

Facility: GRAND GULF NUCLEAR STATION		Date of Examination: 06/28/2010
Examination Level: RO <input checked="" type="checkbox"/> SRO <input type="checkbox"/>		Operating Test Number: 6/10
Administrative Topic (see Note)	Type Code*	Describe activity to be performed
Conduct of Operations	S; N	Perform 06-OP-1R20-W-0001-02, <i>Attachment II of weekly AC/DC Lineup for ESF 21 Transformer Retest</i> GJPM-OPS-ADMR91.00 K/A 2.1.20: 4.6
Conduct of Operations	R; N	Review Cooldown Record GJPM-OPS-ADMR92.00 K/A2.1.25: 3.9
Equipment Control	R; M	Determine Tagging Requirements & Prepare Tagout Tags Sheet for SRV B21-F047L GJPM-OPS-ADMR93.00 K/A 2.2.13: 4.1
Radiation Control		N/A
Emergency Procedures/Plan	R; M	Primary Containment Water Level Determination EOP Attachment 29 GJPM-OPS-ADMR94.00 K/A 2.4.21: 4.0
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.		
* Type Codes & Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) (N)ew or (M)odified from bank (≥ 1) (P)revious 2 exams (≤ 1; randomly selected)		

Facility: GRAND GULF NUCLEAR STATION		Date of Examination: 06/28/2010
Examination Level: RO <input type="checkbox"/> SRO <input checked="" type="checkbox"/>		Operating Test Number: 6/10
Administrative Topic (see Note)	Type Code*	Describe activity to be performed
Conduct of Operations	R; N	Determine Fire Watch Requirements GJPM-OPS-ADMS95.00 K/A 2.1.2: 4.4
Conduct of Operations	R; M	Determine Plant Safety Index Risk Color with the EOOS Risk Monitor Not Available GJPM-OPS-ADMS96.00 K/A2.1.25: 4.2
Equipment Control	R; M	Determine LCO Actions and Initiate a Manual LCO Tracking Report GJPM-OPS-ADMS97.00 K/A 2.2.22: 4.7
Radiation Control	R; M	Determine Protective Action Recommendations GJPM-OPS-ADMS88.00 K/A 2.3.13: 3.8
Emergency Procedures/Plan	R; M	Determine Entry into Site Emergency Plan and Complete Initial Notification Forms as applicable GJPM-OPS-EAL27.00 K/A 2.4.41: 4.6
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.		
* Type Codes & Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) (N)ew or (M)odified from bank (≥ 1) (P)revious 2 exams (≤ 1 ; randomly selected)		

Facility: GRAND GULF NUCLEAR STATION		Date of Examination: 6/28/2010	
Exam Level: RO <input checked="" type="checkbox"/> SRO-I <input type="checkbox"/> SRO-U <input type="checkbox"/>		Operating Test No.: 6/10	
Control Room Systems [@] (8 for RO); (7 for SRO-I); (2 or 3 for SRO-U, including 1 ESF)			
System / JPM Title	Type Code*	Safety Function	
a. 201005/201001/295022 Operate CRD System/RCIS to Achieve Criticality and Commence Reactor Heatup	S; L; M; A	1	
b. 295030/223001 Raise Suppression Pool Level using HPCS	S; M; A	5	
c. 217000 Restore RCIC to Standby following Steam Supply Isolation	S; M	2	
d. 202001 Rapid Restart of Recirc Pump A to Mitigate Thermal Stratification	S; L; M	4	
e. 262001 Align Buses 16AB and 17AC to ESF Transformer	S; M; A	6	
f. 212000 Transfer RPS B to Normal Power Source and RPS A to Alternate Power Source	S; M; A	7	
g. 400000 Align CCW to FPCCU Heat Exchanger A	S; N; A	8	
h. 21272000 Perform Area Radiation Monitor Functional Test	S; N	9	
In-Plant Systems [@] (3 for RO); (3 for SRO-I); (3 or 2 for SRO-U)			
i. 295016 Perform Remote Shutdown Panel Area Actions for Security Code Orange	E; N	7	
j. 295015 EP Attachment 22 – Locally Scram Control Rods	R; E; D	1	
k. 295019 Install Nitrogen Bottle on ADS Air Supply	R; E; D	3	
@ All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.			
* Type Codes	Criteria for RO / SRO-I / SRO-U		
(A)lternate path	4-6 / 4-6 / 2-3		
(C)ontrol room	≤ 9 / ≤ 8 / ≤ 4		
(D)irect from bank	≥ 1 / ≥ 1 / ≥ 1		
(E)mergency or abnormal in-plant	- / - / ≥1 (control room system)		
(EN)gineered safety feature	≥ 1 / ≥ 1 / ≥ 1		
(L)ow-Power / Shutdown	≥ 2 / ≥ 2 / ≥ 1		
(N)ew or (M)odified from bank including 1(A)	≤ 3 / ≤ 3 / ≤ 2 (randomly selected)		
(P)revious 2 exams	≥ 1 / ≥ 1 / ≥ 1		
(R)CA			
(S)imulator			

