / 4 [12]									
000054 (CE/E06) Loss of Main Feedwater / 4 [13]									
000055 Station Blackout / 6 [14]					X		EA2.01: **Ability to determine or interpret the following as they apply to a Station Blackout**: Existing valve positioning on a loss of instrument air system. (CFR: 43.5 / 45.13)	3.4 3.7	
000056 Loss of Off-site Power / 6 [15]									
000057 Loss of Vital AC Inst. Bus / 6 [16]									
000058 Loss of DC Power / 6 [17]						X	2.1.32: **Ability to explain and apply system limits and precautions.** (CFR: 41.10 / 45.5 / 45.12 / 45.13)	3.8 4.0	
000062 Loss of Nuclear Svc Water/4 [18]									
000065 Loss of Instrument Air / 8 [19]									
W/E04 LOCA Outside Containment/3 [20]									
W/E11 Loss of Emergency Coolant Recirc. / 4 [21]									
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink/4 [22]									
000077 Generator Voltage and Electric Grid Disturbances / 6 [23]									
K/A Category Totals:					3	3	Group Point Total:		6

ES-401	PWR Examination Outline Emergency and Abnormal Plant Evolutions - Tier 1/Group 2 (RO / SRO)						Form ES-401-2		
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
000001 Continuous Rod Withdrawal / 1 [1]									
000003 Dropped Control Rod / 1 [2]									
000005 Inoperable/Stuck Control Rod / 1 [3]									
000024 Emergency Boration / 1 [4]					X		AA2.02: **Ability to determine and interpret the following as they apply to the Emergency Boration:** When use of manual boration valve is needed. (CFR: 43.5 / 45.13)	3.9 4.4	
000028 Pressurizer Level Malfunction / 2 [5]									
000032 Loss of Source Range NI / 7 [6]									
000033 Loss of Intermediate Range NI / 7 [7]					X		AA2.10: **Ability to determine and interpret the following as they apply to the Loss of Intermediate Range Nuclear Instrumentation:** Tech-Spec limits if both intermediate range channels have failed. (CFR: 43.5 / 45.13)	3.1 3.8	
000036 (BW/A08) Fuel Handling Accident / 8 [8]									
000037 Steam Generator Tube Leak / 3 [9]									
000051 Loss of Condenser Vacuum / 4 [10]									
000059 Accidental Liquid RadWaste Rel. / 9 [11]						X	G2.4.4: **Ability to recognize abnormal indications for system operating parameters that are entry-level conditions for emergency and abnormal operating procedures.** (CFR: 41.10/43.2/45.6)	4.5 4.7	
000060 Accidental Gaseous Radwaste Rel. / 9 [12]									
000061 ARM System Alarms / 7 [13]									
000067 Plant Fire On-site / 8 [14]									
000068 (BW/A06) Control Room Evac. / 8 [15]									
000069 (W/E14) Loss of CTMT Integrity / 5 [16]									
000074 (W/E06&E07) Inad. Core Cooling / 4 [17]						X	G2.2.42: **Ability to recognize system parameters that are entry-level conditions for Technical Specifications.** (CFR: 41.7/41.10/43.2/43.3/45.3)	3.9 4.6	
000076 High Reactor Coolant Activity / 9 [18]									
W/EO1 & E02 Rediagnosis & SI Termination / 3 [19]									
W/E13 Steam Generator Over-pressure / 4 [20]									
W/E15 Containment Flooding / 5 [21]									
W/E16 High Containment Radiation / 9 [22]									
K/A Category Point Totals:					2	2	Group Point Total:		4

ES-401	PWR Examination Outline Plant Systems - Tier 2/Group 1 (RO / SRO)											Form ES-401-2		
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
003 Reactor Coolant Pump [1]														
004 Chemical and Volume Control [2]														
005 Residual Heat Removal [3]														
006 Emergency Core Cooling [4]														
007 Pressurizer Relief/Quench Tank [5]								X				A2.03: **Ability to (a) predict the impacts of the following malfunctions or operations on the P S; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:** Overpressurization of the PZR (CFR: 41.5 / 43.5 / 45.3 / 45.13)	3.6 3.9	
008 Component Cooling Water [6]														
010 Pressurizer Pressure Control [7]														
012 Reactor Protection[8]														
013 Engineered Safety Features Actuation [9]											X	G2.1.7: **Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior and instrumentation interpretation.** (CFR 41.5, 43.5, 45.12, 45.13)	4.4 4.7	
022 Containment Cooling [10]														
025 Ice Condenser [11]														
026 Containment Spray [12]														
039 Main and Reheat Steam [13]														
059 Main Feedwater [14]														
061 Auxiliary/Emergency Feedwater [15]											X	G2.1.27: **Knowledge of system purpose and/or function.** (CFR: 41.7)	3.9 4.0	
062 AC Electrical Distribution [16]														
063 DC Electrical Distribution [17]														
064 Emergency Diesel Generator [18]														
073 Process Radiation Monitoring [19]														
076 Service Water [20]														

078 Instrument Air [21]							X				A2.01: **Ability to (a) predict the impacts of the following malfunctions or operations on the IAS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations:** Air dryer and filter malfunctions. (CFR: 41.5 / 43.5 / 45.3 / 45.13)	2.4 2.9	
103 Containment [22]										X	G2.4.30: **Knowledge of events related to system operation/status that must be reported to internal organizations or external agencies, such as the State, the NRC, or the transmission system operator.** (CFR: 41.10/43.5/45.11)	2.7 4.1	
K/A Category Point Totals:							2			3	Group Point Total:		5

ES-401 PWR Examination Outline Form ES-401-2
Plant Systems – Tier 2/Group 2 (RO / SRO)

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
001 Control Rod Drive [1]														
002 Reactor Coolant [2]											X	G2.2.40: **Ability to apply Technical Specifications for a system.** (CFR: 41.10/43.2/43.5/45.3)	3.4 4.7	
011 Pressurizer Level Control [3]											X	G2.4.50: **Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.** (CFR: 41.10/43.5/45.3)	4.2 4.0	
014 Rod Position Indication [4]														
015 Nuclear Instrumentation [5]														
016 Non-nuclear Instrumentation [6]														
017 In-core Temperature Monitor [7]														
027 Containment Iodine Removal [8]														
028 Hydrogen Recombiner and Purge Control [9]														
029 Containment Purge [10]														
033 Spent Fuel Pool Cooling [11]														
034 Fuel Handling Equipment [12]														
035 Steam Generator [13]														
041 Steam Dump/Turbine Bypass Control [14]														
045 Main Turbine Generator [15]														
055 Condenser Air Removal [16]														
056 Condensate [17]														
068 Liquid Radwaste [18]														
071 Waste Gas Disposal [19]											X	G2.2.25: **Knowledge of the bases in Technical Specifications for limiting conditions for operations and safety limits.** (CFR: 41.5/41.7/43.2)	3.2 4.2	
072 Area Radiation Monitoring [20]														
075 Circulating Water [21]														
079 Station Air [22]														
086 Fire Protection [23]														
K/A Category Point Totals:											3	Group Point Total:		3

Facility: Date of Exam:

Category	K/A #	Topic	RO		SRO-Only	
			IR	#	IR	#
1. Conduct of Operations	2.1.41	**Knowledge of the refueling process.** (CFR: 41.2/41.10/43.6/45.13)			2.8 3.7	X
	2.1.					
	2.1.					
	2.1.					
	2.1.					
	Subtotal					
2. Equipment Control	2.2.18	**Knowledge of the process for managing maintenance activities during shutdown operations, such as risk assessments, work prioritization, etc.** (CFR: 41.10/43.5/45.13)			2.6 3.9	X
	2.2.15	**Ability to determine the expected plant configuration using design and configuration control documentation, such as drawings, line-ups, tag-outs, etc.** (CFR: 41.10/ 43.3/45.13)			3.9 4.3	X
	2.2.36	**Ability to analyze the effect of maintenance activities, such as degraded power sources, on the status of limiting conditions for operations.** (CFR: 41.10/ 3.2/45.13)			3.1 4.2	X
	2.2.					
	2.2.					
	Subtotal					
3. Radiation Control	2.3.					
	2.3.					
	2.3.					
	2.3.					
	2.3.					
	Subtotal					
4. Emergency Procedures / Plan	2.4.6	**Knowledge of EOP mitigation strategies.** (CFR: 41.10/43.5/45.13)			3.7 4.7	X
	2.4.8	**Knowledge of how abnormal operating procedures are used in conjunction with EOPs.** (CFR: 41.10/43.5/45.13)			3.8 4.5	X
	2.4.18	**Knowledge of the specific bases for EOPs.** (CFR: 41.10/43.1/45.13)			3.3 4.0	X
	2.4.					
	2.4.					
	Subtotal					
Tier 3 Point Total						7

Tier / Group	Randomly Selected K/A	Reason for Rejection
1/1	000025.AA.2.01	RNP does not have indication of individual load amps for RHR Pumps. 480V loads are provided with bus amperage instead of individual load amperage readings. (Replaced with A2.07)
1/1	000026.G2.3.4	Does not meet NUREG 1021, ES-401, D.1.b guidelines. (Replaced with G2.2.38)
1/1	000058.G2.1.9	Does not meet NUREG 1021, ES-401, D.1.b guidelines. (Replaced with G2.1.32)
1/2	000059.G2.3.15	Does not meet NUREG 1021, ES-401, D.1.b guidelines. (Replaced with G2.4.4)
1/2	000074.G2.1.34	Does not meet NUREG 1021, ES-401, D.1.b guidelines. (Replaced with G2.2.42)
2/1	013. G2.1.40	ESF is defeated when the plant enters Mode 5 and will not actuate during refueling activities. (Replaced with G2.1.7)
2/1	061.G2.2.18	Does not meet NUREG 1021, ES-401, D.1.b guidelines. (Replaced with G2.1.27)
2/1	103.G2.3.6	Does not meet NUREG 1021, ES-401, D.1.b guidelines. (Replaced with G2.4.30)
2/2	002.G2.1.15	Does not meet NUREG 1021, ES-401, D.1.b guidelines. (Replaced with G2.2.40)
2/2	011.G2.3.13	Does not meet NUREG 1021, ES-401, D.1.b guidelines. (Replaced with G2.4.50)

H.B. Robinson 2008

ADMINISTRATIVE DOCUMENTS

(Yellow Paper)

