

September 9, 1999

Mr. M. Wadley
President, Nuclear Generation
Northern States Power Company
414 Nicollet Mall
Minneapolis, MN 55401

Dear Mr. Wadley:

SUBJECT: NRC PRAIRIE ISLAND RADIATION SAFETY INSPECTION REPORT
50-282/99011(DRS); 50-306/99011(DRS)

On August 20, 1999, the NRC completed an inspection at your Prairie Island Nuclear Generating Plant, Units 1 and 2. The results of this inspection were discussed on August 20, 1999, with Mr. T. Amundson and other members of your staff. The enclosed report presents the results of that inspection.

The inspection was an examination of activities under your license as they relate to radiation protection and to compliance with the Commission's rules and regulations and with the conditions of your license. Within those areas, the inspection consisted of a selective examination of procedures and representative records, interviews with personnel, and observation of activities in progress. Specifically, this inspection focused on the implementation of your radioactive waste processing and shipping programs and the radiological environmental monitoring program.

During this inspection, the NRC identified one issue of low safety significance that has been entered into your corrective action program and is discussed in the summary of findings and in the body of the attached inspection report. This issue was determined to involve a violation of NRC requirements, but because of its low safety significance the violation is not cited. If you contest this non-cited violation, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington DC 20555-0001; with copies to the Regional Administrator, Region III; the Director, Office of Enforcement, United States Nuclear Regulatory Commission, Washington DC 20555-0001; and the NRC Resident Inspector at the Prairie Island Nuclear Generating Plant.

In addition to the above, the inspection included a review and evaluation of current occupational and public radiation safety Performance Indicators for the first and second quarters of 1999. All Performance Indicators were green in these cornerstones. The records that we reviewed supported the data which had been reported by your staff.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response to this letter, if you should choose to respond, will be placed in the NRC Public Document Room.

M. Wadley

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We will gladly discuss any questions you have concerning this inspection.

Sincerely,

/s/ W. J. Slawinski (for)

Gary L. Shear, Chief
Plant Support Branch

Docket Nos. 50-282; 50-306
License Nos. DPR-42; DPR-60

Enclosure: Inspection Report 50-282/99011(DRS); 50-306/99011(DRS)

cc w/encl: Site General Manager, Prairie Island
Plant Manager, Prairie Island
S. Minn, Commissioner, Minnesota
Department of Public Service
State Liaison Officer, State of Wisconsin
Tribal Council, Prairie Island Dakota Community

M. Wadley

-2-

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos: 50-282; 50-306
License Nos: DPR-42; DPR-60

Report No: 50-282/99011(DRS); 50-306/99011(DRS)

Licensee: Northern States Power Company

Facility: Prairie Island Nuclear Generating Plant
Units 1 & 2

Location: 1717 Wakonade Dr. East
Welch, MN 55089

Dates: August 16 - 20, 1999

Inspectors: John E. House, Senior Radiation Specialist
Steven K. Orth, Senior Radiation Specialist
Mark W. Mitchell, Radiation Specialist

Approved by: Gary L. Shear, Chief, Plant Support Branch
Division of Reactor Safety

SUMMARY OF FINDINGS

Prairie Island Nuclear Generating Plant, Units 1 & 2
NRC Inspection Report 50-282/99011(DRS); 50-306/99011(DRS)

This report covers a one-week period of announced inspection by three regional radiation specialists. This inspection focused on public radiation safety and included a review of the Performance Indicators for occupational and public radiation safety.

Inspection findings were assessed according to potential risk significance and were assigned colors of green, white, yellow or red. Green findings are indicative of issues that, while not necessarily desirable, represent little risk to safety. White findings would indicate issues with some increased risk to safety, and which may require additional NRC inspections. Yellow findings would be indicative of more serious issues with higher potential risk to safe performance and would require the NRC to take additional actions. Red findings represent an unacceptable loss of margin to safety and would result in the NRC taking significant actions that could include ordering the plant shut down. The findings, considered in total with other inspection findings and performance indicators, will be used to determine overall plant performance.

