

William J. Cahill,
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, N Y 10003
Telephone (212) 460-3819

October 16, 1978

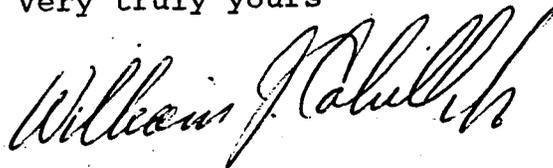
RE: Indian Point Station
Unit No. 2
Docket No. 50-247

REGULATORY DOCKET FILE COPY
Mr. William G. McDonald, Director
Office of Management Information and Program Control
c/o Distribution Services Branch, DDC, ADM
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. McDonald

Enclosed you will find two (2) copies of the monthly operating report relating to Indian Point Unit No. 2 for the month of September, 1978. In as much as Indian Point Unit No. 1 is presently in the defueled shutdown condition with no decision on future operation, we are, with the concurrence of your Mr. T. Cintula, temporarily suspending the submittal of the operating data reports and summary of operating experience for this unit. Any safety related maintenance information for Unit No. 1 will continue to be reported as appropriate.

Very truly yours



enc.

REGULATORY DOCKET FILE COPY

CC: Mr. John G. Davis, Acting Director (40 copies)
Office of Inspection and Enforcement
c/o Distribution Services Branch, DDC, ADM
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. Boyce H. Grier, Director
Office of Inspection and Enforcement
Region 1
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

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A008
S/11

OPERATING DATA REPORT

DOCKET NO. 50-247
 DATE 10-4-78
 COMPLETED BY L. Kawula
 TELEPHONE 914-694-6000
Ext. 209 @
I.P.

OPERATING STATUS

1. Unit Name: Indian Point Unit No. 2
2. Reporting Period: September, 1978
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 885
7. Maximum Dependable Capacity (Net MWe): 849
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes
 Unit was removed from service on September 15th for a scheduled two week maintenance outage.

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720	6 551	37 272
12. Number Of Hours Reactor Was Critical	355.80	3 735.25	24 089.55
13. Reactor Reserve Shutdown Hours	0	51.33	51.33
14. Hours Generator On-Line	355.80	3 666.42	23 390.39
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	985 282	9 863 986	59 815 881
17. Gross Electrical Energy Generated (MWH)	295 680	3 030 490	18 549 016
18. Net Electrical Energy Generated (MWH)	280 208	2 887 140	17 677 997
19. Unit Service Factor	49.4	56.0	62.8
20. Unit Availability Factor	49.4	56.0	62.8
21. Unit Capacity Factor (Using MDC Net)	45.8	51.9	55.9
22. Unit Capacity Factor (Using DER Net)	44.6	50.5	54.3
23. Unit Forced Outage Rate	0	0.4	8.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____
 26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

N.A.

~~7670230450~~

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-247

UNIT I.P. Unit No. 2

DATE 10-4-78

COMPLETED BY L. Kawula

TELEPHONE 914-694-6000
Ext. 209 @ I.P.

MONTH September, 1978

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>827</u>	17	<u>0</u>
2	<u>821</u>	18	<u>0</u>
3	<u>820</u>	19	<u>0</u>
4	<u>827</u>	20	<u>0</u>
5	<u>830</u>	21	<u>0</u>
6	<u>825</u>	22	<u>0</u>
7	<u>828</u>	23	<u>0</u>
8	<u>829</u>	24	<u>0</u>
9	<u>569</u>	25	<u>0</u>
10	<u>823</u>	26	<u>0</u>
11	<u>817</u>	27	<u>0</u>
12	<u>801</u>	28	<u>0</u>
13	<u>795</u>	29	<u>0</u>
14	<u>795</u>	30	<u>0</u>
15	<u>605</u>	31	<u>-</u>
16	<u>0</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH September, 1978

DOCKET NO. 50-247
 UNIT NAME I.P. Unit No. 2
 DATE 10-4-78
 COMPLETED BY L. Kawula
 TELEPHONE 914-694-6000
 Ext. 209 @ I.P.

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
N/A	780909	N/A	0	B	N/A	None	HB	Turbine	Power reduction to perform monthly turbine stop and control valve test.
9	780915	S	364.2	B	1	None	CB	PUMP XX G	Planned outage to repair controlled leakage seal for No. 21 RCP.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER). File (NUREG-0161)

⁵
 Exhibit I - Same Source

Indian Point Station

DOCKET NO. 50-247

UNIT Unit No. 2

DATE October 4, 1978

COMPLETED BY J. Makepeace

TELEPHONE 914-739-8823

Summary of Operating Experience - September, 1978

Unit No. 2 operated at 100% of licensed power until September 9, 1978, when the load was reduced to approximately 46% (310 MWe) to perform the turbine valve exercise test and to permit repairs to a vent line weld attachment on the discharge line from No. 22 main boiler feedwater pump. By the end of the day, the repair had been completed and the load was being returned to full power output.

While the load was reduced for the above repair work, No. 21 main condenser was removed from service because of suspected tube leakage. One leaking tube was identified and plugged.