H. B. R

1. Exam Preparation Checklist ES-201-1
2. Exam Outline Quality Checklist ES-201-2
3. Exam Security Agreement(s) ES-201-3
4. Administrative Topics Outline (Final) ES-301-1*
5. Control Room Systems & Facility Walk-Thru Test Outline (Final) ES-301-2
6. Operating Test Quality Check Sheet ES-301-3
7. Simulator Scenario Quality Check Sheet ES-301-4
8. Transient and Event Checklist ES-301-5
9. Competencies Checklist ES-301-6
10. Written Exam Quality Check Sheet ES-401-6
11. Written Exam Review Worksheet ES-401-9
12. Written Exam Grading Quality Checklist ES-403-1
13. Post-Exam Check Sheet ES-501-1
14. Facility Submittal Letter [1]

Facility: HB Robinson Date of Examination: August 2008

Examinations Developed by: Facility / Written / Operating Test

Target Date*	Task Description (Reference)	Chief Examiner's Initials
-180	1. Examination administration date confirmed (C.1.a; C.2.a and b)	
-120	2. NRC examiners and facility contact assigned (C.1.d; C.2.e)	
-120	3. Facility contact briefed on security and other requirements (C.2.c)	
-120	4. Corporate notification letter sent (C.2.d)	
[-90]	[5. Reference material due (C.1.e; C.3.c; Attachment 2)]	
{-75}	6. Integrated examination outline(s) due, including Forms ES-201-2, ES-201-3, ES-301-1, ES-301-2, ES-301-5, ES-D-1's, ES-401-1/2, ES-401-3, and ES-401-4, as applicable (C.1.e and f; C.3.d)	5/19
{-70}	{7. Examination outline(s) reviewed by NRC and feedback provided to facility licensee (C.2.h; C.3.e)}	
{-45}	8. Proposed examinations (including written, walk-through JPMs, and scenarios, as applicable), supporting documentation (including Forms ES-301-3, ES-301-4, ES-301-5, ES-301-6, and ES-401-6), and reference materials due (C.1.e, f, g and h; C.3.d)	
-30	9. Preliminary license applications (NRC Form 398's) due (C.1.l; C.2.g; ES-202)	
-14	10. Final license applications due and Form ES-201-4 prepared (C.1.l; C.2.i; ES-202)	
-14	11. Examination approved by NRC supervisor for facility licensee review (C.2.h; C.3.f)	
-14	12. Examinations reviewed with facility licensee (C.1.j; C.2.f and h; C.3.g)	
-7	13. Written examinations and operating tests approved by NRC supervisor (C.2.i; C.3.h)	
-7	14. Final applications reviewed; 1 or 2 (if >10) applications audited to confirm qualifications / eligibility; and examination approval and waiver letters sent (C.2.i; Attachment 4; ES-202, C.2.e; ES-204)	
-7	15. Proctoring/written exam administration guidelines reviewed with facility licensee (C.3.k)	
-7	16. Approved scenarios, job performance measures, and questions distributed to NRC examiners (C.3.i)	

* Target dates are generally based on facility-prepared examinations and are keyed to the examination date identified in the corporate notification letter. They are for planning purposes and may be adjusted on a case-by-case basis in coordination with the facility licensee.
[Applies only] {Does not apply} to examinations prepared by the NRC.

ES-201 **Examination Outline Quality Checklist** **Form ES-201-2**

Facility: H.B.Robinson Date of Examination: 8/26/08

Item	Task Description	Initials a	Initials b*	Initials c#
1. WRITTEN	a. Verify that the outline(s) fit(s) the appropriate model, in accordance with ES-401.			
	b. Assess whether the outline was systematically and randomly prepared in accordance with Section D.1 of ES-401 and whether all K/A categories are appropriately sampled.			
	c. Assess whether the outline over-emphasizes any systems, evolutions, or generic topics.			
	d. Assess whether the justifications for deselected or rejected K/A statements are appropriate.			
2. SIMULATOR	a. Using Form ES-301-5, verify that the proposed scenario sets cover the required number of normal evolutions, instrument and component failures, technical specifications, and major transients.			
	b. Assess whether there are enough scenario sets (and spares) to test the projected number and mix of applicants in accordance with the expected crew composition and rotation schedule without compromising exam integrity, and ensure that each applicant can be tested using at least one new or significantly modified scenario, that no scenarios are duplicated from the applicants' audit test(s), and that scenarios will not be repeated on subsequent days.			
	c. To the extent possible, assess whether the outline(s) conform(s) with the qualitative and quantitative criteria specified on Form ES-301-4 and described in Appendix D.			
3. W/T	a. Verify that the systems walk-through outline meets the criteria specified on Form ES-301-2: (1) the outline(s) contain(s) the required number of control room and in-plant tasks distributed among the safety functions as specified on the form (2) task repetition from the last two NRC examinations is within the limits specified on the form (3) no tasks are duplicated from the applicants' audit test(s) (4) the number of new or modified tasks meets or exceeds the minimums specified on the form (5) the number of alternate path, low-power, emergency, and RCA tasks meet the criteria on the form.			
	b. Verify that the administrative outline meets the criteria specified on Form ES-301-1: (1) the tasks are distributed among the topics as specified on the form (2) at least one task is new or significantly modified (3) no more than one task is repeated from the last two NRC licensing examinations			
	c. Determine if there are enough different outlines to test the projected number and mix of applicants and ensure that no items are duplicated on subsequent days.			
4. GENERAL	a. Assess whether plant-specific priorities (including PRA and IPE insights) are covered in the appropriate exam sections.			
	b. Assess whether the 10 CFR 55.41/43 and 55.45 sampling is appropriate.			
	c. Ensure that K/A importance ratings (except for plant-specific priorities) are at least 2.5.			
	d. Check for duplication and overlap among exam sections.			
	e. Check the entire exam for balance of coverage.			
	f. Assess whether the exam fits the appropriate job level (RO or SRO).			

	Printed Name/Signature	Date
a. Author	Richard O. Moore / [signature]	8/10/08
b. Facility Reviewer (*)	James F. Jones / [signature]	8/10/08
c. NRC Chief Examiner (#)	Edwin Lea, Jr / [signature]	8/12/08
d. NRC Supervisor	ANNE-MARIE T. WILDHARTAL / [signature]	08/14/08

Note: # Independent NRC reviewer initial items in Column "c"; chief examiner concurrence required.
* Not applicable for NRC-prepared examination outlines

1. Pre-Examination

I acknowledge that I have acquired specialized knowledge about the NRC licensing examinations scheduled for the week(s) of 8/15 - 8/29 2008 as of the date of my signature. I agree that I will not knowingly divulge any information about these examinations to any persons who have not been authorized by the NRC chief examiner. I understand that I am not to instruct, evaluate, or provide performance feedback to those applicants scheduled to be administered these licensing examinations from this date until completion of examination administration, except as specifically noted below and authorized by the NRC (e.g., acting as a simulator booth operator or communicator is acceptable if the individual does not select the training content or provide direct or indirect feedback). Furthermore, I am aware of the physical security measures and requirements (as documented in the facility licensee's procedures) and understand that violation of the conditions of this agreement may result in cancellation of the examinations and/or an enforcement action against me or the facility licensee. I will immediately report to facility management or the NRC chief examiner any indications or suggestions that examination security may have been compromised.

2. Post-Examination

To the best of my knowledge, I did not divulge to any unauthorized persons any information concerning the NRC licensing examinations administered during the week(s) of 8/15-29/08. From the date that I entered into this security agreement until the completion of examination administration, I did not instruct, evaluate, or provide performance feedback to those applicants who were administered these licensing examinations, except as specifically noted below and authorized by the NRC.

	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE NOTE
1.	J. F. Jones	Exam Team Leader		1/24/08		9/16/08
2.	V. Leeth	Sr Nuc Inst		1/25/08		9/9/08
3.	Lee Sanders	CT Supervisor		1/25/08	Terminated Emp.	
4.	Michael Milbee	Sr NUC Instructor		1/25/08		8/27/08
5.	Paul Harmon	Contract Instructor		2/13/08		8/25/08
6.	R. O. Mathe	Sr. Nuc. Ops Inst.		3/7/08		8/27/08
7.	Dilip Sunthankar	Nuclear Simulator Engineer		4/1/08		8/26/08
8.	Saeed Khalfay	Nuclear Simulator Engineer		4/2/08		8/26/08
9.	W. E. Stover	WCC-SRO		4/18/08		8/29/08
10.	Richard Wiegand	WCC SRO		4/24/08		8/27/08
11.	W. Wonka	STA-SRO		4/24/08		8-27-08
12.	G. Morrison	RO		4/24/08		8/27/08
13.	Greg Kilpatrick	Supt - Ops Trn (HNP)		30Apr08		30Apr08
14.	Joe Pennington	RO		5-7-8		8-27-8
15.	Steve Hebler	SRO		5/19/8		9/2/8

NOTES:

1. **Pre-Examination**

I acknowledge that I have acquired specialized knowledge about the NRC licensing examinations scheduled for the week(s) of 8/15-8/29 as of the date of my signature. I agree that I will not knowingly divulge any information about these examinations to any persons who have not been authorized by the NRC chief examiner. I understand that I am not to instruct, evaluate, or provide performance feedback to those applicants scheduled to be administered these licensing examinations from this date until completion of examination administration, except as specifically noted below and authorized by the NRC (e.g., acting as a simulator booth operator or communicator is acceptable if the individual does not select the training content or provide direct or indirect feedback). Furthermore, I am aware of the physical security measures and requirements (as documented in the facility licensee's procedures) and understand that violation of the conditions of this agreement may result in cancellation of the examinations and/or an enforcement action against me or the facility licensee. I will immediately report to facility management or the NRC chief examiner any indications or suggestions that examination security may have been compromised.

2. **Post-Examination**

To the best of my knowledge, I did not divulge to any unauthorized persons any information concerning the NRC licensing examinations administered during the week(s) of 8/15-8/29. From the date that I entered into this security agreement until the completion of examination administration, I did not instruct, evaluate, or provide performance feedback to those applicants who were administered these licensing examinations, except as specifically noted below and authorized by the NRC.

