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McGuire Nuclear Station
Docket Nos. 50-369 and 50-370
Annual Radiological Environmental Operating Report

Pursuant to the requirements of Technical Specification 5.6.2, attached is the Annual Radiological Environmental Operating Report for the 2009 calendar year.

Questions regarding this report should be directed to Kay Crane, McGuire Regulatory Compliance at (980) 875-4306.

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Page 2

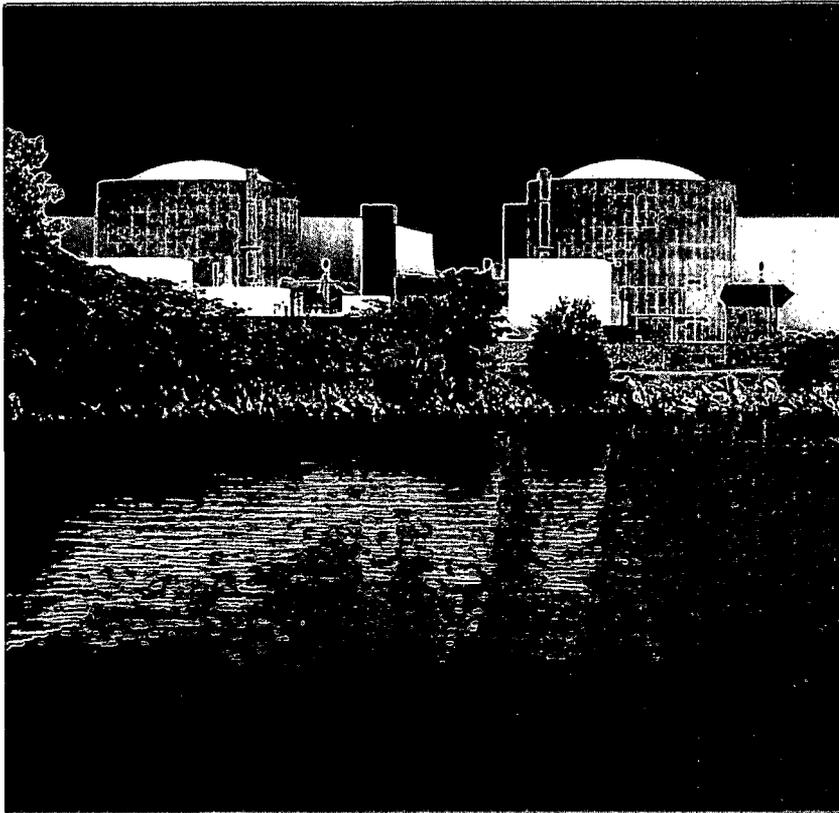
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Units 1 and 2



AREOR

Annual
Radiological Environmental
Operating Report
2009



ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT

DUKE ENERGY CORPORATION
MCGUIRE NUCLEAR STATION
Units 1 and 2

2009

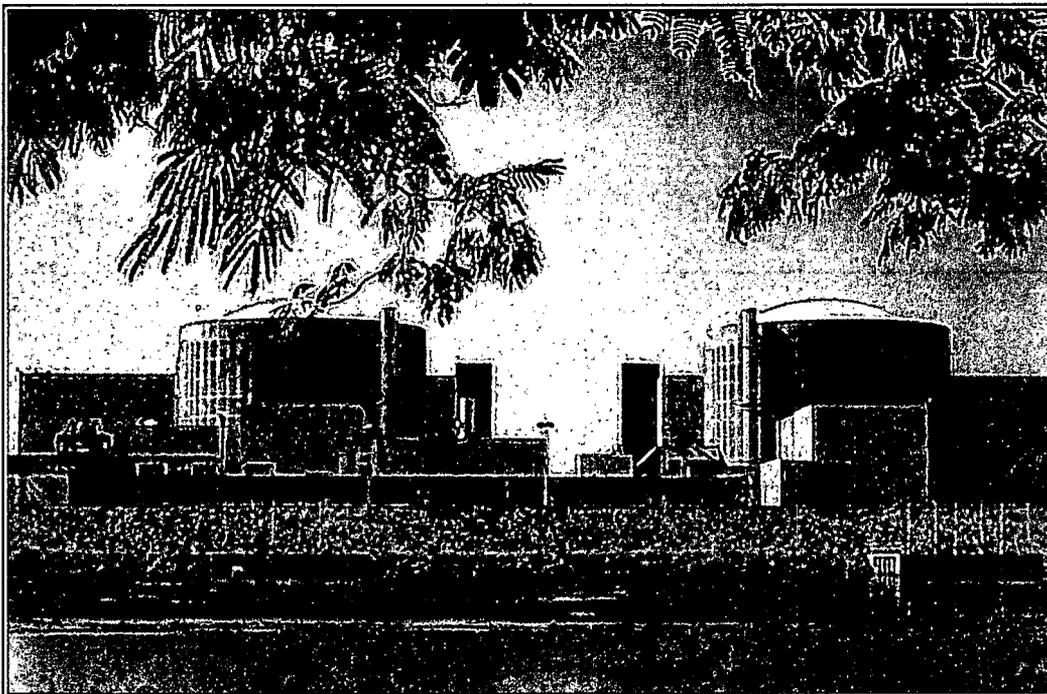


TABLE OF CONTENTS

1.0 Executive Summary	1-1
2.0 Introduction	2-1
2.1 Site Description and Sample Locations	2-1
2.2 Scope and Requirements of the REMP	2-1
2.3 Statistical and Calculational Methodology	2-2
2.3.1 Estimation of the Mean Value	2-2
2.3.2 Lower Level of Detection and Minimum Detectable Activity	2-3
2.3.3 Trend Identification	2-3
3.0 Interpretation of Results	3-1
3.1 Airborne Radioiodine and Particulates	3-2
3.2 Drinking Water	3-5
3.3 Surface Water	3-7
3.4 Milk	3-9
3.5 Broadleaf Vegetation	3-10
3.6 Food Products	3-11
3.7 Fish	3-12
3.8 Shoreline Sediment	3-15
3.9 Direct Gamma Radiation	3-18
3.10 Land Use Census	3-20
4.0 Evaluation of Dose	4-1
4.1 Dose from Environmental Measurements	4-1
4.2 Estimated Dose from Releases	4-1
4.3 Comparison of Doses	4-2
5.0 Quality Assurance	5-1
5.1 Sample Collection	5-1
5.2 Sample Analysis	5-1
5.3 Dosimetry Analysis	5-1
5.4 Laboratory Equipment Quality Assurance	5-1
5.4.1 Daily Quality Control	5-1
5.4.2 Calibration Verification	5-1
5.4.3 Batch Processing	5-2
5.5 Duke Energy Intercomparison Program	5-2
5.6 ERA Proficiency Testing	5-2
5.7 Duke Energy Audits	5-2
5.8 U.S. Nuclear Regulatory Commission Inspections	5-2
5.9 State of North Carolina Intercomparison Program	5-2
5.10 TLD Intercomparison Program	5-3
5.10.1 Nuclear Technology Services Intercomparison Program	5-3
5.10.2 Internal Crosscheck (Duke Energy)	5-3
6.0 References	6-1

Appendices

Appendix A: Environmental Sampling and Analysis Procedures	A-1
I. Change of Sampling Procedures	A-2
II. Description of Analysis Procedures	A-2
III. Change of Analysis Procedures	A-3
IV. Sampling and Analysis Procedures	A-3
A.1 Airborne Particulate and Radioiodine	A-3
A.2 Drinking Water	A-3
A.3 Surface Water	A-4
A.4 Milk	A-4
A.5 Broadleaf Vegetation	A-4
A.6 Food Products	A-4
A.7 Fish	A-4
A.8 Shoreline Sediment	A-5
A.9 Direct Gamma Radiation (TLD)	A-5
A.10 Annual Land Use Census	A-5
V. Global Positioning System (GPS) Analysis	A-6
Appendix B: Radiological Env. Monitoring Program - Summary of Results	B-1
Air Particulate	B-2
Air Radioiodine	B-3
Drinking Water	B-4
Surface Water	B-5
Milk	B-6
Broadleaf Vegetation	B-7
Food Products	B-8
Fish	B-9
Shoreline Sediment	B-10
Direct Gamma Radiation (TLD)	B-11
Appendix C: Sampling Deviations and Unavailable Analyses	C-1
C.1 Sampling Deviations	C-2
C.2 Unavailable Analyses	C-3
Appendix D: Analytical Deviations	D-1
Appendix E: Radiological Environmental Monitoring Program Results	E-1

LIST OF FIGURES

2.1-1	Sampling Locations Map (0.5 Mile Radius)	2-4
2.1-2	Sampling Locations Map (Ten Mile Radius)	2-5
3.1	Concentration of Gross Beta in Air Particulate	3-3
3.2	Concentration of Tritium in Drinking Water	3-5
3.3	Concentration of Tritium in Surface Water	3-7
3.7-1	Concentration of Cs-137 in Fish	3-13
3.7-2	Concentration of Co-60 in Fish	3-13
3.8-1	Concentration of Cs-137 in Shoreline Sediment	3-15
3.8-2	Concentration of Co-60 in Shoreline Sediment	3-16
3.9	Direct Gamma Radiation (TLD) Results	3-18
3.10	2009 Land Use Census Map	3-21

LIST OF TABLES

2.1-A	Radiological Monitoring Program Sampling Locations	2-6
2.1-B	Radiological Monitoring Program Sampling Locations (TLD Sites)	2-7

2.2-A	Reporting Levels for Radioactivity Concentrations in Environmental Samples	2-8
2.2-B	REMP Analysis Frequency	2-8
2.2-C	Maximum Values for the Lower Limits of Detection	2-9
3.1-A	Mean Concentrations of Radionuclides in Air Particulate	3-3
3.1-B	Mean Concentrations of Air Radioiodine (I-131)	3-4
3.2	Mean Concentrations of Radionuclides in Drinking Water	3-6
3.3	Mean Concentrations of Tritium in Surface Water	3-8
3.4	Mean Concentrations of Cs-137 in Milk	3-9
3.5	Mean Concentrations of Cs-137 in Broadleaf Vegetation	3-10
3.6	Mean Concentrations of Cs-137 in Food Products	3-11
3.7	Mean Concentrations of Radionuclides in Fish	3-14
3.8	Mean Concentrations of Radionuclides in Shoreline Sediment	3-16
3.9	Direct Gamma Radiation (TLD) Results	3-19
3.10	Land Use Census Results	3-20
4.1-A	2009 Environmental and Effluent Dose Comparison	4-3
4.1-B	Maximum Individual Dose for 2009 based on Environmental Measurements for McGuire Nuclear Station	4-6
5.0-A	2009 Cross-Check Results for EnRad Laboratories	5-4
5.0-B	2009 Environmental Resource Associates Quik™ Response Program	5-9
5.0-C	2009 Environmental Dosimeter Cross-Check Results	5-11

LIST OF ACRONYMS USED IN THIS TEXT *(in alphabetical order)*

BW	BiWeekly
C	Control
DEHNR	Department of Environmental Health and Natural Resources
DHEC	Department of Health and Environmental Control
EPA	Environmental Protection Agency
ERA	Environmental Resource Associates
GI-LLI	Gastrointestinal – Lower Large Intestine
GPS	Global Positioning System
LLD	Lower Limit of Detection
M	Monthly
MDA	Minimum Detectable Activity
MNS	McGuire Nuclear Station
mrem	millirem
NIST	National Institute of Standards and Technology
NRC	Nuclear Regulatory Commission
ODCM	Offsite Dose Calculation Manual
pCi/kg	picocurie per kilogram
pCi/l	picocurie per liter
pCi/m3	picocurie per cubic meter
PIP	Problem Investigation Process
Q	Quarterly
REMP	Radiological Environmental Monitoring Program
SA	Semiannually
SLCs	Selected Licensee Commitments
SM	Semimonthly
TECH SPECS	Technical Specifications
TLD	Thermoluminescent Dosimeter
μCi/ml	microcurie per milliliter
UFSAR	Updated Final Safety Analysis Report
W	Weekly

1.0 EXECUTIVE SUMMARY

This Annual Radiological Environmental Operating Report describes the McGuire Nuclear Station Radiological Environmental Monitoring Program (REMP), and the program results for the calendar year 2009.



Included are the identification of sampling locations, descriptions of environmental sampling and analysis procedures, comparisons of present environmental radioactivity levels and pre-operational environmental data, comparisons of doses calculated from environmental measurements and effluent data, analysis of trends in environmental radiological data as potentially affected by station operations, and a summary of environmental radiological sampling results. Quality assurance practices, sampling deviations, unavailable samples, and program changes are also discussed.

Sampling activities were conducted as prescribed by Selected Licensee Commitments (SLC's). Required analyses were performed and detection capabilities were met for all collected samples as required by SLC's. Eleven-hundred twenty-eight samples were analyzed comprising 1,582 test results in order to compile data for the 2009 report. Based on the annual land use census, the current number of sampling sites for McGuire Nuclear Station is sufficient.

Concentrations observed in the environment in 2009 for station related radionuclides were generally within the ranges of concentrations observed in the past. Inspection of data showed that radioactivity concentrations in surface water, drinking water, shoreline sediment and fish are higher than the activities reported for samples collected prior to the operation of the station. Measured concentrations were not higher than expected, and all positively identified measurements were within limits as specified in SLC's.

Additionally, environmental radiological monitoring data is consistent with effluents introduced into the environment by plant operations. The total body dose estimated to the maximum exposed member of the public as calculated by environmental sampling data, excluding TLD results, was 8.86E-2 mrem for 2009. It is therefore concluded that station operations has had no significant radiological impact on the health and safety of the public or the environment.

2.0 INTRODUCTION

2.1 SITE DESCRIPTION AND SAMPLE LOCATIONS

McGuire Nuclear Station (MNS) is located geographically near the center of a highly industrialized region of the Carolinas. The land is predominantly rural non-farm with a small amount of land being used for farming. The McGuire site is in northwestern Mecklenburg County, North Carolina, 17 miles north-northwest of Charlotte, North Carolina. The site is bounded to the west by the Catawba River channel and to the north by 32,510 acre Lake Norman. Lake Norman is impounded by Duke Energy Corporation's Cowans Ford Dam Hydroelectric Station. The tailwater of Cowans Ford Dam is the upper limit of Mountain Island Reservoir. Mountain Island Dam is located 15 miles downstream from the site. Lookout Shoals Hydroelectric Station is at the upper reaches of Lake Norman. Marshall Steam Station is located on the western shore of Lake Norman, approximately 16 miles upstream from the site (reference 6.3).

MNS consists of two pressurized water reactors. Each reactor unit is essentially a mirror image of the other joined by an auxiliary building housing both separate and common equipment. Each unit was designed to produce approximately 1200 gross Megawatts of electricity. Unit 1 achieved criticality August 8, 1981 and Unit 2 on May 8, 1983.

Figures 2.1-1 and 2.1-2 are maps depicting the Thermoluminescent Dosimeter (TLD) monitoring locations and the sampling locations. The location numbers shown on these maps correspond to those listed in Tables 2.1-A and 2.1-B. Figure 2.1-1 comprises all sample locations within 0.5 mile radius of MNS. Figure 2.1-2 comprises all sample locations within a ten mile radius of MNS.

2.2 SCOPE AND REQUIREMENTS OF THE REMP

An environmental monitoring program has been in effect at McGuire Nuclear Station since 1977, four years prior to operation of Unit 1 in 1981. The preoperational program provides data on the existing environmental radioactivity levels for the site and vicinity which may be used to determine whether increases in environmental levels are attributable to the station. The operational program provides surveillance and backup support of detailed effluent monitoring which is necessary to evaluate the significance, if any, of the contributions to the existing environmental radioactivity levels that result from station operation.

This monitoring program is based on NRC guidance as reflected in the Selected Licensee Commitments Manual, with regard to sample media, sampling locations, sampling frequency, and analytical sensitivity requirements. Indicator and control locations were established for comparison purposes to distinguish radioactivity of station origin from natural or other "man-made" environmental radioactivity. The environmental monitoring program also verifies projected and anticipated radionuclide concentrations in the environment and related exposures

from releases of radionuclides from McGuire Nuclear Station. This program satisfies the requirements of Section IV.B.2 of Appendix I to 10CFR50 and provides surveillance of all appropriate critical exposure pathways to man and protects vital interests of the company, public, and state and federal agencies concerned with the environment. Reporting levels for radioactivity found in environmental samples are listed in Table 2.2-A. Table 2.2-B lists the REMP analysis and frequency schedule.

The Annual Land Use Census, required by Selected Licensee Commitments, is performed to ensure that changes in the use of areas at or beyond the site boundary are identified and that modifications to the Radiological Environmental Monitoring Program are made if required by changes in land use. This census satisfies the requirements of Section IV.B.3 of Appendix I to 10CFR50. Results are shown in Table 3.10.

Participation in an interlaboratory comparison program as required by Selected Licensee Commitments provides for independent checks on the precision and accuracy of measurements of radioactive material in REMP sample matrices. Such checks are performed as part of the quality assurance program for environmental monitoring in order to demonstrate that the results are valid for the purposes of Section IV.B.2 of Appendix I to 10CFR50. A summary of the results obtained as part of this comparison program are in Section 5 of this annual report.

2.3 STATISTICAL AND CALCULATIONAL METHODOLOGY

2.3.1 ESTIMATION OF THE MEAN VALUE

There was one (1) basic statistical calculation performed on the raw data resulting from the environmental sample analysis program. The calculation involved the determination of the mean value for the indicator and the control samples for each sample medium. The mean is a widely used statistic. This value was used in the reduction of the data generated by the sampling and analysis of the various media in the Radiological Environmental Monitoring Program. "Net activity (or concentration)" is the activity (or concentration) determined to be present in the sample. No "Minimum Detectable Activity", "Lower Limit of Detection", "Less Than Level", or negative activities or concentrations are included in the calculation of the mean. The following equation was used to estimate the mean (reference 6.8):

$$\bar{x} = \frac{\sum_{i=1}^N x_i}{N}$$

Where:

\bar{x} = estimate of the mean,

i = individual sample,

N = total number of samples with a net activity (or concentration),

x_i = net activity (or concentration) for sample i .

2.3.2 LOWER LEVEL OF DETECTION AND MINIMUM DETECTABLE ACTIVITY

The Lower Level of Detection (LLD) and Minimum Detectable Activity (MDA) are used throughout the Environmental Monitoring Program.

LLD - The LLD, as defined in the Selected Licensee Commitments Manual is the smallest concentration of radioactive material in a sample that will yield a net count, above the system background, that will be detected with 95% probability with only 5% probability of falsely concluding that a blank observation represents a "real" signal. The LLD is an *a priori* lower limit of detection. The actual LLD is dependent upon the standard deviation of the background counting rate, the counting efficiency, the sample size (mass or volume), the radiochemical yield, and the radioactive decay of the sample between sample collection and counting. The "required" LLD's for each sample medium and selected radionuclides are given in the Selected Licensee Commitments and are listed in Table 2.2-C.

MDA - The MDA is the net counting rate (sample after subtraction of background) that must be surpassed before a sample is considered to contain a scientifically measurable amount of a radioactive material exceeding background amounts. The MDA is calculated using a sample background and may be thought of as an "actual" LLD for a particular sample measurement.

2.3.3 TREND IDENTIFICATION

One of the purposes of an environmental monitoring program is to determine if there is a buildup of radionuclides in the environment due to the operation of the nuclear station. Visual inspection of tabular or graphical presentations of data (including preoperational) is used to determine if a trend exists. A decrease in a particular radionuclide's concentration in an environmental medium does not indicate that reactor operations are removing radioactivity from the environment but that reactor operations are not adding that radionuclide to the environment in quantities exceeding the preoperational level and that the normal removal processes (radioactive decay, deposition, resuspension, etc.) are influencing the concentration.

Substantial increases or decreases in the amount of a particular radionuclide's release from the nuclear plant will greatly affect the resulting environmental levels; therefore, a knowledge of the release of a radionuclide from the nuclear plant is necessary to completely interpret the trends, or lack of trends, determined from the environmental data. Some factors that may affect environmental levels of radionuclides include prevailing weather conditions (periods of drought, solar cycles or heavier than normal precipitation), construction in or around either the nuclear plant or the sampling location, and addition or deletion of other sources of radioactive materials (such as the Chernobyl accident). Some of these factors may be obvious while others are sometimes unknown. Therefore, how trends are identified will include some judgment by plant personnel.