RADIATION SAFETY

Cornerstone: Public Radiation Safety

- Green. The inspectors identified a non-cited violation (NCV) for the failure to include an emergency response telephone number on shipping papers for radioactive material and waste shipments which satisfied the requirements contained in 49 CFR 172.604. The telephone number entered on the shipping papers was that of an electronic paging system, which did not provide the caller with direct contact with a person knowledgeable of the shipment or with a person who had access to an individual having that knowledge. In addition, the system did not provide any instructions to the caller on how to gain a response. Potentially, this failure could have resulted in delays obtaining emergency response information. (Section 2PS2.3)

Performance Indicators Verification

- Occupational and Public Radiation Safety Performance Indicators (PIs). The inspectors verified that the licensee had properly reported the PIs for these cornerstones, which were in the green band for the first two quarters of 1999. (Section 4OA2)

Report Details

2. RADIATION SAFETY

Cornerstone: Public Radiation Safety

2PS2 Radioactive Material Shipping

.1 Walkdown of Radioactive Waste Systems

a. Inspection Scope

The inspectors reviewed the radioactive waste systems to assess the material condition and operability of the systems. The inspectors also compared the operations of the systems to the descriptions in the Updated Safety Analysis Report (USAR) and the process control program (PCP). During this inspection, the licensee was not conducting waste processing.

b. Observations and Findings

The inspectors observed that major components of the liquid waste volume reduction and solidification equipment were not routinely used. For example, the cement solidification system and the evaporators had not been in service for a number of years, and the applicable system valves were closed to isolate these components from the remaining plant systems. However, this equipment was described as viable waste processing equipment in the USAR and the PCP. Although the licensee indicated that the equipment could be returned to operable service, the inspectors' observation indicated a difference between the USAR and PCP and the plant's normal/routine operations.

Prior to this inspection, the licensee recognized that some of these inconsistencies existed and had begun a project of broad scope to validate the plant's design basis, in accordance with NRC discretionary policies. As part of this review, the licensee evaluated and prioritized the areas of the USAR based on risk significance. Since the radioactive waste systems were of lower risk significance, the licensee planned to review these sections of the USAR in the future (calendar year 2000).

.2 Waste Characterization and Classification

a. Inspection Scope

The inspectors reviewed the licensee's method and procedures for determining the classification of radioactive waste shipments, including the licensee's use of scaling factors to quantify difficult-to-measure radionuclides (e.g., pure alpha or beta emitting radionuclides). The inspectors also reviewed records of radioactive waste shipments to verify that the shipments were properly classified and characterized in accordance with the requirements contained in 10 CFR Part 61.

b. Observations and Findings

There were no findings identified and documented during this inspection.

.3 Shipping Records

a. Inspection Scope

The inspectors reviewed a selection of non-accepted package shipments completed in 1998 and 1999 to verify compliance with NRC and Department of Transportation (DOT) requirements (i.e., 10 CFR Parts 20 and 71 and 49 CFR Parts 172 and 173). During the course of the inspection, the licensee did not perform any shipping activities.

b. Observations and Findings

The inspectors identified a violation of DOT requirements concerning the emergency response telephone number contained on the shipping papers.

On each of the shipment records reviewed, the inspectors identified that the emergency response telephone number, which was required by 49 CFR 172.604, was that of an electronic paging system. As such, an individual using the telephone number would not have been placed in direct contact with a person who was knowledgeable of the hazardous material being shipped or a person who had immediate access to an individual having that knowledge. In addition, the system did not provide instructions to the caller on how to leave a pager message or how to gain a response. Consequently, in the event of a transportation accident, first response information could potentially be delayed. The licensee concurrently identified this as a potential program weakness and had entered the matter into its corrective action program (Condition Report (CR) No. 19992389). Following the onsite portion of this inspection, the licensee notified the inspectors that it had contracted with a commercial service to provide a 24-hour, manned response telephone number, which would be fully consistent with DOT requirements.

Part 172.604 of Title 49 of the Code of Federal Regulations (CFR) states that each licensee who transports licensed material outside the site of usage, as specified in the NRC license, or where transport is on public highways, or who delivers licensed material to a carrier for transport, shall comply with the applicable requirements of the DOT regulations in 49 CFR Parts 170 through 189 appropriate to the mode of transportation. Part 172.604 of Title 49 of the CFR requires, in part, that a person who offers a hazardous material for transportation provide a 24-hour emergency response telephone number for use in the event of an emergency involving hazardous material. The telephone number must be: (1) monitored at all times the hazardous material is in transportation, including storage incidental to transportation, (2) the number of a person who is either knowledgeable of the hazardous material being shipped and has comprehensive emergency response and incident mitigation information for that material, or has immediate access to a person who possesses such knowledge, and (3) entered on the shipping paper. As described above, the licensee failed to provide an emergency response telephone number, which was the number of a person who was either knowledgeable of the hazard or who had immediate access to an individual who

possesses such knowledge. Therefore, the failure is a violation of 10 CFR 71.5. This Severity Level IV violation is being treated as a Non-Cited Violation, consistent with Appendix C of the NRC Enforcement Policy. This violation is in the licensee's corrective action program as CR No. 19992389. (50-282/99011-01 and 50-306/99011-01)