	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
1.	Eric McCartney	Director-Site Operations	Eric McCartney	7/2/08	Eric McCartney	8/26/08	
2.	Greg Lipatrick	Acting Trn Mgr		08/01/08	See Attached	copy	
3.	Joseph A Rackley	SRO		7-16-08		9-12-08	
4.	Richard Orens	RO		7/28/08		9/12/08	
5.	Mike Donithan	SRO		7/28/08		9-2-08	
6.	Brendan Mullaney	SRO		7/28/8		9-2-8	
7.	Tony C. Iannesse	RO		7/28/8		9-15-08	
8.	Valerie B. Gardner	SSO		7/31/08		8/26/08	
9.	Alexander Curlington	SRO candidate		7/31/08		8/26/08	(1)
10.	Robert W Lindsey	NWT Consultant		7/31/08		7/20/08	
11.	Thomas F. Tritt	AO-B		7/31/08		8/28/08	
12.	JOHN KRAUS	SRO		8/7/8		9/2/8	
13.	John R. Littlejohn	SRO		08/07/08		09/12/08	
14.	Martin L. Arnold	SSO	Martin L Arnold	8/7/08		8/29/08	
15.	H Clark Fletcher	Supervisor-Operator Initial Training	H Clark Fletcher	8/13/08	H Clark Fletcher	8/26/08	

NOTES:
(1) SRO candidate for next Initial Class (HLC-10).

1. Pre-Examination

I acknowledge that I have acquired specialized knowledge about the NRC licensing examinations scheduled for the week(s) of 8/15-8/29/08 as of the date of my signature. I agree that I will not knowingly divulge any information about these examinations to any persons who have not been authorized by the NRC chief examiner. I understand that I am not to instruct, evaluate, or provide performance feedback to those applicants scheduled to be administered these licensing examinations from this date until completion of examination administration, except as specifically noted below and authorized by the NRC (e.g., acting as a simulator booth operator or communicator is acceptable if the individual does not select the training content or provide direct or indirect feedback). Furthermore, I am aware of the physical security measures and requirements (as documented in the facility licensee's procedures) and understand that violation of the conditions of this agreement may result in cancellation of the examinations and/or an enforcement action against me or the facility licensee. I will immediately report to facility management or the NRC chief examiner any indications or suggestions that examination security may have been compromised.

2. Post-Examination

To the best of my knowledge, I did not divulge to any unauthorized persons any information concerning the NRC licensing examinations administered during the week(s) of 8/15-8/29. From the date that I entered into this security agreement until the completion of examination administration, I did not instruct, evaluate, or provide performance feedback to those applicants who were administered these licensing examinations, except as specifically noted below and authorized by the NRC.

	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
1.	Ricky Haley	SRO	R Haley	5-9-08	R Haley	9-12-08	
2.	BRYAN C. WALDSMITH	RO/CO	Bryan C Waldsmith	05-09-08	Bryan C Waldsmith	09-05-08	
3.	TS GOULDING	CRSS/SRO	TS Goulding	5-12-08	TS Goulding	8-27-08	
4.	B.H. CLARK	Mgr - TRNG, RNP	[illegible]	5/13/08	[illegible]	8/27/08	
5.	Ken Jones	Supv - Plant Support	Ken Jones	5/19/08	[illegible]	8/26/08	
6.	LEONARD W. PITTS	SRO / STA	[illegible]	5/20/8	[illegible]	6/27/8	
7.	Nick Roh	SRO	[illegible]	5/20/8	[illegible]	9-12-8	
8.	J. McDONALD	SSO - OPERATIONS	[illegible]	6/9/8	[illegible]	8/27/8	
9.	Floyd Lawrence	SOIT - CR3	[illegible]	6/9/08	See Attached Copy		
10.	JOHN DALTON	HARRIS - SOIT	[illegible]	6/9/08	See Attached Copy		
11.	DEREK HAWES	OPS INSTRUCTOR - RNP	[illegible]	6/9/8	[illegible]	8/26/8	
12.	Darryl Foster	Manager-Shift OPS - RNP	[illegible]	6/9/08	[illegible]	8/27/08	
13.	KEITH BROWN	SUPV - OCT	[illegible]	6-9-8	[illegible]	8-27-8	
14.	ARCHIE LUCKY	SNOT-F	Archie Lucky	6-10-08	See Attached Copy		
15.	Vlad Balakhnin	Rx Engineer	[illegible]	6-23-08	[illegible]	9-15-08	

NOTES:
① signed in error. ~~J Goulding~~ 5/9/08

1. **Pre-Examination**

I acknowledge that I have acquired specialized knowledge about the NRC licensing examinations scheduled for the week(s) of 8/15-8/29 as of the date of my signature. I agree that I will not knowingly divulge any information about these examinations to any persons who have not been authorized by the NRC chief examiner. I understand that I am not to instruct, evaluate, or provide performance feedback to those applicants scheduled to be administered these licensing examinations from this date until completion of examination administration, except as specifically noted below and authorized by the NRC (e.g., acting as a simulator booth operator or communicator is acceptable if the individual does not select the training content or provide direct or indirect feedback). Furthermore, I am aware of the physical security measures and requirements (as documented in the facility licensee's procedures) and understand that violation of the conditions of this agreement may result in cancellation of the examinations and/or an enforcement action against me or the facility licensee. I will immediately report to facility management or the NRC chief examiner any indications or suggestions that examination security may have been compromised.

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	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
1.	Stephen K. Ard	OPS INSTRUCTOR		8/15/08		8/26/08	
2.	RICHARD S ALLEN	OIT INSTRUCTOR		8/19/08		8/27/08	
3.	Phillip H. Goss	OIT INSTRUCTOR		8/18/08		8/26/08	
4.	JOSEPH Allen	OIT INSTRUCTOR		8/19/08		8/27/08	
5.							
6.							
7.							
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10.							
11.							
12.							
13.							
14.							
15.							

NO OTHERS

NOTES:

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	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE	NOTE
1.	Ricky Haley	SRO	R Haley	5-9-08	~~R Haley~~	~~5/10/08~~	(1)
2.	Bryan C. Waldsmith	RO/CO	Bryan C. Waldsmith	05-09-08			
3.	TS Gooding	CRSS/SRO	TS Gooding	5-12-08	TS Gooding	8.27.08	
4.	B.H. Clark	Mgr - Training, RNP	[illegible]	5/13/08	[illegible]	8/27/08	
5.	Ken Davis	Supv - Plant Support	[illegible]	5/19/08	[illegible]	8/26/08	
6.	Leonard W. Pitts	SRO ISTA	[illegible]	5/20/8	[illegible]	6/21/8	
7.	Nick Roh	SRO	[illegible]	5/30/08			
8.	J. McDonald	SSD - Operations	[illegible]	6/9/8	[illegible]	8/27/8	
9.	Floyd Lawrence	SOIT - CR3	[illegible]	6/9/08	[illegible]	8/28/08	
10.	John Dalton	Harris - SOIT	[illegible]	6/9/08			
11.	Derek Hawes	OPS Instructor - RNP	[illegible]	6/9/8	[illegible]	8/27/8	
12.	Darryl Foster	Manager-Shift OPS-RNP	[illegible]	6/9/08	[illegible]	8/27/08	
13.	Keith Brown	SUP - OCT	[illegible]	6-9-8	[illegible]	8-27-8	
14.	Archie Lucky	SNOT-F	[illegible]	6-10-08			
15.	Vlad Balakhnin	Rx Engineer	[illegible]	6-23-08			

NOTES:
(1) Signed in error. ~~5/10/08~~

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1.	Ricky Haley	SRO		5-9-08		8/15/08	①
2.	BRYAN C. WALDSMITH	RO/CO		05-09-08			
3.	TS GOULDING	CRSS/SRO		5-12-08		8/27-8	
4.	B.H. CLARK	Mgr - TRAINING, RNP		5/13/08		8/26/08	
5.	Ken Jones	Supv - Plant Support		5/13/08		8/26/08	
6.	LEONARD W. PITTS	SRO ISTA		5/20/08		6/27/08	
7.	NICK Rab	SRO		5/30/08			
8.	J. McDONALD	SRO - OPERATIONS		6/9/08		8/27/08	
9.	Floyd Lawrence	SOIT - CRS		6/9/08		8/28/08	
10.	JOHN DALTON	HARRIS - SOIT		6/9/08		8/28/08	
11.	DEREK HAWES	OPS INSTRUCTOR - RNP		6/9/08		8/28/08	
12.	Doug Foster	Manager-Shift OPS-ENP		6/9/08		8/27/08	
13.	KEITH BROWN	SUP - OCT		6-9-8		8-27-8	
14.	ARCHIE LUCKY	SNOT-F		6-10-08		8-28-08	
15.	Ward Belakovich	Rx Engineer		6-23-08			

NOTES:
① Signed in error. 8/15/08

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	PRINTED NAME	JOB TITLE / RESPONSIBILITY	SIGNATURE (1)	DATE	SIGNATURE (2)	DATE NOTE
1.	Eric McCartney	Director-Site Operations		8/2/08		8/26/08
2.	Craig Lipetzke	Acting Trn Mgr		08/01/08		8/26/08
3.	Joseph A. Rackley	SRO		7-16-08		
4.	Richard Drews	RO		7/24/08		
5.	Mike Donithan	SRO		7/28/08		
6.	Brendan Mulligan	SRO		7/28/08		
7.	Tony C. Jamesse	RO		7/28/08		
8.	Darlene F. Goodnite	SSO		7/24/08		8/26/08
9.	Alexander Curlington	SRO candidate		7/31/08		8/26/08 (1)
10.	Robert Willoughby	NRC Consultant		7/31/08		8/26/08
11.	Thomas F. Tritt	AO-B		7/31/08		8/26/08
12.	JOHN KRANZ	SRO		8/7/08		
13.	John R. Littlefield	SRO		08/07/08		
14.	Martin L. Arnold	SSO		8/7/08		8/26/08
15.	H. Clark Fletcher	Supervisor-Operator Initial Training		8/12/08		8/26/08

NOTES:
(1) SRO candidate for next Initial Class (HLC-10).

ES-301 Administrative Topics Outline Form ES-301-1

Facility: HB ROBINSON Date of Examination: 8/1[illegible]/2008

Examination Level (circle one): RO / **SRO** Operating Test Number:

Administrative Topic (see Note)	Type Code*	Describe activity to be perform[illegible]ed
Conduct of Operations (ADM a)	M	*Manually calculate an Estimated Critical [illegible]ondition.* G2.2.1 (3.9): Given a set of conditions a[illegible] applicable references, perform a Manual Estimated [illegible]ritical Condition Calculation.
Conduct of Operations *(ADM SRO b1)	N	*Review EMP-022, GASEOUS WASTE R[illegible]ELEASE PERMIT.* G2.1.4 (3.8): Given a partially completed [illegible]MP-022, CV Vent Release permit, determine if all [illegible]onditions are met to allow the release.
Equipment Control (ADM SRO c)	N	*Review and approve Technical Specifica[illegible]ion surveillance.* G2.2.39 (4.5): Given a completed, flawe[illegible] OST-020, SHIFTLY SURVEILLANCES, perform th[illegible] review and approval and apply applicable ITS.
Radiation Control (ADM d)	N	*Determine ALARA dose.* G2.3.4 (3.7): Given a set of conditions, c[illegible]lculate the lowest dose path to a job and the lowest [illegible]Stay Time for equipment manipulations.
Emergency Plan (ADM SRO e)	M	*Declare an emergency event.* G2.4.41 (4.6): Given a set of conditions, [illegible]lassify the event IAW the Emergency Action Level [illegible]latrices and complete the Emergency Notification Fo[illegible]n.