Figure 2.1-1

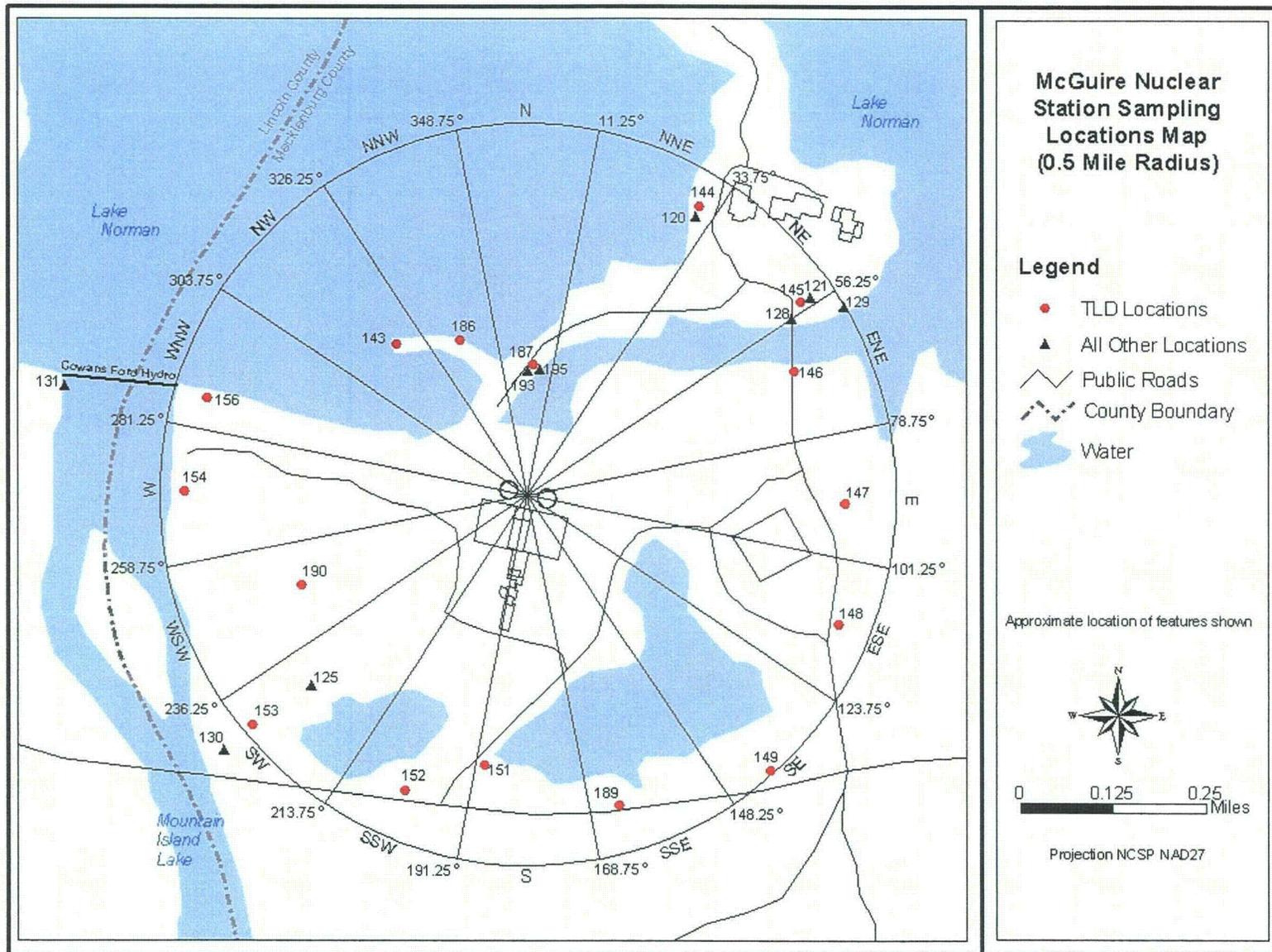


Figure 2.1-2

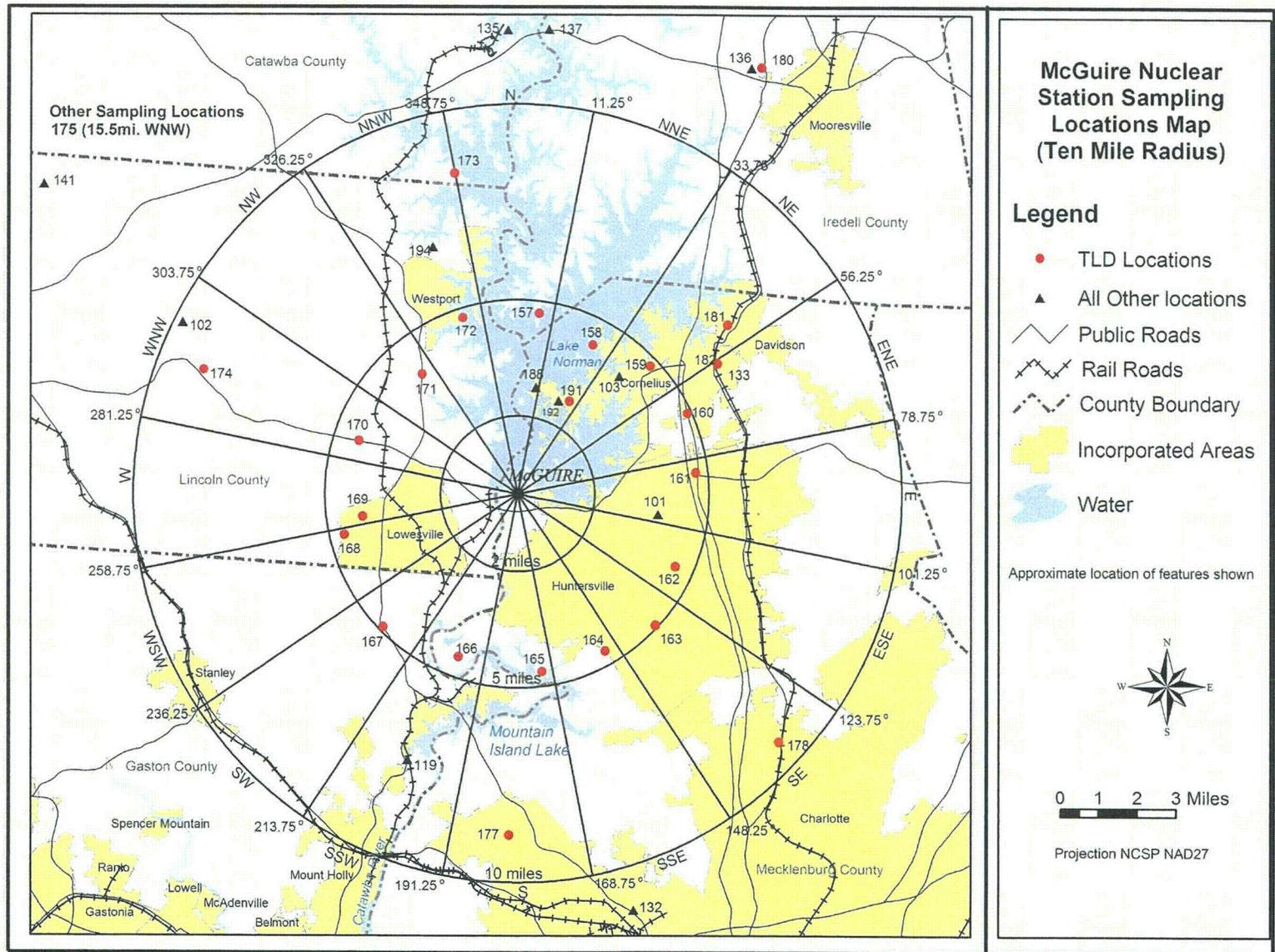


TABLE 2.1-A

**MCGUIRE RADIOLOGICAL MONITORING PROGRAM
SAMPLING LOCATIONS**

W	Weekly	SM	Semimonthly
BW	BiWeekly	Q	Quarterly
M	Monthly	SA	Semiannually
C	Control		

Site #	Location Description*	Air Rad. & Part.	Surface Water	Drinking Water	Shoreline Sediment	Food Products	Fish	Milk	Broad Leaf Veg.
101	North Mecklenburg Water Treatment Facility (3.31 mi E)			M					
102 C	Amity Church Road (9.89 mi WNW)	W							M (b)
103 ⁽¹⁾	Cottonwood Substation (4.20 mi NE)	W							
119	Mt. Holly Municipal Water Supply (7.40 mi SSW)			M					
120	Site Boundary (0.46 mi NNE)	W							M (b)
121	Site Boundary (0.47 mi NE)	W							
125	Site Boundary (0.38 mi SW)	W							M (b)
128	Discharge Canal Bridge (0.45 mi NE)		M						
129	Discharge Canal Entrance to Lake Norman (0.51 mi ENE)				SA		SA		
130	Hwy 73 Bridge Downstream (0.52 mi SW)				SA				
131	Cowans Ford Dam (0.64 mi WNW)		M						
132	Charlotte Municipal Water Supply (11.1 mi SSE)			M					
133	Cornelius (6.23 mi ENE)	W							
135 C	Plant Marshall Intake Canal (11.9 mi N)		M						
136 C	Mooreville Municipal Water Supply (12.7 mi NNE)			M					
137 C	Pinnacle Access Area (12.0 mi N)				SA		SA		
141 C	Lynch Dairy-Cows (14.8 mi WNW)							SM	
188	5 mile radius Gardens (2.79 mi NNE)					M (a)			
192 ⁽¹⁾	Peninsula (2.84 mi NNE)	W							
193	Site Boundary (0.19 mi N)								M (b)
194	East Lincoln County Water Supply (6.73 mi NNW)			M					
195	Fishing Access Road (0.19 mi N)	W							

(1) Location 103 (Air Radioiodine, Air Particulate) was added as a replacement for location 192 (Air Radioiodine, Air Particulate), which was removed from the program. Movement of the air radioiodine and air particulate location is described reference 6.14.

- (a) During Harvest Season
- (b) When Available

* GPS data reflect approximate accuracy to within 2-5 meters. GPS field measurements were taken as close as possible to the item of interest.

TABLE 2.1-B

**MCGUIRE RADIOLOGICAL MONITORING PROGRAM
SAMPLING LOCATIONS**

(TLD SITES)

Site #	Location	Distance*	Sector	Site #	Location	Distance*	Sector
143	SITE BOUNDARY	0.27 miles	NW	164	HAMBRIGHT & BEATTIES FORD ROAD	4.64 miles	SSE
144	SITE BOUNDARY	0.46 miles	NNE	165	ARTHER AUTEN ROAD	4.57 miles	S
145	SITE BOUNDARY	0.47 miles	NE	166	NECK ROAD REFUGUE BOUNDARY	4.44 miles	SSW
146	SITE BOUNDARY	0.42 miles	ENE	167	LUCIA RIVERBEND HWY/ OLD FIREHOUSE	4.87 miles	SW
147	SITE BOUNDARY	0.44 miles	E	168	OLD PLANK ROAD BRIDGE	4.60 miles	WSW
148	SITE BOUNDARY	0.46 miles	ESE	169	GLOVER LANE	4.03 miles	W
149	SITE BOUNDARY	0.50 miles	SE	170	LITTLE EGYPT ROAD	4.32 miles	WNW
151	SITE BOUNDARY	0.37 miles	S	171	TRIANGLE ACE HARDWARE	3.95 miles	NW
152	SITE BOUNDARY	0.44 miles	SSW	172	LAKESHORE SOUTH RD & ISLAND VIEW COURT	4.69 miles	NNW
153	SITE BOUNDARY	0.47 miles	SW	173 SI	KEISTLER STORE / GLENWOOD ROAD	8.39 miles	NNW
154	SITE BOUNDARY	0.45 miles	W	174 SI	EAST LINCOLN JR. HIGH SCHOOL	8.77 miles	WNW
156	SITE BOUNDARY	0.44 miles	WNW	175 C	BOGER CITY	15.5 miles	WNW
189	SITE BOUNDARY	0.43 miles	SSE	177 SI	BELMARRROW ROAD / COULWOOD COMMUNITY	8.77 miles	S
190	SITE BOUNDARY	0.33 miles	WSW	178 SI	FLORIDA STEEL CORPORATION	9.32 miles	SE
157	THE POINTE/MOORESVILLE	4.69 miles	N	180 SI	MOORESVILLE WATER TREATMENT FACILITY	12.7 miles	NNE
158	BETHEL CHURCH ROAD	4.33 miles	NNE	181 SI	OLD DAVIDSON WATER TREATMENT FACILITY	7.02 miles	NE
159	HENDERSON ROAD & WEST CATAWBA AVENUE	4.73 miles	NE	182 SI	CORNELIUS / AIR SITE # 133	6.23 miles	ENE
160	ANCHORAGE MARINE SHOWROOM	4.89 miles	ENE	186 SI	MCGUIRE FISHING ACCESS ROAD ON PENINSULA	0.24 miles	NNW
161	SAM FURR ROAD & HWY 21	4.70 miles	E	187 SI	ENERGY EXPLORIUM / AIR SITE # 195	0.19 miles	N
162	RANSON ROAD	4.53 miles	ESE	191 SI	PENINSULA DEVELOPMENT / AIR SITE # 192	2.84 miles	NNE
163	MCCOY ROAD	4.94 miles	SE				

C = Control
SI = Special Interest

* GPS data reflect approximate accuracy to within 2-5 meters. GPS field measurements were taken as close as possible to the item of interest.

TABLE 2.2-A

**REPORTING LEVELS FOR RADIOACTIVITY
CONCENTRATIONS IN ENVIRONMENTAL SAMPLES**

Analysis	Water (pCi/liter)	Air Particulates or Gases (pCi/m ³)	Fish (pCi/kg-wet)	Milk (pCi/liter)	BroadLeaf Vegetation (pCi/kg-wet)
H-3	20,000 ^{(a),(b)}				
Mn-54	1,000		30,000		
Fe-59	400		10,000		
Co-58	1,000		30,000		
Co-60	300		10,000		
Zn-65	300		20,000		
Zr-Nb-95	400				
I-131	2	0.9		3	100
Cs-134	30	10	1,000	60	1,000
Cs-137	50	20	2,000	70	2,000
Ba-La-140	200			300	

- (a) If no drinking water pathway exists, a value of 30,000 pCi/liter may be used.
- (b) H-3 Reporting level not applicable to surface water

TABLE 2.2-B

REMP ANALYSIS FREQUENCY

Sample Medium	Analysis Schedule	Gamma Isotopic	Tritium	Low Level I-131	Gross Beta	TLD
Air Radioiodine	Weekly	X				
Air	Weekly	X			X	
Direct Radiation	Quarterly					X
Surface Water	Monthly Composite	X				
	Quarterly Composite		X			
Drinking Water	Monthly Composite	X		(a)	X	
	Quarterly Composite		X			
Shoreline Sediment	Semiannually	X				
Milk	Semimonthly	X		X		
Fish	Semiannually	X				
Broadleaf Vegetation	Monthly ^(b)	X				
Food Products	Monthly ^(b)	X				

- (a) Low-level I-131 analysis will be performed if the dose calculated for the consumption of drinking water is > 1 mrem per year. An LLD of 1 pCi/liter will be required for this analysis.
- (b) When Available

TABLE 2.2-C

MAXIMUM VALUES FOR THE LOWER LIMITS OF DETECTION

Analysis	Water (pCi/liter)	Air Particulates or Gases (pCi/m ³)	Fish (pCi/kg-wet)	Milk (pCi/liter)	BroadLeaf Vegetation (pCi/kg-wet)	Sediment (pCi/kg-dry)
Gross Beta	4	0.01				
H-3	2000 ^(a)					
Mn-54	15		130			
Fe-59	30		260			
Co-58, 60	15		130			
Zn-65	30		260			
Zr-Nb-95	15					
I-131	1 ^(b)	0.07		1	60	
Cs-134	15	0.05	130	15	60	150
Cs-137	18	0.06	150	18	80	180
Ba-La-140	15			15		

(a) If no drinking water pathway exists, a value of 3000 pCi/liter may be used.

(b) If no drinking water pathway exists, the LLD of gamma isotopic analysis may be used.

3.0 INTERPRETATION OF RESULTS

Review of 2009 REMP analysis results was performed to detect and identify changes in environmental levels as a result of station operation. The radionuclides with Selected Licensee Commitments reporting levels that indicate consistent detectable activity have been historically trended from preoperation to present. Analyses from 1977 - 1978 have been excluded since these results were much higher than the other preoperational years due to outside influences such as weapons testing. The preoperational analyses from 1981 were combined with the operational analyses from the latter part of 1981 and averaged to give one concentration for each radionuclide for that year.

The highest annual mean concentration of applicable Selected Licensee Commitments radionuclides from the indicator locations for each media type was used for trending purposes. Trending was performed by comparing annual mean concentrations to historical results. Factors evaluated include the frequency of detection and the concentration in terms of the percent of the radionuclide's SLC reporting level (Table 2.2-A). All maximum percent of reporting level values were well below the 100% action level. The highest value reached during 2009 was 5.15% for drinking water tritium at the North Mecklenburg Water Treatment Facility (Location 101). Only Selected Licensee Commitments radionuclides were detected in 2009.

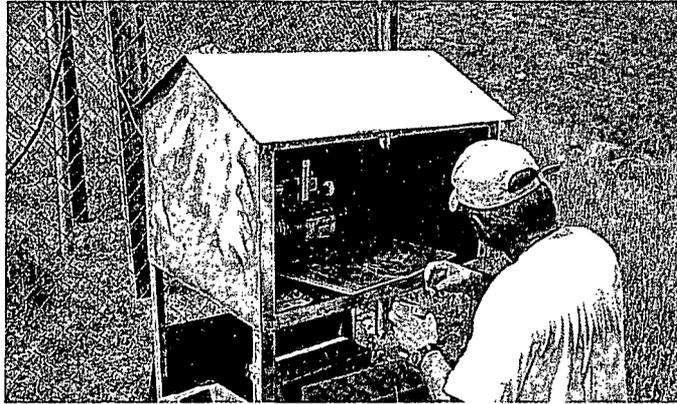
Changes in sample location, analytical technique, and presentation of results must be considered when reviewing for trends. Calculation of the annual mean concentrations has been performed differently over the history of the REMP. During 1979-1986, all net results (sample minus background) positive and negative, were included in the calculation of the mean. Only positive net activity results were used to calculate the mean for the other years. All negative values were replaced with a zero for calculational and graphical purposes to properly represent environmental conditions. A change in gamma spectroscopy analysis systems in 1987 ended a period when many measurements yielded detectable low-level activity for both indicator and control location samples. It is possible that the method the previous system used to estimate net activity may have been vulnerable to false-positive results.

This section includes tables and graphs containing the highest annual mean concentrations of any effluent related radionuclide detected since the change in analysis systems in 1987. Any zero concentrations used in tables or graphs represent activity measurements less than detectable levels. Only the specific radionuclides that represent the highest dose contributors or demonstrate consistent detectable activity are shown graphically.

Data presented in Sections 3.1 through 3.9 support the conclusion that there was no significant increase in radioactivity in the environment around McGuire Nuclear Station due to station operations in 2009. Similarly, there was no significant increase in ambient background radiation levels in the surrounding areas. The 2009 land use census data, shown in Section 3.10, indicates that no program changes are required as a result of the census.

3.1 AIRBORNE RADIOIODINE AND PARTICULATES

In 2009, 363 particulate and radioiodine samples were analyzed, 311 at six indicator locations and 52 at the control location. Particulate samples were analyzed weekly for gamma and gross beta. Radioiodine samples received a weekly gamma analysis.



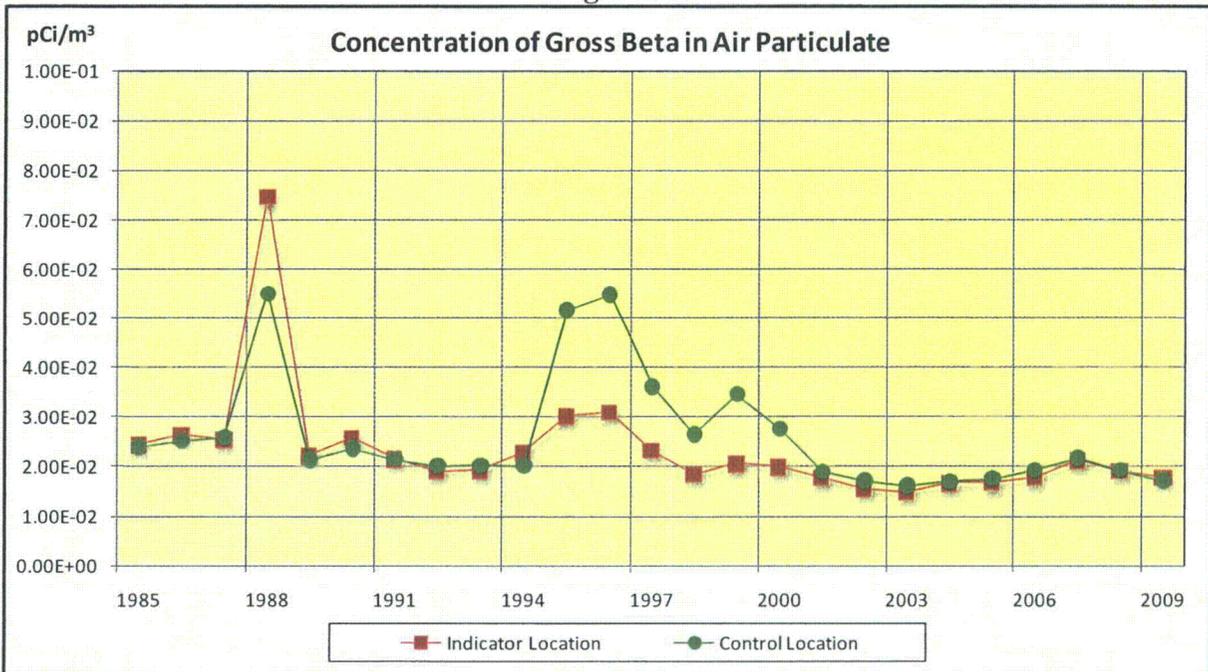
Gross beta analyses indicated $1.79\text{E-}2$ pCi/m³ at the location with the highest annual mean and $1.76\text{E-}2$ pCi/m³ at the control location. The control location was relocated during 2008 (reference 6.14). Detectable gamma emitting particulate activity was last observed in environmental air particulate samples in 2004 (reference 6.15).

No detectable I-131 activity in any environmental air radioiodine samples was found in 2009. K-40 and Be-7 that occur naturally were routinely detected in charcoal cartridges collected during the year. Cs-137 activity was not detected on any cartridges in 2009. Cs-137 detection on the charcoal cartridge was determined in 1990 to be an active constituent of the charcoal. A similar study was performed in 2001 again yielding this conclusion. Therefore, any Cs-137 activities were not used in any dose calculations in Section 4.0 of this report.

Figure 3.1 shows gross beta highest annual mean indicator and control location concentrations since 1985. There is no reporting level for gross beta. Table 3.1-A shows indicator and control location highest annual means for Cs-137 and gross beta.

