NRC FORM 374		S NUCL					Page 1 of 9
MATERIALS LICENSE							w 02 420) and Title 40
Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.						ance on statements and re, acquire, possess, and use(s) and at the place(s) th the regulations of the pomic Energy Act of 1954,	
	Licensee			EG			
Louisiana Energy Se	rvices I.I.C.			License Number: SNM-2010, Amendment 35			
Eodiolaria Eriolgy oo					1. 0		
275 Highway 176				Expiration Dat	e. S	See Condition 13	
P.O. Box 1789				Docket No. 70			0
Eunice, New Mexico	88231						
<ol> <li>Source and/or S Nuclear Materia Byproduct Materia</li> </ol>	and/or	7. Ch Fo	emical and/o rm	r Physical	8.	Maximum amou may possess a under this licen	
	nd depleted) hter products	A.1 A.2	Physical: S and Gas Chemical: U $O_2F_2$ , oxid other compo	UF <sub>6</sub> , UF₄, es and	Α.	136,120,000 kg	Ň
isotope U	enriched in -235 up to by weight and daughters	B.1 B.2	Physical: So and Gas Chemical: $UO_2F_2$ , oxid and other co	UF <sub>6</sub> , UF₄, es, metal	В.	545,000 kg	MIS
C. Tc-99, tra isotopes a contamina	and other 🛛 🔍 💎	C.	Any		C.	Amount that ex contamination a consequence o historical feed o Uranium at othe	as a f the of recycled
D. Deleted		D.	Deleted		D.	Deleted	
E. Deleted		Е.	Deleted		E,	Deleted	
F. Deleted		P	Deleted		F.	Deleted	
G. Co-60		G.	Sealed per	§30.32(g)(1)	G.	1.00E+1 uCi	
H. Deleted		Н.	Deleted		Н.	Deleted	
I. Deleted		I.	Deleted		I.	Deleted	

NRC FORM	374A U.S. NUCLE	AR REGU	LATORY COMMISSION			2
				License N SNM-20		r
	MATERIALS LICE SUPPLEMENTARY			Docket or 70-3103		rence Number
	••••			Amend	men	t 35
				•		
J.	Deleted	J.	Deleted		J.	Deleted
К.	Sr-90	K.	Sealed per §30.32	2(g)(1)	K.	5.00E+0 uCi
L.	Deleted	L.	Deleted	G	L.	Deleted
М.	Deleted	М.	Deleted		М.	Deleted
N.	Deleted	N.	Deleted		N.	Deleted
О.	Deleted	Ο.	Deleted		Ο.	Deleted
Ρ.	Deleted	P.	Deleted		Ρ.	Deleted
Q.	Cs-137	Q.	Sealed per §30.32	2(g)(1)	Q.	5.00E+4 uCi
R.	Deleted	R.	Deleted		R.	Deleted
S.	Po-210	S.	Sealed per §30.32	2(g)(1)	S.	1.00E+1 uCi
Т.	Th-230	Т.)	Sealed per §30.32	2(g)(1)	Т.	1.00E+1 uCi
U.	U-232	U.	Sealed per §30.32	2(g)(1)	U.	1.00E+1 uCi
۷.	U-233	V.	Sealed per §30.32	2(g)(1)	V.	1.00E+1 uCi
W.	U-234	W.	Sealed per §30.32	2(g)(1)	W.	1.00E+1 uCi
Х.	U-235	<b>X</b> .	Sealed per §30.32	2(g)(1)	Х.	1.00E+1 uCi
Υ.	U-236	Y	Sealed per §30.32	2(g)(1)	Y.	1.00E+1 uCi
Ζ.	U-238	Ζ.	Sealed per §30.32	2(g)(1)	Z.	1.00E+1 uCi 1.00E+1 uCi . 5.00E+4 uCi
AA.	Am-241	AA.	Sealed per §30.32	2(g)(1)	AA	. 5.00E+4 uCi
BB.	Cf-252	BB.	Sealed per §30.32	2(g)(1)	BB	. 5.00E+2 uCi
CC.	Ce-139	CC.	Sealed per §30.32	2(g)(1)	СС	c.1.00E+1 uCi
DD.	Co-60	DD.	Unsealed per §30.	.32(i)(1)(ii	i) DE	D. 5.00E+0uCi
EE.	Sr-90	EE.	Unsealed per §30.	.32(i)(1)(ii	i) E <mark>r</mark>	<mark>5.00E+0 uCi</mark>
FF.	Cs-137	FF.	Unsealed per §30.	.32 <mark>(i)(</mark> 1)(ii	i) FF	. 1.00E+1 uCi
GG.	Po-210	GG.	Unsealed per §30.	.32(i)(1)(ii	i) G(	G.1.00E+1 uCi

