

June 17, 2010

LICENSEE: Pacific Gas and Electric Company
FACILITY: Diablo Canyon Nuclear Power Plant, Units 1 and 2
SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON JUNE 3, 2010,
BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND PACIFIC
GAS AND ELECTRIC COMPANY CONCERNING DRAFT REQUEST FOR
ADDITIONAL INFORMATION RELATED TO THE DIABLO CANYON NUCLEAR
POWER PLANT, UNITS 1 AND 2, LICENSE RENEWAL APPLICATION—
AGING MANAGEMENT PROGRAMS

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Pacific Gas and Electric Company (PG&E) held a telephone conference call on June 3, 2010, to obtain clarification on the staff's draft request for additional information (D-RAI) regarding the Diablo Canyon Nuclear Power Plant license renewal application (LRA).

By emails dated May 17, 2010 and May 20, 2010, the staff sent D-RAIs to PG&E regarding aging management programs. PG&E reviewed the information contained therein, and requested a telephone conference call. The telephone conference call was useful in clarifying the intent of the staff's D-RAIs. Enclosure 2 provides discussions on D-RAIs for which the applicant requested clarification. No changes to other D-RAIs were necessary as a result of this telephone conference call. Formal RAIs will be issued by a separate letter.

Enclosure 1 provides a listing of the participants.

The applicant had an opportunity to comment on this summary.

/RA/

Nathaniel Ferrer, Safety Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket Nos. 50-275 and 50-323

Enclosures:
As stated

cc w/encl: See next page

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Memorandum to Pacific Gas and Electric Company from N. Ferrer dated June 17, 2010

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON JUNE 3, 2010, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND PACIFIC GAS AND ELECTRIC COMPANY CONCERNING DRAFT REQUEST FOR ADDITIONAL INFORMATION RELATED TO THE DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2, LICENSE RENEWAL APPLICATION—AGING MANAGEMENT PROGRAMS

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**TELEPHONE CONFERENCE CALL
DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION**

**LIST OF PARTICIPANTS
June 3, 2010**

PARTICIPANTS

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Kim Green	U.S. Nuclear Regulatory Commission (NRC)
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Diablo Canyon Nuclear Power Plant (DCPP), Units 1 and 2
License Renewal Application (LRA)
Draft Request for Additional Information
Aging Management Programs

D-RAI B2.1.1-1

Generic Aging Lessons Learned (GALL) Report AMP XI.M1 “scope of program” element contains a broad class of components for inservice inspection (ISI) with respective standards for flaw acceptance and flaw evaluation. Also, the “detection of aging effects” program element covers the inspections of Class 1 small-bore piping and socket welds.

In its description of the ISI program under LRA Section B2.1.1, the applicant stated that DCPP evaluates every indication. However, the acceptance standards IWD-3400 and IWD-3500 and the flaw evaluation standard IWD-3600, for Class 3 components, are not included in the “program description” of LRA AMP B2.1.1. Also, Class 1 small-bore piping and socket welds for the aging management program are covered under different AMPs for DCPP.

Explain how the “program description” includes the use of acceptance and evaluation standards for Class 3 components. Also, indicate under the “detection of aging effects” element in which AMPs the inspection of Class 1 small-bore piping and socket welds are covered or supplemented.

Discussion: Based on the discussion with the applicant, the staff agreed to revise this question as follows. The revised question will be sent as a formal RAI.

RAI B2.1.1-1

Generic Aging Lessons Learned (GALL) Report AMP XI.M1 “scope of program” element contains a broad class of components for inservice inspection (ISI) with respective standards for flaw acceptance and flaw evaluation. Also, the “detection of aging effects” program element covers the inspections of Class 1 small-bore piping and socket welds.

In its description of the ISI program under LRA Section B2.1.1, the applicant stated that DCPP evaluates every indication. However, the acceptance standards IWD-3400 and IWD-3500 and the flaw evaluation standard IWD-3600, for Class 3 components, are not included in the “program description” of LRA AMP B2.1.1. Also, Class 1 small-bore piping and socket welds for the aging management program are covered under different AMPs for DCPP.

Explain how the “program description” includes the use of acceptance and evaluation standards for Class 3 components. Also, indicate in which AMP the inspection of Class 1 small-bore piping and socket welds are covered or supplemented, including a justification for using this program.

D-RAI B2.1.16-3

LRA Table 3.3.1, Item 3.3.1.35 refers to the aging evaluation of pump casings in auxiliary systems fabricated of steel with stainless steel cladding and exposed to treated borated water. The aging effect identified is loss of material due to cladding breach, and the GALL Report recommends the use of a plant-specific aging management program. The applicant proposes to manage this aging effect using its Water Chemistry Program, with its One-Time Inspection Program to be used for verification of program effectiveness.

The staff notes that treated borated water is corrosive with respect to the ferritic steel underlying the stainless steel cladding in these pump casings. This means that even if proper water chemistry is maintained under the applicant's Water Chemistry Program, the ferritic steel substrate will undergo corrosive attack in the event of a cladding breach. The applicant must therefore rely on its proposed One-Time Inspection Program to detect this aging effect in these components. Since loss of material in the underlying ferritic steel is not normally surface-intersecting, a volumetric inspection technique such as ultrasonic testing would normally be used for detection purposes. However, the complex microstructure and geometry of this weld-clad component makes ultrasonic inspection difficult and potentially unreliable.

Describe the details of the inspection technique to be used to perform the one-time inspection of these components and provide relevant plant or industry experience to demonstrate the effectiveness and reliability of this technique.

Discussion: The applicant referred the staff to Section 3.3.2.2.14 and Table A4-1 of the LRA. The staff determined that sufficient information is included in the LRA to address the staff's concern. Therefore, this question is withdrawn and will not be sent as a formal RAI.

Diablo Canyon Nuclear Power Plant
Units 1 and 2

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