Table 3.1-B gives indicator location highest annual means and control means since 1979 for I-131. Preoperational and ten year averages are also shown. No I-131 activity has been detected since 1989. Since no activity was detected in 2009, no reporting levels were approached.

Figure 3.1



There is no reporting level for Gross Beta in air particulate

Table 3.1-A Mean Concentrations of Radionuclides in Air Particulate

YEAR	Cs-137 Indicator (pCi/m ³)	Cs-137 Control (pCi/m ³)	Beta Indicator (pCi/m ³)	Beta Control (pCi/m ³)
1979*	4.40E-3	1.47E-3	**	**
1980*	6.70E-3	4.53E-3	**	**
1981*	6.16E-3	5.32E-3	**	**
1982*	3.82E-3	2.29E-3	**	**
1983*	2.93E-3	3.21E-3	**	**
1984	1.74E-3	8.29E-4	**	**
1985	1.86E-3	1.32E-3	2.44E-2	2.40E-2
1986	4.98E-3	3.03E-3	2.64E-2	2.52E-2
1987	1.07E-2	7.91E-3	2.54E-2	2.59E-2
1988	0.00E0	0.00E0	7.49E-2	5.51E-2
1989	0.00E0	0.00E0	2.22E-2	2.14E-2
1990	0.00E0	0.00E0	2.58E-2	2.37E-2
1991	0.00E0	0.00E0	2.16E-2	2.15E-2
1992	0.00E0	0.00E0	1.92E-2	2.02E-2
1993	0.00E0	0.00E0	1.93E-2	2.04E-2
1994	0.00E0	0.00E0	2.28E-2	2.02E-2
1995	0.00E0	0.00E0	3.02E-2	5.17E-2
1996	0.00E0	0.00E0	3.11E-2	5.49E-2
1997	0.00E0	0.00E0	2.34E-2	3.62E-2
1998	0.00E0	0.00E0	1.86E-2	2.66E-2
1999	0.00E0	0.00E0	2.06E-2	3.47E-2
2000	0.00E0	0.00E0	2.00E-2	2.77E-2
2001	0.00E0	0.00E0	1.79E-2	1.91E-2

Table 3.1-A continued

YEAR	Cs-137 Indicator (pCi/m ³)	Cs-137 Control (pCi/m ³)	Beta Indicator (pCi/m ³)	Beta Control (pCi/m ³)
2002	0.00E0	0.00E0	1.57E-2	1.72E-2
2003	0.00E0	0.00E0	1.50E-2	1.63E-2
2004	0.00E0	0.00E0	1.67E-2	1.71E-2
2005	0.00E0	0.00E0	1.68E-2	1.77E-2
2006	0.00E0	0.00E0	1.79E-2	1.94E-2
2007	0.00E0	0.00E0	2.12E-2	2.18E-2
2008	0.00E0	0.00E0	1.92E-2	1.93E-2
Average (1999 – 2008)	NOT APPLICABLE	NOT APPLICABLE	1.81E-2	2.10E-2
2009	0.00E0	0.00E0	1.79E-2	1.76E-2

0.00E0 = no detectable measurements

* Radioiodines and Particulates analyzed together

** Gross Beta analysis not performed

Table 3.1-B Mean Concentrations of Air Radioiodine (I-131)

Year	Indicator Location (pCi/m ³)	Control Location (pCi/m ³)
1979*	3.28E-3	1.04E-3
1980*	2.01E-3	1.10E-3
1981*	4.17E-3	6.27E-4
1982*	1.42E-3	2.48E-3
1983*	1.99E-3	2.01E-4
1984	3.17E-3	0.00E0
1985	3.15E-3	1.04E-3
1986	1.27E-2	6.10E-3
1987	1.07E-2	6.60E-3
1988	0.00E0	0.00E0
1989	2.18E-2	0.00E0
1990	0.00E0	0.00E0
1991	0.00E0	0.00E0
1992	0.00E0	0.00E0
1993	0.00E0	0.00E0
1994	0.00E0	0.00E0
1995	0.00E0	0.00E0
1996	0.00E0	0.00E0
1997	0.00E0	0.00E0
1998	0.00E0	0.00E0
1999	0.00E0	0.00E0
2000	0.00E0	0.00E0
2001	0.00E0	0.00E0
2002	0.00E0	0.00E0
2003	0.00E0	0.00E0
2004	0.00E0	0.00E0
2005	0.00E0	0.00E0
2006	0.00E0	0.00E0
2007	0.00E0	0.00E0
2008	0.00E0	0.00E0
2009	0.00E0	0.00E0

0.00E0 = no detectable measurements

* Radioiodines and Particulates analyzed together.

3.2 DRINKING WATER

In 2009, 65 drinking water samples were analyzed for gross beta and gamma emitting radionuclides. Fifty-two samples were from the four indicator locations and 13 from the control location. Tritium (H-3) analyses were performed on 20 composite samples, 16 at indicator locations and four at the control location.

No detectable gamma activity was found in drinking water samples in 2009 and has not been detected since 1987. Gross beta analyses indicated 1.90 pCi/l at the location with the highest annual mean and 1.81 pCi/l at the control location. Tritium was detected in all of the 16 indicator composite samples taken in 2009 with the highest annual mean resulting in only 5.15% of the reporting level. Tritium was detected in one of the four control location samples. The dose for consumption of water was less than one mrem per year, historically and for 2009; therefore low-level iodine analysis is not required.

Figure 3.2 shows tritium highest annual mean indicator and control location concentrations with comparisons to 20% of the reporting level. Table 3.2 gives indicator location highest annual means and control means since 1979 for tritium and gross beta. There is no reporting level for gross beta.

Drinking water Location 101 was added to the sampling program in 1999. Figure 3.2 shows an increase beginning in that year. There was an increase in drinking water tritium in 2006 due to silica removal from the spent fuel pools. This resulted in additional water volume being released from the plant. Controlled tritium releases to Lake Norman have continued in an effort to balance the plant inventory. In addition, an extreme drought during the second half of 2007 and much of 2008 affecting the Catawba River Basin resulted in less dilution volume available in Lake Norman.

Figure 3.2

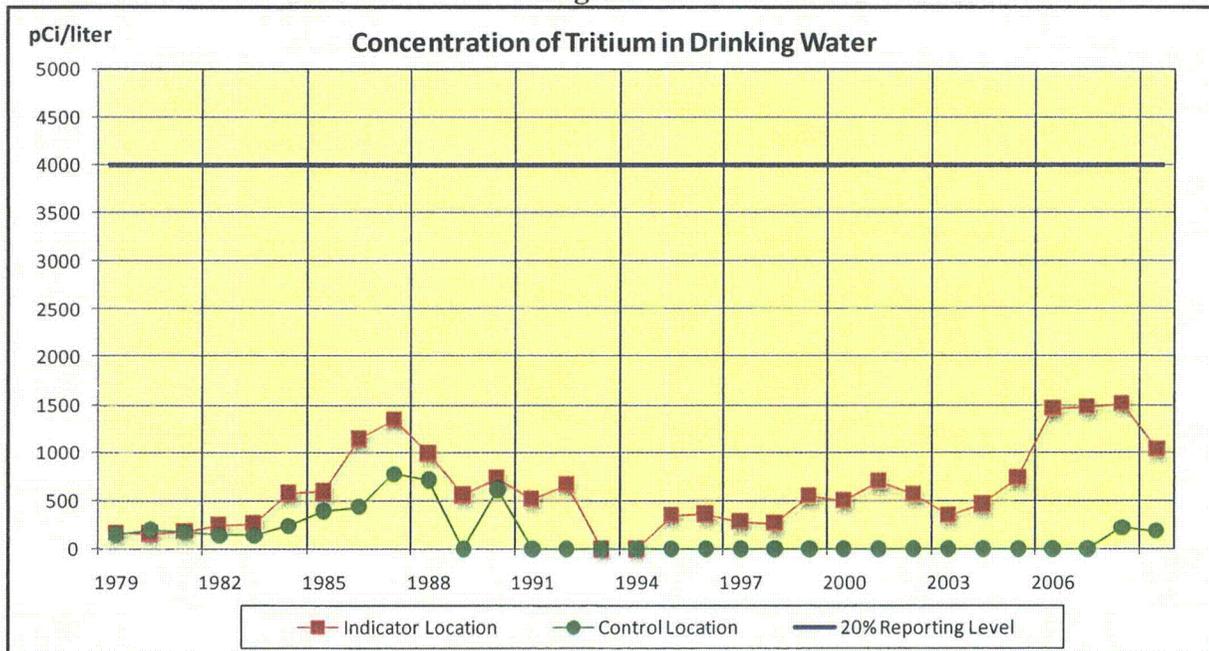


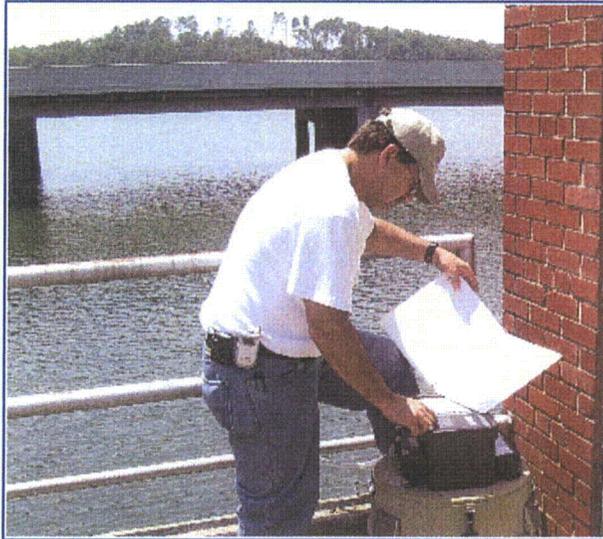
Table 3.2 Mean Concentrations of Radionuclides in Drinking Water

YEAR	Gross Beta (pCi/l)		Tritium (pCi/l)	
	Indicator Location	Control Location	Indicator Location	Control Location
1979	2.40E0	2.03E0	1.65E2	1.50E2
1980	2.34E0	1.87E0	1.63E2	2.05E2
1981	2.79E0	2.41E0	1.88E2	1.78E2
1982	2.62E0	2.43E0	2.43E2	1.45E2
1983	1.80E0	1.87E0	2.65E2	1.45E2
1984	2.78E0	1.81E0	5.77E2	2.45E2
1985	1.88E0	1.90E0	5.93E2	4.00E2
1986	2.13E0	2.15E0	1.14E3	4.37E2
1987	2.30E0	2.00E0	1.35E3	7.75E2
1988	2.00E0	2.00E0	9.92E2	7.11E2
1989	2.80E0	2.70E0	5.62E2	0.00E0
1990	3.70E0	4.30E0	7.32E2	6.11E2
1991	2.40E0	2.50E0	5.22E2	0.00E0
1992	2.00E0	1.70E0	6.73E2	0.00E0
1993	2.80E0	2.40E0	0.00E0	0.00E0
1994	2.47E0	2.90E0	0.00E0	0.00E0
1995	4.20E0	3.30E0	3.58E2	0.00E0
1996	2.75E0	2.11E0	3.60E2	0.00E0
1997	2.70E0	2.24E0	2.90E2	0.00E0
1998	2.75E0	2.33E0	2.68E2	0.00E0
1999	2.48E0	2.17E0	5.49E2	0.00E0
2000	2.66E0	1.99E0	5.04E2	0.00E0
2001	2.48E0	2.19E0	6.98E2	0.00E0
2002	2.47E0	2.08E0	5.64E2	0.00E0
2003	1.81E0	1.52E0	3.51E2	0.00E0
2004	1.68E0	1.29E0	4.61E2	0.00E0
2005	1.74E0	1.30E0	7.35E2	0.00E0
2006	1.75E0	1.80E0	1.46E3	0.00E0
2007	1.81E0	1.76E0	1.48E3	0.00E0
2008	2.40E0	1.87E0	1.52E3	2.26E2
2009	1.90E0	1.81E0	1.03E3	1.86E2

0.00E0 = no detectable measurements

3.3 SURFACE WATER

In 2009, 39 surface water samples were analyzed for gamma emitting radionuclides, 26 at the two indicator locations and 13 at the control location. Analyses for H-3 were performed on 12 samples, eight at indicator locations and four at the control location.



No detectable gamma activity was found in surface water samples in 2009 and has not been detected since 1988. Tritium was detected in all of the eight indicator composite samples taken in 2009. Tritium was detected in one of the control location composite samples in 2009.

Figure 3.3 shows tritium highest annual mean indicator and control location concentrations. Table 3.3 gives indicator and control location highest annual means since 1979 for tritium.

There was an increase in surface water tritium in 2006 due to silica removal from the spent fuel pools. This resulted in additional water volume being released from the plant. Controlled tritium releases to Lake Norman have continued in an effort to balance the plant inventory. In addition, an extreme drought during the second half of 2007 and much of 2008 affecting the Catawba River Basin resulted in less dilution volume available in Lake Norman.

Figure 3.3

There is no reporting level for tritium in surface water

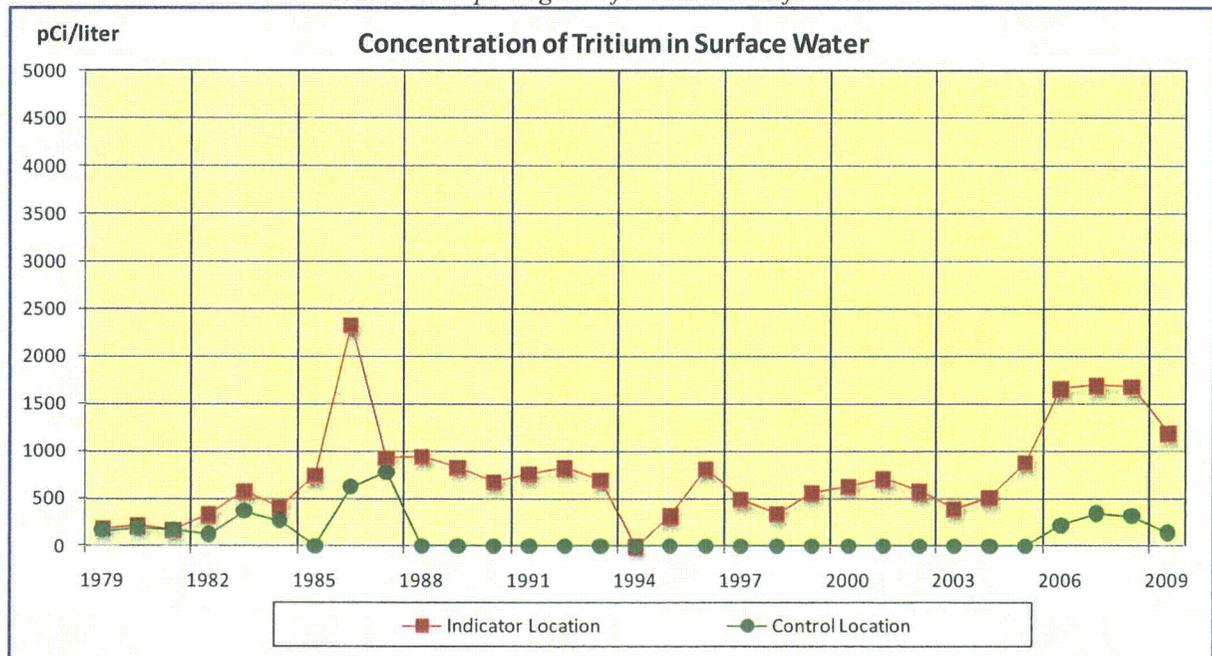


Table 3.3 Mean Concentrations of Tritium in Surface Water

YEAR	H-3 Indicator (pCi/l)	H-3 Control (pCi/l)
1979	1.85E2	1.66E2
1980	2.13E2	1.93E2
1981	1.75E2	1.70E2
1982	3.30E2	1.23E2
1983	5.75E2	3.67E2
1984	4.10E2	2.65E2
1985	7.33E2	0.00E0
1986	2.33E3	6.13E2
1987	9.20E2	7.70E2
1988	9.40E2	0.00E0
1989	8.22E2	0.00E0
1990	6.77E2	0.00E0
1991	7.53E2	0.00E0
1992	8.13E2	0.00E0
1993	6.85E2	0.00E0
1994	0.00E0	0.00E0
1995	3.15E2	0.00E0
1996	8.08E2	0.00E0
1997	4.85E2	0.00E0
1998	3.40E2	0.00E0
1999	5.60E2	0.00E0
2000	6.22E2	0.00E0
2001	6.98E2	0.00E0
2002	5.65E2	0.00E0
2003	3.91E2	0.00E0
2004	5.04E2	0.00E0
2005	8.74E2	0.00E0
2006	1.65E3	2.19E2
2007	1.68E3	3.42E2
2008	1.67E3	3.13E2
2009	1.18E3	1.41E2

0.00E0 = no detectable measurements

3.4 MILK

In 2009, 26 milk samples were analyzed for low level I-131 and other gamma emitting radionuclides. One control location was sampled. No indicator dairies were identified by the 2009 land use census.

No detectable activity was found in milk samples in 2009 other than naturally-occurring K-40. Cs-137 has not been detected in milk samples since 1990 and all other radionuclides have not been detected since 1987.

Table 3.4 gives indicator location highest annual means and control means since 1979 for Cs-137. Since no activity was detected in 2009, no reporting levels were approached.

Table 3.4 Mean Concentrations of Cs-137 in Milk

YEAR	Cs-137 Indicator (pCi/l)	Cs-137 Control (pCi/l)
1979	2.48E1	6.04E0
1980	1.72E1	4.13E0
1981	2.04E1	4.15E0
1982	1.21E1	5.20E0
1983	2.01E1	2.82E0
1984	1.48E1	2.56E0
1985	1.42E1	2.72E0
1986	3.74E0	3.45E0
1987	5.20E0	8.60E0
1988	3.40E0	2.90E0
1989	6.00E0	5.60E0
1990	5.30E0	2.60E0
1991	0.00E0	0.00E0
1992	0.00E0	0.00E0
1993	0.00E0	0.00E0
1994	0.00E0	0.00E0
1995	0.00E0	0.00E0
1996	0.00E0	0.00E0
1997	0.00E0	0.00E0
1998	0.00E0	0.00E0
1999	0.00E0	0.00E0
2000	0.00E0	0.00E0
2001	0.00E0	0.00E0
2002	0.00E0	0.00E0
2003	0.00E0	0.00E0
2004	0.00E0	0.00E0
2005	0.00E0	0.00E0
2006	0.00E0	0.00E0
2007	0.00E0	0.00E0
2008	0.00E0	0.00E0
2009	0.00E0	0.00E0

0.00E0 = no detectable measurements

3.5 BROADLEAF VEGETATION

In 2009, 48 broadleaf vegetation samples were analyzed, 36 at the three indicator locations and twelve at the control location.

The control location was relocated during 2008 (reference 6.14). There were no gamma emitting radionuclides identified in any indicator location or control location broadleaf vegetation samples during 2009.

No airborne Cs-137 has been released from the plant since 1998. Cs-137 attributable to past nuclear weapons testing is known to exist in many environmental media at low and highly variable levels.

Table 3.5 gives indicator and control location highest annual means since 1979 for Cs-137. Since no activity was detected in 2009, no reporting levels were approached.

Table 3.5 Mean Concentrations of Cs-137 in Broadleaf Vegetation

YEAR	Cs-137 Indicator (pCi/kg)	Cs-137 Control (pCi/kg)
1979	2.19E1	1.93E1
1980	2.30E1	1.92E1
1981	3.04E1	2.02E1
1982	2.46E1	1.22E1
1983	9.07E0	7.85E0
1984	1.02E1	1.05E1
1985	8.05E0	2.37E-2
1986	4.03E1	1.27E1
1987	2.20E1	1.70E1
1988	3.90E1	3.40E1
1989	9.60E1	0.00E0
1990	4.00E1	0.00E0
1991	3.30E1	0.00E0
1992	4.90E1	0.00E0
1993	1.60E1	0.00E0
1994	0.00E0	0.00E0
1995	0.00E0	0.00E0
1996	0.00E0	0.00E0
1997	0.00E0	0.00E0
1998	0.00E0	2.69E1
1999	0.00E0	0.00E0
2000	0.00E0	0.00E0
2001	0.00E0	0.00E0
2002	0.00E0	0.00E0
2003	0.00E0	0.00E0
2004	0.00E0	0.00E0
2005	0.00E0	0.00E0
2006	2.98E1	0.00E0
2007	1.34E1	0.00E0
2008	0.00E0	0.00E0
2009	0.00E0	0.00E0

0.00E0 = no detectable measurements

3.6 FOOD PRODUCTS

In 2009, 12 food products (crops) samples were analyzed, all at one indicator location. There is no control location for this media.

No detectable activity has been detected in this media since 1987. Table 3.6 shows Cs-137 indicator highest annual means with preoperational data. Since no activity was detected in 2009, no reporting levels were approached.