Facility: GRAND GULF NUCLEAR STATION		Date of Examination: 6/28/2010	
Exam Level: RO <input type="checkbox"/> SRO-I <input checked="" type="checkbox"/> SRO-U <input type="checkbox"/>		Operating Test No.: 6/10	
Control Room Systems [@] (8 for RO); (7 for SRO-I); (2 or 3 for SRO-U, including 1 ESF)			
System / JPM Title	Type Code*	Safety Function	
a. 201005/201001/295022 Operate CRD System/RCIS to Achieve Criticality and Commence Reactor Heatup	S; L; M; A	1	
b. 295030/223001 Raise Suppression Pool Level using HPCS	S; M; A	5	
c. 217000 Restore RCIC to Standby following Steam Supply Isolation	S; M	2	
d. 202001 Rapid Restart of Recirc Pump A to Mitigate Thermal Stratification	S; L; M	4	
e. <i>262001 Align Buses 16AB and 17AC to ESF Transformer 21</i>	<i>S; M; A</i>	<i>6</i>	
f. 212000 Transfer RPS B to Normal Power Source and RPS A to Alternate Power Source	S; M; A	7	
g. 400000 Align CCW to FPCCU Heat Exchanger A	S; N; A	8	
h. N/A			
In-Plant Systems [@] (3 for RO); (3 for SRO-I); (3 or 2 for SRO-U)			
i. 295016 Perform Remote Shutdown Panel Area Actions for Security Code Orange	E; N	7	
j. 295015 EP Attachment 22 – Locally Scram Control Rods	R; E; D	1	
k. 295019 Install Nitrogen Bottle on ADS Air Supply	R; E; D	3	
@ All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.			
* Type Codes	Criteria for RO / SRO-I / SRO-U		
(A)lternate path	4-6 / 4-6 / 2-3		
(C)ontrol room			
(D)irect from bank	≤ 9 / ≤ 8 / ≤ 4		
(E)mergency or abnormal in-plant	≥ 1 / ≥ 1 / ≥ 1		
(EN)gineered safety feature	- / - / ≥1 (control room system)		
(L)ow-Power / Shutdown	≥ 1 / ≥ 1 / ≥ 1		
(N)ew or (M)odified from bank including 1(A)	≥ 2 / ≥ 2 / ≥ 1		
(P)revious 2 exams	≤ 3 / ≤ 3 / ≤ 2 (randomly selected)		
(R)CA	≥ 1 / ≥ 1 / ≥ 1		
(S)imulator			

Scenario 2

Facility: **GRAND GULF NUCLEAR STATION** Scenario No.: **2** Op-Test No.: **06/10**

Examiners: _____ Operators: _____

Objectives: To evaluate the candidates' ability to operate the facility in response to the following evolutions:

1. Place the second Reactor Feed Pump (A) on the Master Level Controller.
2. Raise reactor power by withdrawing control rods.
3. Respond to a control rod drifting *out*.
4. Respond to a spurious HPCS initiation followed by HPCS pump trip.
5. Respond to loss of Service Transformers 11 and 21.
6. Respond to a trip of Division 1 Diesel Generator.
7. Respond to LOCA in the Drywell.
8. Respond to a failure Division 2 ECCS to automatically initiate.