NRC FORM 374A	U.S. NUCLEAR REGULATORI	COMMISSION 3
		License Number SNM-2010
	IALS LICENSE IENTARY SHEET	Docket or Reference Number 70-3103
		Amendment 35
HH. Th-230	HH. Unsea	led per §30.32(i)(1)(ii) HH. 1.00E+1 uCi
II. U-232	II. Unsea	led per §30.32(i)(1)(ii) II. 1.00E+1 uCi
JJ. U-233	JJ. Unsea	ıled per §30.32(i)(1)(ii) JJ. 1.00E+1 uCi
KK. U-234	KK. Unsea	aled per §30.32(i)(1)(ii) KK. 1.00E+1 uCi
LL. U-235	LL. Unsea	led per §30.32(i)(1)(ii) LL. 1.00E+1 uCi
MM. U-236	MM. Unsea	ıled per §30.32(i)(1)(ii) MM.1.00E+1 uCi
NN. U-238	NN. Unsea	ıled per §30.32(i)(1)(ii) NN. 1.00E+1 uCi
OO. Am-241	OO. Unsea	led per §30.32(i)(1)(ii) OO. 5.00E+0 uCi
PP. Ce-139	PP. Unsea	aled per §30.32(i)(1)(ii) PP. 1.00E+1 uCi
<ul> <li>and conditions, or as rev CFR 40.35(f), 10 CFR 5</li> <li>a. Application for Ma</li> <li>b. Safety Analysis Re 2004; September 3</li> <li>February 28, 2006</li> <li>October 12, 2007; 28, 2008; November January 29, 2010; 2010.</li> <li>c. Environmental Re 2004; September 2007; April 10, 200</li> <li>2008; January 23, March 31, 2010.</li> <li>d. Physical Security December 10, 200</li> <li>February 5, 2010,</li> <li>e. Fundamental Nuc 27, 2004; July 30, 2007; November 3 14, 2010.</li> <li>f. Quality Assurance October 23, 2006; 2007; July 31, 200</li> </ul>	uct authorized activities at f vised in accordance with Se 1.22, 10 CFR 70.32, 10 CF terial License, NRC Form 3 eport dated December 12, 30, 2004; April 22, 2005; Ap 5; March 16, 2006; March 2 October 19, 2007; Novem ber 19, 2008; January 23, 2 March 31, 2010; May 2, 2 port dated December 12, 2 30, 2004; April 22, 2005; J 07; July 30, 2007, October 2009; March 5, 2009; Sep Plan dated December 12, 2 04; January 12, 2005; Febr as amended by license co lear Material Control Plan of 2004; October 7, 2004; De 30, 2007; September 4, 200 e Program Description date November 12, 2007; July	lated December 12, 2003, as revised by letters dated February cember 7, 2004; April 22, 2005; October 23, 2006; October 19, 09; and September 24, 2009; January 13, 2010; and January d April 9, 2004, as revised by letter dated April 22, 2005; 30, 2007, October 12, 2007, October 19, 2007; November 12, h 2, 2009; March 5, 2009; September 24, 2009; November 25,