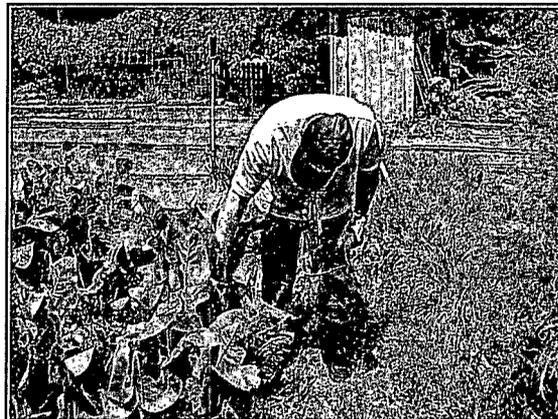


Table 3.6 Mean Concentrations of Cs-137 in Food Products

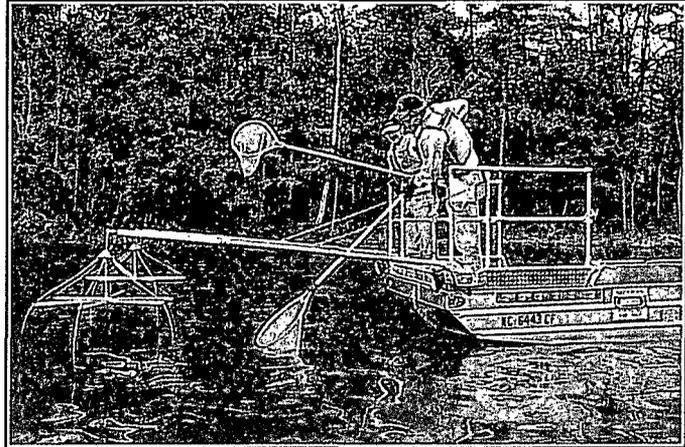
YEAR	Cs-137 Indicator (pCi/kg)
1979	2.19E1
1980	2.30E1
1981	3.04E1
1982	2.46E1
1983	9.07E0
1984	8.45E0
1985	7.99E0
1986	2.15E1
1987	2.90E1
1988	0.00E0
1989	0.00E0
1990	0.00E0
1991	0.00E0
1992	0.00E0
1993	0.00E0
1994	0.00E0
1995	0.00E0
1996	0.00E0
1997	0.00E0
1998	0.00E0
1999	0.00E0
2000	0.00E0
2001	0.00E0
2002	0.00E0
2003	0.00E0
2004	0.00E0
2005	0.00E0
2006	0.00E0
2007	0.00E0
2008	0.00E0
2009	0.00E0

0.00E0 = no detectable measurements

3.7 FISH

In 2009, 12 fish samples were analyzed for gamma emitting radionuclides, six at the indicator location and six at the control location.

Figure 3.7-1 shows Cs-137 highest annual mean indicator and control location concentrations with comparisons to 5% of the reporting level. Figure 3.7-2 shows Co-60 highest annual mean indicator and control location concentrations also with comparisons to 5% of the reporting level. Table 3.7 gives indicator location highest annual means since 1980 for all radionuclides detected since the analysis change in 1988.



Co-58 activity was not detected in 2009 in any of the indicator or control samples. Cs-137 activity was detected in one of the six indicator samples taken at Location 129 with a mean concentration of 17.6 pCi/kg, which is 1.76% of the reporting level. Cs-137 was detected in one of the six control samples.

All other radionuclides not shown in the table have demonstrated no detectable activity since 1986.

Figure 3.7-1

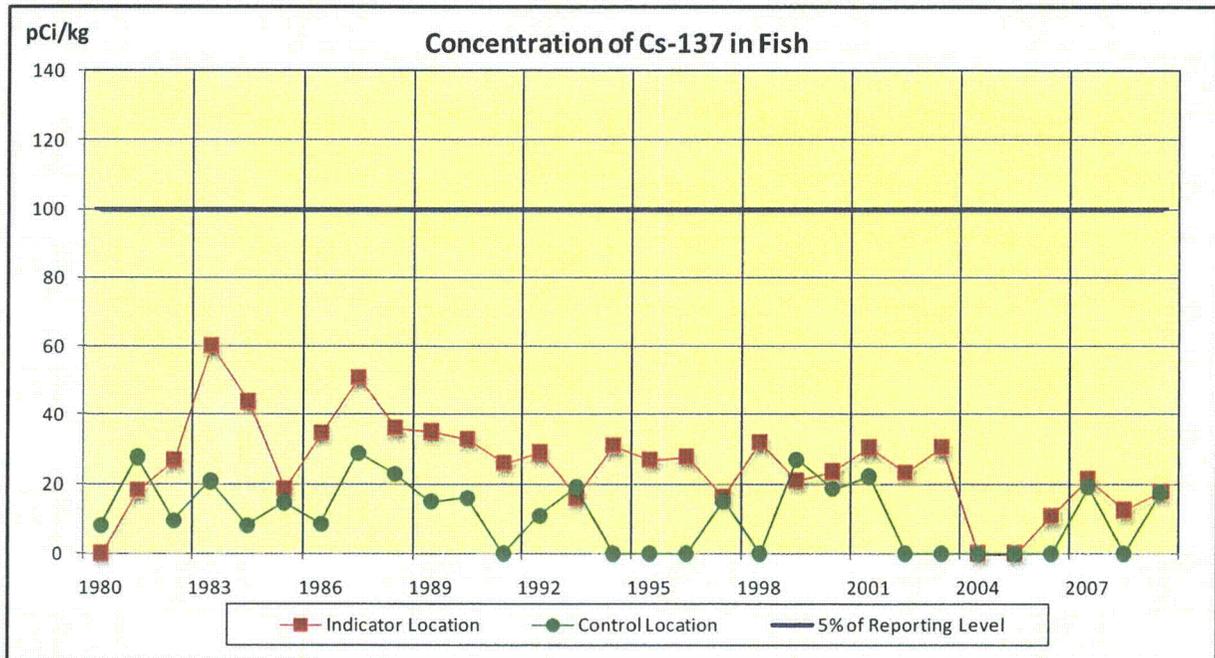


Figure 3.7-2

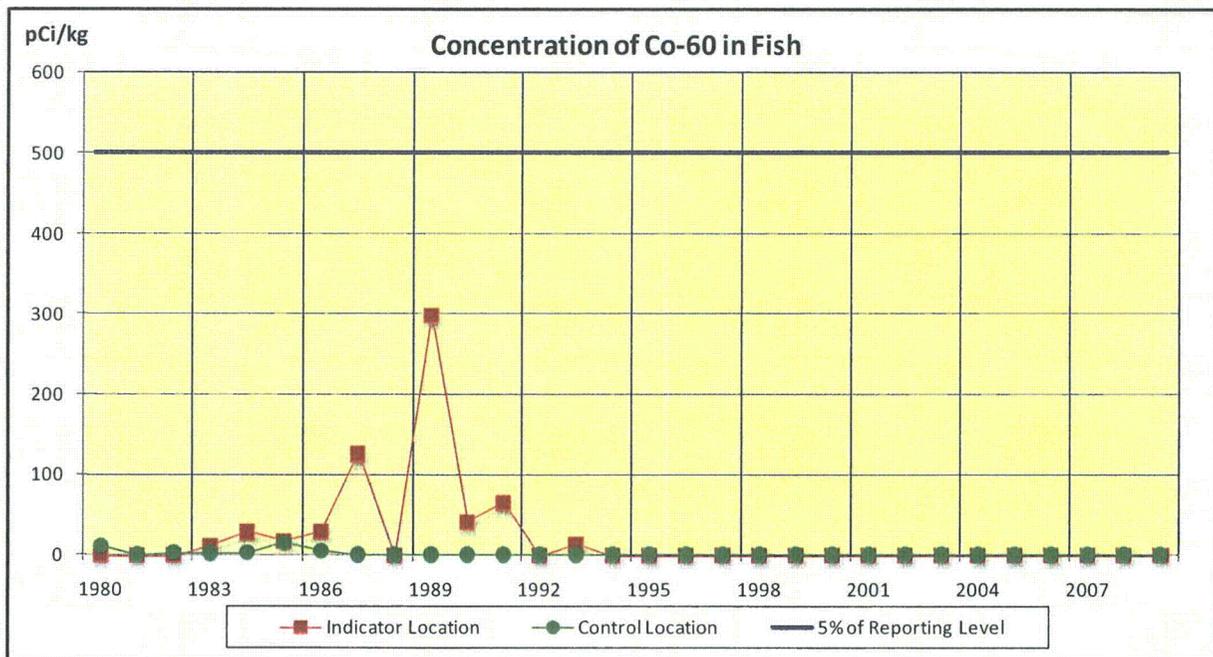


Table 3.7 Mean Concentrations of Radionuclides in Fish (pCi/kg)

YEAR	Mn-54 Indicator	Co-58 Indicator	Co-60 Indicator	Cs-134 Indicator	Cs-137 Indicator
1980	-1.97E1	8.36E0	-2.25E1	-2.70E1	-4.13E0
1981	-2.71E0	-2.98E0	-2.65E0	-1.99E0	1.80E1
1982	-3.83E0	8.16E0	-4.34E-1	-8.22E-1	2.69E1
1983	-2.60E0	2.60E1	1.11E1	-1.32E0	6.03E1
1984	3.61E0	1.45E2	2.82E1	3.11E1	4.38E1
1985	2.53E-1	7.19E0	1.72E1	-1.56E0	1.86E1
1986	1.03E0	3.17E1	2.96E1	1.67E1	3.49E1
1987	0.00E0	2.71E2	1.25E2	2.60E1	5.10E1
1988	1.20E1	7.70E1	0.00E0	2.70E1	3.60E1
1989	9.00E1	4.05E2	2.99E2	1.10E1	3.50E1
1990	0.00E0	5.60E1	4.10E1	0.00E0	3.30E1
1991	6.20E0	1.40E1	6.50E1	5.90E0	2.60E1
1992	0.00E0	0.00E0	0.00E0	0.00E0	2.90E1
1993	0.00E0	8.20E1	1.30E1	0.00E0	1.60E1
1994	0.00E0	0.00E0	0.00E0	0.00E0	3.10E1
1995	0.00E0	0.00E0	0.00E0	0.00E0	2.70E1
1996	0.00E0	0.00E0	0.00E0	0.00E0	2.78E1
1997	0.00E0	0.00E0	0.00E0	0.00E0	1.62E1
1998	0.00E0	0.00E0	0.00E0	0.00E0	3.21E1
1999	0.00E0	3.53E1	0.00E0	0.00E0	2.10E1
2000	0.00E0	4.28E1	0.00E0	0.00E0	2.34E1
2001	0.00E0	1.32E1	0.00E0	0.00E0	3.04E1
2002	0.00E0	0.00E0	0.00E0	0.00E0	2.33E1
2003	0.00E0	0.00E0	0.00E0	0.00E0	3.05E1
2004	0.00E0	0.00E0	0.00E0	0.00E0	0.00E0
2005	0.00E0	0.00E0	0.00E0	0.00E0	0.00E0
2006	0.00E0	0.00E0	0.00E0	0.00E0	1.08E1
2007	0.00E0	0.00E0	0.00E0	0.00E0	2.11E1
2008	0.00E0	0.00E0	0.00E0	0.00E0	1.24E1
2009	0.00E0	0.00E0	0.00E0	0.00E0	1.76E1

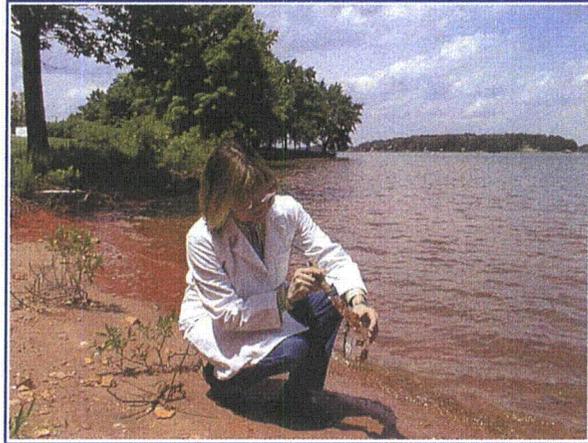
0.00E0 = no detectable measurements

All negative values have been replaced with zeros for calculational purposes

3.8 SHORELINE SEDIMENT

In 2009, six shoreline sediment samples were analyzed, four from two indicator locations and two at the control location.

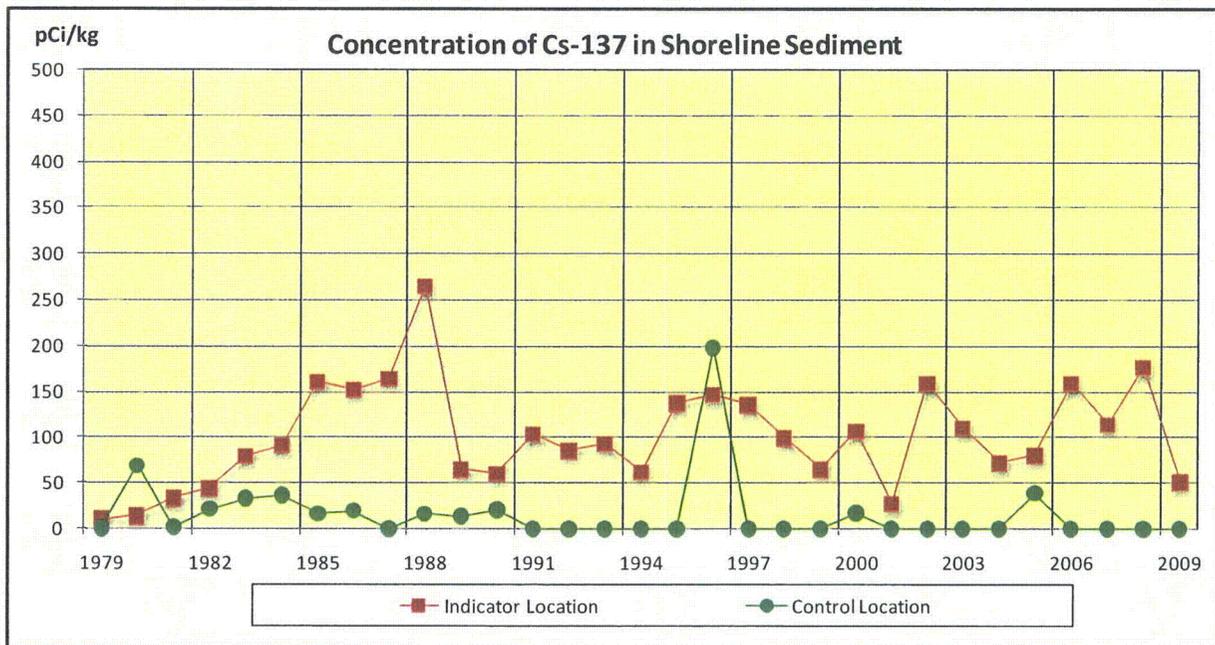
Figure 3.8-1 shows Cs-137 highest annual mean indicator and control location concentrations since 1979. Figure 3.8-2 shows Co-60 highest annual mean indicator and control location concentrations since 1979.



Cs-137 activity was detected in two of the four indicator samples taken. The shoreline sediment location with the highest annual mean was Location 130 with a mean concentration of 50.8 pCi/kg. Cs-137 was not detected in any control location samples.

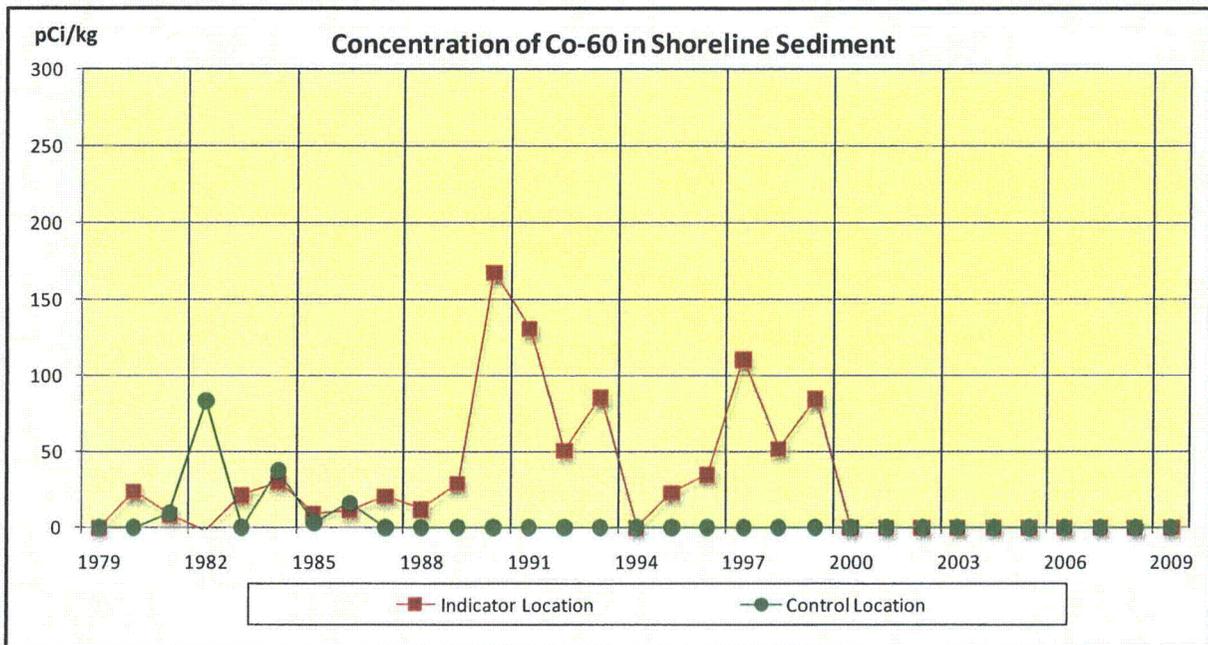
Table 3.8 gives indicator location highest annual means since 1979 for all radionuclides detected since the analysis change in 1988. There is no reporting level for shoreline sediment.

Figure 3.8-1



There is no reporting level for Cs-137 in shoreline sediment

Figure 3.8-2



There is no reporting level for Co-60 in shoreline sediment

Table 3.8 Mean Concentrations of Radionuclides in Shoreline Sediment (pCi/kg)

YEAR	Mn-54 Indicator	Co-58 Indicator	Co-60 Indicator	Cs-134 Indicator	Cs-137 Indicator
1979	-1.07E1	2.25E1	-6.50E0	0.00E0	1.20E1
1980	1.06E1	-8.74E0	2.36E1	-3.53E0	1.44E1
1981	2.13E1	1.20E1	8.21E0	3.97E1	3.36E1
1982	5.38E1	1.66E1	-1.69E0	7.67E1	4.40E1
1983	4.40E0	3.43E1	2.12E1	7.65E1	8.02E1
1984	1.19E1	7.11E1	3.04E1	3.34E1	9.13E1
1985	4.77E0	1.46E1	9.20E0	2.02E1	1.61E2
1986	1.37E1	1.02E1	1.16E1	6.35E1	1.53E2
1987	0.00E0	1.06E2	2.10E1	4.20E1	1.65E2
1988	6.50E0	9.20E1	1.20E1	9.10E0	2.66E2
1989	2.90E1	3.80E1	2.90E1	5.30E1	6.50E1
1990	3.80E1	2.70E1	1.68E2	0.00E0	6.10E1
1991	2.80E1	5.30E1	1.31E2	0.00E0	1.03E2
1992	9.40E0	0.00E0	5.10E1	9.20E0	8.60E1
1993	0.00E0	2.20E1	8.60E1	0.00E0	9.30E1
1994	4.10E1	0.00E0	0.00E0	0.00E0	8.00E1
1995	1.70E1	0.00E0	2.30E1	0.00E0	1.38E2
1996	2.90E1	1.78E1	3.50E1	0.00E0	1.47E2
1997	0.00E0	0.00E0	1.11E2	3.10E1	1.36E2
1998	0.00E0	0.00E0	5.21E1	0.00E0	9.97E1
1999	0.00E0	2.47E1	8.49E1	0.00E0	6.51E1
2000	0.00E0	3.04E1	0.00E0	0.00E0	1.08E2

Table 3.8 continued

YEAR	Mn-54 Indicator	Co-58 Indicator	Co-60 Indicator	Cs-134 Indicator	Cs-137 Indicator
2001	0.00E0	0.00E0	0.00E0	0.00E0	2.77E1
2002	2.24E1	0.00E0	0.00E0	0.00E0	1.59E2
2003	0.00E0	0.00E0	0.00E0	0.00E0	1.11E2
2004	0.00E0	0.00E0	0.00E0	0.00E0	7.17E1
2005	0.00E0	0.00E0	0.00E0	0.00E0	8.08E1
2006	0.00E0	0.00E0	0.00E0	0.00E0	1.59E2
2007	0.00E0	0.00E0	0.00E0	0.00E0	1.14E2
2008	0.00E0	0.00E0	0.00E0	0.00E0	1.77E2
2009	0.00E0	0.00E0	0.00E0	0.00E0	5.08E1

3.9 DIRECT GAMMA RADIATION

In 2009, 162 TLDs were analyzed, 158 at indicator locations, four at the control location. TLDs are collected and analyzed quarterly. The highest annual mean exposure for an indicator location was 108 milliroentgen. The annual mean exposure for the control location was 91.2 milliroentgen.