The inspectors determined that this finding was green using the significance determination process (SDP) for the public radiation safety cornerstone because the actual risk significance was low. Although the emergency response telephone contact did not fully meet the requirements of 49 CFR 172.604, the licensee provided appropriate emergency response information on the shipping papers in accordance with 49 CFR 172.602 that could have been used by emergency responders. Also, the inspectors did not identify any occasions in which the licensee failed to respond to an actual request for emergency response information. Consequently, the finding was determined to be green.

.4 Unrestricted Release of Materials

a. Inspection Scope

The inspectors reviewed the licensee's program for unconditionally releasing material from the radiologically controlled area (RCA), which included the selection of survey instrumentation and the sensitivity of that instrumentation. The inspectors also observed radiation protection (RP) technicians performing surveys of potentially contaminated hoses and other materials for unconditional release, reviewed records of bulk materials released from the RCA, and reviewed confirmatory surveys of materials outside of the RCA.

b. Observations and Findings

The inspectors found that the licensee was not fully implementing NRC guidance concerning the sensitivity (i.e., lower limit of detection (LLD)) of surveys for bulk material released from the RCA.

The RP staff analyzed representative samples of potentially contaminated bulk materials (i.e., both liquids and granular solids) via gamma spectroscopy and reviewed the gamma isotopic results in accordance with radiation protection implementing procedure (RPIP) 1302 (Revision 12), "Unconditional Release of Materials." Procedure RPIP 1302 stated that samples of bulk materials were to be analyzed at an LLD consistent with the LLDs used in the licensee's environmental monitoring program. However, the procedure only specified the use of the LLD for dry sediment (i.e., nominal LLD of 150 picocuries per liter (pCi/l) for manganese-54, cobalt-58, cobalt-60, and cesium-134 and 180 pCi/l for cesium-137). This sediment LLD is a factor of ten higher than the LLD used by the licensee to analyze liquid environmental samples. The NRC's health physics position No. 221 contained in NUREG/CR-5569, ORNL/TM-12067, "Health Physics Positions Data Base," Revision 1, describes the use of environmental LLDs for unconditional release of bulk materials and specifies that the environmental LLDs for water should be used for the survey of liquids (e.g., oils). Since the licensee was not surveying potentially contaminated oils to the liquid LLDs, the potential existed for the inadvertent release of small amounts of licensed materials. Although no violations of

NRC requirements were identified, the licensee planned to enter this observation into its corrective action system and to review the NRC guidance described above and its procedures.

The inspectors reviewed the licensee's survey results and did not identify any evidence of the inappropriate release of radioactive materials from the RCA.

2PS3 Radiological Environmental Monitoring

.1 Review of Annual Report and Environmental Impact

a. Inspection Scope

The inspectors reviewed the 1998 Annual Radiological Environmental Operating Report and the implementation of the radiological environmental monitoring program (REMP) to verify compliance with the Offsite Dose Calculation Manual (ODCM). In addition, the inspectors reviewed the adequacy of offsite dose calculations.

b. Observations and Findings

There were no findings identified and documented during this inspection.

.2 Sample Location Walkdowns and Instrument Maintenance

a. Inspection Scope

The inspectors observed the location and operability of selected environmental monitors and the meteorological instrumentation, including the routine maintenance and calibration. The inspectors also observed the contract environmental monitoring technician collecting environmental samples. Specifically, the inspectors verified that the following sampling and thermoluminescence dosimeter (TLD) locations were as described in the ODCM:

<u>Sample Type</u>	<u>Location Number</u>
• air particulate/iodine cartridge	P-2, P-3, P-4, P-6
• surface water	P-6
• TLD	P-08A, P-09A, P-10A, P-01A, P-01S, P-02S

b. Observations and Findings

Air sampling and meteorological instrumentation maintenance and calibrations were performed at the required frequencies and in accordance with the vendors' procedures.