NRC	FORM	374A U.S. NUCLEAR REGULATORY COMMISSION	4
			License Number SNM-2010
		MATERIALS LICENSE	Docket or Reference Number
		SUPPLEMENTARY SHEET	70-3103
			Amendment 35
	g.	Emergency Plan dated December 12, 2003, as revised b April 22, 2005; October 23, 2006; July 30, 2007; October September 4, 2008; September 30, 2008, February 19, 2 2009; November 25, 2009; January 29, 2010; and March	19, 2007; November 2, 2007; March 10, 2008; 009, March 5, 2009; April 16, 2009; September 24,
	h.	Standard Practice Procedure Plan for the Protection of C revised by correspondence dated July 30, 2004; March 1 March 20, 2007; April 27, 2007; July 19, 2007; October 1 May 1, 2008; May 7, 2008; June 26, 2008; July 7, 2008; J 2008; October 6, 2008; October 16, 2008; November 20, March 2, 2009; and December 29, 2009.	lassified Matter dated December 12, 2003, as 6, 2006; November 21, 2006; November 22, 2006; 2, 2007; November 30, 2007; February 4, 2008; August 4, 2008; September 4, 2008; September 5,
	i. j.	Standard Practice Procedure Plan for the Protection of C United States. Location of the National Enrichment Facili dated December 3, 2007; April 21, 2008; July 2, 2008; Oc 2008; March 26, 2009; August 30, 2009; December 11, 2 Information System Security Plan (SSP) for Plant Control	ity dated October 11, 2007, as revised by letters ctober 8, 2008; October 29, 2008; November 26, 009; December 29, 2009; and January 15, 2010.
	k.	revised by letters dated January 29, 2008 and April 2, 20 Movement Plan for Transportation of Classified Centrifug	08.
	I.	Countries and the US dated February 26, 2008, as revise Information System Security Plan for the Hot Acceptance	
	m.	revised by correspondence dated December 12, 2008; M Fundamental Nuclear Material Control Plan Attachment for 2008, as revised by letters dated December 23, 2008; Ja and February 4, 2009.	or the Hot Acceptance Testing, dated December 1,
	n.	Information System Security Plan for the Centrifuge Asse 2008, as revised by correspondence dated December 12 March 13, 2009.	
	0.	Information System Security Plan for the Plant Control an Control Centers dated February 3, 2009; as revised by co 18, 2009.	
	p.	Information System Security Plan for the Classified Plant revised by correspondence dated November 25, 2009.	Management Network dated April 30, 2009; as
	q.	Notwithstanding the commitments in Sections 2.0 and 3.0 identified in Condition 10 to use certified reference standard fulfill the above stated commitments relative to the use of calibration identified in the February 1, 2010, request letter	ards, the licensee shall have until August 1, 2010 to well characterized materials for its instrument
	r.	Information System Security Plan for the High Assurance revised by correspondence dated March 19, 2010.	Guard, LES ISSP 3.0, dated February 1, 2010, as
	S.	Information Security Program Guidelines for the Protectic 26, 2010, as revised by correspondence dated April 15, 2	
11.	readi comp facilit The I	duction of $UF_6$ into any module of the NEF shall not occur in ness and management measures verification review to ver- pliance with the performance requirements of 10 CFR 70.6 y has been constructed and will be operated safely and in icensee shall provide the Commission with 120 days adva- ule of the NEF.	rify that management measures that ensure 1 have been implemented and confirms that the accordance with the requirements of the license.

12. The licensee is hereby granted the special authorizations and exemptions identified in Section 1.2.3.6 of the National Enrichment Facility Safety Evaluation Report, dated June 2005.