NOTE: All items (5 total) are requ[illegible]red for SROs. RO applicants require only 4 [illegible]tems unless they are retaking only the administrative topics, when 5 are required.

*Type Codes & Criteria:
(C)ontrol room
(D)irect from bank (≤ 3 for ROs; ≤ for SROs & RO ret[illegible]kes)
(N)ew or (M)odified from bank (> 1)
(P)revious 2 exams (≤ 1; randomly selected)
(S)imulator

* Approval for replacement JPM

8/21/08
Facility Representative

8/21/2008
NRC Chief Examiner

08/21/08
NRC Branch Chief

ES-301 Administrative Topics Outline Form ES-301-1

Facility: HB ROBINSON Date of Examination: 8/1[illegible]/2008

Examination Level (circle one): RO / SRO Operating Test Number: [illegible]

Administrative Topic (see Note)	Type Code*	Describe activity to be perform[illegible]ed
Conduct of Operations (ADM a)	M	*Manually calculate an Estimated Critical [illegible]ondition.* G2.2.1 (3.9): Given a set of conditions a[illegible] applicable references, perform a Manual Estimated [illegible]ritical Condition Calculation.
Conduct of Operations *(ADM SRO b1)	N	*Review EMP-022, GASEOUS WASTE R[illegible]ELEASE PERMIT.* G2.1.4 (3.8): Given a partially completed [illegible]MP-022, CV Vent Release permit, determine if all [illegible]onditions are met to allow the release.
Equipment Control (ADM SRO c)	N	*Review and approve Technical Specifica[illegible]ion surveillance.* G2.2.39 (4.5): Given a completed, flawe[illegible] OST-020, SHIFTLY SURVEILLANCES, perform th[illegible] review and approval and apply applicable ITS.
Radiation Control (ADM d)	N	*Determine ALARA dose.* G2.3.4 (3.7): Given a set of conditions, c[illegible]lculate the lowest dose path to a job and the lowest [illegible]Stay Time for equipment manipulations.
Emergency Plan (ADM SRO e)	M	*Declare an emergency event.* G2.4.41 (4.6): Given a set of conditions, [illegible]lassify the event IAW the Emergency Action Level [illegible]latrices and complete the Emergency Notification Fo[illegible]n.

NOTE: All items (5 total) are requ red for SROs. RO applicants require only 4 [illegible]tems unless they are retaking only the administrative topics, when 5 are required.

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(C)ontrol room
(D)irect from bank (≤ 3 for ROs; ≤ for SROs & RO ret[illegible]kes)
(N)ew or (M)odified from bank (> 1)
(P)revious 2 exams (≤ 1; randomly selected)
(S)imulator

* Approval for replacement JPM

8/21/08 8/21/2008 08/21/08

Facility Representative NRC Chief Examiner NRC Branch Chief

Facility: HB ROBINSON	Date of Examination: ***8/18/2008***
Examination Level (circle one): RO / **SRO**	Operating Test Number:

Administrative Topic (see Note)	Type Code*	Describe activity to be performed
Conduct of Operations (ADM a)	M	*Manually calculate an Estimated Critical Condition.* G2.2.1 (3.9): Given a set of conditions and applicable references, perform a Manual Estimated Critical Condition Calculation.
Conduct of Operations *(ADM SRO b1)	N	*Review EMP-022, GASEOUS WASTE RELEASE PERMIT.* G2.1.4 (3.8): Given a partially completed EMP-022, CV Vent Release permit, determine if all conditions are met to allow the release.
Equipment Control (ADM SRO c)	N	*Review and approve Technical Specification surveillance.* G2.2.39 (4.5): Given a completed, flawed OST-020, SHIFTLY SURVEILLANCES, perform the review and approval and apply applicable ITS.
Radiation Control (ADM d)	N	*Determine ALARA dose.* G2.3.4 (3.7): Given a set of conditions, calculate the lowest dose path to a job and the lowest Stay Time for equipment manipulations.
Emergency Plan (ADM SRO e)	M	*Declare an emergency event.* G2.4.41 (4.6): Given a set of conditions, classify the event IAW the Emergency Action Level Matrices and complete the Emergency Notification Form.

NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when 5 are required.

*Type Codes & Criteria:
(C)ontrol room
(D)irect from bank (≤ 3 for ROs; ≤ for SROs & RO retakes)
(N)ew or (M)odified from bank (> 1)
(P)revious 2 exams (≤ 1; randomly selected)
(S)imulator

* Approval for replacement JPM

8/21/08 8/21/2008

Facility Representative NRC Chief Examiner NRC Branch Chief

HB ROBINSON NRC SRO EXAMINATION

CONDUCT OF OPERATIONS: Given a set of conditions and applicable references, perform a Manual Estimated Critical Condition Calculation. The applicant will be given all applicable data curves and GP-003, NORMAL PLANT STARTUP FROM HOT SHUTDOWN TO CRITICAL, Attachment 10.1, and will be required to calculate an ECP to within $\pm$ 250 pcm of actual Reactor Engineering calculation. 250 pcm is the tolerance contained in GP-003 between a manually calculated ECP and one that would be received from Reactor Engineering. This JPM will be performed by both RO and SRO candidates. (Modified bank JPM for this exam)

CONDUCT OF OPERATIONS: Given a partially completed EMP-022, CV Vent Release permit, determine if all conditions are met to allow the release. The applicant will be required to determine from the given conditions, what is incorrect and if it must be corrected to allow the CV vent to be performed. This JPM will be performed by SRO applicants only. (New JPM for this exam)

EQUIPMENT CONTROL: Given a completed portion of OST-020, SHIFTLY SURVEILLANCES, that has faulted data, perform the review and approval of the surveillance. The applicant will be required to identify that 2 Safety Injection Accumulators are out of tolerance and initiate action IAW ITS. Since 2 SI accumulators are out of the specified tolerance, LCO 3.0.3 must be applied. This JPM will be performed by SRO candidates only. (New JPM for this exam)

RADIATION CONTROL: Given a set of conditions, the applicant will determine the most efficient method of performing a job to receive the lowest dose for work in an RCA. The applicant will be given 2 possible paths to get to a work site and the option of using 1 or 2 workers. This JPM will be performed by both RO and SRO candidates. (New JPM for this exam)

EMERGENCY PLAN: Given a set of conditions, classify the event IAW the Emergency Action Level Matrices. The applicant will be required to classify a set of conditions using EPCLA-01, EMERGENCY CONTROL, and the EAL Matrices as guidance. Upon completion of the event classification, the applicant will be required to manually fill out an Emergency Notification Form. This JPM is Time Critical; the classification must be made within 15 minutes and the ENF must be completed within 15 minutes from completion of the classification. This JPM will be performed by SRO candidates only. (Modified bank JPM for this exam)

Facility: HB ROBINSON	Date of Examination: ***8/18/2008***
Examination Level (circle one): RO / **SRO**	Operating Test Number:

Administrative Topic (see Note)	Type Code*	Describe activity to be performed
Conduct of Operations (ADM a)	M	*Manually calculate an Estimated Critical Condition.* G2.2.1 (3.9): Given a set of conditions and applicable references, perform a Manual Estimated Critical Condition Calculation.
Conduct of Operations (ADM SRO b)	M	*Determine if required shift manning is met.* G2.1.4 (3.8): Given a set of circumstances, determine whether the shift complement requirements are met.
Equipment Control (ADM SRO c)	N	*Review and approve Technical Specification surveillance.* G2.2.39 (4.5): Given a completed, flawed OST-020, SHIFTLY SURVEILLANCES, perform the review and approval and apply applicable ITS.
Radiation Control (ADM d)	N	*Determine ALARA dose.* G2.3.4 (3.7): Given a set of conditions, calculate the lowest dose path to a job and the lowest Stay Time for equipment manipulations.
Emergency Plan (ADM SRO e)	M	*Declare an emergency event.* G2.4.41 (4.6): Given a set of conditions, classify the event IAW the Emergency Action Level Matrices and complete the Emergency Notification Form.

NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when 5 are required.

*Type Codes & Criteria:
(C)ontrol room
(D)irect from bank (≤ 3 for ROs; ≤ for SROs & RO retakes)
(N)ew or (M)odified from bank (> 1)
(P)revious 2 exams (≤ 1; randomly selected)
(S)imulator

Facility: HB ROBINSON	Date of Examination: ***8/18/2008***
Examination Level (circle one): **RO** / SRO	Operating Test Number:

Administrative Topic (see Note)	Type Code*	Describe activity to be performed
Conduct of Operations (ADM a)	M	*Manually calculate an Estimated Critical Condition.* G2.2.1 (3.9): Given a set of conditions and applicable references, perform a Manual Estimated Critical Condition Calculation.
Conduct of Operations (ADM RO b)	M	*Overtime extension determination.* G2.1.5 (2.9): Given a set of circumstances, determine if work hour limits will be exceeded and notify supervision.
Equipment Control (ADM RO c)	N	*Determine CVCS Blender controls potentiometer settings.* G2.2.12 (3.7): Given a set of conditions, perform the Administrative Daily Checks to determine potentiometer settings for FCV-113A, Boric Acid Flow and HFC-114, Primary Water Flow Auto Mode.
Radiation Control (ADM d)	N	*Determine ALARA dose.* G2.3.4 (3.2): Given a set of conditions, calculate the lowest dose path to a job and the lowest Stay Time for work on equipment.
Emergency Plan		Not selected for RO.

NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when 5 are required.