However, the inspectors observed that the meteorology engineering staff had identified a wind speed indicator problem in the primary, secondary, and backup systems. In 12 of the last 24 months the licensee performed the required calibrations of the wind speed sensors and found the high end wind speed as-found parameter to be out of specification. The staff performed corrective actions to remedy the problem. However,

due to the sporadic nature of the equipment performance problem and the interval between the findings, the staff was unable to fully correct the problem in a more timely manner. The staff tracked this issue as a corrective work order, which was a part of the licensee's corrective action program. The inspectors concluded that the wind speed performance problem did not affect the licensee's ability to use the meteorology data for operational or emergency situations.

.3 Offsite Dose Calculation Manual

a. Inspection Scope

The inspectors reviewed the licensee's changes to the ODCM that were implemented since the last NRC inspection of this area.

b. Observations and Findings

There were no findings identified and documented during this inspection.

.4 Quality Control of Environmental Radiochemical Measurements

a. Inspection Scope

The inspectors reviewed portions of the licensee's vendor laboratory's quality assurance program which verify the accuracy of radiochemical analyses of environmental samples. Specifically, the inspectors reviewed the interlaboratory comparison program results and audits of the vendor's laboratory and results.

b. Observations and Findings

There were no findings identified and documented during this inspection.

4 OTHER ACTIVITIES

4OA1 Identification and Resolution of Problems

a. Inspection Scope

The inspectors reviewed the licensee's self assessments, audits, and CRs concerning the radioactive waste management and shipping programs and the REMP. In addition, the inspectors interviewed members of the RP staff to evaluate their knowledge and use of the licensee's corrective action system.

b. Observations and Findings

The quality assurance staff was effectively reviewing the radioactive waste management and shipping programs and the REMP. The licensee entered applicable audit findings into its corrective action system, which ensured that the findings were evaluated and corrected.

The RP staff did not have a formal, internal self assessment program. Although the staff indicated that program assessments and improvements were performed on an ongoing basis, the staff did not routinely document assessments and improvements. Consequently, the inspectors were unable to evaluate the level of the RP staff's reviews. For example, in the REMP area, the REMP coordinator performed and documented field observations, which critiqued the sample collectors' performance. However, the licensee's documentation for other assessment activities was limited. Licensee management indicated that site-wide enhancements were being implemented to address this issue.

During interviews with the RP staff, personnel demonstrated a basic knowledge of the licensee's corrective action system. However, personnel indicated some reluctance to initiate corrective action program documents (i.e., CRs). Consistently, the staff stated that identified problems were corrected; however, the staff also stated that they may not always enter these problems into the CR system. Licensee management indicated that site-wide enhancements were being implemented to improve the CR system.

4OA2 Performance Indicator Verification

a. Inspection Scope

The inspectors verified the licensee's assessment of its performance indicators (PIs) for the occupational and public radiation safety cornerstones that the licensee reported in the first and second quarters of 1999. Specifically, the inspectors reviewed quarterly offsite dose assessments, dosimetry reports, and radiological occurrence reports for the applicable periods of time.

b. Observations and Findings

The licensee reported green PIs in both of the radiation safety cornerstones for the first two quarters of 1999. Based on the licensee's evaluation, one occurrence in the occupational safety PI was identified by the licensee in the fourth quarter of 1998. The inspectors independently reviewed the licensee's records and found no problems with the accuracy or completeness of the licensee's PI data.

4OA4 Other

- .1 (Closed) Inspection Follow-up Item Nos. 50-282/98019-01 and 50-306/98019-01: The licensee planned to change the LLDs contained in the ODCM for drinking water and ground water samples to be consistent with NRC guidance for drinking water pathways. In addition, the licensee planned to evaluate the periodic discharge of water drained from the condenser as a potential effluent release pathway (i.e., for inclusion into the ODCM). The inspectors screened these issues using the SDP and determined that the issues were of low risk significance (i.e., green). At the time of this inspection, the issues were tracked in the REMP action item tracking system, and the licensee was implementing an ODCM change which would include the above issues. Consequently, this item is closed.