NRC	FORM 3	374A U.S. NUCLEAR REGULATORY COMMISSIO	
			License Number SNM-2010
		MATERIALS LICENSE SUPPLEMENTARY SHEET	Docket or Reference Number 70-3103
			Amendment 35
13.	This li	icense will expire 30 years after the date of license issua	nce.
4.		ne disposition of depleted UF <sub>6</sub> , the licensee shall not use ess that results in the production of anhydrous hydrofluor	
15.	b.	The licensee shall provide proof of \$5 million liability ins 30 days prior to the planned date for obtaining possesside pleted or natural uranium hexafluoride The licensee shall provide proof of full liability insurance days prior to the planned date for obtaining feed materia licensee is proposing to provide less than \$300 million of provide, to the NRC for review and approval, an evaluat amounts less than \$300 million, at least 120 days prior to	on of test material (less than or equal to 50 kg) of e, as required under 10 CFR 140.13b, at least 30 al (greater than 50 kg uranium hexafluoride). If the of liability insurance coverage, the licensee shall ion supporting liability insurance coverage in
16.	TEC	The licensee shall provide an updated Decommissioning of the proposed financial assurance instruments to NRC date for obtaining test material (less than or equal to 50 NRC final executed copies of the reviewed financial assoreceipt of test material. In this Decommissioning Fundir for decontamination and decommissioning of the Centric Cylinder Receipt and Dispatch Building, and all other pla amount of the financial assurance instrument shall be u applicable changes to the decommissioning cost estimat	For review at least six months prior to the planned kilograms of uranium hexafluoride), and provide to burance instruments at least 21 days prior to the ng Plan update, the licensee shall provide full funding fuge Test Facility, the Post Mortem Facility, the ant areas where licensed material is used. The pdated to current year dollars and include any
	S	The licensee shall provide an updated Decommissioning of the proposed financial assurance instruments to NRC date for obtaining feed material (greater than 50 kilogran Separations Building Module (SBM) 1001, and provide financial assurance instruments at least 21 days prior to Decommissioning Funding Plan update, the licensee sh decommissioning of SBM 1001 and all other plant areas	c for review at least six months prior to the planned ms of uranium hexafluoride) for initial production in to NRC final executed copies of the reviewed the receipt of feed material. In this all provide full funding for decontamination and
		In addition, the licensee shall provide funding for the dis needed to disposition the first three years of depleted un estimate shall include an update to the U.S. Departmen estimate. The total amount funded for depleted uraniun cost estimate. The amount of the financial assurance in include any applicable changes to the decommissioning	anium tails generation. The decommissioning cost t of Energy (DOE) depleted uranium disposition cost n disposition shall be no less than the updated DOE astrument shall be updated to current year dollars and
		The licensee shall provide an updated Decommissioning of the proposed financial assurance instruments to NRC feed material in SBM 1003, and provide to NRC final ex- instruments at least 21 days prior to introducing feed material Funding Plan update, the licensee shall provide full func SBM 1003 and all other plant areas where licensed material instrument shall be updated to current year dollars and decommissioning cost estimate.	c for review at least six months prior to introducing ecuted copies of the reviewed financial assurance aterial into SBM 1003. In this Decommissioning ling for decontamination and decommissioning of erial is used. The amount of the financial assurance

