



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 175  
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated April 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 175, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael J. Case, Acting Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 23, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 175

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

Table 4.1-1 (page 4 of 7)

Table 4.1-1 (page 5 of 7)

Insert Pages

Table 4.1-1 (page 4 of 7)

Table 4.1-1 (page 5 of 7)

Table 4.1-1

Minimum Frequencies for Checks, Calibrations and  
Tests of Instrument Channels

Channel Description	Check	Calibrate	Test	Remarks
27. Turbine Trip				
a. Low Auto Stop Oil Pressure	N.A.	R	N.A.	
28. Control Rod Protection (for use with LOPAR fuel)	N.A.	R	*	
29. Loss of Power				
a. 480v Emergency Bus Undervoltage (Loss of Voltage)	N.A.	R	R	
b. 480v Emergency Bus Undervoltage (Degraded Voltage)	N.A.	R	R	
c. 480v Emergency Bus Undervoltage (Alarm)	N.A.	R	M	
30. Auxiliary Feedwater				
a. Steam Generator Water Level (Low-Low)	S	R#	R#	
b. Low-Low Level AFWS Automatic Actuation Logic	N.A.	N.A.	M	Test one logic channel per month on an alternating basis.
c. Station Blackout (Undervoltage)	N.A.	R	R	

\* Within 31 days prior to entering a condition in which the Control Rod Protection System is required to be operable unless the reactor trip breakers are manually opened during RCS cooldown prior to T<sub>cold</sub> decreasing below 350°F and the breakers are maintained opened during RCS cooldown when T<sub>cold</sub> is less than 350°F.

Table 4.1-1

Minimum Frequencies for Checks, Calibrations and  
Tests of Instrument Channels

Channel Description	Check	Calibrate	Test	Remarks
d. Trip of Main Feedwater Pumps	N.A.	N.A.	R#	
31. Reactor Coolant System Subcooling Margin Monitor	M	R	N.A.	
32. PORV Position Indicator (Limit Switch)	M	R#	R#	
33. PORV Block Valve Position Indicator (Limit Switch)	M*	R#	R#	
34. Safety Valve Position Indicator (Acoustic Monitor)	M	R#	R#	
35. Auxiliary Feedwater Flow Rate	M	R	R	
36. PORV Actuation/ Reclosure Setpoints	N.A.	R#	N.A.	
37. Overpressure Protection System (OPS)	N.A.	R#	**	

\* Except when block valve operator is deenergized.

\*\* Within 31 days prior to entering a condition in which OPS is required to be operable and at monthly intervals thereafter when OPS is required to be operable.