4OA5 Management Meetings

.1 Exit Meeting Summary

The inspectors presented the inspection results to Mr. T. Amundson and other members of licensee management and staff at the conclusion of the inspection on August 20, 1999. The licensee acknowledged the findings presented and did not identify any information discussed as proprietary.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

T. Amundson, General Superintendent, Engineering, Acting Plant Manager
T. Beard, Corporate Health Physicist
J. Friedrich, Radiation Protection Engineering Supervisor
J. Hill, Quality Manager
J. Hopkins, Radioactive Waste Liquid Engineer
A. Johnson, General Superintendent, Radiation Protection and Chemistry
S. Lappegaard, Superintendent, Chemistry and Environmental
D. Larimer, Radiochemistry Supervisor
M. Loosbrock, Radiation Protection/Chemistry Engineer
D. Molback, Meteorology System Engineer

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

50-282/99011-01	NCV	Failure to provide an adequate emergency response telephone number on shipping papers for radioactive shipments (Section 2PS2.3).
50-306/99011-01		

Closed

50-282/98019-01	IFI	Planned changes to the ODCM to be consistent with NRC guidance regarding drinking water LLDs and effluent pathways (Section 4OA4.1).
50-306/98019-01		
50-282/99011-01	NCV	Failure to provide an adequate emergency response telephone number on shipping papers for radioactive shipments (Section 2PS2.3).
50-306/99011-01		

Discussed

None.

LIST OF ACRONYMS USED

CFR	Code of Federal Regulations
CR	Condition Report
DOT	Department of Transportation
DRS	Division of Reactor Safety
LLD	Lower Limit of Detection
NCV	Non-Cited Violation
NRC	Nuclear Regulatory Commission
ODCM	Offsite Dose Calculation Manual
PCP	Process Control Program
PDR	Public Document Room
PI	Performance Indicator
RCA	Radiologically Controlled Area
REMP	Radiological Environmental Monitoring Program
RPIP	Radiation Protection Implementing Procedures
RP	Radiation Protection
SDP	Significance Determination Process
TLD	Thermoluminescence Dosimeter
USAR	Updated Safety Analysis Report

LIST OF DOCUMENTS REVIEWED

Assessments and Audits

Internal Audit Report No. AG 1999-S-2, dated August 2, 1999.
"NUPIC Joint Audit of Teledyne-Brown Engineering Services, Northbrook, Illinois," completed April 29, 1999.
Observation Report No. 1998142, "Effluent Samples/Annual REMP Report," dated June 8, 1998.
Observation Report No. 1998150, "PINGP REMP Training," dated June 17, 1998.
Observation Report No. 1998154, "REMP Field Sampling/Rad Waste Reduction," dated June 19, 1998.
Observation Report No. 1998159, "Followup to OR 1996252 Concerning REMP Report Wording Not Matching Technical Specification Wording," dated June 24, 1998.
Observation Report No. 1999087, "Radwaste Material Release Controls," dated May 11, 1999.
Observation Report No. 1999089, "PINGP REMP," dated May 13, 1999.
Observation Report No. 1999093, "PI - Radioactive Waste and Radwaste Reduction Program," dated May 14, 1999.
Observation Report No. 1999094, "PI - REMP/Radioactive Waste Self Assessment," dated May 14, 1999.
Observation Report No. 1999096, "PINGP REMP Annual Report," dated May 4, 1999.
Observation Report No. 1999098, "PI REMP/Process Control Program Changes," dated May 13, 1999.
PINGP 1260 (Revision 1), "Chemistry/Rad Protection Observation Sheet," performed By M. Agen, dated December 22, 1998.
Prairie Island Field Observation Cards, performed by M. Agen, dated April 28, 1999, and May 11, 1999.

Instrument Calibrations

Certificate of Calibration of Rotameter Model F & P 10A3565B, completed on October 27, 1998.
PINGP 999 (Revision 3), "Calibration Data Sheet for NNC Tool Monitor," performed on April 22, 1999.