NRC FO	ORM 374A	
--------	----------	--

## U.S. NUCLEAR REGULATORY COMMISSION

## MATERIALS LICENSE SUPPLEMENTARY SHEET

License Number SNM-2010 Docket or Reference Number

70-3103

Amendment 35

- d. After the first three years of initial plant production, subsequent updated decommissioning cost estimates and revised funding instruments for depleted uranium disposition shall be provided annually on a forwardlooking basis to reflect projections of depleted uranium byproduct generation. The depleted uranium disposition cost estimate shall include an update to the DOE depleted uranium disposition cost estimate. The total amount funded for depleted uranium disposition shall be no less than the updated DOE cost estimate.
- 17. Deleted
- 18. Deleted
- 19. To define the boundaries of each item relied on for safety (IROFS), the licensee shall utilize its procedure, "IROFS Boundary Definitions." Completed IROFS boundaries for all IROFS shall be available for inspection at the time of the operational readiness review.
- 20. Currently, there are no IROFS that have been specified as using software, firmware, microcode, programmable logic controllers, and/or any digital device, including hardware devices which implement data communication protocols (such as fieldbus devices and Local Area Network controllers), etc. Should the design of any IROFS be changed to include any of the preceding features, the licensee shall obtain Commission approval prior to implementing the change(s). The licensee's design change(s) shall adhere to accepted best practices in software and hardware engineering, including software quality assurance controls as discussed in the Quality Assurance Program Description throughout the development process and the applicable guidance of the following industry standards and regulatory guides as specified in Safety Analysis Report Chapter 3:
  - a. American Society of Mechanical Engineers (ASME) NQA-1-1994, Part II, subpart Part 2.7, "Quality Assurance Requirements of Computer Software for Nuclear Facility Applications," as revised by NQA-1a-1995 Addenda of NQA-1-1994 and ASME NQA-1-1994, Part 1, Supplement 11S-2, "Supplementary Requirements for Computer Program Testing." (Refer to SAR Chapter 11, Appendix A, Section 3.)
  - b. Electric Power Research Institute (EPRI) NP-5652, "Guideline for the Utilization of Commercial Grade Items in Nuclear Safety Grade Applications," June 1988.
  - c. EPRI Topical Report (TR) -102323, "Guidelines for Electromagnetic Interference Testing in Power Plants," Revision 1, December 1996.
  - d. EPRI TR-106439, "Guideline on Evaluation and Acceptance of Commercial Grade Digital Equipment for Nuclear Safety Applications," October 1996.
  - e. Regulatory Guide 1.152, "Criteria for Digital Computers in Safety Systems in Nuclear Power Plants," Revision 1, January 1996.
  - f. Regulatory Guide 1.168, "Verification, Validation, Reviews, and Audits for Digital Software Used in Safety Systems of Nuclear Power Plants," Revision 1, February 2004.
  - g. Regulatory Guide 1.169, "Configuration Management Plans for Digital Computer Software Used in Safety Systems of Nuclear Power Plants," September 1997.

6

N		RM 374A U.S. NUCLEAR REGULATORY COMMISSION	7
			License Number SNM-2010
		MATERIALS LICENSE SUPPLEMENTARY SHEET	Docket or Reference Number 70-3103
			Amendment 35
	h.	Regulatory Guide 1.170, "Software Test Documentation Systems of Nuclear Power Plants," September 1997.	for Digital Computer Software Used in Safety
	i.	Regulatory Guide 1.172, "Software Requirements Specif Safety Systems of Nuclear Power Plants," September 19	
	j.	Regulatory Guide 1.173, "Developing Software Life Cycle Safety Systems of Nuclear Power Plants," September 19	
	hu Int	any above changes result in IROFS requiring operator action uman-system interfaces shall be conducted using the applical terface Design Review Guidelines," Revision 2, dated May 20 actors Engineering Program Review Model," Revision 2, date	ble guidance in NUREG-0700, "Human-System 002 (NRC, 2002d), and NUREG-0711, "Human
2	eq cyl lim DL	nsite storage of $DUF_6$ generated at the NEF shall be limited to quivalent amount of uranium stored in other NRC accepted ar linder types) of $DUF_6$ . The generation of any additional DUF nit shall constitute noncompliance with the license. The licen $UF_6$ for onsite storage until this noncompliance is remedied. the NEF in New Mexico other than at the NEF.	nd Department of Transportation ("DOT") certified <sub>6</sub> to be stored onsite by the licensee beyond this see shall suspend production of any additional
2	be sto Th	nsite storage of any one cylinder of $\text{DUF}_6$ generated at the Ni eginning from the date that each cylinder is filled in accordance orage of any one $\text{DUF}_6$ cylinder beyond this limit by the licens he licensee shall suspend production of any additional $\text{DUF}_6$ medied. In no event shall the licensee store $\text{DUF}_6$ generated	ce with the licensee's standard procedures. The see shall constitute noncompliance with the license. for onsite storage until this noncompliance is
2		ne licensee shall provide financial assurance for the offsite dis ontingency factor of twenty-five percent (25%).	sposal of DUF <sub>6</sub> from the NEF using a minimum