Initial Conditions: Reactor Power is 50%.

INOPERABLE Equipment

None

Turnover:

The plant is at 50% power during startup. Reactor Recirc pumps are in fast speed with core flow 60%. **Both Circ Water Pumps are in service.** After assuming the shift, place RFP A on the Master Level Controller per IOI 03-1-01-2 step 5.16.1 and SOI 04-1-01-N21 section 4.6.4 immediately following turnover. Then, power ascension is to continue with withdrawing control rods in accordance with the Control Rod Movement Sequence per IOI 03-1-01-2 and the Reactivity Management Plan for startup. There is no out of service equipment, and EOOS is green. A Division 1 work week is in effect.

Scenario 2

Event No.	Malf. No.	Event Type*	Event Description
1		N(BOP)	Place Reactor Feed Pump A on the Master Level Controller (IOI 03-1-01-2 step 5.16.1 and SOI 04-1-01-N21-1 section 4.6.4)
2		R(ACRO)	Raise reactor power to approximately 55% by withdrawing control rods. (Control Rod Movement Sequence)
3	<i>z161161_40_25</i>	C(ACRO)	Respond to control rod 40- <i>25</i> drift <i>out</i> (ONEP 05-1-02-IV-1 sections 2.2/3.2; Tech Spec 3.1.3)
4	e22052 e22053	C(SS)	Respond to a spurious HPCS initiation and HPCS pump trip (02-S-01-27 step 6.6.3, ARI 04-1-02-1H13-P601-16A-H3, Tech Spec 3.5.1)
5	r21133a r21133b	M(ALL)	Respond to a trip of Service Transformers 11 and 21 (ONEPs 05-1-02-I-4, 05-1-02-I-1, 05-1-02-I-2; EP-2)
6	n41141a	C (BOP)	Respond to trip of Division 1 Diesel Generator (ONEP 05-1-02-I-4)
7	rr063a@4.92	M(ALL)	Respond to a LOCA in the drywell (EP-2, EP-3)
8	rr040f@0 rr041f@100	I(ACRO)	Respond to failure of Division 2 ECCS to automatically initiate (EN-OP-115)

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Critical Tasks

- *Open at least seven SRVs before RPV level drops to -210" Fuel Zone Range.*
- *Manually initiate Division 2 ECCS prior to RPV pressure going below 300 psig.*

Facility: **GRAND GULF NUCLEAR STATION** Scenario No.: **5** Op-Test No.: **06/10**

Examiners: _____ Operators: _____

Objectives: To evaluate the candidates' ability to operate the facility in response to the following evolutions:

1. Raise reactor power to 100% using Recirc Flow Control Valves.
2. Respond to failure of RPS RPV Level Transmitter A.
3. Respond to Thyristor Voltage Regulator (TVR) trip to manual.
4. Respond to Recirc Pump A trip.
5. Respond to Thermal Hydraulic Instability.
6. Respond to a failure of the Reactor Mode Switch to initiate Scram.
7. Respond to Hydraulic Block ATWS above 4% power.
8. Respond to a SLC piping rupture.

Initial Conditions: Reactor Power is 90%.

INOPERABLE Equipment

None

Turnover:

The plant is at 90% power. Reactor power is to be raised to 100%. Pre-conditioning conditions are established for 100% power operation. There is no out of service equipment and EOOS is green. A Division 1 work week is in effect.