REMP Calibration/Maintenance Form 1117, Revision 4:

Air Sampler	S/N 5065	Site P-1	Completed 3/3/99
Air Sampler	S/N 5798	Site P-2	Completed 3/3/99
Air Sampler	S/N 5625	Site P-3	Completed 3/3/99
Air Sampler	S/N 5797	Site P-4	Completed 3/3/99
Air Sampler	S/N 5798	Site P-6	Completed 3/3/99
Air Sampler	S/N 4019	Site P-1	Completed 6/8/99
Air Sampler	S/N 5798	Site P-2	Completed 6/8/99
Air Sampler	S/N 5068	Site P-3	Completed 6/8/99
Air Sampler	S/N 5798	Site P-4	Completed 6/8/99
Air Sampler	S/N 5064	Site P-6	Completed 6/8/99

TP 1677 (Revision 12), "Meteorological Instrumentation Monthly Test," performed during 1997 through 1999.

TP 1676 (Revision 7), "Meteorological Instruments Calibration," performed during 1997 through 1999.

Miscellaneous

Gamma Spectrum Analyses Nos. 99-10047 (January 13, 1999), 99-10128 (February 5, 1999), 99-10169 (February 15, 1999), 99-10936 (May 11, 1999), 99-11161 (June 3, 1999), 99-11190 (June 7, 1999), 99-11199 (June 8, 1999), and 99-11200 (June 8, 1999).

Met/Vent Raw Data Review for August 16, 1999.

Prairie Island Updated Safety Analysis Report Appendix H (Rev 4).

Prairie Island Nuclear Generating Station 1998 Annual Radiological Environmental Monitoring Report, completed May 3, 1999.

Radiation Protection Survey Records: "S/G Mock-up," (April 14, 1999); "New Admin Building Including Lunch Rooms. All Elevations," (April 16, 1999); "Scrap Yard," (April 25, 1999); "Construction Fab Shop," (May 8, 1999); "Const. Elec. Shop," (May 15, 1999); "695' U-1 Aux Bldg," (May 17, 1999); "Wylie Trailer," (May 25, 1999); "Access Control," (July 26, 1999); "715' Access Control," (August 3, 1999); "I & C Shop Clean Side Areas," (August 9, 1999); "Access Control," (August 13, 1999); and "Turbine Bldg -- 2 Rugs Checked in Friskall," (August 17, 1999).

REMP Action Item List, July 27, 1999.

1998 Critical Receptor Identification Report, completed November 6, 1998.

1998 Land Use Census, completed May 27, 1999.

Condition Report Forms

19981046, "P.O. # PL7389 RIF# 42131 Radlok 179 Containers Rec'd May 11, 1998 Were Missing Seals and Manway Covers Were Not Torqued,"

19991395, "Radiation Survey Instrument CM-7A had Expired Cal Sticker,"

19991596, "RAMSHP Software Contains an Error for Waste Classification,"

19992310, "REMP Mislabeling of Fish Samples, May 27, 1999," and

19992369, "REMP Samples Destroyed in Shipment, August 3, 1999."

Procedures

RPIP 1302 (Revision 12), "Unconditional Release of Materials,"

RPIP 1307 (Revision 4), "Rad Waste Classification,"

RPIP 1310 (Revision 4), "Rad Waste Streams/ Scaling Factors,"

RPIP 1322 (Revision 1), "RADMAN For Windows™ to Generate Scaling Factors,"

RPIP 1323 (Revision 1), "RADMAN for Windows to Classify and Characterize Rad Waste,"

RPIP 1508 (Revision 5), "Integral Tool Monitor Description, Operation and Calibration,"

RPIP 4730 (Revision 2), "REMP Sampling Procedure,"
RPIP 4731 (Revision 5), "REMP Air Sampling Procedure,"
RPIP 4732 (Revision 5), "REMP Water Sampling Procedure,"
Radiation Protection Procedure (RPP) D11.4 (Revision 20), "Radioactive Materials Shipment
Greater Than Typ A Quantities in Exclusive Use Vehicle to Barnwell, SC Using ATG Cask and
HIC Liner,"
RPP D11.7 (Revision 11), "Radioactive Materials Shipment LSA/SCO/LTD QTY Not Exceeding
Type A in Exclusive Use Vehicle -To a Licensed Facility,"
RPP D11.11 (Revision 6), "Radioactive Materials Shipment LSA/SCO/LTD QTY Not Exceeding
Type A in Exclusive Use Vehicle -To a Licensed Processing Facility,"
RPP D59 (Revision 7), "Process Control Program for Solidification/Dewatering of Radioactive
Waste from Liquid Systems,"
TP 1677 (Revision 12), "Meteorological Instrumentation Monthly Test," and
TP 1676 (Revision 7), "Meteorological Instruments Calibration."