Upon reaching 15,727 cylinders of  $DUF_6$  in 48Y cylinders (or the equivalent amount of uranium stored in other NRC accepted and DOT certified cylinder types) in onsite storage, the licensee shall immediately increase the financial assurance to provide a fifty percent (50%) contingency factor for disposition of  $DUF_6$  stored at the NEF unless: (a) an application to construct and operate a deconversion facility outside of New Mexico that is specifically designated to deconvert the  $DUF_6$  stored onsite at the NEF has been docketed by the agency responsible for reviewing the application; (b) an application for such a facility has been approved by the agency responsible for reviewing the application; or (c) the licensee is using another alternate method for removing the  $DUF_6$  stored onsite.

In addition, upon reaching the limit of 15,727 cylinders of  $DUF_6$  in 48Y cylinders (or the equivalent amount of uranium stored in other NRC accepted and DOT certified cylinder types) in onsite storage, the licensee shall immediately increase the financial assurance to provide fifty percent (50%) contingency factor for disposition of  $DUF_6$  stored at NEF if the contingency factor has not already been increased to fifty percent (50%). The contingency factor shall remain at fifty percent (50%) until the number of cylinders stored onsite is reduced to ninety-eight percent (98%) of the 15,727 limit and either: (a) an application to construct and operate a deconversion facility outside of New Mexico that is specifically designated to deconvert the  $DUF_6$  stored onsite at the NEF has been docketed by the agency responsible for reviewing the application; (b) an application for such a

NRC	FORM 374A U.S. NUCLEAR REGULATORY COMMISSION	8
		License Number SNM-2010
	MATERIALS LICENSE SUPPLEMENTARY SHEET	Docket or Reference Number 70-3103
		Amendment 35
	facility has been approved by the agency responsible for review another alternate method for removing the DUF <sub>6</sub> from New Met	
	Nothing herein shall release the licensee from other financial a regulations.	ssurance obligations set forth in applicable laws and
24.	The licensee shall maintain and follow the Fundamental Nuclea accounting and measurement control of uranium source materia to 10 CFR 74.33(b). The licensee shall make no change to ma safeguarding of uranium source material or special nuclear ma material control and accounting program implemented pursuan Commission. If the licensee desires to make changes that wou and accounting program or its measurement control program, to amendment to its license pursuant to 10 CFR 70.34. The licensee shall maintain records of changes to the material Commission approval a period of five years from the date of the	ial and special nuclear material at the NEF pursuant aterial control procedures essential for the terial that would decrease the effectiveness of the t to 10 CFR 74.33(b) without prior approval of the uld decrease the effectiveness of its material control the licensee shall submit an application for control and accounting program made without prior e change. The licensee shall furnish to the Director,
25.	Division of Nuclear Security, Office Nuclear Security and Incide 10 CFR 70.5(a), a report containing a description of each chan uranium enriched less than 20 percent in the uranium-235 isoto If there are any revisions to the nuclear criticality safety validati	ge within six months of the change if it pertains to ope.
	NRC describing the changes and shall provide the revised valid implement the changes in the revised validation report until NR	
26.	The licensee shall not use, process, store, reproduce, transmit provided by applicable personnel and facility clearances as req	
27.	The licensee shall be limited to possession of no greater than s	50 kg of UF <sub>6</sub> in the Centrifuge Assembly Building.
28.	The Licensee is exempted from the definitions of "commercial characteristics," dedicating entity," and "dedication" in 10 CFR	
	Commercial grade item: A commercial grade item means a str affects its IROFS function that was not designed and manufact items do not include items where the design and manufacturing verifications to ensure that defects or failures to comply are ide characteristics of the item cannot be verified).	ured as a basic component. Commercial grade process require in-process inspections and
	Basic component: A basic component means a structure, syste IROFS function, that is directly procured by the licensee or acti- which a defect or failure to comply with any applicable regulation Commission would create a substantial safety hazard (i.e., exc In all cases, basic components include IROFS-related design, a replacement parts, or consulting services that are associated we are performed by the component supplier or others.	vity subject to the regulations in part 70 and in on in this chapter, order, or license issued by the eed performance requirements of 10 CFR 70.61). analysis, inspection, testing, fabrication,