Scenario 5

Event No.	Malf. No.	Event Type*	Event Description
1		R(ACRO)	Raise reactor power to 100% using Recirc Flow Control Valves (IOI 03-1-01-2 Attachment VIII)
2	lt b21n080a_b	C(ACRO)	Respond to failure of RPS RPV Level Transmitter A. (ARI 04-1-02-1H13-P6804A2-D4; 5A-C1; 7A-A2; Tech Spec 3.3.1.1; 3.3.6.1; TR3.1.5).
3	n41102	C(BOP)	Respond to TVR trip from Automatic to Manual (ARI 04-1-02-1H13-P680-9A-C15, SOI 04-1-01-N40-1 section 5.2) (GGNS Scram 10/2008)
4	rr012a	C(BOP)	Respond to trip of Recirc Pump A (ARI 04-1-02-P680-3A-B4, ONEP 05-1-02-III-3; Tech Spec 3.4.1)
5	rr165 @ 5 – 100 over 5 minutes c71162	C(ACRO)	Respond to Thermal Hydraulic Instability (ONEP 05-1-02-III-3, 05-1-02-I-1, 05-1-02-I-2)
6	di_1c71m602 RUN	C(ACRO)	Respond to a failure of the Reactor Mode Switch to scram the Reactor. (EN-OP-115)
7	c11164 @ 30	M(ALL)	Respond to ATWS to insert control rods with power above 4% (EP-2A)
8	c41263 @ 60	C (BOP)	Respond to leak in SLC piping (EP-2A)

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Critical Tasks

- *When Thermal Hydraulic Instability is observed, inserts a manual scram.*
- *Insert control rods by manual scram and/or normal rod insertion via RCIS.*

Facility: **GRAND GULF NUCLEAR STATION** Scenario No.: **6** Op-Test No.: **06/10**

Examiners: _____ Operators: _____

Objectives: To evaluate the candidates' ability to operate the facility in response to the following evolutions:

1. Raise reactor power to 85% using Recirc Flow Control Valves.
2. Place the third Condensate and Condensate Booster Pump in service.
3. Respond to APRM A failed upscale.
4. [Respond to trip of ESF Transformer11.](#)
5. Respond to a loss of Main Condenser Vacuum.
6. Respond to a LOCA with reduced injection systems.
7. Respond to failure of RCIC to auto start.

Initial Conditions: Reactor Power is 80%.

INOPERABLE Equipment

None

Turnover:

The plant is at 80% power. Reactor power is to be raised to 85%. Pre-conditioning conditions are established for 100% power operation.

Condensate Pump B and Condensate Booster Pump B pre-start checks are complete.

Condensate Cleanup is ready for the third Condensate and Booster Pump operation. IOI 03-1-01-2 Attachment II step 6.6 is complete. There is no out of service equipment and EOOS is green. A Division 1 work week is in effect.

Event No.	Malf. No.	Event Type*	Event Description
1		R(ACRO)	Raise reactor power to 85% using Recirc Flow Control Valves (IOI 03-1-01-2 Attachment II)
2		N(BOP)	Place the third Condensate and Condensate Booster Pump in service. (SOI 04-1-01-N19-1 section 4.3).
3	c51009a	C(ACRO)	Respond to APRM A failed upscale (ARI 04-1-02-P680-5A-A11; B10; Tech Spec/TR 3.3.1.1; TR 3.3.2.1)
4	r21134g	C(BOP)	Respond to trip of ESF Transformer 11. (ARI 04-1-02-1H13-P807 4A-B2; E6-, ONEP 05-1-02-I-4; Tech Spec 3.8.1) (SOER 10-1 Large Power Transformer Reliability)
5	fw163a @ 5 ramp to 100 over 14 minutes	C(ACRO)	Respond to a loss of Main Condenser Vacuum (ONEP 05-1-02-V-8)
6	rr063a@ 10 over 2 minutes	M(ALL)	Respond to a LOCA with reduced injection systems. (EP-2/3)
7	e51043	C(ACRO)	Respond to failure of RCIC to auto start (EP-2; SOI 04-1-01-E51-1)

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Critical Tasks

- *Reduce RPV Pressure to within capability of Condensate or Low Pressure ECCS to allow RPV level restoration.*
- *Restore and maintain RPV level above -191" using available injection systems.*