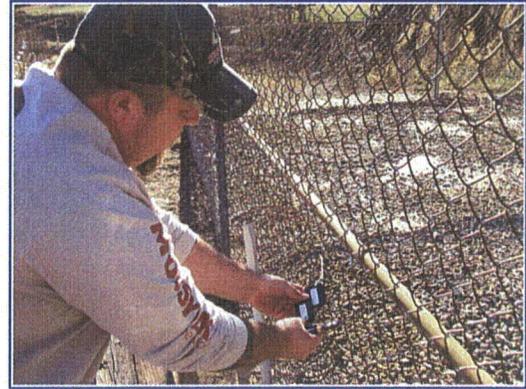
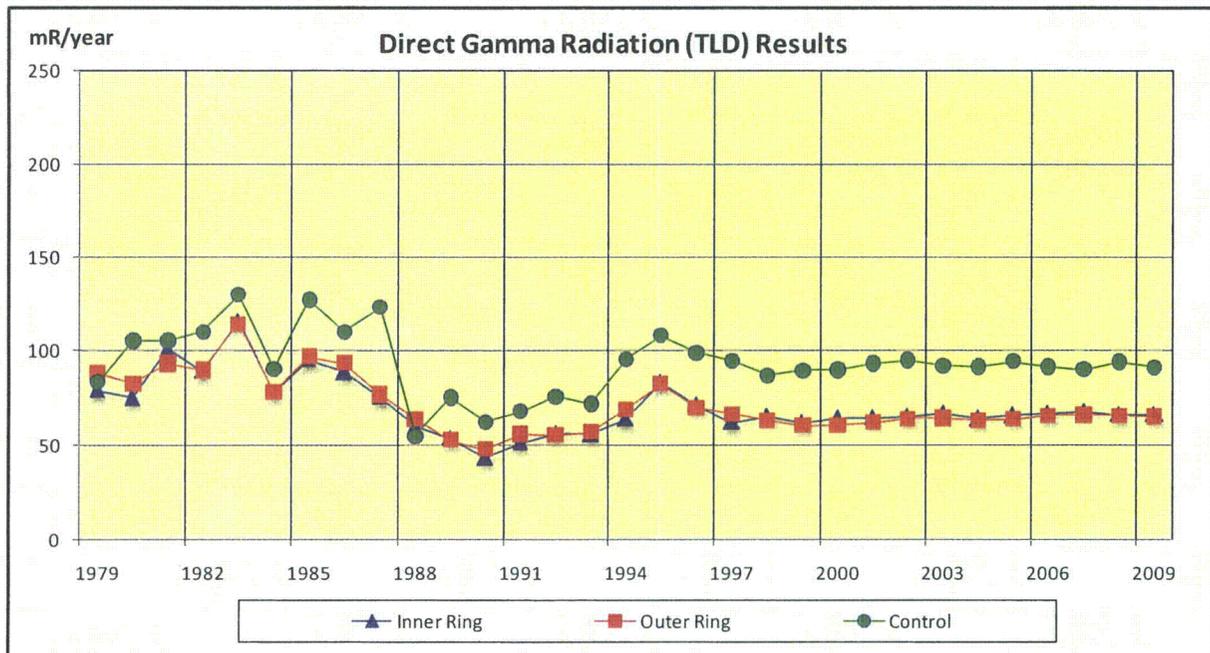


Figure 3.9 and Table 3.9 show TLD inner ring (site boundary), outer ring (4-5 miles), and control location annual averages in milliroentgen per year. Preoperational data and ten year rolling averages are also given. As shown in the graph, inner and outer ring averages historically compare closely, with control data somewhat higher. Inner and outer ring averages comprise a number of data points with the control average representing only one location.

The calculated total body dose from gaseous effluents for 2009 was 1.98E-1 millirem, which is 0.30% of the average inner ring TLD values. Therefore, it can be concluded that discharges from the plant had very little impact on the measured TLD values.

Figure 3.9



There is no reporting level for Direct Radiation (TLD)

Table 3.9 Direct Gamma Radiation (TLD) Results

YEAR	Inner Ring Average (mR/yr)	Outer Ring Average (mR/yr)	Control (mR/yr)
1979	7.91E1	8.82E1	8.32E1
1980	7.54E1*	8.29E1*	1.05E2
1981	1.01E2	9.31E1	1.05E2
1982	8.95E1	8.97E1	1.10E2
1983	1.16E2	1.14E2	1.30E2
1984	7.85E1	7.83E1	9.02E1
1985	9.54E1	9.69E1	1.27E2
1986	8.91E1	9.35E1	1.10E2
1987	7.58E1	7.71E1	1.23E2
1988	6.03E1	6.42E1	5.48E1
1989	5.37E1	5.30E1	7.55E1
1990	4.34E1	4.78E1	6.25E1
1991	5.14E1	5.59E1	6.80E1
1992	5.65E1	5.55E1	7.60E1
1993	5.61E1	5.71E1	7.20E1
1994	6.40E1	6.93E1	9.55E1
1995	8.36E1	8.25E1	1.08E2
1996	7.18E1	7.02E1	9.88E1
1997	6.22E1	6.68E1	9.45E1
1998	6.59E1	6.32E1	8.69E1
1999	6.23E1	6.05E1	8.96E1
2000	6.50E1	6.08E1	8.97E1
2001	6.51E1	6.22E1	9.33E1
2002	6.57E1	6.43E1	9.48E1
2003	6.74E1	6.45E1	9.20E1
2004	6.46E1	6.33E1	9.16E1
2005	6.62E1	6.34E1	9.44E1
2006	6.75E1	6.58E1	9.17E1
2007	6.84E1	6.60E1	9.00E1
2008	6.69E1	6.58E1	9.14E1
Average (1999 – 2008)	6.59E1	6.37E1	9.19E1
2009	6.67E1	6.53E1	9.12E1

* Values are based on two quarters due to change in TLD locations.

3.10 LAND USE CENSUS

The land use census was conducted May 18 and May 19, 2009 as required by SLC 16.11.14. Table 3.10 summarizes census results. A map indicating identified locations is shown in Figure 3.10.

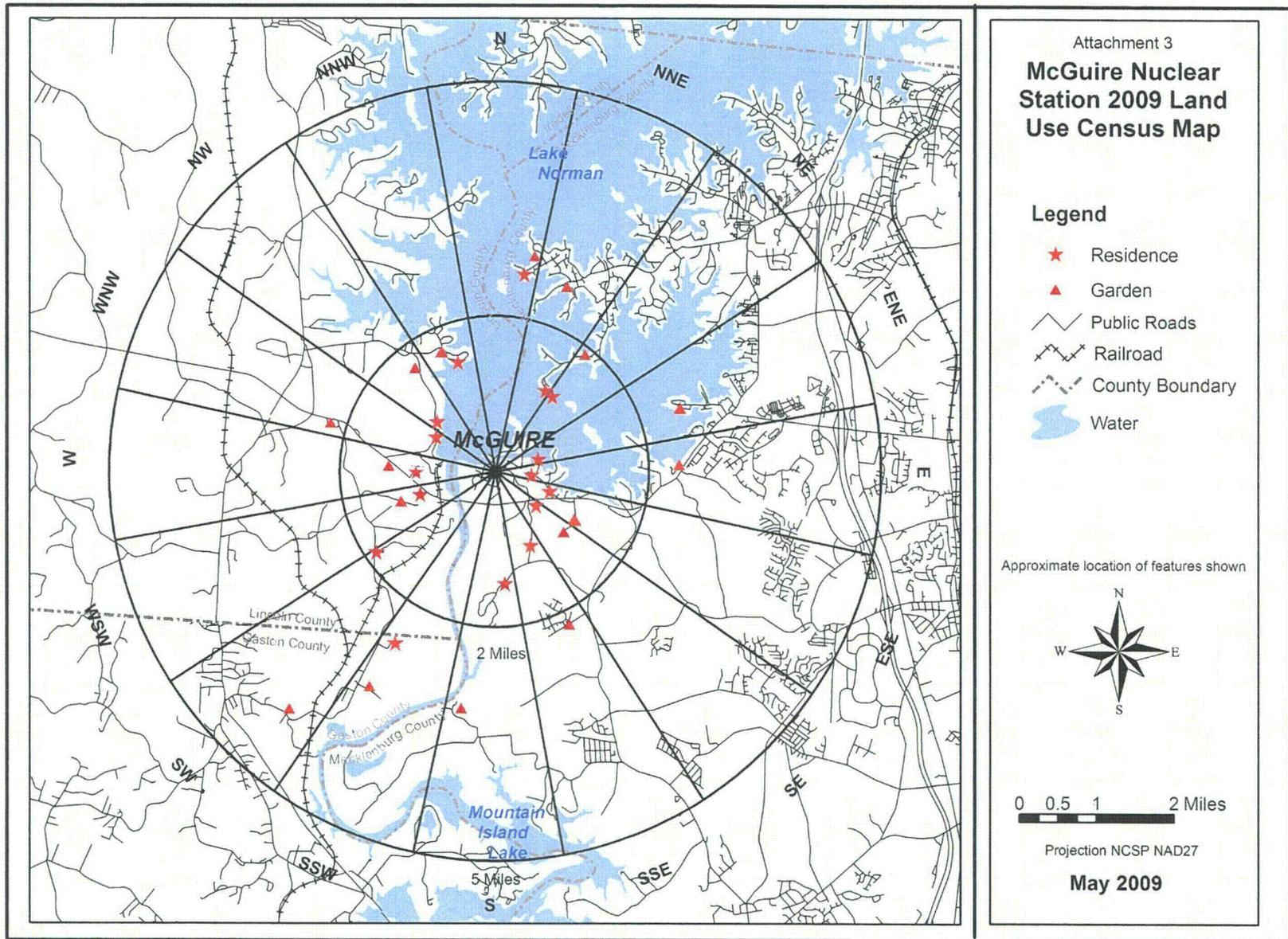
During the 2009 census, no new residences, irrigated gardens (superior to existing gardens) or milk locations were identified. The nearest residence is located in the East sector at 0.48 miles. No environmental program changes were required as a result of the 2009 land use census.

Table 3.10 McGuire 2009 Land Use Census Results

Sector	Distance (Miles)	Sector	Distance (Miles)		
N	Nearest Residence	2.53	S	Nearest Residence	1.45
	Nearest Garden (irrigated)	2.79		Nearest Garden	3.12
	Nearest Milk Animal	-		Nearest Milk Animal	-
NNE	Nearest Residence	1.23	SSW	Nearest Residence	2.56
	Nearest Garden (irrigated)	2.53		Nearest Garden	3.15
	Nearest Milk Animal	-		Nearest Milk Animal	-
NE	Nearest Residence	1.21	SW	Nearest Residence	1.85
	Nearest Garden	1.80		Nearest Garden	4.00
	Nearest Milk Animal	-		Nearest Milk Animal	-
ENE	Nearest Residence	0.57	WSW	Nearest Residence	1.01
	Nearest Garden	2.54		Nearest Garden	1.33
	Nearest Milk Animal	-		Nearest Milk Animal	-
E	Nearest Residence	0.48	W	Nearest Residence	1.15
	Nearest Garden	2.10		Nearest Garden	1.23
	Nearest Milk Animal	-		Nearest Milk Animal	-
ESE	Nearest Residence	0.65	WNW	Nearest Residence	0.88
	Nearest Garden	1.20		Nearest Garden	2.15
	Nearest Milk Animal	-		Nearest Milk Animal	-
SE	Nearest Residence	0.67	NW	Nearest Residence	0.95
	Nearest Garden	1.18		Nearest Garden	1.68
	Nearest Milk Animal	-		Nearest Milk Animal	-
SSE	Nearest Residence	1.06	NNW	Nearest Residence	1.48
	Nearest Garden	2.18		Nearest Garden (irrigated)	1.69
	Nearest Milk Animal	-		Nearest Milk Animal	-

“-“ indicates no occurrences within the 5 mile radius

Figure 3.10



4.0 EVALUATION OF DOSE

4.1 DOSE FROM ENVIRONMENTAL MEASUREMENTS

Annual doses to maximum exposed individuals were estimated based on measured concentrations of radionuclides in 2009 MNS REMP samples. The primary purpose of estimating doses based on sample results is to allow comparison to effluent program dose estimates.

Doses based on sample results were calculated using the methodology and data presented in NRC Regulatory Guide 1.109. Measured radionuclide concentrations, averaged over the entire year for a specific radionuclide, indicator location and sample type, were used to calculate REMP-based doses. Where applicable, average background concentration at the corresponding control location was subtracted. Regulatory Guide 1.109 consumption rates for the maximum exposed individual were used in the calculations. When the guide listed "NO DATA" as the dose factor for a given radionuclide and organ, a dose factor of zero was assumed.

Maximum dose estimates (Highest Annual Mean Concentration) based on drinking water, fish, and shoreline sediment sample results are reported in Table 4.1-A. The individual critical population and pathway dose calculations are reported in Table 4.1-B.

REMP-based dose estimates are not reported for airborne radioiodine or milk sample types because no radionuclides other than naturally occurring K-40 and Be-7 were detected in the samples. Dose estimates are not reported for surface water because sampled surface water is not considered to be a potable drinking water source although surface water tritium concentrations are used in calculating doses from fish. Exposure estimates based upon REMP TLD results are discussed in Section 3.9.

The maximum environmental organ dose estimate for any single sample type (other than direct radiation from gaseous effluents) collected during 2009 was $8.73E-2$ mrem to the maximum exposed child liver, total body, thyroid, kidney, lung, and GI-LLI from the consumption of drinking water.

4.2 ESTIMATED DOSE FROM RELEASES

Throughout the year, dose estimates were calculated based on actual 2008 liquid and gaseous effluent release data. Effluent-based dose estimates were calculated using the RETDAS computer program which employs methodology and data presented in NRC Regulatory Guide 1.109. These doses are shown in Table 4.1-A along with the corresponding REMP-based dose estimates. Summaries of RETDAS dose calculations are reported in the Annual Radioactive Effluent Release Report (reference 6.6).

The effluent-based liquid release doses are summations of the dose contributions from the drinking water, fish, and shoreline pathways. The effluent-based gaseous release doses report noble gas exposure separately from iodine, particulate, and tritium exposure. For noble gas exposure there is no critical age group; as the maximum exposed individuals are assumed to receive the same doses, regardless of their age group. For iodine, particulate, and tritium exposure the effluent-based gaseous release doses are summations of the dose contributors from ground/plane, inhalation, milk and vegetation pathways.

4.3 COMPARISON OF DOSES

The environmental and effluent dose estimates given in Table 4.1-A agree reasonably well. The similarity of the doses indicate that the radioactivity levels in the environment do not differ significantly from those expected based on effluent measurements and modeling of the environmental exposure pathways. This indicates that effluent program dose estimates are both valid and reasonably conservative.

There are some differences in how effluent and environmental doses are calculated that affect the comparison. Doses calculated from environmental data are conservative because they are based on a mean that includes only samples with a net positive activity versus a mean that includes all sample results (i.e. zero results are not included in the mean). Also, airborne tritium is not measured in environmental samples but is used to calculate effluent doses.

In calculations based on liquid release pathways, drinking water and fish consumption were the predominant dose pathways based on environmental and effluent data. The maximum total organ dose based on 2009 environmental sample results was 8.86E-2 mrem to the child liver, total body, thyroid, kidney, lung, and GI-LLI. The maximum total organ dose of 1.76E-1 mrem for liquid effluent-based estimates was to the child liver.

In calculations based on gaseous release pathways, inhalation was the predominant dose pathway for effluent samples. The maximum organ dose for gaseous effluent estimates was 1.98E-1 mrem to the teen liver, total body, thyroid, kidney, lung, and GI-LLI. No radioactivity was detected from gaseous pathways in environmental samples; therefore, there is no calculated dose.

Noble gas samples are not collected as part of the REMP, preventing an analogous comparison of effluent-based noble gas exposure estimates.

The doses calculated do not exceed the 40CFR190 dose commitment limits for members of the public. Doses to members of the public attributable to the operation of MNS are being maintained well within regulatory limits.

TABLE 4.1-A

**MCGUIRE NUCLEAR STATION
2009 ENVIRONMENTAL AND EFFLUENT DOSE COMPARISON**

LIQUID RELEASE PATHWAY

Organ	Environmental or Effluent Data	Critical Age ⁽¹⁾	Critical Pathway ⁽²⁾	Location	Maximum Dose ⁽³⁾ (mrem)
Skin	Environmental	Teen	Shoreline Sediment	130 (0.52 mi SW)	1.33E-04
Skin	Effluent	Teen	Shoreline Sediment	Discharge Pt.	6.67E-04
Bone	Environmental	-	-	-	0.00E+00
Bone	Effluent	Child	Fish	Discharge Pt.	2.81E-03
Liver	Environmental	Child	Drinking Water	101 (3.31 mi E)	8.86E-02
Liver	Effluent	Child	Drinking Water	3.31 mi E	1.76E-01
T. Body	Environmental	Child	Drinking Water	101 (3.31 mi E)	8.86E-02
T. Body	Effluent	Child	Drinking Water	3.31 mi E	1.73E-01
Thyroid	Environmental	Child	Drinking Water	101 (3.31 mi E)	8.86E-02
Thyroid	Effluent	Child	Drinking Water	3.31 mi E	1.73E-01
Kidney	Environmental	Child	Drinking Water	101 (3.31 mi E)	8.86E-02
Kidney	Effluent	Child	Drinking Water	3.31 mi E	1.74E-01
Lung	Environmental	Child	Drinking Water	101 (3.31 mi E)	8.86E-02
Lung	Effluent	Child	Drinking Water	3.31 mi E	1.73E-01
GI-LLI	Environmental	Child	Drinking Water	101 (3.31 mi E)	8.86E-02
GI-LLI	Effluent	Child	Drinking Water	3.31 mi E	1.75E-01

(1) Critical Age is the highest total dose (all pathways) to an age group.

(2) Critical Pathway is the highest individual dose within the identified Critical Age group.

(3) Maximum dose is a summation of the fish, drinking water and shoreline sediment pathways.

GASEOUS RELEASE PATHWAY**IODINE, PARTICULATE, and TRITIUM**

Organ	Environmental or Effluent Data	Critical Age ⁽¹⁾	Critical Pathway ⁽²⁾	Location	Maximum Dose ⁽³⁾ (mrem)
Skin	Environmental	-	-	-	0.00E+00
Skin	Effluent	All	Ground Plane	0.5 mi. ENE	7.18E-06
Bone	Environmental	-	-	-	0.00E+00
Bone	Effluent	Child	Ground Plane	0.5 mi. ENE	6.96E-06
Liver	Environmental	-	-	-	0.00E+00
Liver	Effluent	Teen	Inhalation	0.5 mi. ENE	1.98E-01
T. Body	Environmental	-	-	-	0.00E+00
T. Body	Effluent	Teen	Inhalation	0.5 mi. ENE	1.98E-01
Thyroid	Environmental	-	-	-	0.00E+00
Thyroid	Effluent	Teen	Inhalation	0.5 mi. ENE	1.98E-01
Kidney	Environmental	-	-	-	0.00E+00
Kidney	Effluent	Teen	Inhalation	0.5 mi. ENE	1.98E-01
Lung	Environmental	-	-	-	0.00E+00
Lung	Effluent	Teen	Inhalation	0.5 mi. ENE	1.98E-01
GI-LLI	Environmental	-	-	-	0.00E+00
GI-LLI	Effluent	Teen	Inhalation	0.5 mi. ENE	1.98E-01

(1) Critical Age is the highest total dose (all pathways) to an age group.

(2) Critical Pathway is the highest individual dose within the identified Critical Age group.

(3) Maximum dose is a summation of the ground/plane, inhalation, milk and vegetation pathways.

NOBLE GAS

Air Dose	Environmental or Effluent Data	Critical Age	Critical Pathway	Location	Maximum Dose (mrad)
Beta	Environmental	-	-	-	Not Sampled
Beta	Effluent	N/A	Noble Gas	0.5 mi. NNE	6.97E-02
Gamma	Environmental	-	-	-	Not Sampled
Gamma	Effluent	N/A	Noble Gas	0.5 mi. NNE	2.51E-02

TABLE 4.1-B

Maximum Individual Dose for 2009 based on Environmental Measurements (mrem) for McGuire Nuclear Station

Age	Sample Medium	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Skin
Infant	Airborne	0.00E+00							
	Drinking Water	0.00E+00	8.57E-02	8.57E-02	8.57E-02	8.57E-02	8.57E-02	8.57E-02	0.00E+00
	Milk	0.00E+00							
	<u>TOTAL</u>	0.00E+00	8.57E-02	8.57E-02	8.57E-02	8.57E-02	8.57E-02	8.57E-02	0.00E+00
Child	Airborne	0.00E+00							
	Drinking Water	0.00E+00	8.73E-02	8.73E-02	8.73E-02	8.73E-02	8.73E-02	8.73E-02	0.00E+00
	Milk	0.00E+00							
	Broadleaf Vegetation	0.00E+00							
	Fish	0.00E+00	1.31E-03	1.31E-03	1.31E-03	1.31E-03	1.31E-03	1.31E-03	0.00E+00
	<u>Shoreline Sediment</u>	0.00E+00	0.00E+00	2.39E-05	0.00E+00	0.00E+00	0.00E+00	0.00E+00	2.79E-05
<u>TOTAL</u>	0.00E+00	8.86E-02	8.86E-02	8.86E-02	8.86E-02	8.86E-02	8.86E-02	2.79E-05	
Teen	Airborne	0.00E+00							
	Drinking Water	0.00E+00	4.56E-02	4.56E-02	4.56E-02	4.56E-02	4.56E-02	4.56E-02	0.00E+00
	Milk	0.00E+00							
	Broadleaf Vegetation	0.00E+00							
	Fish	0.00E+00	1.59E-03	1.59E-03	1.59E-03	1.59E-03	1.59E-03	1.59E-03	0.00E+00
	<u>Shoreline Sediment</u>	0.00E+00	0.00E+00	1.14E-04	0.00E+00	0.00E+00	0.00E+00	0.00E+00	1.33E-04
<u>TOTAL</u>	0.00E+00	4.72E-02	4.73E-02	4.72E-02	4.72E-02	4.72E-02	4.72E-02	1.33E-04	
Adult	Airborne	0.00E+00							
	Drinking Water	0.00E+00	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02	0.00E+00
	Milk	0.00E+00							
	Broadleaf Vegetation	0.00E+00							
	Fish	0.00E+00	2.06E-03	2.06E-03	2.06E-03	2.06E-03	2.06E-03	2.06E-03	0.00E+00
	<u>Shoreline Sediment</u>	0.00E+00	0.00E+00	2.05E-05	0.00E+00	0.00E+00	0.00E+00	0.00E+00	2.39E-05
<u>TOTAL</u>	0.00E+00	6.67E-02	6.67E-02	6.67E-02	6.67E-02	6.67E-02	6.67E-02	2.39E-05	

Note: Dose tables are provided for sample media displaying positive nuclide occurrence.