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(C)ontrol room
(D)irect from bank (≤ 3 for ROs; ≤ for SROs & RO retakes)
(N)ew or (M)odified from bank (> 1)
(P)revious 2 exams (≤ 1; randomly selected)
(S)imulator

Facility: **HB ROBINSON**	Date of Examination: ***8/18/2008***
Exam Level (circle one): RO / SRO(I) / **SRO (U)**	Operating Test No.:

Control Room Systems@ (8 for RO; 7 for SRO-I; 2 or 3 for SRO-U, including 1 ESF)			
	System / JPM Title	Type Code*	Safety Function
a.	**(IRPI/014): Perform Rod Cluster Exercise IAW OST-011.**	**A, M, S**	**1**
b.	(ECCS/006): Fill a Safety Injection Accumulator IAW OP-202.	D, S	2
c.	(SGTR/038): Isolate ruptured S/G IAW PATH-2.	A, D, E, L, S	3
d.	**(CSS/026): Manually initiate Containment Spray IAW PATH-1.**	**A, E, N, S**	**5**
e.	(W/E03): Perform a Post-LOCA Cooldown and Depressurization IAW EPP-8.	A, D, E, L, S	4P
f.	**(NIS/015): Remove N-44 from service IAW OWP-011.**	**D, S**	**7**
g.	(CCW/026): Respond to a Loss of Component Cooling Water.	D, E, S,	8
h.	(SW/076): Limit Radiation Exposure in response to a Radiation alarm IAW AOP-005. (SRO-I do not perform).	D, S	9
In-Plant Systems@ (3 for RO; 3 for SRO-I; 3 or 2 for SRO-U)			
i.	(Rod Control/001): Trip the Reactor from the Rod Drive MG Set Room.	D, E, R	1
j.	**(PZR Pressure/010): Energize PZR Heaters from Emergency busses IAW EPP-21.**	**D, E, L, R**	**3**
k.	**(EDG/064): Manually start EDG using Air Start Solenoids.**	**A, E, N, R**	**6**

@ All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.

* Type Codes	Criteria for RO / SRO-I / SRO-U
(A)lternate path	4-6 / 4-6 / 2-3
(C)ontrol room	
(D)irect from bank	≤ 9 / ≤ 8 / ≤ 4
(E)mergency or abnormal in-plant	≥ 1 / ≥ 1 / ≥ 1
(L)ow-Power / Shutdown	≥ 1 / ≥ 1 / ≥ 1
(N)ew or (M)odified from bank including 1(A)	≥ 2 / ≥ 2 / ≥ 1
(P)revious 2 exams	≤ 3 / ≤ 3 / ≤ 2 (randomly selected)
(R)CA	≥ 1 / ≥ 1 / ≥ 1
(S)imulator	

ES-301 **Operating Test Quality Checklist** **Form ES-301-3**

Facility: H.B.Robinson Date of Examination: 8/18/08 Operating Test Number:

	1. General Criteria	Initials a	b*	c#
a.	The operating test conforms with the previously approved outline; changes are consistent with sampling requirements (e.g., 10 CFR 55.45, operational importance, safety function distribution).			
b.	There is no day-to-day repetition between this and other operating tests to be administered during this examination.			
c.	The operating test shall not duplicate items from the applicants' audit test(s). (see Section D.1.a.)			
d.	Overlap with the written examination and between different parts of the operating test is within acceptable limits.			
e.	It appears that the operating test will differentiate between competent and less-than-competent applicants at the designated license level.			
	2. Walk-Through Criteria	--	--	--
a.	Each JPM includes the following, as applicable: • initial conditions • initiating cues • references and tools, including associated procedures • reasonable and validated time limits (average time allowed for completion) and specific designation if deemed to be time-critical by the facility licensee • operationally important specific performance criteria that include: – detailed expected actions with exact criteria and nomenclature – system response and other examiner cues – statements describing important observations to be made by the applicant – criteria for successful completion of the task – identification of critical steps and their associated performance standards – restrictions on the sequence of steps, if applicable			
b.	Ensure that any changes from the previously approved systems and administrative walk-through outlines (Forms ES-301-1 and 2) have not caused the test to deviate from any of the acceptance criteria (e.g., item distribution, bank use, repetition from the last 2 NRC examinations) specified on those forms and Form ES-201-2.			
	3. Simulator Criteria	--	--	--
	The associated simulator operating tests (scenario sets) have been reviewed in accordance with Form ES-301-4 and a copy is attached.			

		Printed Name / Signature	Date
a.	Author	Richard O. Moore /	8/10/08
b.	Facility Reviewer(*)	James F. Jones /	8/10/2008
c.	NRC Chief Examiner (#)	Edwin Lea, Jr. /	8/12/08
d.	NRC Supervisor	Michael T. Widmann /	08/14/08

NOTE: * The facility signature is not applicable for NRC-developed tests.
Independent NRC reviewer initial items in Column "c"; chief examiner concurrence required.

ES-301 **Simulator Scenario Quality Checklist** **Form ES-301-4**

Facilty: H.B.Robinson Date of Exam: 8/18/08 Scenario Numbers: 1 / 2 /3 Operating Test No.:

QUALITATIVE ATTRIBUTES	Initials a	b*	c#
1. The initial conditions are realistic, in that some equipment and/or instrumentation may be out of service, but it does not cue the operators into expected events.			
2. The scenarios consist mostly of related events.			
3. Each event description consists of • the point in the scenario when it is to be initiated • the malfunction(s) that are entered to initiate the event • the symptoms/cues that will be visible to the crew • the expected operator actions (by shift position) • the event termination point (if applicable)			
4. No more than one non-mechanistic failure (e.g., pipe break) is incorporated into the scenario without a credible preceding incident such as a seismic event.			
5. The events are valid with regard to physics and thermodynamics.			
6. Sequencing and timing of events is reasonable, and allows the examination team to obtain complete evaluation results commensurate with the scenario objectives.			
7. If time compression techniques are used, the scenario summary clearly so indicates. Operators have sufficient time to carry out expected activities without undue time constraints. Cues are given.			
8. The simulator modeling is not altered.			
9. The scenarios have been validated. Pursuant to 10 CFR 55.46(d), any open simulator performance deficiencies or deviations from the referenced plant have been evaluated to ensure that functional fidelity is maintained while running the planned scenarios.			
10. Every operator will be evaluated using at least one new or significantly modified scenario. All other scenarios have been altered in accordance with Section D.5 of ES-301.			
11. All individual operator competencies can be evaluated, as verified using Form ES-301-6 (submit the form along with the simulator scenarios).			
12. Each applicant will be significantly involved in the minimum number of transients and events specified on Form ES-301-5 (submit the form with the simulator scenarios).			
13. The level of difficulty is appropriate to support licensing decisions for each crew position.			

Target Quantitative Attributes (Per Scenario; See Section D.5.d)	Actual Attributes	--	--	--
1. Total malfunctions (5–8)	10 / 9 / 9			
2. Malfunctions after EOP entry (1–2)	4 / 4 / 4			
3. Abnormal events (2–4)	4 / 3 / 3			
4. Major transients (1–2)	2 / 2 / 2			
5. EOPs entered/requiring substantive actions (1–2)	2 / 1 / 2			
6. EOP contingencies requiring substantive actions (0–2)	1 / 1 / 1			
7. Critical tasks (2–3)	2 / 2 / 2			

ES-301 **Simulator Scenario Quality Checklist** **Form ES-301-4**

Facilty: H.B.Robinson Date of Exam: 8/18/08 Scenario Numbers: 4 /-/ - Operating Test No.:

QUALITATIVE ATTRIBUTES		Initials a	b*	c#
1.	The initial conditions are realistic, in that some equipment and/or instrumentation may be out of service, but it does not cue the operators into expected events.			
2.	The scenarios consist mostly of related events.			
3.	Each event description consists of • the point in the scenario when it is to be initiated • the malfunction(s) that are entered to initiate the event • the symptoms/cues that will be visible to the crew • the expected operator actions (by shift position) • the event termination point (if applicable)			
4.	No more than one non-mechanistic failure (e.g., pipe break) is incorporated into the scenario without a credible preceding incident such as a seismic event.			
5.	The events are valid with regard to physics and thermodynamics.			
6.	Sequencing and timing of events is reasonable, and allows the examination team to obtain complete evaluation results commensurate with the scenario objectives.			
7.	If time compression techniques are used, the scenario summary clearly so indicates. Operators have sufficient time to carry out expected activities without undue time constraints. Cues are given.			
8.	The simulator modeling is not altered.			
9.	The scenarios have been validated. Pursuant to 10 CFR 55.46(d), any open simulator performance deficiencies or deviations from the referenced plant have been evaluated to ensure that functional fidelity is maintained while running the planned scenarios.			
10.	Every operator will be evaluated using at least one new or significantly modified scenario. All other scenarios have been altered in accordance with Section D.5 of ES-301.			
11.	All individual operator competencies can be evaluated, as verified using Form ES-301-6 (submit the form along with the simulator scenarios).			
12.	Each applicant will be significantly involved in the minimum number of transients and events specified on Form ES-301-5 (submit the form with the simulator scenarios).			
13.	The level of difficulty is appropriate to support licensing decisions for each crew position.			

	Target Quantitative Attributes (Per Scenario; See Section D.5.d)	Actual Attributes	--	--	--
1.	Total malfunctions (5–8)	1 / 1 / - / -			
2.	Malfunctions after EOP entry (1–2)	5 / - / -			
3.	Abnormal events (2–4)	4 / - / -			
4.	Major transients (1–2)	3 / - / -			
5.	EOPs entered/requiring substantive actions (1–2)	2 / - / -			
6.	EOP contingencies requiring substantive actions (0–2)	2 / - / -			
7.	Critical tasks (2–3)	2 / - / -			

Facility:	Robinson					Date of Exam:	8/18/2008					Operating Test No.:	NRC			
Applicant	Event Type	Scenarios														
		1 (Day 1) (Sim 1)			2 (Day 2) (Sim 2)			3 (Day 3) (Sim 3)			4 (Day 4) (Sim 4)			TOTAL	Minimum		
		CREW POSITION			CREW POSITION			CREW POSITION			CREW POSITION						
		SRO	ATC	BOP	SRO	ATC	BOP	SRO	ATC	BOP	SRO	ATC	BOP		R	I	U
SRO-U (1)	RX													**0**	1	1	0
	NOR	4												**1**	1	1	1
	I/C	13 56					34 68 9							**9**	4	4	2
	MAJ	79					57							**4**	2	2	1
	TS	23												**2**	0	2	2
SRO-I (1)	RX		4											**1**	1	1	0
	NOR													**0**	1	1	1
	I/C		56		13 4									**5**	4	4	2
	MAJ		79		57									**4**	2	2	1
	TS				12 3									**3**	0	2	2
RO (1)	RX					4								**1**	1	1	0
	NOR			4										**1**	1	1	1
	I/C			13 8 10		15								**6**	4	4	2
	MAJ			79		57								**4**	2	2	1
	TS													**0**	0	2	2

Instructions:

1. Circle the applicant level and enter the operating test number and Form ES-D-1 event numbers for each event type; TS are not applicable for RO applicants. ROs must service in both the "at-the-controls (ATC)" and "balance-of-plant (BOP)" positions; Instant SROs must do one scenario, including at least two instrument or component (I/C) malfunctions and one major transient, in the ATC position.

2. Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.5.d) but must be significant per Section C.2.a of Appendix D. (*) Reactivity and normal evolutions may be replaced with additional instrument or component malfunctions on a 1-for-1 basis.

3. Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements specified for the applicant's license level in the right-hand columns.

Facility:	Robinson					Date of Exam:	8/18/2008					Operating Test No.:	NRC			
Applicant	Event Type	Scenarios														
		1 (Day 1) (Sim 1)			2 (Day 2) (Sim 2)			3 (Day 3) (Sim 3)			4 (Day 4) (Sim 4)			TOTAL	Minimum		
		CREW POSITION			CREW POSITION			CREW POSITION			CREW POSITION						
		SRO	ATC	BOP	SRO	ATC	BOP	SRO	ATC	BOP	SRO	ATC	BOP		R	I	U
SRO-U (2)	RX													0	1	1	0
	NOR	4												1	1	1	1
	I/C	13 56												9	4	4	2
	MAJ	79												4	2	2	1
	TS	23												2	0	2	2
SRO-U (3)	RX		4											1	1	1	0
	NOR													0	1	1	1
	I/C		56		13 4									5	4	4	2
	MAJ		79		57									4	2	2	1
	TS				12 3									3	0	2	2
RO (2)	RX					4			2					2	1	1	0
	NOR			4										1	1	1	1
	I/C			13 8 10		15			48					8	4	4	2
	MAJ			79		57			56					6	2	2	1
	TS													0	0	2	2

Instructions:

1. Circle the applicant level and enter the operating test number and Form ES-D-1 event numbers for each event type; TS are not applicable for RO applicants. ROs must service in both the "at-the-controls (ATC)" and "balance-of-plant (BOP)" positions; Instant SROs must do one scenario, including at least two instrument or component (I/C) malfunctions and one major transient, in the ATC position.

2. Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.5.d) but must be significant per Section C.2.a of Appendix D. (*) Reactivity and normal evolutions may be replaced with additional instrument or component malfunctions on a 1-for-1 basis.

3. Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements specified for the applicant's license level in the right-hand columns.

Facility:	Robinson						Date of Exam:	8/18/2008					Operating Test No.:	NRC		
Applicant	Event Type	Scenarios														
		1 (Day 1) (Sim 1)			2 (Day 2) (Sim 2)			3 (Day 3) (Sim 3)			4 (Day 4) (Sim 4)			TOTAL	Minimum		
		CREW POSITION			CREW POSITION			CREW POSITION			CREW POSITION						
		SRO	ATC	BOP	SRO	ATC	BOP	SRO	ATC	BOP	SRO	ATC	BOP		R	I	U
SRO-U (4)	RX													**0**	1	1	0
	NOR							1						**1**	1	1	1
	I/C						34 68 9	23						**7**	4	4	2
	MAJ						57	56						**4**	2	2	1
	TS							46						**2**	0	2	2
	RX													**0**	1	1	0
	NOR													**0**	1	1	1
	I/C													**0**	4	4	2
	MAJ													**0**	2	2	1
	TS													**0**	0	2	2
	RX													**0**	1	1	0
	NOR													**0**	1	1	1
	I/C													**0**	4	4	2
	MAJ													**0**	2	2	1
	TS													**0**	0	2	2

Instructions:

1. Circle the applicant level and enter the operating test number and Form ES-D-1 event numbers for each event type; TS are not applicable for RO applicants. ROs must service in both the "at-the-controls (ATC)" and "balance-of-plant (BOP)" positions; Instant SROs must do one scenario, including at least two instrument or component (I/C) malfunctions and one major transient, in the ATC position.

2. Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.5.d) but must be significant per Section C.2.a of Appendix D. (*) Reactivity and normal evolutions may be replaced with additional instrument or component malfunctions on a 1-for-1 basis.

3. Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements specified for the applicant's license level in the right-hand columns.

Facility: Robinson Date of Examination: 8/18/08 Operating Test No.

Competencies	SRO				RO (ATC)				BOP			
	SCENARIO				SCENARIO				SCENARIO			
	1	2	3	4	1	2	3	4	1	2	3	4
Interpret/Diagnose Events and Conditions	1,3,4, 5,6,7, 9	1,2,3, 4,6,7	2,3,4, 5,6,7	1,2,4, 5,6,7, 8	4,5,6, 7,9	1,2,5	2,4,5, 6,8	1,4,6, 7,8,9, 10	1,3,4, 7,8,9, 10	3,4,6, 7,8,9	1,3,5, 6,7,9	1,3,5, 6,7,8 11
Comply With and Use Procedures (1)	ALL	ALL	ALL	ALL	4,5,6	1,2,4, 5,7	2,4,5, 6,8	1,4,6, 7,8,9, 10	1,3,4, 7,8,9, 10	3,4,6, 7,8,9	1,3,5, 6,7,9	1,3,5, 6,7,8 11
Operate Control Boards (2)	N/A	N/A	N/A	N/A	4,5,6, 7,8,9	1,2,4, 5,7	2,4,5, 6,8	1,4,6, 7,8,9, 10	1,3,4, 8,10	3,4,6, 7,8,9	1,3,5, 6,7,9	1,3,5, 6,7,8 11
Communicate and Interact	ALL	ALL	ALL	ALL	ALL	ALL	ALL	ALL	ALL	ALL	ALL	ALL
Demonstrate Supervisory Ability (3)	ALL	ALL	ALL	ALL	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A
Comply With and Use Tech. Specs. (3)	2,3	1,2,3	4,6	2,3,4	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A

Notes:

(1) Includes Technical Specification compliance for an RO.

(2) Optional for an SRO-U.

(3) Only applicable to SROs.

Instructions:

Circle the applicants' license type and enter one or more event numbers that will allow the examiners to evaluate every applicable competency for every applicant.

ES-401 **Written Examination Quality Checklist** **Form ES-401-6**

Facility: H.B.Robinson Date of Exam: 8/26/08 Exam Level: RO ☒ SRO ☒

Item Description					Initial a	b*	c#
1.	Questions and answers are technically accurate and applicable to the facility.					j	EJ
2.	a. NRC K/As are referenced for all questions. b. Facility learning objectives are referenced as available.					j	EJ
3.	SRO questions are appropriate in accordance with Section D.2.d of ES-401					j	EJ
4.	The sampling process was random and systematic (If more than 4 RO or 2 SRO questions were repeated from the last 2 NRC licensing exams, consult the NRR OL program office).						EJ
5.	Question duplication from the license screening/audit exam was controlled as indicated below (check the item that applies) and appears appropriate: __ the audit exam was systematically and randomly developed; or __ the audit exam was completed before the license exam was started; or __ the examinations were developed independently; or X the licensee certifies that there is no duplication; or __ other (explain)					j	EJ
6.	Bank use meets limits (no more than 75 percent from the bank, at least 10 percent new, and the rest new or modified); enter the actual RO / SRO-only question distribution(s) at right.	Bank	Modified	New		j	EJ
		15/3	7/1	53/24			
7.	Between 50 and 60 percent of the questions on the RO exam are written at the comprehension/ analysis level; the SRO exam may exceed 60 percent if the randomly selected K/As support the higher cognitive levels; enter the actual RO / SRO question distribution(s) at right.	Memory		C/A		j	EJ
		37/9		38/16			
8.	References/handouts provided do not give away answers or aid in the elimination of distractors.					j	EJ
9.	Question content conforms with specific K/A statements in the previously approved examination outline and is appropriate for the tier to which they are assigned; deviations are justified.					j	EJ
10.	Question psychometric quality and format meet the guidelines in ES Appendix B.					j	EJ
11.	The exam contains the required number of one-point, multiple choice items; the total is correct and agrees with the value on the cover sheet.					j	EJ

	Printed Name / Signature	Date
a. Author	Richard O. Moore /	8/10/08
b. Facility Reviewer (*)	James F. Jones /	8/10/08
c. NRC Chief Examiner (#)	Edwin Lea, Jr /	8/12/08
d. NRC Regional Supervisor	MARCOLLET. WIDREVAL /	08/13/08

Note: * The facility reviewer's initials/signature are not applicable for NRC-developed examinations.
Independent NRC reviewer initial items in Column "c"; chief examiner concurrence required.

Facility: HB Robinson Date of Exam: 8/26/08 Exam Level: RO [X] SRO [X]

Item Description	Initials a	b	c
1. Clean answer sheets copied before grading			
2. Answer key changes and question deletions justified and documented			N/A
3. Applicants' scores checked for addition errors (reviewers spot check > 25% of examinations)			
4. Grading for all borderline cases (80 ±2% overall and 70 or 80, as applicable, ±4% on the SRO-only) reviewed in detail			
5. All other failing examinations checked to ensure that grades are justified			N/A
6. Performance on missed questions checked for training deficiencies and wording problems; evaluate validity of questions missed by half or more of the applicants			

	Printed Name/Signature	Date
a. Grader	R.O. Mont /	8/27/08
b. Facility Reviewer(*)	J.F. Jones /	8/29/08
c. NRC Chief Examiner (*)	Edwin Lea, Jr. /	9/9/2008
d. NRC Supervisor (*)	Malcolm T. Widmann /	09/09/08

(*) The facility reviewer's signature is not applicable for examinations graded by the NRC; two independent NRC reviews are required.

Post-Examination Check Sheet: H. B. Robinson	
Task Description	**Date Complete**
1. Facility written exam comments or graded exams received and verified complete	9/9/2008
2. Facility written exam comments reviewed and incorporated and NRC grading completed, if necessary	N/A
3. Operating tests graded by NRC examiners	9/16/2008
4. NRC chief examiner review of operating test and written exam grading completed	9/17/2008
5. Responsible supervisor review completed	9/23/2008
6. Management (licensing official) review completed	9/23/2008
7. License and denial letters mailed	9/23/2008
8. Facility notified of results	9/23/2008
9. Examination report issued (refer to NRC MC 0612)	10/6/2008
10. Reference material returned after final resolution of any appeals	N/A

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6.	7.
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/ units	Back-ward	Q= K/A	SRO Only	U/E/S	Explanation

Instructions

[Refer to Section D of ES-401 and Appendix B for additional information regarding each of the following concepts.]

