UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated December 28, 1988, as clarified May 10, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 142 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective no later than 30 days after its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 19, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 142

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages	Insert Pages
4.16-1	4.16-1
4.16-2	4.16-2

4.16 REACTOR COOLANT SYSTEM AND CONTAINMENT FREE VOLUME LEAKAGE DETECTION AND REMOVAL SYSTEMS SURVEILLANCE

Applicability

Applies to the surveillance and monitoring of leakage detection and removal systems provided for determining and removing reactor coolant leakage and leakage into the containment free volume. Applies to the testing of certain LPI/RHR check valves(1,2).

Objective

To verify compliance with operational leakage limits of Specification 3.1.F. To specify a test to check for RCS leakage through certain check valves.

Specifications

A. For the purposes of demonstrating compliance with the operational leakage limits of Specification 3.1.F., the following shall be performed:

1. At least once a shift, monitor the leakage detection systems required by Specification 3.1.F.1.a(6).

2. At least once a shift, monitor the containment sump inventory and discharge.

3. At least once a shift, monitor the recirculation sump inventory and the reactor cavity inventory.

4. At least once daily, perform a reactor coolant system water inventory balance.

5. For the RCS/RHR pressure isolation valves, periodic leakage testing* shall be accomplished every time the plant is placed

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* To satisfy ALARA requirements, leakage may be measured indirectly (as from the performance of pressure indicators) if accomplished in accordance with approved procedures and supported by computations showing that the method is capable of demonstrating valve compliance with the leakage criteria. Minimum test differential pressure shall not be less than 150 psid.

in the cold shutdown condition for refueling, each time the plant is placed in a cold shutdown condition for at least 72 consecutive hours if testing has not been accomplished in the preceding 9 months, and prior to returning the valve to service after maintenance, repair or replacement work is performed.

B. A test shall be performed, whenever the RCS pressure decreases to 700 psig (i.e. within 100 psig of the RHR design pressure) or whenever the RHR is secured to go to hot shutdown, to check for leakage through SIS low head injection line check valves 897A-D and RHR check valves 838A-D.

C. The containment sump pumps required to be operable by Specification 3.1.F.1.a(1) shall be demonstrated to be operable by performance of the following surveillance program:

1. At monthly intervals, each sump pump shall be started and a discharge flow of at least 25 gpm verified.

2. At refueling intervals, each sump pump shall be operated under visual observation to verify that the pumps start and stop at the appropriate setpoints and that the discharge flow is at least 25 gpm per pump.

Basis

Specifications 4.16.A and 4.16.C establish the surveillance program for monitoring reactor coolant system leakage and leakage into the containment free volume during plant operation and ensure compliance with Specification 3.1.F. These specifications also establish surveillance requirements for the containment sump pumps. Surveillance requirements for the various leakage detection instrumentation systems are contained in Table 4.1-1 of these specifications.

Specification 4.16.B was added to the Technical Specifications in response to NRC's July 5, 1985 rescission of our February 11, 1980 Confirmatory Order Item A.5. Item A.5 was developed to address the intersystem loss-of-coolant accident (Event V) identified in WASH-1400(1,2). The RHR system design pressure is 600 psig.

References

(1) NRC Letters dated July 5, 1985, and February 11, 1980
(2) Con Edison Letter dated March 14, 1980