



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 8, 2010

Mr. Jon Franke, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing & Regulatory Programs
15760 W. Power Line Street
Crystal River, FL 34428-6708

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE
CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT, LICENSE
RENEWAL APPLICATION (TAC NO. ME0274)

Dear Mr. Franke:

By letter dated December 16, 2009, Florida Power Corporation submitted an application pursuant to Title 10 of the *Code of Federal Regulations* Part 54, to renew the operating license for Crystal River Unit 3 Nuclear Generating Plant (CR-3), for review by the U.S. Nuclear Regulatory Commission (NRC or the staff). The staff is reviewing the information contained in the license renewal application and has identified, in the enclosure, areas where additional information is needed to complete the review. Further requests for additional information may be issued in the future.

Items in the enclosure were discussed with Mr. Michael Heath, and a mutually agreeable date for the response is within 30 days from the date of this letter. If you have any questions, please contact me at 301-415-3733 or by e-mail at robert.kuntz@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to be "R. Kuntz", written over a large, stylized "R" that serves as a background or initial.

Robert F. Kuntz, Sr. Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosure:
As stated

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION
LICENSE RENEWAL APPLICATION FOR
CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT
DOCKET NO: 50-302

RAI 3.2.2.2.6-1

Background:

“Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants” (SRP-LR), Section 3.2.2.2.6, Loss of Material Due to Erosion, discusses stainless steel orifice plates in the High Pressure Safety Injection minimum flow lines. The referenced document in the GALL Report discusses the excessive flow rate due, in part, to erosion of the orifice plate, causing potential cavitation concerns for the centrifugal charging pumps. License renewal application (LRA) Section 3.2.2.2.6 states that CR-3 manages this aging effect with the “Inspection of Internal Surfaces in Miscellaneous Piping and Ducting Components Program.” According to the associated aging management program (AMP), in LRA Section B.2.23, this program will be implemented using existing “preventive maintenance, surveillance testing and periodic testing work order tasks that provide opportunities for the visual inspection of internal surfaces of piping and ducting components.”

Issue:

SRP-LR states that the acceptance criteria for this item are described in Branch Technical Position RSLB-1 in Appendix A.1. Section A.1.2.2.2, states, in part, that all activities that are credited for managing a certain aging effect for a specific component should be described. Based on the information in LRA Section 3.2.2.2.6 and Section B.2.23, it is unclear what specific activities are credited for managing the loss of material due to erosion for the orifice plates in question.

Request:

Describe the existing preventive maintenance, surveillance testing or periodic testing work order task(s) that provide the opportunity to perform a visual inspection, such that material loss due to erosion will be adequately managed for the HPI Make Up (charging) pump miniflow recirculation orifices.

ENCLOSURE

RAI B.2.21-4

Background:

The GALL Report, Section XI.M35 recommends the use of the One-Time Inspection of ASME Code Class 1 Small-Bore Piping only for those plants that have not experienced cracking of ASME Code Class 1 small-bore piping resulting from stress corrosion or thermal and mechanical loading. For those plants that have experienced cracking, the GALL Report recommends periodic inspection of the subject piping to be managed by a plant-specific AMP. The GALL Report also recommends the use of volumetric technique in the examination of ASME Code Class 1 small bore piping.

Issue:

By letter dated September 11, 2009, the staff issued RAI B.2.21-2 requesting information regarding the examination technique on Class 1 socket welds. In its response on October 13, 2009, and as supplemented on March 3, 2010, the applicant stated that it will perform periodic volumetric examinations of Class 1 socket welds. The applicant further stated that the examination "will begin at such a time as an acceptable nuclear industry methodology for nondestructive socket weld examination becomes available." However, the applicant did not clearly identify when "such a time" would be, nor did the applicant describe the details of "an acceptable nuclear industry methodology."

Request:

Provide detailed information on the committed inspection schedule and methodology of volumetric examinations on socket welds.

Letter to J. Franke from R. Kuntz dated July 8, 2010

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Robert F. Kuntz, Sr. Project Manager
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