1. Enter the level of knowledge (LOK) of each question as either (F)undamental or (H)igher cognitive level.
2. Enter the level of difficulty (LOD) of each question using a 1 – 5 (easy – difficult) rating scale (questions in the 2 – 4 range are acceptable).
3. Check the appropriate box if a psychometric flaw is identified:
   - The stem lacks sufficient focus to elicit the correct answer (e.g., unclear intent, more information is needed, or too much needless information).
   - The stem or distractors contain cues (i.e., clues, specific determiners, phrasing, length, etc).
   - The answer choices are a collection of unrelated true/false statements.
   - The distractors are not credible; single implausible distractors should be repaired, more than one is unacceptable.
   - One or more distractors is (are) partially correct (e.g., if the applicant can make unstated assumptions that are not contradicted by stem).
4. Check the appropriate box if a job content error is identified:
   - The question is not linked to the job requirements (i.e., the question has a valid K/A but, as written, is not operational in content).
   - The question requires the recall of knowledge that is too specific for the closed reference test mode (i.e., it is not required to be known from memory).
   - The question contains data with an unrealistic level of accuracy or inconsistent units (e.g., panel meter in percent with question in gallons).
   - The question requires reverse logic or application compared to the job requirements.
5. Check questions that are sampled for conformance with the approved K/A and those that are *designated SRO-only* (K/A and license level mismatches are unacceptable).
6. Based on the reviewer's judgment, is the question as written (U)nsatisfactory (requiring repair or replacement), in need of (E)ditorial enhancement, or (S)atisfactory?
7. At a minimum, explain any "U" ratings (e.g., how the Appendix B psychometric attributes are not being met).

1			X											S?	Should not use the word approximately. Show how you determine the points used. CHANGED STEM 35 TO 18 PSIG. REMOVED APPROXIMATELY. CHANGED DISTRACTORS - OK
2														U	A & B are not plausible. Are there any scenario, where an automatic valve alignment would occur while an operator is performing a manual transfer or alignment on that system? CHANGED STEM AND REWORDED DISTRACTORS. – OK

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6.	7.
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/ units	Back-ward	Q= K/A	SRO Only	U/E/S	Explanation
3		1												U	LOD. This question is very simple. What indication are provided to indicate there might be a failure of the #2 seal? CHANGED STEM AND DISTRACTORS.– OK
4		2												S	MADE A CHANGE TO THE STEM – OK
5		1												U	Easy question. Fluctuating – cycling. All of the other distractors would result in constant flow or no flow. Very little system knowledge is needed to answer this question. CHANGED STEM – OK
6			X											S/E	Crew "is"– ..... Path-1 and is unable..... MADE CHANGES TO THE STEM – OK
7		2												S	CHANGED QUESTION BECAUSE IT WAS TO CLOSE TO 10 QUESTION. – OK
8		2				X								E	In distractor C you are increasing the possibility of a release. Why would this be plausible? CHANGED DISTRACTOR – OK.
9		2												S	ADDED WORDS TO THE STEM. – OK
10			X			X								U	Look at question 7. Very little difference in the two questions. CHANGED WORDING IN THE STEM AND DISTRACTORS. CHANGED QUESTION 7. – OK
11		2												S	OK
12			X			X								U	The stem says that "Both PAM operable." There is no need for the applicant to say that he must verify PAM is operable as stated in the distractors A & B. Two implausible distractors A & B. CHANGED DISTRACTORS AND STEM – OK.
13	H	2												S	OK
14	2	H												S	MADE CHANGES TO THE STEM AND DISTRACTORS. OK.
15	F	2												S	CHANGED DISTRACTOR "C" – OK
16		1												S	This is a memory level question. Could be improved. CHANGED WORDING IN DISTRACTORS. – OK

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6.	7.
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/ units	Back-ward	Q= K/A	SRO Only	U/E/S	Explanation
17	F	2												S	CHANGED WORDING IN STEM – OK
18	F	2												S	REWORDED ONE OF THE DISTRACTORS – OK –WILL RE-VALIDATE QUESTION
19	H	1/2												U	Based on the information provided, limited information is needed to answer the question — IRPI indications increasing and Tavg 1.5 degrees higher than Tref and rods in AUTO = UNC ROD withdrawal. Distractors are not plausible. As written LOD. CHANGED DISTRACTORS – OK
20						X								U	There may be two correct answer or the answer identified as correct is not. If there is a rod that failed to insert, some actions must be taken to insert that rod as some point. The answer you provided as correct states "No actions required." CHANGED STEM DISTRACTORS – OK
21	H	2												S	OK
22		2				X								E/S	Need to look at plausibility of distractor A -CHANGED DISTRACTOR A. – OK
23	F?	2				X								E	Distractor A is not plausible. REWORDED STEM AND REWORDED DISTRACTORS. – XXXXXX — WILL TAKE ANOTHER LOOK AT THIS QUESTION. PROBLEM WITH DISTRACTOR D. 7/22/08 CHANGED DISTRACTOR D. – OK.
24	F	2												S	OK
25	F?	1												U	LOD. The question meets the K/A. Please explain the operational value of this question. How often and in what procedures require the RO to make this determination/perform such a calculation? LICENSEE WANTS TO KEEP QUESTION – OK
26														S	OK
27						X								U	Distractors A & B are not plausible. ADDED INFO TO THE STEM, REWORDED DISTRACTOR A. OK.
28														S	OK

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6.	7.
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/ units	Back-ward	Q= K/A	SRO Only	U/E/S	Explanation
29						X						X		U?	Distractor B is not plausible. You said that it was isolated. K/A not matched. NEW QUESTION – XXXXXXX NEED TO LOOK AT DISTRACTOR D. WILL REVISIT ON TUESDAY. XXXXXXX – 7/22/08 – WROTE NEW QUESTION – OK
30														S	OK
31		3												S	OK
32		2												S	MADE CHANGES TO DISTRACTORS. – OK
33		2												S	OK
34		2												S	MADE A CHANGE TO STEM – OK 7/22/08
35		2												S	WILL LOOK AT CHANGING DISTRACTOR B XXXXXXXXXX CHANGED DISTRACTOR B – 7/1/31/2008 –OK
36		1				X								U	Distractor A is not plausible. Distractor B is not plausible. Distractors not related to question asked. XXXX WILL TAKE BACK TO LOOK AT ADDING WORDS TO THE DISTRACTOR TO MAKE QUESTION MATCH THE K/A — XXXXXXXXXXX CHANGED DISTRACTORS – OK. 7/31/2008
37		1/2				X								E	LOD. In distractor C& D are you attempting to say auto start? If so, why not use Auto-start/automatically started? Distractor D plausible? Would such a condition exist for the given plant equipment lineup? Maintenance is in progress – if proper safety precautions are in place, precautions would be in place to prevent equipment from starting - personnel safety is one of the first things addressed when working on equipment – that means making sure that equipment will not auto start..... CHANGED DISTRACTORS – OK
38		2				X								S/E	Distractors are weak. All systems functioned as designed - please explain why anyone would expect the a design limit to be exceed. CHANGED STEM AND DISTRACTORS. – OK
39		2												S	
40		2												S	

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6.	7.
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/ units	Back-ward	Q= K/A	SRO Only	U/E/S	Explanation
41		1													LOD To answer this question all one need to remember is a required flow of 300 GPM is needed. The 300 GPM flow requirement is considered common knowledge. Flow is increased by opening a valve/valves. MADE CHANGES TO STEM – OK.
42		3												S	
43		3												S	
44		2												S	
45		1				X									LOD. Easy distractors. I am not sure if you can do much with this question. Is there a valve that can not be closed form the Panel????
46						X								U	As written the question has two correct answers (A & B)
47		2												S	Easy
48						X								U	Two correct answers (A & B would solve the problem).
49		2				X								U	Two correct answers - A & D – REWROTE QUESTION – 7/31/2008
50		1/2												S	Easy question – CHANGED STEM & DISTRACTORS – OK 7/31/2008
51		2				X								S?	Could C also be correct?
52		2												S	
53		1				X						?		U	LOD. We know that SG A is isolated. A & C removed immediately. Distractors do not appear to be plausible. Make sure K/A matches.
54												X		U?	**Please explain why you consider this a K/A match. STILL NEED TO WORK ON – 7/31/2008 XXXXXXXXX WROTE A NEW QUESTION OK 8/5/2008**
55		1										X		U	LOD. K/A not matched. Setpoint question. As written there is more than one correct answer.
56		2												S	
57		2												S	

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6.	7.
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/ units	Back-ward	Q= K/A	SRO Only	U/E/S	Explanation
58		2												S	Question could be improved!!
59		1	X			X								U	What are you asking? The stem says that Train A Plasma display is inop..... Then use Train B . Then you ask how to determine subcooling and CET - answer CETC temperatures from Train A and B CETCs. Distractors A & D are not plausible. Are temperatures gotten from a junction box? Why would one be required to look at reading from only one train to make a key determination?
60		2												S	
61		2												S	
62						X								U	Distractors A & D do not appear to be plausible
63		2												S	
64		2												S	
65		2												S	
66		1			X									E	Distractor C is not plausible. What are Bosun's used for?
67		2												S?	See if this question is on the SRO exam.
68		1										X		U/E?	LOD Do you think this question addresses the knowledge of a process? SAT – OK 7/31/2008
69														U	LOD It appears that this question can be answered by answering which system/component is most important to plant safety/operation.
70		1												U	LOD General rad worker question. Is this specifically RO knowledge?
71		1												?	LOD
72		1												?	LOD
73		1				X								?	LOD Simple memory - Distractor C.
74		1												?	LOD

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6.	7.
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/ units	Back-ward	Q= K/A	SRO Only	U/E/S	Explanation
75			X			X								U	"A" could be correct. As written the stem states that all immediate actions have been completed. Therefore, is it not right to say that PATH-1 actions have been completed? Have you not already ensured that the reactor and turbine .....? If only one train of SI and RHR is required, then there may be not correct answer. In your reference you did not provide the bases. Distractors C & D are not plausible. When is an operator required to wait a period of time before attempting to start/make happen an automatic action once it was observed as not having occurred? **STILL NEED TO WORK ON 7/31/2008 SWAPPED QUESTIONS – 73 BECAME 74; 74 BECAME 75, AND REPLACED 73. OK – 8/5/2008**
															54 & 75 NEED TO BE WORKED ON – 7/31/2008

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6. U/E/S	7. Explanation
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/ units	Back-ward	Q= K/A	SRO Only		

Instructions

[Refer to Section D of ES-401 and Appendix B for additional information regarding each of the following concepts.]