*McGuire Nuclear Station
Dose from Drinking Water Pathway for 2009 Data
Maximum Exposed Infant*

Infant Dose from Drinking Water Pathway (mrem) = Usage (l) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year) = 330 l

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator Location	Water (pCi/l)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	1.99E-05	4.51E-06	NO DATA	4.41E-06	NO DATA	7.31E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	3.60E-06	8.98E-06	NO DATA	NO DATA	NO DATA	8.97E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	3.08E-05	5.38E-05	2.12E-05	NO DATA	NO DATA	1.59E-05	2.57E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	1.08E-05	2.55E-05	NO DATA	NO DATA	NO DATA	2.57E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	1.84E-05	6.31E-05	2.91E-05	NO DATA	3.06E-05	NO DATA	5.33E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	4.20E-08	1.73E-08	1.00E-08	NO DATA	1.24E-08	NO DATA	1.46E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	2.06E-07	5.02E-08	3.56E-08	NO DATA	5.41E-08	NO DATA	2.50E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	3.59E-05	4.23E-05	1.86E-05	1.39E-02	4.94E-05	NO DATA	1.51E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	3.77E-04	7.03E-04	7.10E-05	NO DATA	1.81E-04	7.42E-05	1.91E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	5.22E-04	6.11E-04	4.33E-05	NO DATA	1.64E-04	6.64E-05	1.91E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BaLa-140	1.71E-04	1.71E-07	8.81E-06	NO DATA	4.06E-08	1.05E-07	4.20E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
H-3	NO DATA	3.08E-07	3.08E-07	3.08E-07	3.08E-07	3.08E-07	3.08E-07	101	843	0.00E+00	8.57E-02	8.57E-02	8.57E-02	8.57E-02	8.57E-02	8.57E-02
Dose Commitment (mrem) =										0.00E+00	8.57E-02	8.57E-02	8.57E-02	8.57E-02	8.57E-02	8.57E-02

McGuire Nuclear Station
Dose from Drinking Water Pathway for 2009 Data
Maximum Exposed Child

Child Dose from Drinking Water Pathway (mrem) = Usage (l) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year) = 510 l

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator Location	Water (pCi/l)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	1.07E-05	2.85E-06	NO DATA	3.00E-06	NO DATA	8.98E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	1.80E-06	5.51E-06	NO DATA	NO DATA	NO DATA	1.05E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	1.65E-05	2.67E-05	1.33E-05	NO DATA	NO DATA	7.74E-06	2.78E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	5.29E-06	1.56E-05	NO DATA	NO DATA	NO DATA	2.93E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	1.37E-05	3.65E-05	2.27E-05	NO DATA	2.30E-05	NO DATA	6.41E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	2.25E-08	8.76E-09	6.26E-09	NO DATA	8.23E-09	NO DATA	1.62E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	1.16E-07	2.55E-08	2.27E-08	NO DATA	3.65E-08	NO DATA	2.66E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	1.72E-05	1.73E-05	9.83E-06	5.72E-03	2.84E-05	NO DATA	1.54E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	2.34E-04	3.84E-04	8.10E-05	NO DATA	1.19E-04	4.27E-05	2.07E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	3.27E-04	3.13E-04	4.62E-05	NO DATA	1.02E-04	3.67E-05	1.96E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BaLa-140	8.31E-05	7.28E-08	4.85E-06	NO DATA	2.37E-08	4.34E-08	4.21E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
H-3	NO DATA	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	101	843	0.00E+00	8.73E-02	8.73E-02	8.73E-02	8.73E-02	8.73E-02	8.73E-02
Dose Commitment (mrem) =										0.00E+00	8.73E-02	8.73E-02	8.73E-02	8.73E-02	8.73E-02	8.73E-02

*McGuire Nuclear Station
Dose from Fish Pathway for 2009 Data
Maximum Exposed Child*

Child Dose from Fish Pathway (mrem) = Usage (kg) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/kg)

H-3 Concentration in Fish = Surface Water pCi/l x Bioaccumulation Factor 0.9 pCi/kg per pCi/l = 1040 pCi/l x 0.9 = 936 pCi/kg

Usage (intake in one year) = 6.9 kg

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator Location	Fish (pCi/kg)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	1.07E-05	2.85E-06	NO DATA	3.00E-06	NO DATA	8.98E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	1.80E-06	5.51E-06	NO DATA	NO DATA	NO DATA	1.05E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	1.65E-05	2.67E-05	1.33E-05	NO DATA	NO DATA	7.74E-06	2.78E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
C0-60	NO DATA	5.29E-06	1.56E-05	NO DATA	NO DATA	NO DATA	2.93E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	1.37E-05	3.65E-05	2.27E-05	NO DATA	2.30E-05	NO DATA	6.41E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	2.34E-04	3.84E-04	8.10E-05	NO DATA	1.19E-04	4.27E-05	2.07E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	3.27E-04	3.13E-04	4.62E-05	NO DATA	1.02E-04	3.67E-05	1.96E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
H-3	NO DATA	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	128	936	0.00E+00	1.31E-03	1.31E-03	1.31E-03	1.31E-03	1.31E-03	1.31E-03
Dose Commitment (mrem) =										0.00E+00	1.31E-03	1.31E-03	1.31E-03	1.31E-03	1.31E-03	1.31E-03

McGuire Nuclear Station
Dose from Shoreline Sediment Pathway for 2009 Data
Maximum Exposed Child

Shoreline Recreation = 14 hr (in one year)
 Shore Width Factor = 0.3 (lake shore - location 129)
 Shore Width Factor = 0.2 (river shoreline - location 130)
 Sediment Surface Mass = 40 kg/m²

Child Dose from Shoreline Sediment Pathway (mrem) = Shoreline Recreation (hr) x External Dose Factor (mrem/hr per pCi/m²) x Shore Width Factor x Sediment Surface Mass (kg/m²) x Sediment Concentration (pCi/kg)

Radionuclide	External Dose Factor Standing on Contaminated Ground		Indicator Location	Sediment Concentration (pCi/kg)	Highest Annual Net Mean Concentration		Dose	
	(mrem/hr per pCi/m ²) T. Body	(mrem/hr per pCi/m ²) Skin			(mrem) T. Body	(mrem) Skin		
Mn-54	5.80E-09	6.80E-09	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	
Co-58	7.00E-09	8.20E-09	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	
Co-60	1.70E-08	2.00E-08	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	
Cs-134	1.20E-08	1.40E-08	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	
Cs-137	4.20E-09	4.90E-09	130	50.8	2.39E-05	2.79E-05		
Dose Commitment (mrem) =					2.39E-05	2.79E-05		

*McGuire Nuclear Station
Dose from Drinking Water Pathway for 2009 Data
Maximum Exposed Teen*

Teen Dose from Drinking Water Pathway (mrem) = Usage (l) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year) = 510 l

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator Location	Water (pCi/l)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	5.90E-06	1.17E-06	NO DATA	1.76E-06	NO DATA	1.21E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	9.72E-07	2.24E-06	NO DATA	NO DATA	NO DATA	1.34E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	5.87E-06	1.37E-05	5.29E-06	NO DATA	NO DATA	4.32E-06	3.24E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	2.81E-06	6.33E-06	NO DATA	NO DATA	NO DATA	3.66E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	5.76E-06	2.00E-05	9.33E-06	NO DATA	1.28E-05	NO DATA	8.47E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	8.22E-09	4.56E-09	2.51E-09	NO DATA	4.42E-09	NO DATA	1.95E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	4.12E-08	1.30E-08	8.94E-09	NO DATA	1.91E-08	NO DATA	3.00E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	5.85E-06	8.19E-06	4.40E-06	2.39E-03	1.41E-05	NO DATA	1.62E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	8.37E-05	1.97E-04	9.14E-05	NO DATA	6.26E-05	2.39E-05	2.45E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	1.12E-04	1.49E-04	5.19E-05	NO DATA	5.07E-05	1.97E-05	2.12E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BaLa-140	2.84E-05	3.48E-08	1.83E-06	NO DATA	1.18E-08	2.34E-08	4.38E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
H-3	NO DATA	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	101	843	0.00E+00	4.56E-02	4.56E-02	4.56E-02	4.56E-02	4.56E-02	4.56E-02
Dose Commitment (mrem)=										0.00E+00	4.56E-02	4.56E-02	4.56E-02	4.56E-02	4.56E-02	4.56E-02

*McGuire Nuclear Station
Dose from Fish Pathway for 2009 Data
Maximum Exposed Teen*

Teen Dose from Fish Pathway (mrem) = Usage (kg) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/kg)

H-3 Concentration in Fish = Surface Water pCi/l x Bioaccumulation Factor 0.9 pCi/kg per pCi/l = 1357 pCi/l x 0.9 = 1221 pCi/kg

Usage (intake in one year) = 16 kg

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Location	(pCi/kg)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	5.90E-06	1.17E-06	NO DATA	1.76E-06	NO DATA	1.21E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	9.72E-07	2.24E-06	NO DATA	NO DATA	NO DATA	1.34E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	5.87E-06	1.37E-05	5.29E-06	NO DATA	NO DATA	4.32E-06	3.24E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	2.81E-06	6.33E-06	NO DATA	NO DATA	NO DATA	3.66E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	5.76E-06	2.00E-05	9.33E-06	NO DATA	1.28E-05	NO DATA	8.47E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	8.37E-05	1.97E-04	9.14E-05	NO DATA	6.26E-05	2.39E-05	2.45E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	1.12E-04	1.49E-04	5.19E-05	NO DATA	5.07E-05	1.97E-05	2.12E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
H-3	NO DATA	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	128	936	0.00E+00	1.59E-03	1.59E-03	1.59E-03	1.59E-03	1.59E-03	1.59E-03
Dose Commitment (mrem) =										0.00E+00	1.59E-03	1.59E-03	1.59E-03	1.59E-03	1.59E-03	1.59E-03

McGuire Nuclear Station
Dose from Shoreline Sediment Pathway for 2009 Data
Maximum Exposed Teen

Shoreline Recreation = 67 hr (in one year)
 Shore Width Factor = 0.3 (lake shore - location 129)
 Shore Width Factor = 0.2 (river shoreline - location 130)
 Sediment Surface Mass = 40 kg/m²

Teen Dose from Shoreline Sediment Pathway (mrem) = Shoreline Recreation (hr) x External Dose Factor (mrem/hr per pCi/m²) x Shore Width Factor x Sediment Surface Mass (kg/m²) x Sediment Concentration (pCi/kg)

Radionuclide	External Dose Factor Standing on Contaminated Ground (mrem/hr per pCi/m ²)		Indicator Location	Sediment (pCi/kg)	Dose (mrem)	
	T. Body	Skin			T. Body	Skin
Mn-54	5.80E-09	6.80E-09	ALL	0.00	0.00E+00	0.00E+00
Co-58	7.00E-09	8.20E-09	ALL	0.00	0.00E+00	0.00E+00
Co-60	1.70E-08	2.00E-08	ALL	0.00	0.00E+00	0.00E+00
Cs-134	1.20E-08	1.40E-08	ALL	0.00	0.00E+00	0.00E+00
Cs-137	4.20E-09	4.90E-09	130	50.8	1.14E-04	1.33E-04
Dose Commitment (mrem) =					1.14E-04	1.33E-04

McGuire Nuclear Station
Dose from Drinking Water Pathway for 2009 Data
Maximum Exposed Adult

Adult Dose from Drinking Water Pathway (mrem) = Usage (l) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year) = 730 l

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator Location	Water (pCi/l)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	4.57E-06	8.72E-07	NO DATA	1.36E-06	NO DATA	1.40E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	7.45E-07	1.67E-06	NO DATA	NO DATA	NO DATA	1.51E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	4.34E-06	1.02E-05	3.91E-06	NO DATA	NO DATA	2.85E-06	3.40E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	2.14E-06	4.72E-06	NO DATA	NO DATA	NO DATA	4.02E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	4.84E-06	1.54E-05	6.96E-06	NO DATA	1.03E-05	NO DATA	9.70E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	6.22E-09	3.46E-09	1.86E-09	NO DATA	3.42E-09	NO DATA	2.10E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	3.04E-08	9.75E-09	6.60E-09	NO DATA	1.53E-08	NO DATA	3.09E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	4.16E-06	5.95E-06	3.41E-06	1.95E-03	1.02E-05	NO DATA	1.57E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	6.22E-05	1.48E-04	1.21E-04	NO DATA	4.79E-05	1.59E-05	2.59E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	7.97E-05	1.09E-04	7.14E-05	NO DATA	3.70E-05	1.23E-05	2.11E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BaLa-140	2.03E-05	2.55E-08	1.33E-06	NO DATA	8.67E-09	1.46E-08	4.18E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
H-3	NO DATA	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	101	843	0.00E+00	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02
Dose Commitment (mrem) =										0.00E+00	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02	6.46E-02

McGuire Nuclear Station
Dose from Fish Pathway for 2009 Data
Maximum Exposed Adult

Adult Dose from Fish Pathway (mrem) = Usage (kg) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/kg)

H-3 Concentration in Fish = Surface Water pCi/l x Bioaccumulation Factor 0.9 pCi/kg per pCi/l = 1040 pCi/l x 0.9 = 936 pCi/kg

Usage (intake in one year) = 21 kg

Radionuclide	<u>Ingestion Dose Factor</u>							<u>Highest Annual Net Mean Concentration</u>		<u>Dose (mrem)</u>						
	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Location	(pCi/kg)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	4.57E-06	8.72E-07	NO DATA	1.36E-06	NO DATA	1.40E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	7.45E-07	1.67E-06	NO DATA	NO DATA	NO DATA	1.51E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	4.34E-06	1.02E-05	3.91E-06	NO DATA	NO DATA	2.85E-06	3.40E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	2.14E-06	4.72E-06	NO DATA	NO DATA	NO DATA	4.02E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	4.84E-06	1.54E-05	6.96E-06	NO DATA	1.03E-05	NO DATA	9.70E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	6.22E-05	1.48E-04	1.21E-04	NO DATA	4.79E-05	1.59E-05	2.59E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	7.97E-05	1.09E-04	7.14E-05	NO DATA	3.70E-05	1.23E-05	2.11E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
H-3	NO DATA	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	128	936	0.00E+00	2.06E-03	2.06E-03	2.06E-03	2.06E-03	2.06E-03	2.06E-03
Dose Commitment (mrem) =										0.00E+00	2.06E-03	2.06E-03	2.06E-03	2.06E-03	2.06E-03	2.06E-03

McGuire Nuclear Station
Dose from Shoreline Sediment Pathway for 2009 Data
Maximum Exposed Adult

Shoreline Recreation = 12 hr (in one year)
 Shore Width Factor = 0.3 (lake shore - location 129)
 Shore Width Factor = 0.2 (river shoreline - location 130)
 Sediment Surface Mass = 40 kg/m²

Adult Dose from Shoreline Sediment Pathway (mrem) = Shoreline Recreation (hr) x External Dose Factor (mrem/hr per pCi/m²) x Shore Width Factor x Sediment Surface Mass (kg/m²) x Sediment Concentration (pCi/kg)

Radionuclide	External Dose Factor Standing on Contaminated Ground (mrem/hr per pCi/m ²)		Highest Annual Net Mean Concentration		Dose (mrem)	
	T. Body	Skin	Indicator Location	Sediment (pCi/kg)	T. Body	Skin
Mn-54	5.80E-09	6.80E-09	ALL	0.00	0.00E+00	0.00E+00
Co-58	7.00E-09	8.20E-09	ALL	0.00	0.00E+00	0.00E+00
Co-60	1.70E-08	2.00E-08	ALL	0.00	0.00E+00	0.00E+00
Cs-134	1.20E-08	1.40E-08	ALL	0.00	0.00E+00	0.00E+00
Cs-137	4.20E-09	4.90E-09	130	50.8	2.05E-05	2.39E-05
Dose Commitment (mrem) =					2.05E-05	2.39E-05

5.0 QUALITY ASSURANCE

5.1 SAMPLE COLLECTION

EnRad Laboratories, Fisheries, and Aquatic Ecology performed the environmental sample collections as specified by approved sample collection procedures.

5.2 SAMPLE ANALYSIS

EnRad Laboratories performed the environmental sample analyses as specified by approved analysis procedures. EnRad Laboratories is located in Huntersville, North Carolina, at Duke Energy Corporation's Environmental Center.



Duke Energy Corporation's
Environmental Center

5.3 DOSIMETRY ANALYSIS

The Radiation Dosimetry and Records group performed environmental dosimetry measurements as specified by approved dosimetry analysis procedures.

5.4 LABORATORY EQUIPMENT QUALITY ASSURANCE

5.4.1 DAILY QUALITY CONTROL

EnRad Laboratories has an internal quality assurance program which monitors each type of instrumentation for reliability and accuracy. Daily quality control checks ensure that instruments are in proper working order and these checks are used to monitor instrument performance.

5.4.2 CALIBRATION VERIFICATION

National Institute of Standards and Technology (NIST) standards that represent counting geometries are analyzed as unknowns at various frequencies ranging from weekly to annually to verify that efficiency calibrations are valid. The frequency is dependent upon instrument use and performance. Investigations are performed and documented should calibration verification data fall out of limits.

5.4.3 BATCH PROCESSING

Method quality control samples are analyzed with sample analyses that are processed in batches. These include gross beta in drinking water and all tritium analyses.

5.5 DUKE ENERGY INTERCOMPARISON PROGRAM

EnRad Laboratories participated in the Duke Energy Nuclear Generation Department Intercomparison Program during 2009. Interlaboratory cross-check standards, including, Marinelli beakers, air filters, air cartridges, gross beta on smears, and tritium in water samples were analyzed at various times of the year. A summary of the EnRad Laboratory program results for 2009 is documented in Table 5.0-A.

5.6 ERA PROFICIENCY TESTING

EnRad Laboratories performed method proficiency testing through a program administered by Environmental Resource Associates (ERA) of Arvada, CO. ERA supplied requested method proficiency samples for analysis and nuclide concentration determination. ERA reported proficiency test results to the North Carolina Department of Health and Human Services, North Carolina Public Health Drinking Water Laboratory Certification Program. A summary of these proficiency test data for 2009 is documented in Table 5.0-B.

5.7 DUKE ENERGY AUDITS

The McGuire Radiation Protection Section was not audited by the Quality Assurance Group in 2009, but was audited in 2008 (reference 6.16). There were some TLD laboratory procedural enhancements identified as a result of the 2008 audit.

EnRad Laboratories was not audited by the Quality Assurance Group in 2009, but was audited in 2008 (reference 6.17). There were some REMP recommendations as a result of the 2008 audit.

5.8 U.S. NUCLEAR REGULATORY COMMISSION INSPECTIONS

The McGuire Nuclear Station Radiological Environmental Monitoring Program was audited by the NRC in 2009 (reference 6.12). There were no findings identified by the audit.

5.9 STATE OF NORTH CAROLINA INTERCOMPARISON PROGRAM

EnRad Laboratories routinely participates with the State of North Carolina Department of Environmental Health and Natural Resources (DEHNR) in an intercomparison

program. EnRad Laboratories sends air, surface water, milk, crops, vegetation, sediment, and fish samples which have been collected to the State of North Carolina Radiation Protection Section.

5.10 TLD INTERCOMPARISON PROGRAM

5.10.1 NUCLEAR TECHNOLOGY SERVICES INTERCOMPARISON PROGRAM

Radiation Dosimetry and Records participates in a quarterly TLD intercomparison program administered by Nuclear Technology Services, Inc. of Roswell, GA. Nuclear Technology Services irradiates environmental dosimeters quarterly and sends them to the Radiation Dosimetry and Records group for analysis of the unknown estimated delivered exposure. A summary of the Nuclear Technology Services Intercomparison Report is documented in Table 5.0-C.

5.10.2 INTERNAL CROSSCHECK (DUKE ENERGY)

Radiation Dosimetry and Records participates in a quarterly TLD intracomparison program administered internally by the Dosimetry Lab. The Dosimetry Lab Staff irradiates environmental dosimeters quarterly and submits them for analysis of the unknown estimated delivered exposure. A summary of the Internal Cross Check (Duke Energy) Result is documented in Table 5.0-C.

TABLE 5.0-A

DUKE ENERGY INTERLABORATORY COMPARISON PROGRAM

2009 CROSS-CHECK RESULTS FOR ENRAD LABORATORIES

Cross-Check samples are normally analyzed a minimum of three times. A status of "3 Pass" indicates that all three analyses yielded results within the designated acceptance range. A status of "1 Pass" indicates that one analysis of the cross check was performed

If applicable, footnote explanations are included following this table.

