

June 29, 2010

Mr. Thomas Joyce
President and Chief Nuclear Officer
PSEG Nuclear LLC
P.O. Box 236
Hancocks Bridge, NJ 08038

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR SALEM NUCLEAR
GENERATING STATION UNITS 1 AND 2 LICENSE RENEWAL APPLICATION
REGARDING SUBSECTION 3.1.2.2.14 (TAC NO ME1834 / ME1836)

Dear Mr. Joyce:

By letter dated August 18, 2009, as supplemented by letter dated January 23, 2009, Public Service Enterprise Group Nuclear, LLC, submitted an application pursuant to Title 10 of the *Code of Federal Regulations* Part 54 for renewal of Operating License Nos. DPR-70 and DPR-75 for Salem Nuclear Generating Station Units 1 and 2, respectively. The staff of the U.S. Nuclear Regulatory Commission (NRC or the staff) is reviewing this application in accordance with the guidance in NUREG-1800, "Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants." During its review, the staff has identified areas where additional information is needed to complete the review. The staff's request for additional information is included in the Enclosure. Further requests for additional information may be issued in the future.

Items in the enclosure were provided to John Hufnagel and other members of your staff, and a mutually agreeable date for the response is within 30 days from the date of this letter. If you have any questions, please contact me by telephone at 301-415-2981 or by e-mail at bennett.brady@nrc.gov.

Sincerely,

/RA/

Bennett M. Brady, Project Manager
Projects Branch 1
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-272 and 50-311

Enclosure:
As stated

cc w/encl: See next page

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ADAMS Accession No.: ML101680402

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DATE	6/29/10	6/21/10	06/28/10	06/29/10

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Letter to T. Joyce from B. Brady dated June 29, 2010

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GENERATING STATION UNITS 1 AND 2 LICENSE RENEWAL APPLICATION
REGARDING SUBSECTION 3.1.2.2.14 (TAC NO ME1834 / ME 1836)

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RConte, RI

MModes, RI

DTiff, RI

NMcNamara, RI

Salem Nuclear Generating Station,
Units 1 and 2

cc:

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Senior Vice President Nuclear
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Mr. Brian Booth
Director Nuclear Oversight
PSEG Nuclear
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Mr. Ali Fakhar
Manager, License Renewal
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Township Clerk
Lower Alloways Creek Township
Municipal Building, P.O. Box 157
Hancocks Bridge, NJ 08038

Mr. Paul Bauldauf, P.E., Asst. Director
Radiation Protection Programs
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Mr. Brian Beam
Board of Public Utilities
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Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
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Mr. Paul Davison
Vice President, Operations Support
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One Alloway Creek Neck Road
Hancocks Bridge, NJ 08038

Salem Nuclear Generating Station,
Units 1 and 2

- 2 -

cc:

Ms. Christine Neely
Director – Regulator Affairs
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One Alloway Creek Neck Road
Hancocks Bridge, NJ 08038

Senior Resident Inspector
Hope Creek Generating Station
U.S. Nuclear Regulatory Commission
Drawer 0509
Hancocks Bridge, NJ 08038

Mr. Earl R. Gage
Salem County Administrator
Administration Building
94 Market Street
Salem, NJ 08079

REQUEST FOR ADDITIONAL INFORMATION FOR SALEM NUCLEAR GENERATING STATION
UNITS 1 AND 2 LICENSE RENEWAL APPLICATION REGARDING SUBSECTION 3.1.2.2.14
(TAC NO ME1834 / ME 1836)

SNGS RAI 3.1.2.2.14-01

SNGS LRA Table 3.1.1, Item 3.1.1-32
SNGS LRA Subsection 3.1.2.2.14
SNGS LRA Subsection B.2.1.10, Steam Generator Tube Integrity
GALL AMP XI.M19, Steam Generator Tube Integrity
GALL AMP XI.M17, Flow-Accelerated Corrosion

Background:

SNGS LRA Table 3.1.1, item 3.1.1-32 and subsection 3.1.2.2.14 state that the Steam Generator Tube Integrity Program will be used to manage the aging effect of wall thinning in the steam generator steel feedwater inlet ring and supports. LRA Subsection B.2.1.10 states that the Steam Generator Tube Integrity Program will manage the aging effect of wall thinning, but it does not specify for which components, nor what techniques would be used.

Issue:

GALL AMP XI.M17, which is credited with managing wall thinning due to flow accelerated corrosion for many steel components, includes analysis, inspection, and verification to ensure that flow accelerated corrosion is not occurring at an unacceptable rate and that components are repaired or replaced before wall thinning becomes unacceptable. However, it is not clear to the staff whether the Steam Generator Tube Integrity Program uses both analysis and inspection to verify that unacceptable wall thinning is not occurring.

Request:

- 1) Describe the analytical methodology and inspection technique used to manage wall thinning in the steam generator feedwater inlet ring and supports.
- 2) Justify that the analytical methodology, together with verification by inspection, are adequate to ensure that loss of component intended function does not occur during the period of extended operation.
- 3) Alternatively, if inspection alone is credited to manage the aging effect, justify that the inspection and its associated acceptance criteria are adequate to ensure that need for corrective action is timely identified so that corrective actions are taken before loss of component intended function occurs.

ENCLOSURE