NRC	FORM 374A U.S. NUCLEAR REGULATORY COMMISSIO	v 9
		License Number SNM-2010
	MATERIALS LICENSE SUPPLEMENTARY SHEET	Docket or Reference Number 70-3103
		Amendment 35
	<i>Critical characteristics</i> : Critical characteristics are those impor characteristics of a commercial grade item that, once verified, perform its intended IROFS function.	
	<i>Dedication</i> : Dedication is an acceptance process undertaken a grade item to be used as a basic component will perform its in deemed equivalent to an item designed and manufactured und program. This assurance is achieved by identifying the critical acceptability by inspections, tests, or analyses performed by th delivery, supplemented as necessary by one or more of the fol inspections or witness at holdpoints at the manufacturer's facil performance. In all cases, the dedication process must be cor of 10 CFR Part 50, Appendix B. The process is considered co basic component.	tended IROFS function and, in this respect, is der a 10 CFR 50, Appendix B, quality assurance characteristics of the item and verifying their ne purchaser or third-party dedicating entity after lowing: commercial grade surveys; product ity, and analysis of historical records for acceptable inducted in accordance with the applicable provisions
	Dedicating entity: Dedicating entity means the organization the be performed by the manufacturer of the item, a third-party ded entity, pursuant to Section 21.21(c) of this part, is responsible defects and failure to comply for the dedicated item, and maint In cases where the Licensee applies the commercial grade item process, the Licensee would assume full responsibility as the o	dicating entity, or the licensee itself. The dedicating for identifying and evaluating deviations, reporting aining auditable records of the dedication process. m procurement strategy and performs the dedication dedicating entity.
	Prior to implementing the above commercial grade procureme shall submit a license amendment request to the NRC for appr Description to include its commitments described in its exempt and November 24, 2008.	oval amending its Quality Assurance Program
29.	The licensee shall ensure that:	
	<ul> <li>a. The Emergency Exit and Control Point will be placed in freceipt of feed material on site, to allow for operational to operators to become fully trained and familiar with the op LES will ensure that during the period of reduced Control Standard Practices and Procedures Plan for the protecti</li> <li>c. In accordance with 10 CFR 70.32(e), a revised PSP is s that were suspended by the licensee's February 5, 2010</li> </ul>	esting, operator training, and sufficient time for beration of equipment and procedures; Illed Access Area security, the requirements of the on of classified matter will not be compromised; and ubmitted to reinstate the original security procedures
30.	No changes shall be made, without prior NRC approval, to speresult in modifying the current values for criticality-based analy 3 sections include 3.2.5.2 related to Safe-By Design and Table Chapter 5 sections include 5.0, 5.1.1 through 5.1.5, 5.2.1.2 thr sections contain data and discussions related to safe-by-desig safety parameters, commitments, and the margin of safety for use of absorbers in structural material without strictly adhering SAR and prior review and approval by the NRC.	sis in a less conservative direction. Specific Chapter 3.1-9, "Failure Frequency Index Numbers." Specific ough 5.2.1.7, and Tables 5.1.1 and 5.1-2. The above n, nuclear criticality safety analysis, nuclear criticality subcriticality. Any additional analysis that credits the