Gamma in Water 3.5 liters

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
3/5/2009	Q091GWL	Cr-51	1.54 - 2.73 E5	2.05 E5	2.06 E5	3 Pass
		Mn-54	4.68 - 8.30 E4	6.24 E4	6.47 E4	3 Pass
		Co-58	4.88 - 8.65 E4	6.50 E4	6.52 E4	3 Pass
		Fe-59	4.40 - 7.80 E4	5.87 E4	6.15 E4	3 Pass
		Co-60	5.08 - 9.02 E4	6.78 E4	7.05 E4	3 Pass
		Zn-65	5.79 - 10.26 E4	7.72 E4	8.01 E4	3 Pass
		Cs-134	3.38 - 5.99 E4	4.50 E4	4.13 E4	3 Pass
		Cs-137	3.96 - 7.03 E4	5.28 E4	5.05 E4	3 Pass
		Ce-141	4.55 - 8.07 E4	6.07 E4	5.99 E4	3 Pass
7/29/2009	Q093GWS	Co-57	1.22 - 2.16 E5	1.62 E5	1.73 E5	2 Pass
		Co-60	1.88 - 3.33 E5	2.50 E5	2.65 E5	2 Pass
		Y-88	1.22 - 2.16 E5	1.62 E5	1.63 E5	2 Pass
		Sn-113	1.91 - 3.38 E5	2.54 E5	2.46 E5	2 Pass
		Cs-137	3.28 - 5.81 E5	4.37 E5	4.46 E5	2 Pass

Gamma in Water 1.0 liter

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
3/5/2009	Q091GWL	Cr-51	1.54 - 2.73 E5	2.05 E5	2.04 E5	3 Pass
		Mn-54	4.68 - 8.30 E4	6.24 E4	6.53 E4	3 Pass
		Co-58	4.88 - 8.65 E4	6.50 E4	6.48 E4	3 Pass
		Fe-59	4.40 - 7.80 E4	5.87 E4	6.17 E4	3 Pass
		Co-60	5.08 - 9.02 E4	6.78 E4	6.96 E4	3 Pass
		Zn-65	5.79 - 10.26 E4	7.72 E4	7.98 E4	3 Pass
		Cs-134	3.38 - 5.99 E4	4.50 E4	4.03 E4	3 Pass
		Cs-137	3.96 - 7.03 E4	5.28 E4	5.03 E4	3 Pass
		Ce-141	4.55 - 8.07 E4	6.07 E4	5.94 E4	3 Pass

Gamma in Water 1.0 liter, continued

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
7/29/2009	Q093GWS	Co-57	1.22 - 2.16 E5	1.62 E5	1.64 E5	3 Pass
		Co-60	1.88 - 3.33 E5	2.50 E5	2.59 E5	3 Pass
		Y-88	1.22 - 2.16 E5	1.62 E5	1.58 E5	3 Pass
		Sn-113	1.91 - 3.38 E5	2.54 E5	2.40 E5	3 Pass
		Cs-137	3.28 - 5.81 E5	4.37 E5	4.28 E5	3 Pass

Gamma in Water 0.5 liter

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
3/5/2009	Q091GWL	Cr-51	1.54 - 2.73 E5	2.05 E5	2.05 E5	3 Pass
		Mn-54	4.68 - 8.30 E4	6.24 E4	6.35 E4	3 Pass
		Co-58	4.88 - 8.65 E4	6.50 E4	6.27 E4	3 Pass
		Fe-59	4.40 - 7.80 E4	5.87 E4	6.10 E4	3 Pass
		Co-60	5.08 - 9.02 E4	6.78 E4	6.94 E4	3 Pass
		Zn-65	5.79 - 10.26 E4	7.72 E4	8.01 E4	3 Pass
		Cs-134	3.38 - 5.99 E4	4.50 E4	3.88 E4	3 Pass
		Cs-137	3.96 - 7.03 E4	5.28 E4	4.90 E4	3 Pass
		Ce-141	4.55 - 8.07 E4	6.07 E4	5.80 E4	3 Pass
7/29/2009	Q093GWS	Co-57	1.22 - 2.16 E5	1.62 E5	1.61 E5	3 Pass
		Co-60	1.88 - 3.33 E5	2.50 E5	2.55 E5	3 Pass
		Y-88	1.22 - 2.16 E5	1.62 E5	1.58 E5	3 Pass
		Sn-113	1.91 - 3.38 E5	2.54 E5	2.36 E5	3 Pass
		Cs-137	3.28 - 5.81 E5	4.37 E5	4.27 E5	3 Pass

Gamma in Water 0.25 liter

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
3/5/2009	Q091GWL	Cr-51	1.54 - 2.73 E5	2.05 E5	2.08 E5	3 Pass
		Mn-54	4.68 - 8.30 E4	6.24 E4	6.51 E4	3 Pass
		Co-58	4.88 - 8.65 E4	6.50 E4	6.43 E4	3 Pass
		Fe-59	4.40 - 7.80 E4	5.87 E4	6.15 E4	3 Pass
		Co-60	5.08 - 9.02 E4	6.78 E4	7.13 E4	3 Pass
		Zn-65	5.79 - 10.26 E4	7.72 E4	7.93 E4	3 Pass
		Cs-134	3.38 - 5.99 E4	4.50 E4	4.00 E4	3 Pass
		Cs-137	3.96 - 7.03 E4	5.28 E4	5.07 E4	3 Pass
		Ce-141	4.55 - 8.07 E4	6.07 E4	5.97 E4	3 Pass

Gamma in Water 0.25 liter, continued

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
7/29/2009	Q093GWS	Co-57	1.22 - 2.16 E5	1.62 E5	1.58 E5	2 Pass
		Co-60	1.88 - 3.33 E5	2.50 E5	2.57 E5	2 Pass
		Y-88	1.22 - 2.16 E5	1.62 E5	1.58 E5	2 Pass
		Sn-113	1.91 - 3.38 E5	2.54 E5	2.32 E5	2 Pass
		Cs-137	3.28 - 5.81 E5	4.37 E5	4.13 E5	2 Pass

Gamma in Filter

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi	Reference Value pCi	Mean Reported Value pCi	Cross Check Status
6/18/2009	E6701-37	Cr-51	0.29 - 10.17 E2	1.73 E2	1.61 E2	2 Pass
		Mn-54	4.45 - 7.89 E1	5.93 E1	5.96 E1	2 Pass
		Co-58	2.94 - 5.36 E1	3.97 E1	3.95 E1	2 Pass
		Fe-59	2.06 - 13.54 E1	5.29 E1	4.50 E1	2 Pass
		Co-60	1.01 - 1.80 E2	1.35 E2	1.36 E2	2 Pass
		Zn-65	5.68 - 10.07 E1	7.57 E1	8.01 E1	2 Pass
		Cs-134	5.38 - 9.54 E1	7.17 E1	6.50 E1	2 Pass
		Cs-137	6.23 - 11.04 E1	8.30 E1	7.61 E1	2 Pass
		Ce-141	0.92 - 1.64 E2	1.23 E2	1.30 E2	2 Pass

Iodine in Water

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
7/21/2009	Q093LIW1	I-131	1.99 - 3.53 E2	2.65 E2	2.29 E2	3 Pass
7/21/2009	Q093LIW2	I-131	0.87 - 1.55 E1	1.16 E1	0.89 E1	1/3 Low ⁽¹⁾
7/21/2009	Q093LIW3	I-131	5.09 - 9.03 E2	6.79 E2	5.99 E2	3 Pass

Iodine in Milk

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
3/4/2009	Q091LIM2	I-131	3.30 - 5.85 E3	4.40 E3	2.65 E3	3/3 Low ⁽²⁾
3/4/2009	Q091LIM3	I-131	1.41 - 2.51 E2	1.88 E2	1.38 E2	2/3 Low ⁽³⁾
6/8/2009	Q092LIM1	I-131	3.36 - 5.95 E2	4.48 E2	4.54 E2	3 Pass
6/8/2009	Q092LIM2	I-131	0.86 - 1.53 E3	1.15 E3	1.08 E3	3 Pass
6/8/2009	Q092LIM3	I-131	7.49 - 13.28 E1	9.99 E1	9.61 E1	3 Pass

Iodine on Cartridge

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi	Reference Value pCi	Mean Reported Value pCi	Cross Check Status
6/18/2009	E6702-37	I-131	7.24 - 12.83 E1	9.65 E1	9.53 E1	3 Pass

Beta Air Particulate

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi	Reference Value pCi	Mean Reported Value pCi	Cross Check Status
12/10/2009	E6903-37	Cs-137	1.97 - 3.50 E2	2.63 E2	2.36 E2	3 Pass

Tritium in Water

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
1/28/2009	Q091TWR1	H-3	4.30 - 7.62 E3	5.73 E3	5.54 E3	3 Pass
1/28/2009	Q091TWR2	H-3	0.86 - 1.53 E4	1.15 E4	1.09 E4	3 Pass
7/22/2009	Q093TWS1	H-3	0.88 - 1.56 E4	1.18 E4	1.13 E4	3 Pass
7/22/2009	Q093TWS2	H-3	3.43 - 6.61 E2	4.76 E2	5.10 E2	3 Pass
11/12/2009	Q094TWS1	H-3	5.30 - 9.40 E8	7.07 E8	7.82 E8	3 Pass
11/12/2009	Q094TWS2	H-3	0.77 - 1.36 E5	1.02 E5	0.97 E5	3 Pass

Table 5.0-A Footnote Explanations

- (1) Iodine in Water, Sample ID Q093LIW2, Reference Date 7/21/2009

One of three results was outside of the acceptance range (reference 6.18).

- (2) Iodine in Milk, Sample ID Q091LIM2, Reference Date 3/4/2009

Three results for this cross-check were reported. All three of the reported results trended low and were outside of the acceptance range (reference 6.19).

- (3) Iodine in Milk, Sample ID Q091LIM3, Reference Date 3/4/2009

Three results for this cross-check were reported. Two of the three reported results trended low and were outside of the acceptance range (reference 6.19).

TABLE 5.0-B

ENVIRONMENTAL RESOURCE ASSOCIATES (ERA) QUIK™ RESPONSE PROGRAM

2009 PROFICIENCY TEST RESULTS FOR ENRAD LABORATORIES

ERA LABORATORY CODE: D242401

Proficiency test samples are received, prepared, analyzed, and reported to Environmental Resource Associates as described in the "Quik" Response instruction package within the study period. Proficiency test data are reported to ERA for evaluation. ERA reports proficiency test results to the North Carolina Department of Health and Human Services, North Carolina Public Drinking Water Laboratory Certification Program.

If applicable, footnote explanations are included following this data table.

Gamma Emitters in Water

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Reported Value pCi/l	Proficiency Check Status
4/6/2009	RAD-77*	Ba-133	4.34 - 5.83 E1	5.27 E1	6.18 E1	High ⁽¹⁾
		Cs-134	5.95 - 8.02 E1	7.29 E1	6.86 E1	Pass
		Cs-137	1.51 - 1.87 E2	1.68 E2	1.52 E2	Pass
		Co-60	8.00 - 10.0 E1	8.89 E1	9.56 E1	Pass
		Zn-65	7.60 - 10.1 E1	8.44 E1	9.97 E1	Pass
10/5/2009	RAD-79**	Ba-133	7.83 - 10.2 E1	9.29 E1	9.35 E1	Pass
		Cs-134	6.50 - 8.73 E1	7.94 E1	7.60 E1	Pass
		Cs-137	4.91 - 6.29 E1	5.46 E1	5.51 E1	Pass
		Co-60	1.05 - 1.31 E2	1.17 E2	1.15 E2	Pass
		Zn-65	8.96 - 11.9 E1	9.95 E1	1.11 E2	Pass

Tritium in Water

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Reported Value pCi/l	Proficiency Check Status
4/6/2009	RAD-77*	H-3	1.78 - 2.23 E4	2.03 E4	1.87 E4	Pass
10/5/2009	RAD-79**	H-3	1.43 - 1.80 E4	1.64 E4	1.53 E4	Pass

* ERA study period 4/6/2009 - 5/21/2009, ERA data report issue date 6/3/2009

** ERA study period 10/5/2009 - 11/19/2009, ERA data report issue date 12/4/2009

Table 5.0-B Footnote Explanations

- (1) Gamma Emitters in Water, Sample ID RAD-77, Reference Date 4/6/2009
Reported result for Ba-133 was above the acceptance range limit (reference 6.20).

TABLE 5.0-C

2009 ENVIRONMENTAL DOSIMETER CROSS-CHECK RESULTS

Nuclear Technology Services

1st Quarter 2009						2nd Quarter 2009					
TLD Number	Delivered (mrem)	Reported (mrem)	Bias (% diff)	Pass/Fail Criteria	Pass/Fail	TLD Number	Delivered (mrem)	Reported (mrem)	Bias (% diff)	Pass/Fail Criteria	Pass/Fail
100125	110.1	104.0	5.87	<+/-15%	Pass	102077	106	101.0	4.95	<+/-15%	Pass
100203	109.2	104.0	5.00	<+/-15%	Pass	102243	100	101.0	-0.99	<+/-15%	Pass
101141	105.4	104.0	1.35	<+/-15%	Pass	102244	108	101.0	6.93	<+/-15%	Pass
100210	104.5	104.0	0.48	<+/-15%	Pass	102323	102	101.0	0.99	<+/-15%	Pass
100386	108.6	104.0	4.42	<+/-15%	Pass	102329	101	101.0	0.00	<+/-15%	Pass
Average Bias (B)			3.42			Average Bias (B)			2.38		
Standard Deviation (S)			2.37			Standard Deviation (S)			3.40		
Measure Performance B +S			5.79	<15%	Pass	Measure Performance B +S			5.78	<15%	Pass
3rd Quarter 2009						4th Quarter 2009					
TLD Number	Delivered (mrem)	Reported (mrem)	Bias (% diff)	Pass/Fail Criteria	Pass/Fail	TLD Number	Delivered (mrem)	Reported (mrem)	Bias (% diff)	Pass/Fail Criteria	Pass/Fail
102228	69.5	70.2	-1.00	<+/-15%	Pass	100140	14.0	14.0	0.00	<+/-15%	Pass
102492	71.1	70.2	1.28	<+/-15%	Pass	100358	14.0	14.0	0.00	<+/-15%	Pass
102007	69.9	70.2	-0.43	<+/-15%	Pass	100404	13.0	14.0	-7.14	<+/-15%	Pass
102041	73.4	70.2	4.56	<+/-15%	Pass	100405	14.0	14.0	0.00	<+/-15%	Pass
102078	70.6	70.2	0.57	<+/-15%	Pass	100415	15.0	14.0	7.14	<+/-15%	Pass
Average Bias (B)			1.00			Average Bias (B)			0.00		
Standard Deviation (S)			2.18			Standard Deviation (S)			5.05		
Measure Performance B +S			3.17	<15%	Pass	Measure Performance B +S			5.05	<15%	Pass

Internal Crosscheck (Duke Energy)

1st Quarter 2009						2nd Quarter 2009					
TLD Number	Delivered (mrem)	Reported (mrem)	Bias (% diff)	Pass/Fail Criteria	Pass/Fail	TLD Number	Delivered (mrem)	Reported (mrem)	Bias (% diff)	Pass/Fail Criteria	Pass/Fail
101297	18.3	19.0	-3.95	<+/-15%	Pass	102325	49.1	50.0	-1.78	<+/-15%	Pass
101286	18.5	19.0	-2.58	<+/-15%	Pass	102491	48.7	50.0	-2.68	<+/-15%	Pass
101258	18.2	19.0	-4.26	<+/-15%	Pass	102374	48.4	50.0	-3.26	<+/-15%	Pass
101216	18.4	19.0	-3.42	<+/-15%	Pass	102084	47.6	50.0	-4.88	<+/-15%	Pass
101252	18.1	19.0	-4.74	<+/-15%	Pass	102404	51.3	50.0	2.56	<+/-15%	Pass
101356	18.7	19.0	-1.74	<+/-15%	Pass	102396	47.8	50.0	-4.32	<+/-15%	Pass
101339	18.2	19.0	-4.32	<+/-15%	Pass	102346	48.7	50.0	-2.60	<+/-15%	Pass
101127	17.6	19.0	-7.47	<+/-15%	Pass	102485	48.6	50.0	-2.80	<+/-15%	Pass
101278	19.4	19.0	1.89	<+/-15%	Pass	102059	48.2	50.0	-3.68	<+/-15%	Pass
101305	18.4	19.0	-3.05	<+/-15%	Pass	102263	50.0	50.0	-0.08	<+/-15%	Pass
Average Bias (B)			-3.36			Average Bias (B)			-2.35		
Standard Deviation (S)			2.40			Standard Deviation (S)			2.18		
Measure Performance B +S			5.76	<15%	Pass	Measure Performance B +S			4.53	<15%	Pass
3rd Quarter 2009						4th Quarter 2009					
TLD Number	Delivered (mrem)	Reported (mrem)	Bias (% diff)	Pass/Fail Criteria	Pass/Fail	TLD Number	Delivered (mrem)	Reported (mrem)	Bias (% diff)	Pass/Fail Criteria	Pass/Fail
102400	25.9	27.0	-3.93	<+/-15%	Pass	100955	20.4	22.0	-7.14	<+/-15%	Pass
102486	25.7	27.0	-4.70	<+/-15%	Pass	100050	20.0	22.0	-9.18	<+/-15%	Pass
102402	25.9	27.0	-4.04	<+/-15%	Pass	100885	21.2	22.0	-3.68	<+/-15%	Pass
102406	26.9	27.0	-0.44	<+/-15%	Pass	101409	19.2	22.0	-12.77	<+/-15%	Pass
102435	26.6	27.0	-1.63	<+/-15%	Pass	100389	20.3	22.0	-7.82	<+/-15%	Pass
102442	25.3	27.0	-6.48	<+/-15%	Pass	100401	20.4	22.0	-7.18	<+/-15%	Pass
102436	25.9	27.0	-3.93	<+/-15%	Pass	101383	21.0	22.0	-4.45	<+/-15%	Pass
102440	25.1	27.0	-7.19	<+/-15%	Pass	100551	20.4	22.0	-7.14	<+/-15%	Pass
102479	25.2	27.0	-6.81	<+/-15%	Pass	100748	20.9	22.0	-4.95	<+/-15%	Pass
102384	26.0	27.0	-3.67	<+/-15%	Pass	100263	20.9	22.0	-4.91	<+/-15%	Pass
Average Bias (B)			-4.28			Average Bias (B)			-6.92		
Standard Deviation (S)			2.17			Standard Deviation (S)			2.68		
Measure Performance B +S			6.45	<15%	Pass	Measure Performance B +S			9.61	<15%	Pass

6.0 REFERENCES

- 6.1 McGuire Selected License Commitment Manual
- 6.2 McGuire Technical Specifications
- 6.3 McGuire Updated Final Safety Analysis Report
- 6.4 McGuire Offsite Dose Calculation Manual
- 6.5 McGuire Annual Radiological Environmental Operating Report 1979 - 2008
- 6.6 McGuire Annual Radioactive Effluent Release Report 2008
- 6.7 Probability and Statistics in Engineering and Management Science, Hines and Montgomery, 1969, pages 287-293.
- 6.8 Practical Statistics for the Physical Sciences, Havilcek and Crain, 1988, pages 83-93.
- 6.9 Nuclear Regulatory Commission Regulatory Guide 1.109, Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purposes of Evaluating Compliance with 10CFR50, Appendix I.
- 6.10 EnRad Laboratories Operating Procedures
- 6.11 RETDAS, Radiological Effluent Tracking and Dose Assessment Software, Canberra Version 3.5.1, DPC Revision #4.0
- 6.12 NRC Integrated Inspection Report (50-369/2009004, 50-370/2009004)
- 6.13 Duke Energy Corporation EnRad Laboratory Charcoal Cartridge Study, performed 2001
- 6.14 Problem Investigation Process Database, V 3.2.27, Duke Power Company, G-07-00556
- 6.15 Problem Investigation Process Database V 3.2.27, Duke Power Company, G-04-00134
- 6.16 Radiological Effluents Controls Audit 08-22(INOS)(REC)(MNS)
- 6.17 Radiological Effluent Controls Audit 08-23(INOS)(REC)(NGO)
- 6.18 Problem Investigation Process Database, V 3.2.27, Duke Power Company, G-09-01278
- 6.19 Problem Investigation Process Database, V 3.2.27, Duke Power Company, G-09-00423
- 6.20 Problem Investigation Process Database, V 3.2.27, Duke Power Company, G-09-01295

APPENDIX A

ENVIRONMENTAL SAMPLING
&
ANALYSIS PROCEDURES

APPENDIX A

ENVIRONMENTAL SAMPLING AND ANALYSIS PROCEDURES

Adherence to established procedures for sampling and analysis of all environmental media at McGuire Nuclear Station was required to ensure compliance with Station Selected Licensee Commitments. Analytical procedures were employed to ensure that Selected Licensee Commitments detection capabilities were achieved.

Environmental sampling and analyses were performed by EnRad Laboratories, Dosimetry and Records, and Fisheries and Aquatic Ecology.

This appendix describes the environmental sampling frequencies and analysis procedures by media type.

I. CHANGE OF SAMPLING PROCEDURES

Indicator location 103 (Air Radioiodine, Air Particulate) was added as a replacement for indicator location 192 (Air Radioiodine, Air Particulate), which was removed from the program (reference 6.14).

II. DESCRIPTION OF ANALYSIS PROCEDURES

Gamma spectroscopy analyses are performed using high purity germanium gamma detectors and Canberra analytical software. Designated sample volumes are transferred to appropriate counting geometries and analyzed by gamma spectroscopy. Perishable samples such as fish and broadleaf vegetation are ground to achieve a homogeneous mixture. Soils and sediments are dried, sifted to remove foreign objects (rocks, clams, glass, etc.) then transferred to appropriate counting geometry.

Low-level iodine analyses are performed by passing a designated sample aliquot through a pre-weighed amount of ion exchange resin to remove and concentrate any iodine in the aqueous sample (milk). The resin is then dried, mixed thoroughly, and a net resin weight determined before being transferred to appropriate counting geometry and analyzed by gamma spectroscopy.

Tritium analyses are performed quarterly by using low-level environmental liquid scintillation analysis technique on a Packard 2550 liquid scintillation system or Perkin-Elmer 2900TR liquid scintillation system. Tritium samples are distilled and batch processed with a tritium spike and blank to verify instrument performance and sample preparation technique are acceptable.

Gross beta analysis is performed by concentrating a designated aliquot of sample precipitate and analyzing by Tennelec XLB Series 5 gas-flow proportional counters. Samples are batch processed with a blank to ensure sample contamination has not occurred.

III. CHANGE OF ANALYSIS PROCEDURES

No analysis procedures were changed during 2009.

