



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 199 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

1.0 INTRODUCTION

By letter dated October 9, 1998, Consolidated Edison Company of New York, Inc., (the licensee) submitted a request to change the Administrative Controls Section of the Technical Specifications (TSs) for Indian Point Nuclear Generating Unit No. 2 (IP2). The proposed changes would correct typographical errors in TS Sections 6.5.2.1.j, 6.7.1.c, and 6.8.1.a. In addition, TS Section 6.5.2.2 would be revised to allow changes the membership of the Nuclear Facility Safety Committee (NFSC). This change would provide Consolidated Edison (Con Ed) with the flexibility to obtain industry experts outside of Con Ed to perform as members of the NFSC.

2.0 EVALUATION

Section 13.4, "Operational Review", of NUREG-0800, the "Standard Review Plan" (SRP), provides the acceptance criteria used by the staff to evaluate TS provisions related to the plant staff review of operational activities performed by the NFSC. The SRP stipulates that this review function should meet the provisions described in Section 4.3 of ANSI N18.7 (ANS 3.2), and the qualification requirements for those performing these reviews should meet or exceed those described in Section 4.7 of ANSI/ANS-3.1-1978 and the regulatory position of Regulatory Guide (RG) 1.8.

These acceptance criteria are based on meeting the relevant requirements of 10 CFR 50, Section 50.40(b) as it relates to the licensee being technically qualified to engage in licensed activities, and of Appendix B to 10 CFR Part 50 as it relates to the review and audit functions required by the licensee's quality assurance program.

RG 1.8, Revision 2 (April 1987) conditionally endorsed both ANSI/ANS N18.1-1971, and ANSI/ANS-3.1-1981, "Selection and Training of Nuclear Power Plant Personnel," as industry standards which provide acceptable methods to the NRC staff for meeting the applicable regulatory requirements in 10 CFR 50, Section 50.34(b)(6)(i). Additionally, RG 1.33, Revision 2 (February 1978) conditionally endorsed ANSI N18.7 (ANS-3.2)-1976, "Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants," as an industry standard which provides acceptable methods to the NRC staff for meeting the applicable regulatory requirements in Appendix B to 10 CFR Part 50.

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2.1 NFSC Membership Changes

Currently, in accordance with Section 6.5.2.2 of the TS, the Chairman and Vice Chairman will be senior officials of the Company experienced in the field of nuclear energy. In addition, the Committee members are from departments having a direct interest in nuclear plant design, construction and operation or in nuclear safety and shall be designated by the Vice President of the Company, who is responsible for the functioning of the department subject to the approval of the Chairman. Committee members from other departments may be appointed by the Chairman with the concurrence of the Vice President of that department. The proposed revision reads as follows: "The Chairman or Vice Chairman will be a senior official of the Company. The Chairman and Vice Chairman will be experienced in the field of nuclear energy." and "Committee members from departments of Consolidated Edison shall be designated by the NFSC Chairman with concurrence of the appropriate department Vice President. Committee members from outside Consolidated Edison shall be designated by the NFSC Chairman with the concurrence of the Senior Vice President, Nuclear Operations."

The licensee indicated that the proposed change would provide flexibility in retaining industry experts, not Company officials, as either the Chairman or Vice Chairman, and as NFSC committee members.

The NRC staff has reviewed the proposed changes to Section 6.5.2.2 of the TS for the IP2 resulting from the new NFSC membership criteria and has concluded that prospective NFSC members will continue to collectively possess the range of technical competence, in accordance with Section 13.4 of the SRP and as defined in RG 1.8, Revision 2, necessary to assess the overall safety of operating phase activities at the facility.

Based on the considerations discussed above, the staff concludes that the revisions to Section 6.0 (Administrative Controls) of the TSs for IP2, as described in the October 9, 1998, letter by the Consolidated Edison Company of New York, Inc. are acceptable, as the TS continue to meet the guidance in Section 13.4, "Operational Review," of the Standard Review Plan (NUREG-0800).

2.2 Correction of Typographical Errors

TS 6.5.2.1.j currently has the phrase "nuclear power plant" printed at the top of page 6-7. However this phrase is a repeat from the bottom of page 6-6. This error was introduced during document word processing.

TS 6.7.1.c currently has the phrase "systems or structures, and (3) corrective action taken to prevent recurrence." printed at the top of page 6-12. However this phrase is a repeat from the bottom of page 6-11. This error was introduced during document word processing.

TS 6.8.1.a provides the requirements and recommendation of Sections 5.1 and 5.3 of ANSI N18.7-1976 and Appendix A of RG 1.33, Revision 2 except as provided in 6.8.2 and 6.8.3. Section 5.1 of ANSI N18.7-1976 provides a description of the program for providing the administrative controls. Section 5.2 provides the rules of practice for the organization. The

licensee is proposing to change the reference from 5.1 to 5.2. This change is necessary to accurately reflect the specific section of the referenced ANSI Standard.

These changes are administrative in nature and correct typographical errors, therefore, the above proposed changes are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration (63 FR 69337), and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Harold

Date: March 8, 1999