1. Enter the level of knowledge (LOK) of each question as either (F)undamental or (H)igher cognitive level.
2. Enter the level of difficulty (LOD) of each question using a 1 – 5 (easy – difficult) rating scale (questions in the 2 – 4 range are acceptable).
3. Check the appropriate box if a psychometric flaw is identified:
   - The stem lacks sufficient focus to elicit the correct answer (e.g., unclear intent, more information is needed, or too much needless information).
   - The stem or distractors contain cues (i.e., clues, specific determiners, phrasing, length, etc).
   - The answer choices are a collection of unrelated true/false statements.
   - The distractors are not credible; single implausible distractors should be repaired, more than one is unacceptable.
   - One or more distractors is (are) partially correct (e.g., if the applicant can make unstated assumptions that are not contradicted by stem).
4. Check the appropriate box if a job content error is identified:
   - The question is not linked to the job requirements (i.e., the question has a valid K/A but, as written, is not operational in content).
   - The question requires the recall of knowledge that is too specific for the closed reference test mode (i.e., it is not required to be known from memory).
   - The question contains data with an unrealistic level of accuracy or inconsistent units (e.g., panel meter in percent with question in gallons).
   - The question requires reverse logic or application compared to the job requirements.
5. Check questions that are sampled for conformance with the approved K/A and those that are *designated SRO-only* (K/A and license level mismatches are unacceptable).
6. Based on the reviewer's judgment, is the question as written (U)nsatisfactory (requiring repair or replacement), in need of (E)ditorial enhancement, or (S)atisfactory?
7. At a minimum, explain any "U" ratings (e.g., how the Appendix B psychometric attributes are not being met).

76			X	X									X	U	There appears to be unnecessary information in the stem. You identify procedures, then ask what procedures should be used. I do not see this as a SRO only question. If the controller was in AUTO, and the RO noticed that the controller was operating erratically, he/she would take manual control, realize what procedure should be entered, perform immediate actions from memory to correct the problem based on plant conditions — SRO directions/instructions would not be required to assure that the immediate actions are complete.

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6.	7.
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/ units	Back-ward	Q= K/A	SRO Only	U/E/S	Explanation
77	H					X								U	Distractor D is not plausible. There is no information in the stem which would indicate that a loss of IA had occurred. If we allow the applicant to make that assumption, then the answer could be correct if the failed close position is closed. Assumptions should not be made in selecting the answer. How can Distractor A be plausible, if you say"no action required" then conclude with saying that you must "Ensure....." This would be an action
78	H					X							X	U	This question can be answered with system knowledge only. Distractor D is not plausible. If a pump is cavitating, why would anyone think that reducing flow would solve the problem.
79	H													S	
80	H		X	X		X							X	U	Two correct answers - C & D. Based on the information given and the current plant conditions, why would anyone think that they would exit LOSS of instrument air? The stem asked what procedure is required to restore cool cooling - you only need to remember what procedure number. Do reactor operators not know this? Explain why you consider this SRO only.
81	F													S/E/U ?	Who is required to know the bases of the cautions in procedures. Can this question be answered based on system knowledge (how system operate for specific conditions)? The basis states "The Caution is provided to warn the Operator of the possibility of equipment performing uncontrolled starts."
82	H	2				X							X	U	Not SRO only. System question that an RO can answer. Distractor D not plausible.
83	H					X								S/E	Take a look at distractor B. Make sure that it is not correct. Why do you consider distractor D plausible?
84	F	2				X								U	Distractors C & D are not plausible. Why do you consider A plausible. The reactor is shut down - Why would one consider Reactor Core Safety Limits a concern when in FRCP-C.1? WROTE NEW QUESTIONS – OK 7/31/2008

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6.	7.
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/ units	Back-ward	Q= K/A	SRO Only	U/E/S	Explanation
85	H					X						?		U	What is this question asking? It appears that the question is asking which safety limit is affected once a cool-down is started while implementing FRP-C.1. (Pressure / Temperature) This is RO knowledge, in that they know that pressure and temperature limits are of a concern. They may not know the TS number, but they can identify the limit. Distractors A & D are not plausible. STILL NEED TO WORK ON THIS QUESTION 7/31/2008 – CHANGED K/A – WROTE NEW QUESTION – OK – 8/5/2008
86	H												X	U	Not an SRO question. System knowledge question. For the conditions given, after the standby pump is started, observations noted by the RO after starting the pump. He/she recognizes that pressure is increasing and takes appropriate actions according to APP. Why would one think there is a malfunction of the RCS pressure control when we said that the speed controller for the pump was set at maximum. Is the SRO really providing directions or is the RO performing actions and the SRO is agreeing? Would the RO not respond by stopping the activities that started the pressure increase ..... Stopping the standby pump....? CHANGED K/S – NEW QUESTION – OK 7/31/2008
87	H												X	U	This is an RO question. To answer the question, based on information in the stem and the way the distractors are written, all you need to realize is the fact that an SI did not occur. and the distractors, all you need to know to answer this question is ..... Is SI required, yes - go to Path -1. – WROTE NEW QUESTION – LICENSEE IS HAPPY – OK 8/5/2008
88	F												X	U	Please explain why you consider this a SRO only question.
89	H												X	U	As written this is a systems question requiring only RO knowledge to answer.
90	F	1												U	LOD
91	H													S?	Need to make sure there are not two correct answers (D & B)

Q#	1. LOK (F/H)	2. LOD (1-5)	3. Psychometric Flaws					4. Job Content Flaws				5. Other		6. U/E/S	7. Explanation
			Stem Focus	Cues	T/F	Cred. Dist.	Partial	Job-Link	Minutia	#/ units	Back-ward	Q= K/A	SRO Only		
92	H													S?	May be RO Knowledge. Based on the information provided could an RO not answer the question once it is concluded that RCS leakage is occurring. You could leave on the procedure number and selected correct answer based on name of procedure only Excessive Leakage. REWROTE QUESTION – OK – 8/5/2008
93	F	1												U	LOD. Identify a system where oxygen limits is required to be maintained to prevent corrossion.
94	F													S?	Could C be correct?
95	F?												X	U?	Not SRO only. Who makes the entry? CHANGED STEM AND DISTRACTOR – OK 7/31/2008
96	F	1												U	LOD. No knowledge of procedures is required. Distractors B not plausible. Distractor A could be correct.
97	F	1				X						?		U	Distractor C & D are not plausible. CHANGED DISTRACTORS – OK 7/31/2008
98	H	3												S	
99	F	1													LOD Common knowledge, RO would know this, but it is the SRO responsibility to know what to do given the conditions. Memory question.... Which AOPs are considered concurrent AOPs? Which AOPs should be performed concurrently while performing procedures in the EOP network?
100	F	1				X						X		E	Could distractor C be correct? Is knowledge of strategy of actions mean to describe the bases?

Progress Energy

Serial: RNP-RA/08-0055

MAY 27 2008

Mr. Luis A. Reyes
Regional Administrator, Region II
U. S. Nuclear Regulatory Commission - Region II
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H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261/LICENSE NO. DPR-23

REACTOR AND SENIOR REACTOR OPERATOR INITIAL EXAMINATION OUTLINES

Dear Mr. Reyes:

In response to NRC letter dated April 23, 2008, Carolina Power and Light Company, now doing business as Progress Energy Carolinas, Inc., has submitted the requested examination outlines to your staff for H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2. The reactor and senior reactor operator initial examination outlines were mailed directly to Mr. Edwin Lea on May 15, 2008.

If you have any questions concerning this matter, please contact Mr. C. A. Castell at (843) 857-1626.

Sincerely,

C. T. Baucom
Manager – Support Services – Nuclear

CTB/cac

c: Document Control Desk
NRC Resident Inspector, HBRSEP
Ms. M. G. Vaaler, NRC, NRR
Mr. M. T. Widmann, NRC, Region II

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MAY 27 2008

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NRC Resident Inspector, HBRSEP
Ms. M. G. Vaaler, NRC, NRR
Mr. M. T. Widmann, NRC, Region II

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Serial: RNP-RA/08-0056

JUL 02 2008

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H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261/LICENSE NO. DPR-23

REACTOR OPERATOR AND SENIOR REACTOR OPERATOR INITIAL EXAMINATIONS

Dear Mr. Reyes:

In response to NRC letter dated April 23, 2008, Carolina Power and Light Company, now doing business as Progress Energy Carolinas, Inc., has submitted the operating and written examination materials identified in Attachment 2 of ES-201 to your staff for H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2. The operating and written examination materials were shipped directly to Mr. Edwin Lea on June 23, 2008.

If you have any questions concerning this matter, please contact Mr. C. A. Castell at (843) 857-1626.

Sincerely,

C. T. Baucom
Manager – Support Services – Nuclear

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NRC Resident Inspector, HBRSEP
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JUL 10 2008

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POST-EXAM COMMENTS

(Green Paper)

Licensee Submitted Post-Exam Comments

[ ] Letter Attached With Comments

[ ] Comments Only - No Letter

[ ] Letter Stating "No Comments"

[✓] None

Progress Energy

No Post-exam comments letter

Serial: RNP-RA/08-0101

SEP 2 4 2008

Mr. Luis A. Reyes
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Atlanta, Georgia 30303-8931

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261/LICENSE NO. DPR-23

POST EXAMINATION COMMENTS FOR OPERATOR
INITIAL WRITTEN EXAMINATIONS ADMINISTERED ON AUGUST 26, 2008

Dear Mr. Reyes:

In accordance with NUREG-1021, "Operator Licensing Examination Standards for Power Reactors," ES-402, "Administering Initial Written Examinations," H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2, offers no challenges or formal comments to questions found on the operator initial written examinations administered on August 26, 2008. HBRSEP, Unit No. 2, Operations Training personnel have conducted an examination review with each candidate to ensure any missed questions are understood and that no knowledge deficiencies exist.

If you have any questions concerning this matter, please contact me at (843) 857-1626.

Sincerely,

C. A. Castell
Supervisor – Licensing/Regulatory Programs

CAC/cac

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SEP 3 0 2008