IV. SAMPLING AND ANALYSIS PROCEDURES

A.1 AIRBORNE PARTICULATE AND RADIOIODINE

Airborne particulate and radioiodine samples at each of seven locations were composited continuously by means of continuous air samplers. Air particulates were collected on a particulate filter and radioiodines were collected in a charcoal cartridge positioned behind the filter in the sampler. The samplers are designed to operate at a constant flow rate (in order to compensate for any filter loading) and are set to sample approximately 2 cubic feet per minute. Filters and cartridges were collected weekly. A separate weekly gamma analysis was performed on each charcoal cartridge and air particulate. A weekly gross beta analysis was performed on each filter. The continuous composite samples were collected from the locations listed below.

Location 102	=	Amity Church Road (9.89 mi. WNW)
Location 103	=	Cottonwood (4.20 mi. NE)
		<i>Programmatic sampling initiated 10/5/2009, reference 6.14</i>
Location 120	=	Site Boundary (0.46 mi. NNE)
Location 121	=	Site Boundary (0.47 mi. NE)
Location 125	=	Site Boundary (0.38 mi. SW)
Location 133	=	Cornelius (6.23 mi. ENE)
Location 192	=	Peninsula (2.84 mi. NNE)
		<i>Programmatic sampling terminated 10/5/2009, reference 6.14</i>
Location 195	=	Fishing Access Road (0.19 mi. N)

A.2 DRINKING WATER

Monthly composite samples were collected. A gross beta and gamma analysis was performed on monthly composites. Tritium analysis was performed on the quarterly composites. The composites were collected monthly from the locations listed below.

Location 101	=	North Mecklenburg Water Treatment Facility (3.31 mi E)
Location 119	=	Mt. Holly Municipal Water Supply (7.40 mi. SSW)
Location 132	=	Charlotte Municipal Water Supply (11.1 mi. SSE)
Location 136	=	Mooresville Municipal Water Supply (12.7 mi. NNE)
Location 194	=	East Lincoln County Water Supply (6.73 mi. NNW)

A.3 SURFACE WATER

Monthly composite samples were collected. A gamma analysis was performed on the monthly composites. Tritium analysis was performed on the quarterly composites sample. The composites were collected monthly from the locations listed below.

- Location 128 = Discharge Canal Bridge (0.45 mi. NE)
- Location 131 = Cowans Ford Dam (0.64 mi. WNW)
- Location 135 = Plant Marshall Intake Canal (11.9 mi. N)

A.4 MILK

Biweekly grab samples were collected at one location. A gamma and low-level Iodine-131 analysis was performed on each sample. The biweekly grab samples were collected from the location listed below.

- Location 141 = Lynch Dairy - Cows (14.8 mi. WNW)

A.5 BROADLEAF VEGETATION

Monthly samples were collected as available and a gamma analysis was performed on each sample. The samples were collected from the locations listed below.

- Location 102 = Amity Church Road (9.89 mi. WNW)
- Location 120 = Site Boundary (0.46 mi. NNE)
- Location 125 = Site Boundary (0.38 mi. SW)
- Location 193 = Site Boundary (0.19 mi. N)

A.6 FOOD PRODUCTS

Samples were collected monthly when available during the harvest season and a gamma analysis was performed on each. The samples were collected at the location listed below.

- Location 188 = 5 mile radius Gardens (2.79 mi NNE)

A.7 FISH

Semiannual samples were collected and a gamma analysis was performed on the edible portions of each sample. Boney fish (i.e. Sunfish) were prepared whole minus the head and tail portions. The samples were collected from the locations listed below.

Location 129 = Discharge Canal Entrance to Lake Norman (0.51 mi. ENE)
Location 137 = Pinnacle Access Area (12.0 mi. N)

A.8 SHORELINE SEDIMENT

Semiannual samples were collected and a gamma analysis was performed on each following the drying and removal of rocks and clams. The samples were collected from the locations listed below.

Location 129 = Discharge Canal Entrance to Lake Norman (0.51 mi. ENE)
Location 130 = Highway 73 Bridge Downstream (0.52 mi. SW)
Location 137 = Pinnacle Access Area (12.0 mi. N)

A.9 DIRECT GAMMA RADIATION (TLD)

Thermoluminescent dosimeters (TLD) were collected quarterly at forty-one locations. A gamma exposure rate was determined for each TLD. TLD locations are listed in Table 2.1-B. The TLDs were placed as indicated below.

- * An inner ring of 14 TLDs at the site boundary, one in each available meteorological sector. The site boundary locations in the N and NNW sectors are over water; however, two special interest TLD's were placed in these sectors inside the site boundary in March, 1991.
- * An outer ring of 16 TLDs, one in each meteorological sector in the 6 to 8 kilometer range.
- * The remaining TLDs were placed in special interest areas such as population centers, residential areas, schools, and control locations.

A.10 ANNUAL LAND USE CENSUS

An annual Land Use Census was conducted to identify within a distance of 8 kilometers (5.0 miles) from the station, the nearest location from the site boundary in each of the sixteen meteorological sectors, the following:

- * The Nearest Residence
- * The Nearest Garden greater than 50 square meters or 500 square feet
- * The Nearest Milk-giving Animal (cow, goat, etc.)

The census was conducted during the growing season from 5/18 to 5/19/2009. Results are shown in Table 3.10. No changes were made to the sampling procedures during 2009 as a result of the 2009 census.

In the environmental program, the air deposition parameters (D/Q) are used to determine air, broadleaf vegetation and milk sampling locations. McGuire's sectors with the three highest values did not change in 2009.

V. GLOBAL POSITIONING SYSTEM (GPS) ANALYSIS

The McGuire site centerline used for GPS measurements was referenced from the McGuire Nuclear Station Updated Final Safety Analysis Report (UFSAR), section 2.1.1, Site Location. Waypoint coordinates used for MNS GPS measurements were latitude 35°-25'-59"N and longitude 80°-56'-55"W. Maps and tables were generated using North American Datum (NAD) 27. Data normally reflect accuracy to within 2 to 5 meters from point of measurement. GPS field measurements were taken as close as possible to the item of interest. Distances for the locations are displayed using three significant figures.

APPENDIX B

**RADIOLOGICAL
ENVIRONMENTAL MONITORING
PROGRAM**

SUMMARY OF RESULTS

2009

Environmental Radiological Monitoring Program Summary

Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

Report Period: 01-JAN-2009 to 31-DEC-2009

Medium or Pathway Sampled Unit of Measurement	Type and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Fraction) Range	Location with Highest Annual Mean Name, Distance, Direction		Control Location Mean (Fraction) Range	No. of Non-Routine Report Meas.
				Location Code	Mean (Fraction) Range		
Air Particulate (pCi/m3)				102 (9.89 mi WNW)			
BETA	363	1.00E-02	1.74E-2 (311/311) 6.81E-3 - 2.75E-2	125 (0.38 mi SW)	1.79E-2 (51/51) 7.64E-3 - 2.71E-2	1.76E-2 (52/52) 8.99E-3 - 2.72E-2	0
CS-134	363	5.00E-02	0.00 (0/311) 0.00 - 0.00		0.00 (0/51) 0.00 - 0.00	0.00 (0/52) 0.00 - 0.00	0
CS-137	363	6.00E-02	0.00 (0/311) 0.00 - 0.00		0.00 (0/51) 0.00 - 0.00	0.00 (0/52) 0.00 - 0.00	0
I-131	363	7.00E-02	0.00 (0/311) 0.00 - 0.00		0.00 (0/51) 0.00 - 0.00	0.00 (0/52) 0.00 - 0.00	0

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

Environmental Radiological Monitoring Program Summary

Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

Report Period: 01-JAN-2009 to 31-DEC-2009

Medium or Pathway Sampled Unit of Measurement	Type and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Fraction) Range	Location with Highest Annual Mean Name, Distance, Direction		Control Location Mean (Fraction) Range	No. of Non-Routine Report Meas.
				Location Code	Mean (Fraction) Range		
Air Radioiodine (pCi/m3)				102 (9.89 mi WNW)			
	CS-134 363	5.00E-02	0.00 (0/311) 0.00 - 0.00		0.00 (0/52) 0.00 - 0.00	0.00 (0/52) 0.00 - 0.00	0
	CS-137 363	6.00E-02	0.00 (0/311) 0.00 - 0.00		0.00 (0/52) 0.00 - 0.00	0.00 (0/52) 0.00 - 0.00	0
	I-131 363	7.00E-02	0.00 (0/311) 0.00 - 0.00		0.00 (0/52) 0.00 - 0.00	0.00 (0/52) 0.00 - 0.00	0

Mean and range based upon detectable measurements only

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Zero range indicates no detectable activity measurements

Environmental Radiological Monitoring Program Summary

Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

Report Period: 01-JAN-2009 to 31-DEC-2009

Medium or Pathway Sampled	Type and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Fraction) Range	Location with Highest Annual Mean Name, Distance, Direction		Control Location Mean (Fraction) Range	No. of Non-Routine Report Meas.
				Location Code	Mean (Fraction) Range		
Drinking Water (pCi/liter)						136 (12.7 mi NNE)	
BALA-140	65	15	0.00 (0/52) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
BETA	65	4	1.81 (50/52) 0.67 - 3.00	119 (7.40 mi SSW)	1.90 (12/13) 1.08 - 2.40	1.81 (13/13) 1.29 - 2.27	0
CO-58	65	15	0.00 (0/52) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
CO-60	65	15	0.00 (0/52) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
CS-134	65	15	0.00 (0/52) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
CS-137	65	18	0.00 (0/52) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
FE-59	65	30	0.00 (0/52) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
H-3	20	2000	700 (16/16) 189 - 1170	101 (3.31 mi E)	1029 (4/4) 922 - 1170	186 (1/4) 186 - 186	0
I-131	65	15	0.00 (0/52) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
MN-54	65	15	0.00 (0/52) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
NB-95	65	15	0.00 (0/52) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
ZN-65	65	30	0.00 (0/52) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
ZR-95	65	15	0.00 (0/52) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

Environmental Radiological Monitoring Program Summary

Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

Report Period: 01-JAN-2009 to 31-DEC-2009

Medium or Pathway Sampled	Type and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Fraction) Range	Location with Highest Annual Mean Name, Distance, Direction		Control Location Mean (Fraction) Range	No. of Non-Routine Report Meas.
				Location Code	Mean (Fraction) Range		
Surface Water (pCi/liter)						135 (11.9 mi N)	
BALA-140	39	15	0.00 (0/26) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
CO-58	39	15	0.00 (0/26) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
CO-60	39	15	0.00 (0/26) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
CS-134	39	15	0.00 (0/26) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
CS-137	39	18	0.00 (0/26) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
FE-59	39	30	0.00 (0/26) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
H-3	12	2000	937 (8/8) 384 - 1610	128 (0.45 mi NE)	1181 (4/4) 904 - 1610	141 (1/4) 141 - 141	0
I-131	39	15	0.00 (0/26) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
MN-54	39	15	0.00 (0/26) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
NB-95	39	15	0.00 (0/26) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
ZN-65	39	30	0.00 (0/26) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0
ZR-95	39	15	0.00 (0/26) 0.00 - 0.00		0.00 (0/13) 0.00 - 0.00	0.00 (0/13) 0.00 - 0.00	0

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

Environmental Radiological Monitoring Program Summary

Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

Report Period: 01-JAN-2009 to 31-DEC-2009

Medium or Pathway Sampled Unit of Measurement	Type and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Fraction) Range	Location with Highest Annual Mean Name, Distance, Direction		Control Location Mean (Fraction) Range	No. of Non-Routine Report Meas.
				Location Code	Mean (Fraction) Range		
Milk (pCi/liter)			NO INDICATOR LOCATION		141 (14.8 mi WNW)		
	BALA-140 26	15	0.00 (0/26) 0.00 - 0.00		0.00 (0/26) 0.00 - 0.00	0.00 (0/26) 0.00 - 0.00	0
	CS-134 26	15	0.00 (0/26) 0.00 - 0.00		0.00 (0/26) 0.00 - 0.00	0.00 (0/26) 0.00 - 0.00	0
	CS-137 26	18	0.00 (0/26) 0.00 - 0.00		0.00 (0/26) 0.00 - 0.00	0.00 (0/26) 0.00 - 0.00	0
	I-131 26	15	0.00 (0/26) 0.00 - 0.00		0.00 (0/26) 0.00 - 0.00	0.00 (0/26) 0.00 - 0.00	0
	LLI-131 26	1	0.00 (0/26) 0.00 - 0.00		0.00 (0/26) 0.00 - 0.00	0.00 (0/26) 0.00 - 0.00	0

Mean and range based upon detectable measurements only

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Environmental Radiological Monitoring Program Summary

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Medium or Pathway Sampled Unit of Measurement	Type and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Fraction) Range	Location with Highest Annual Mean Name, Distance, Direction		Control Location Mean (Fraction) Range	No. of Non-Routine Report Meas.
				Location Code	Mean (Fraction) Range		
Broadleaf Vegetation (pCi/kg-wet)				102 (9.89 mi WNW)			
	CS-134	48	60	0.00 (0/36)	0.00 (0/12)	0.00 (0/12)	0
				0.00 - 0.00	0.00 - 0.00	0.00 - 0.00	
	CS-137	48	80	0.00 (0/36)	0.00 (0/12)	0.00 (0/12)	0
				0.00 - 0.00	0.00 - 0.00	0.00 - 0.00	
	I-131	48	60	0.00 (0/36)	0.00 (0/12)	0.00 (0/12)	0
			0.00 - 0.00	0.00 - 0.00	0.00 - 0.00		

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

Environmental Radiological Monitoring Program Summary

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Medium or Pathway Sampled Unit of Measurement	Type and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Fraction) Range	Location with Highest Annual Mean Name, Distance, Direction		Control Location Mean (Fraction) Range	No. of Non-Routine Report Meas.
				Location Code	Mean (Fraction) Range		
Food Products (pCi/kg-wet)				NO CONTROL LOCATION			
	CS-134	12	60	0.00 (0/12) 0.00 - 0.00	0.00 (0/12) 0.00 - 0.00	0.00 (0/0) 0.00 - 0.00	0
	CS-137	12	80	0.00 (0/12) 0.00 - 0.00	0.00 (0/12) 0.00 - 0.00	0.00 (0/0) 0.00 - 0.00	0
	I-131	12	60	0.00 (0/12) 0.00 - 0.00	0.00 (0/12) 0.00 - 0.00	0.00 (0/0) 0.00 - 0.00	0

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

Environmental Radiological Monitoring Program Summary

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			Mean (Fraction) Range		Location Code	Mean (Fraction) Range		
Fish (pCi/kg-wet)							137 (12.0 mi N)	
CO-58	12	130	0.00 (0/6) 0.00 - 0.00			0.00 (0/6) 0.00 - 0.00	0.00 (0/6) 0.00 - 0.00	0
CO-60	12	130	0.00 (0/6) 0.00 - 0.00			0.00 (0/6) 0.00 - 0.00	0.00 (0/6) 0.00 - 0.00	0
CS-134	12	130	0.00 (0/6) 0.00 - 0.00			0.00 (0/6) 0.00 - 0.00	0.00 (0/6) 0.00 - 0.00	0
CS-137	12	150	17.6 (1/6) 17.6 - 17.6		129 (0.51 mi ENE)	17.6 (1/6) 17.6 - 17.6	17.6 (1/6) 17.6 - 17.6	0
FE-59	12	260	0.00 (0/6) 0.00 - 0.00			0.00 (0/6) 0.00 - 0.00	0.00 (0/6) 0.00 - 0.00	0
MN-54	12	130	0.00 (0/6) 0.00 - 0.00			0.00 (0/6) 0.00 - 0.00	0.00 (0/6) 0.00 - 0.00	0
ZN-65	12	260	0.00 (0/6) 0.00 - 0.00			0.00 (0/6) 0.00 - 0.00	0.00 (0/6) 0.00 - 0.00	0

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

Environmental Radiological Monitoring Program Summary

Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

Report Period: 01-JAN-2009 to 31-DEC-2009

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Fraction) Range	Location with Highest Annual Mean Name, Distance, Direction		Control Location Mean (Fraction) Range	No. of Non-Routine Report Meas.
				Location Code	Mean (Fraction) Range		
Shoreline Sediment (pCi/kg-dry)						137 (12.0 mi N)	
	MN-54	6	0	0.00 (0/4) 0.00 - 0.00	0.00 (0/2) 0.00 - 0.00	0.00 (0/2) 0.00 - 0.00	
	CO-58	6	0	0.00 (0/4) 0.00 - 0.00	0.00 (0/2) 0.00 - 0.00	0.00 (0/2) 0.00 - 0.00	0
	CO-60	6	0	0.00 (0/4) 0.00 - 0.00	0.00 (0/2) 0.00 - 0.00	0.00 (0/2) 0.00 - 0.00	0
	CS-134	6	150	0.00 (0/4) 0.00 - 0.00	0.00 (0/2) 0.00 - 0.00	0.00 (0/2) 0.00 - 0.00	0
	CS-137	6	180	50.8 (2/4) 38.2 - 63.4	130 (0.52 mi SW) 50.8 (2/2) 38.2 - 63.4	0.00 (0/2) 0.00 - 0.00	0

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

If LLD is equal to 0.00, then the LLD is not required by Selected Licensee Commitments

Environmental Radiological Monitoring Program Summary

Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

Report Period: 01-JAN-2009 to 31-DEC-2009

Medium or Pathway Sampled	Type and Total Number of Analyses Performed	Lower Limit of Detection (LLD)	All Indicator Locations Mean (Fraction) Range	Location with Highest Annual Mean Name, Distance, Direction		Control Location Mean (Fraction) Range	No. of Non-Routine Report Meas.
				Location Code	Mean (Fraction) Range		
Direct Radiation TLD (mR/standard quarter)	162	0.00E+00	17.1 (158/158) 9.30 - 30.7	180	27.0 (4/4)	175 (15.5 mi WNW)	0
				(12.7 mi NNE)	23.2 - 30.7	22.8 (4/4) 19.5 - 26.5	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

APPENDIX C

**SAMPLING DEVIATIONS
&
UNAVAILABLE ANALYSES**

APPENDIX C

MCGUIRE NUCLEAR STATION SAMPLING DEVIATIONS & UNAVAILABLE ANALYSES

DEVIATION & UNAVAILABLE REASON CODES			
BF	Blown Fuse	PO	Power Outage
FZ	Sample Frozen	PS	Pump out of service / Undergoing Repair
IW	Inclement Weather	SL	Sample Loss/Lost due to Lab Accident
LC	Line Clog to Sampler	SM	Motor / Rotor Seized
OT	Other	TF	Torn Filter
PI	Power Interrupt	VN	Vandalism
PM	Preventive Maintenance	CN	Construction

C.1 SAMPLING DEVIATIONS

Air Particulate and Air Radioiodines

Location	Scheduled Collection Dates	Actual Collection Dates	Reason Code	Corrective Action
125	7/20 - 7/27/2009	7/20 - 7/27/2009	PI	Power interrupted to sampling equipment during composite period for about 10.9 hours. Suspected cause inclement weather.
195	9/21 - 9/28/2009	9/21 - 9/28/2009	PI	Power interrupted to sampling equipment during composite period for about 8.62 hours due to indeterminate cause.
195	9/28 - 10/5/2009	9/28 - 10/5/2009	PI	Power interrupted to sampling equipment during composite period for about 27.3 hours. Work request 72478 written to investigate power delivery stability.
125	12/7 - 12/14/2009	12/8 - 12/14/2009	PO	Breaker mistakenly turned off by unknown individual. PIP G-09-01255 written. Sample for preceding composite period (11/30 - 12/7/2009) unavailable due to this problem. Breaker labeled, reset, and normal sampling resumed.

Drinking Water

Location	Scheduled Collection Dates	Actual Collection Dates	Reason Code	Corrective Action
119	8/24 - 9/21/2009	8/24 - 9/21/2009	PS	Water plant maintenance work interrupted water supply during composite period from 9/12 - 9/16/2009.

Surface Water

Location	Scheduled Collection Dates	Actual Collection Dates	Reason Code	Corrective Action
128	12/15 - 1/12/2009 1/12 - 2/9/2009	12/15 - 1/12/2009 1/12 - 2/9/2009	PS	Submersible pump malfunction. Work request 64072 written. Grab sample taken 1/12/2009 and combined with composite. Repair work spanned two composite periods. Submersible pump replaced and water flow restored 1/13/2009.
135	2/9 - 3/9/2009 3/9 - 4/6/2009	2/9 - 3/9/2009 3/9 - 4/6/2009	PO	Power outage to site. Work request 65833 written. Grab sample taken 3/9/2009 and combined with composite. Repair work spanned two composite periods. Power restored, sampling resumed 3/10/2009.

C.2 UNAVAILABLE ANALYSES

Air Particulate and Air Radioiodines

Location	Scheduled Collection Dates	Reason Code	Corrective Action
125	11/30 - 12/7/2009	PO	Power interruption due to breaker trip. Insufficient volume collected. Work request 74373 written. Breaker reset did not restore power to sampling equipment. Maintenance determined power supply had been inadvertently turned off in building 7478 at breaker CKT5. PIP G-09-01255 written. Power interruption affected the start of the next scheduled composite period (12/7 - 12/14/2009).

TLD

Location	Scheduled Collection Dates	Reason Code	Corrective Action
158	9/17 - 12/17/2009	CN	TLD missing. 1 st quarter 2010 TLD placed in field.
159	9/17 - 12/17/2009	CN	TLD missing. 1 st quarter 2010 TLD placed in field.

APPENDIX D

ANALYTICAL DEVIATIONS

No Analytical deviations were incurred for the
2009 Radiological Environmental Monitoring Program

APPENDIX E

**RADIOLOGICAL
ENVIRONMENTAL MONITORING
PROGRAM RESULTS**

This appendix includes all of the sample analysis reports generated from each sample medium for 2009. Appendix E is located separately from this report and is permanently archived at the Duke Energy Corporation Environmental Center radiological environmental master file, located at the McGuire Nuclear Station Site in Huntersville, North Carolina.