At 0510 hours that same day, a step increase from 2.5 gpm to 5.5 gpm was observed in the No. 1 seal return flow from No. 21 reactor coolant pump. There was a simultaneous decrease in the ΔP across the thermal barrier from about 40 inches of water to approximately 15 inches. All other pump parameters remained essentially unchanged.

At 0140 hours on September 11, 1978, No. 24 main condenser was removed from service because of suspected tube leakage. A single tube leak was identified and plugged. The condenser was returned to service at approximately 0640 hours on the same day.

A single tube leak in No. 22 main condenser was identified and plugged on September 13, 1978. On the same day, No. 25 condenser was removed from service because of a suspected tube leak. However, no leak could be identified and the condenser was returned to service on the following day.

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Unit No. 2 was removed from service at 1948 hours on September 15, 1978, for a planned two week maintenance outage. The principal purpose of the outage was to remove, inspect and repair as necessary the controlled leakage seal package of No. 21 reactor coolant pump and to locate and eliminate sources of condenser air in-leakage. The reactor was brought to the cold shutdown condition on September 16th, and reactor coolant system drain down began the next day.

Unit No. 2 was maintained in the cold shutdown condition for the remainder of the report period. Items of particular interest during this period were as follows:

1. Repairs to the expansion joint between No. 23 low pressure turbine and its condenser were completed.
2. Repairs to the No. 1 seal on No. 21 reactor coolant pump were completed and the pump reassembled. The cause of the high seal return flow from the No. 1 seal of this pump was found to be a defective "O" ring between the No. 1 seal runner and the pump shaft.
3. Photographs of the tube support plate flow slots in No. 23 steam generator were taken through the lower eight-inch inspection opening and one of the six-inch inspection openings immediately above the tube sheet. Preliminary analysis of the photographs indicates no appreciable change in the flow slot widths since the unit was returned to service following the Cycle 2/3 refueling outage.
4. A leak was observed in the RHR miniflow line immediately downstream of valve 1870. The leak was in the form of several pin holes through the wall of a 2" x 3" enlarging close radius elbow. Efforts to pad weld the affected area were unsuccessful and replacement of the elbow was necessary.

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A new elbow was installed in the RHR miniflow line replacing the one found to have several pinhole leaks in it. To facilitate replacement and to minimize personnel exposures, a prefabricated section of piping containing the new elbow and a manual valve replacement for valve 743 was installed. This section of piping include the section of piping downstream of valve 1870 up to and including the upstream bolted flange for flow element 642. Following installation of this new piping section, the line was successfully hydrottested at 625 psig. Appendix J Type "C" tests were also satisfactorily performed on the new manual globe valve 743.

5. All twelve (12) of the condenser steam dump valves were replaced.

On September 27, 1978, heatup of the reactor coolant system was commenced.

On September 28, 1978, the reactor coolant system was successfully hydrottested at 2335 psig. During the test, slight leakage was observed at the pump flange joint for No. 23 reactor coolant pump. This leakage was subsequently eliminated by retorquing the flange studs.

At the close of this report period, Unit No. 2 was being maintained in the hot shutdown condition, in preparation for a return to service early in October.

Unit No. 2Mechanical and Electrical Maintenance

<u>Date</u>	<u>Component</u>	<u>MWR #</u>	<u>Malfunction</u>	<u>Corrective Action</u>
8-1-78	No. 21 Instrument Air Compressor	2C27733	Stuffing box leakage.	Installed new packing assembly.
8-2-78	No. 22 Waste Gas Compressor	2N27673	Mechanical seal fail- ure.	Replaced mechanical seal.
8-9-78	Valve RV-1475	2N27717	Valve leaks past seat.	Disassembled, cleaned, reinstalled valve.
8-15-78	No. 22 Zurn Strainer	2C27691	Strainer trips on overload	Renewed heater element.
8-17-78	No. 22 Auxiliary Boiler Feed Pump	2C27749	Indicating lights in- correct.	Adjusted limit switch.
8-17-78	No. 23 Component Cooling Pump	2N27757	Mechanical seals fail- ure.	Replaced mechanical seal.
8-18-78	Valve 1705	2N27762	Indicating light in- correct.	Adjusted limit trip arm.
8-19-78	Valve 1702	2N27761	Indicating light in- correct.	Renewed limit switch.
8-22-78	No. 21 Component Cooling Heat Exchanger	2N27793	Tube leakage.	Plugged two tubes.
8-25-78	No. 23 Safety Injection Pump	2N27777	Head bolts leakage.	Replaced head bolts.
8-26-78	No. 22 Safety Injection Pump	2N27776	Head bolts leakage.	Replaced head bolts.

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Instrumentation and Control Repair

<u>Date</u>	<u>Component</u>	<u>MWR #</u>	<u>Malfunction</u>	<u>Corrective Action</u>
8-16-78	Power Range Channel 42	2C22481I	Delta-Flux Indicator Inoperative.	Loose connections tightened.

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